

Draft Submittal

(Pink Paper)

1. Reactor Operator Written Exam

WATTS BAR EXAM 2002-301

50-390

**NOVEMBER 26 &
DECEMBER 9 - 13, 2002**

QUESTIONS REPORT
for WBfindraft

1. 001A4.03 001/T2G1/T2G1/BORATION/C/A 4.1/3.9/B/WB020301/R/RLM

During a plant heatup, the control rods were withdrawn 5 steps in the BANK-SELECT position to prevent "thermal lock-up" of the rods during the heatup.

Which ONE of the following would result if the control rods were NOT fully inserted using BANK-SELECT prior to withdrawing rods using MANUAL?

- A. Rod upper limit stop malfunction.
- B. ✓ Rod bank overlap malfunction.
- C. Rod bottom lights malfunction.
- D. Rod Position Indication malfunction.

Ref: WB exam bank

QUESTIONS REPORT
for WBfindraft

2. 003A2.02 001/T2/G1/T2/G1/RCP/C/A 3.7/3.9/N/WB02301/R/RLM

- Unit 1 is at 39% power
- Annunciator 100D, RCP Seal Leak Off Flow Hi has alarmed, due to #4 RCP.
- #1 RCP Lower Bearing Temperature is at 220°F and increasing.

Per AOI-24, which ONE of the following describes the correct actions for the control room crew?

- A. Trip the #1 RCP, close #1 Seal Return FCV, trip the Rx and go to E-0, Rx Trip or Safety Injection.
- B. Trip the #1 RCP, close #1 Seal Return FCV and go to GO-5, UNIT SHUTDOWN FROM 30% REACTOR POWER TO HOT STANDBY.
- C. Trip the #1 RCP, close #1 Seal Return FCV and go to AOI-5, Unscheduled Removal of One RCP Below P-8
- D. Trip the #1 RCP, close #1 Seal Return FCV and go to AOI-39, Rapid Load Reduction to remove the Unit from service.

Ref: WB lesson plan 3-OT-AOI2400, obj. 10
WB AOI-24, section 3.2, step 2

Distractor analysis:

A is incorrect because AOI 24, section 3.2, step 2 says to trip Rx IF power >48%, otherwise, trip the RCP

B is incorrect (same as A) and because once Rx is tripped in AOI 24, then E-0 becomes controlling procedure.

C is correct based on AOI 24, section 3.2, step 2 RNO for rising bearing temp directs tripping pump, closing seal leakoff and going to AOI-5

D is incorrect because power is less than P-8, 48% and AOI 24 directs transition to AOI- 5, which after several other actions can direct to AOI-39 or other power reduction procedures.

QUESTIONS REPORT
for WBfindraft

3. 004K4.04 001/T2G1/T2G1/CONTROL TRANSFER/C/A 3.2/3.1/N/WB020301/R/RLM

Unit 1 is recovering from a Main Control Room evacuation in accordance with AOI-27, Main Control Room Inaccessibility and control has been formally transferred from the Auxiliary Control Room.

The UO attempted to perform a blended makeup to the VCT when annunciator 112-E, PW to Blender Flow Deviation alarmed.

Which ONE of the following describes the cause of the alarm?

- A. Transfer switch for Primary Water Pumps left in Aux position
- B. Transfer switch for FCV-62-143, Blender Primary Water Flow Control Valve left in Aux position
- C. Transfer switch for VCT LEVEL XMTR 1-LCV-62-118 left in Aux position
- D. Transfer switch for FCV-62-144, Makeup to CCP Suction Flow Control Valve left in Aux position

Ref: WB procedure AOI-27
No learning objective found

Distractor analysis:

A is incorrect, switch does not exist

B is correct, in Aux position at the Auxiliary Control Room, FCV-143 remains closed regardless of activities on the Main Control Board

C is incorrect because level is not an input when performing a manual makeup

D is incorrect because this switch does not exist

QUESTIONS REPORT

for WBfindraft

4. 005AK3.02 001/T1G1/T1G1//C/A (2.5/2.6)/B/WB02301/R/EL

Given the following plant conditions:

- Reactor power is at 75% with a power rise in progress using control rods.
- The OAC determines that Control Bank D rod H-12 is immovable and is 14 steps below the other rods in Bank D.
- Crew is performing AOI-2, "Malfunction of Reactor Control System"

Which ONE of the following describes how control rod H-12 will be realigned to control bank D and how control bank insertion limit will change following the realignment?

