Entergy

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Fred Dacimo
Vice President, Operations

September 12, 2003

Re: Indian Point, Unit No. 2
Docket No. 50-247

NL-03-143

Document Control Desk
U.S. Nuclear Regulatory Commission
Mail Stop O-P1-17
Washington, DC 20555-0001

Subject: Monthly Operating Report for August 2003

Dear Sir:

This letter provides the Monthly Operating Report for Indian Point 2 for the month of August 2003, in accordance with Technical Specification 6.9.1.7. There are no commitments contained in this correspondence.

Should you or your staff have any questions regarding this matter, please contact Mr. John McCann, Manager, Licensing, Indian Point Energy Center at (914) 734-5074.

Sincerely,

Fred R. Dacimo
Vice President, Operations
Indian Point Energy Center

IE24

Attachments

cc:

Mr. Hubert J. Miller
Regional Administrator – Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406-1498

Mr. Patrick D. Milano, Project Manager
Project Directorate I
Division of Reactor Projects I/II
U.S. Nuclear Regulatory Commission
Mail Stop O-8-C2
Washington, DC 20555-0001

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Indian Point Unit 2
P.O. Box 38
Buchanan, NY 10511-0038

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Indian Point Unit 3
P.O. Box 337
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Mr. Paul Eddy
State of New York Department of Public Service
3 Empire Plaza
Albany, NY 12223

OPERATING DATA REPORT

DOCKET NO. 50-247
DATE September 5, 2003
COMPLETED BY S. Smith
TELEPHONE (914)736-8304

OPERATING STATUS

1. Unit Name : INDIAN POINT UNIT No. 2
2. Reporting Period : August-2003
3. Licensed Thermal Power (MWt) : 3114.4
4. Nameplate Rating (Gross Mwe) : 1008
5. Design Electrical Rating (Net Mwe) : 993
6. Maximum Dependable Capacity (Gross Mwe) : 988
7. Maximum Dependable Capacity (Net Mwe) : 956

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report , Give Reasons :
9. Power Level To Which Restricted , If Any (Net Mwe) :
10. Reasons For Restrictions , If Any :

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5,831	255,696
12. Number Of Hours Reactor Was Critical	682.18	5,735.86	180,237.48
13. Reactor Reserve Shutdown Hours	0	0	4,566.64
14. Hours Generator On-Line	627.87	5,669.65	176,241.95
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,907,956	17,410,014	497,853,133
17. Gross Electrical Energy Generated (MWH)	602,034	5,684,767	155,763,123
18. Net Electrical Energy Generated (MWH)	577,360	5,490,600	149,270,072
19. Unit Service Factor	84.4	97.2	68.9
20. Unit Availability Factor	84.4	97.2	68.9
21. Unit Capacity Factor (Using MDC Net)	81.2	99.2	65.3
22. Unit Capacity Factor (Using DER Net)	78.1	95.2	63.1
23. Unit Forced Outage Rate	15.6	2.8	13.5

24. Shutdowns Scheduled Over Next 6 Months (Type , Date , and Duration Of Each) :

25. If Shut Down At End Of Report Period , Estimated Date Of Startup :

26. Units In Test Status (Prior to Commercial Operation) :	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-247
UNIT I.P. Unit #2
DATE September 5, 2003
COMPLETED BY S. Smith
TELEPHONE (914)736-8304

MONTH August-2003

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	960	17	414
2	959	18	953
3	161	19	953
4	0	20	955
5	28	21	953
6	882	22	952
7	957	23	956
8	957	24	959
9	957	25	958
10	957	26	959
11	959	27	957
12	958	28	958
13	957	29	959
14	643	30	959
15	0	31	959
16	0		

INSTRUCTIONS

On this format , list the average daily unit power level in Mwe-Net for each day in the reporting month. Compute to the nearest whole megawatt .

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-247
UNIT I.P. Unit #2
DATE September 5, 2003
COMPLETED BY S. Smith
TELEPHONE (914)736-8304

REPORT MONTH August-2003

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	030803	F	59.47	H	3	2003-004	XX	XXXXXX	Reactor trip initiated by a turbine trip (TT) on auto stop oil. The TT was caused by a generator trip on over-frequency due to a loss of electrical load as a result of a lightning strike on the tower of 345 kV feeder W93.
3	030814	F	56.67	H	3	2003-005	XX	XXXXXX	Reactor trip (RT) on low Reactor Coolant Pump (RCP) loop flow due to trip of a RCP breaker as a result of under-frequency due to an unstable power transmission grid. RT initiated a turbine trip.

1
F : Forced
S : Scheduled

2
Reason :
A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & License Examination
F - Administrative
G - Operational Error (Explain)
H - Other (Explain)

3
Method :
1 - Manual
2 - Manual Scram
3 - Automatic Scram
4 - Other (Explain)

4
Exibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exibit I - Same Source

(9/77)

SUMMARY OF OPERATING EXPERIENCE
August 2003

Indian Point Unit No. 2 Nuclear Power Plant was synchronized to the bus for a total of 627.87 hours, producing a gross electrical generation of 602,034 MWH.

On August 3, 2003, at approximately 0430 hours, an automatic reactor trip was initiated by a turbine auto stop oil. The turbine trip was caused by a generator trip on over-frequency due to a loss of electrical load as a result of a lightning strike on the tower of 345 kV feeder W93. Following the reactor trip, the resultant cooldown caused letdown flow to isolate when pressurizer level reached approximately 18% as designed. Since charging flow remained in service, the subsequent increase in pressurizer level and pressure caused the Pressurizer Power Operated Relief Valve (PORV) 455C to actuate automatically to control the pressure transient. The charging/level mismatch after letdown isolation caused a total of ten (10) PORV actuations from 0514 hours to 0526 hours. After letdown was restored, pressurizer level was stabilized and the PORV actuations terminated. The unit was stabilized in the hot shutdown condition.

The reactor was brought critical on August 4, 2003, at 0813 hours, and the unit synchronized to the bus on August 5, 2003, at 1558 hours, with full power achieved at 1230 hours, on August 6, 2003.

On August 14, at approximately 1612 hours, an automatic reactor trip was initiated on low reactor coolant pump (RCP) loop flow. Low RCP loop flow was caused by the trip of a RCP supply breaker as a result of 6.9 kV bus underfrequency due to an unstable power transmission grid. The grid disturbance was associated with a blackout which affected much of the northeastern United States. The unit was stabilized in the hot shutdown condition.

The reactor was brought critical on August 16, 2003, at 0218 hours, and the unit synchronized to the bus on August 17, 2003, at 0052 hours, with full power achieved at 2235 hours on August 17, 2003. The unit remained on line at full power for the remainder of the reporting period.