- A. Control Bank D will be realigned to control rod H-12 and control bank D insertion limit will be higher.
- B. ✓ Control Bank D will be realigned to control rod H-12 and control bank D insertion limit will be lower
- C. Control rod H-12 will be realigned to Control Bank D and control bank D insertion limit will be lower.
- D. Control rod H-12 will be realigned to Control Bank D and control bank D insertion limit will be higher.

The correct answer is B

a. incorrect - the method of realignment is correct but the change in insertion limit is in the wrong direction since the bank will be inserted to match H-12 position.

b. correct - control bank D will be inserted to match rod H-12 position and since the bank is being inserted the insertion limit for control bank D will be lower after the bank is aligned with rod H-12.

c. incorrect - the method of realignment is incorrect and the change in insertion limit is wrong since the bank will not move the insertion limit will not change.

d. incorrect - the method of realignment is incorrect and the change in insertion limit is wrong since the bank will not move the insertion limit will not change.

REFERENCE:

AOI-2, section 3.4 p. 23. Lesson plan 3-OT-SYS085A pp. 36-37.

10CFR55.41.5, 10

RO - 1 SRO - 1

QUESTIONS REPORT

for WBfindraft

5. 005K5.03 001/T2G3/T2G3/BORATION/C/A 2.9/3.1/B/WB020301/R/RLM

Unit 1 is in Mode 4 with #1 Reactor Coolant Pump running. "A" train RHR has JUST been placed in service when the Reactor Operator notes source range counts suddenly increasing.

Which ONE of the following describes the correct action?

- A. Commence a maximum boration per AOI-34 Immediate Boration to add negative reactivity.
- B. Start the #2 Reactor Coolant Pump and secure the #1 Reactor Coolant Pump to ensure through mixing of the RCS volume.
- C. Start "B" Train RHR and secure "A" train RHR to reestablish pre-event conditions.
- D. Commence a normal boration to decrease the source range counts.

Ref: TP Exam Bank, no specific learning objective found
Need copy of AOI 34 to verify this question.

Distractor Analysis:

A is correct because AOI provides guidance to immediately borate when uncontrolled +reactivity is inserted.

B is incorrect because swapping RCP's will not add negative reactivity to the core.

C is incorrect same as answer B.

D is incorrect because there is no guidance for a normal boration.

QUESTIONS REPORT
for WBfindraft

6. 008K1.02 001/T2G3/T2G3/CCW LOADS/M 3.3/3.4/B/WB020301/R/RLM

Which ONE of the following correctly describes the uses of CCS in the Reactor Building?

- A. RCP oil coolers, Seal Water Htx. and RCP thermal barriers.
- B. RCP oil coolers, RCP thermal barriers and Regen Htx.
- C. RCP oil coolers, RCP thermal barriers and Non-Regen Htx.
- D. RCP oil coolers, RCP thermal barriers and Excess Letdown Htx.

Ref: WB Lesson Plan 3-OT-SYS070A, obj. 2&15

Dwg. 47W859-2

Distractor analysis:

See attached dwg. Note dwg says Containment, and question call it Rx bldg?

QUESTIONS REPORT

for WBfindraft

7. 009EA2.39 001/T1G2/T1G2/RCP TRIP CRITERIA/M 4.3/4.7/B/WB020301/R/RLM

Which ONE of the following explains why it is preferable to leave the RCP's running during a small break LOCA if the RCS pressure RCP trip criteria on the foldout page are met but there is no SI flow?

- A. To provide heat removal through the break and the S/G's.
- B. To maintain two phase mixture level above the break longer.
- C. To limit single phase inventory loss out of the break.
- D. To prevent boron stratification in the core.

Ref: WB Lesson Plan 3-OT-0100, obj. 3

WB Lesson Plan 3-OT-0100, pp. 14

DC Cook exam bank

A is correct because the pumps will pump two phase (or steam) flow and cool by reflux in S/G's

B is incorrect because the two phase mixture will continue to exit the vessel.

C is incorrect same as B.

D is incorrect because there will be boron stratification in the core due to severe inventory depletion.

QUESTIONS REPORT
for WBfindraft

8. 013K6.01 001/T2G1/T2G1/CHANNEL FAILURE/C/A 2.7/3.1/B/WB020301/R/RLM

Given the following plant conditions:

- Reactor power is 100%
- Pressurizer pressure channel I, 1-PT-68-340, has been removed from service for surveillance testing with it's associated bistables tripped.
- Pressurizer pressure channel IV, 1-PT-68-322, fails low.

Which ONE of the following describes the result of these conditions?

- A. Reactor trip, but NO Safety Injection
Pzr PORV, 1-PCV-68-340 remains closed
- B. Reactor trip and Safety Injection
Pzr PORV, 1-PCV-68-340 remains closed.
- C. Reactor trip but NO Safety Injection
Pzr PORV, 1-PCV-68-340 opens.
- D. Reactor trip and Safety Injection
Pzr PORV, 1-PCV-68-340 opens.

Ref: WB Lesson Plan 3-OT-SYS099A

47W611-63-1

Did not find specific lesson plan on ESFAS.

QUESTIONS REPORT
for WBfindraft

9. 015G2.2.12 001/T2G1/T2G1/SURVEILLANCE/C/A 3.0/3.4/N/WB020301/R/RLM

Unit 1 has been ramping up in power in accordance with GO-4, Normal Power Operation and has stabilized at 53% power to perform 1-SI-92-1, NIS daily comparison.

Results of the surveillance are as follows:

Calorimetric Power - 49%

N-41 indication - 54%

N-42 indication - 52%

N-43 indication - 52%

N-44 indication - 53%

Which ONE of the following describes the required action?

- A. ✓ Shall obtain concurrence from the System 92 System Engineer and adjust all NI's to 49%.
- B. Make no adjustments
- C. Shall obtain concurrence from the System 92 System Engineer and adjust all NI's to 51%.
- D. Adjust N-41 to 51%

Ref: WB procedure 1-SI-92-1

No specific learning objective

This is a daily surveillance, but may want to provide precautions and limitations.

Distractor analysis:

A is correct in that it is permissible to downward adjust NI's to equal calorimetric, but N-41 requires concurrence from the engineer because it's adjustment will exceed 4%

B is incorrect because T/S requires NI's to be + or - 2% or less

C is incorrect because no NI will be adjusted more than 4%

D is incorrect because the other NI's will still be out more than 2%.

QUESTIONS REPORT
for WBfindraft

10. 016K3.02 001/T2G2/T2G2/PZR LEVEL TRANSMITTE/C/A 3.4/3.5/M/WB020301/R/RLM

With Pressurizer Level Control Channel Select switch 1-XS-68-339E selected to LI-68-339 & 335, the following SEQUENTIAL plant events occur due to a failure without operator action.

- Charging flow increases to maximum
- Pressurizer level begins to rise
- Letdown isolates and heaters turn off
- Pressurizer level eventually rises to the high level reactor trip

Which ONE of the following failures occurred?

- A. Level channel 1-LT-68-339 failed high.
- B. Level channel 1-LT-68-335 failed high.
- C. Level channel 1-LT-68-339 failed low.
- D. Level channel 1-LT-68-335 failed low.

Ref: WB lesson plan 3-OT-SYS068C, Obj. 12

Distractor analysis:

A is incorrect because charging "see" the level as high, throttles to minimum. Actual level decreases and channel LT 335 shut letdown valve 62-69 and isolates heaters. With no letdown, even an minimum charging, actual level rises to the high level trip setpoint.

B is incorrect because LT-68-335 can't be selected for level control

C is correct because failing low makes charging increase and level would initially go up rapidly due to letdown also isolating and cause a trip the Rx

D is incorrect same as B

QUESTIONS REPORT
for WBfindraft

11. 022K4.03 001/T2G1/T2G1/ESF ACTUATION/M 3.6/4.0/B/WB020301/R/RLM

Given the following plant conditions:

- Reactor trip & Safety Injection occurred due to Large Break LOCA.
- Containment ØB isolation has occurred.
- All systems responded normally.

Which ONE of the following describes the response of the Lower Compartment Coolers when ØB is reset?

- A. Fans in A-P-AUTO remain off.
Cooler ERCW isolation valves remain closed.
- B. Fans in A-P-AUTO remain off.
Cooler ERCW isolation valves open.
- C. Fans in A-P-AUTO start.
Cooler ERCW isolation valves remain closed.
- D. Fans in A-P-AUTO start .
Cooler ERCW isolation valves open.

Ref: WB Lesson Plan 3-OT-SYS030C, obj. 5
WB Exam Bank

Need licensee to verify this question. Lesson plan a bit vague on this point.

QUESTIONS REPORT
for WBfindraft

12. 051AA2.02 001/T1G1/T1G1//M (3.9/4.1) 43.5/45./B/WB02301/R/EL/KFO

Which ONE of the following combinations requires ONLY the turbine (NOT the reactor) to be tripped per AOI-11, " Loss of Condenser Vacuum"?

- A. Generator load 700 MWe; condenser pressure 5.3" Hga.
- B. Generator load 400 MWe; condenser pressure 5.0" Hga.
- C. ✓ Generator load 300 MWe; condenser pressure 4.0" Hga.
- D. Generator load 200 MWe; condenser pressure 3.0" Hga.

REF WB bank, validated in AOI-11

- A. incorrect - per back pressure limit table in AOI-11
- B. incorrect - per back pressure limit table in AOI-11
- c. correct
- d. incorrect - per back pressure limit table in AOI-11

QUESTIONS REPORT
for WBfindraft

13. 056AK1.01 001/T1G3/T1G3/ES-0.2/C/A 3.7/4.2/N/WB02301/R/RLM

-Unit 1 has experienced a Loss of Offsite Power

-The operating crew is currently performing a cooldown, in accordance with ES-0.2,
Natural Circulation Cooldown

-Prior to initiating the cooldown, two CRDM fans trip

Which ONE of the following describes the maximum allowable cooldown rate?

A. <100°F in any one hour

B. <50°F in any one hour

C. ✓ <25°F in any one hour

D. <10°F in any one hour

Ref: Watts Bar Lesson Plan 3-OT-EOP0000, obj. 15
Watts Bar Lesson Plan 3-OT-EOP0000, pp. 44 of 95

Distractor analysis:

Answer A is incorrect, but is normal max cooldown with forced RCS flow

Answer B is incorrect, but is correct for 3 or more CRDM fans operating (total of 4)

Answer C is correct per ES-0.2, step 7 RNO

Answer D just rounds out the 4

QUESTIONS REPORT
for WBfindraft

14. 057AA2.18 001/T1G1/T1G1/VITAL BUS/M 3.1/3.1/B/WB020301/R/RLM

Given the following:

- Unit 1 at 100% power
- Alarms received indicate a failed electrical board
- Other indications are:
 - All trip status lights out
 - Low seal flow to RCP's due to FCV-62-89 failing open
 - High charging flow due to FCV-62-93 failing open

Which ONE of the following identifies which electrical board that was lost?

- A. 125 V DC Vital Battery Board I
- B. 120 VAC Vital Instrument Power Board 1-I.
- C. 125 V DC Vital Battery Board II
- D. 120 VAC Vital Instrument Power Board 1-II.

Ref: WB Lesson Plan 3-OT-AOI2500, obj. 1
WB procedures AOI-25.1,,2 and 21.1,,2

WB exam bank

Distractor analysis:

See attached procedures which show loads from each of the above boards.

QUESTIONS REPORT
for WBfindraft

15. 059K4.11 001/T2G1/T2G1/FEEDWATER ISOLATION/C/A 3.1/3.3/M/WB020301/R/RLM

After which one of the following events can the feedwater isolation be reset by operating the feedwater isolation reset pushbuttons without performing any other actions?

- A. A spray valve fails open causing pressurizer pressure to drop to 1725 psig. The spray valve is closed and pressure returns to 2235.
- B. ✓ From 10% Rx power, the operator overfeeds a single steam generator to the High-High Level setpoint and subsequently clears the High-High Level by reducing level.
- C. A turbine trip from 65% Rx power, causing a reactor trip. Steam dumps open to control Tavg at 557°F.
- D. A high steam line flow causes a low Tavg and an SI. Main Steam isolation terminates high flow condition and allows Tavg to return to 557°F.

Ref: WB Lesson Plan 3-OT-SYS003A, obj. 13
WB dwg. 1-47W611-3-2

Distractor analysis:

Answers A and D are incorrect because the SI input must also be reset.

Answer C is incorrect because both the FWI switches AND pushbuttons must reset.

Answer B is correct because when only a Hi-Hi S/G level input is present, when it clears, only the pushbutton needs depressing to break the seal-in. See attached logic drawing.

Note: This needs to be discussed with the licensee carefully in that the logic diagram seems to conflict with a statement from the startup procedure (also attached).

Need to verify that HS 3-99A2 on the dwg is the button mentioned in the lesson plan. If it is a switch, the stem will need to be changed.

QUESTIONS REPORT
for WBfindraft

16. 061K5.01R 001/T2G1/T2G1//C/A/M/WB02301/R/KFO

Given the following:

- The Unit 1 reactor tripped from 100% pwr
- All systems responded as designed

The heat transfer rate between RCS and the steam generators will

- A. increase as RCS temperature increases and AFW flow increases.
- B. increase as RCS temperature stabilizes and AFW flow increases.
- C. increase as AFW temperature increases and AFW flow increases.
- D. decrease as AFW temperature stabilizes and AFW flow increases.

Modified from INPO bank -

- a. correct - as the temperature differential increases the heat transfer rate increases
- b. incorrect - as the temperature differential stabilizes the heat transfer rate stabilizes
- c. incorrect - as the temperature differential decreases the heat transfer rate decreases
- d. incorrect - as the temperature differential stabilizes the heat transfer rate stabilizes

QUESTIONS REPORT
for WBfindraft

17. 072A1.01 001/T2G1/T2G1/SFP/M 3.4/3.6/N/WB020301/R/RLM

The following conditions exist:

- Unit 1 is defueled
- Fuel movement activites in progress in the Fuel Pool
- Radiation readings on 0-RM-90-102 and 103, Spent Fuel Pit Area Monitors have been steadily rising.

If the radiation levels continue to rise, which ONE of the following should occur?

- A. Aux Building General Supply and Exhaust Fans and the Fuel Handling Area Exhaust Fans STOP and the ABGTS Fans START
- B. Aux Building General Supply and Exhaust Fans STOP and the Fuel Handling Area Exhaust Fans START
- C. Aux Building General Supply Fans and ABGTS Fans STOP and the Fuel Handling Area Exhaust Fans START
- D. Aux Building General Supply Fans and the Fuel Handling Area Exhaust Fans STOP and ABGTS Fans START

Ref: WB Lesson Plan 3-OT-SYS030B, Obj.13
3-OT-SYS090A, Obj.7

ARI 184-B

Distractor Analysis:

B, C, and D are incorrect because they should occur when the high radiation alarms actuate
(See attached ARI-B)

A is correct because it should NOT occur
(See attached ARI-B)

QUESTIONS REPORT
for WBfindraft

18. 073A2.01 001/T2G2/T2G2/WASTE RELEASE/M 3.1/3.5/N/WB020301/R/RLM

Given the following:

- Unit 1 is at 100% power
- A liquid waste release is in progress
- Annunciator 181-A, WDS RELEASE LINE 0-RM-122 LIQ RAD HI alarms.
- The reading on RM-90-122 has decreased to less than background

Which ONE of the following describes the initial required actions by the control room operators?

- A. Have the detector assembly cleaned.
- B. Notify RADCON to take radiation readings in the detector area.
- C. Verify liquid rad waste release valve RCV - 77 - 43 shut.
- D. Initiate a Maintenance Work Order to repair RM-90-122.

Ref: WB lesson plan 3-OT-SYS090A, obj. 7
ARI 181-A

Distractor analysis:

A is incorrect because there is no guidance to clean the detector although low readings could be the result of fouling.

B is incorrect due to no guidance to confirm that areas readings are suddenly less than typical background

D is correct based on the guidance in the ARI and the philosophy to secure releases to the environment when process monitors fail.

D is incorrect as an initial action although ultimately the detector will need repair at a later time.

***Need to verify term that WB uses for Maintenance Work Order.

QUESTIONS REPORT
for WBfindraft

19. 074EA2.01 001/T1G1/T1G1/SUBCOOLING/C/A/B/WB02301/R/KFO/LRM

Given the following:

- The Unit has tripped from 100% power with a LOCA in progress
- Pzr pressure is 900 psig
- RCPs are tripped
- Core Exit thermocouples indicate 720 degrees F
- RVLIS indicates 36%
- T_{hot} for all loops range between 512 °F and 525 °F

Which of the following describes the conditions existing in the core as applicable to the EOPs?

- A. Subcooled conditions, which present no challenge to the fuel matrix and fuel cladding as long as the hot leg temperatures remain below saturated conditions.
- B. Saturated conditions, which does not present a challenge to the fuel matrix and fuel cladding as long as the cold leg temperatures remain at saturated conditions.
- C. Saturated conditions, which present a potential challenge to the fuel matrix and fuel cladding.
- D✓ Super heated conditions, which present a imminent challenge to the fuel matrix and fuel cladding.

WB bank - validated on steam tables

- a. incorrect - not saturated conditions
- b. incorrect - is super heated but does challenge the fuel matrix and fuel cladding
- c. incorrect - not saturated conditions
- d. correct - superheated conditions that present a challenge to fuel matrix and cladding

QUESTIONS REPORT
for WBfindraft

20. 078G2.4.11 001/T2G3/T2G3/LOSS OF AIR/C/A 3 .4/3.6/B/WB020301/R/RLM

Given the following conditions:

- Plant in Mode 5.
- Total Loss of Control Air has occurred.
- Train "A" RHR in Shutdown Cooling mode.
- The OAC reports that Pressurizer level is 92% and slowly rising.
- CCS is inservice with flow to the RCP thermal barriers.

Which ONE of the following actions should be taken to control Pressurizer level per AOI-10, "Loss of Control Air"?

- A. Locally isolate charging and letdown and place excess letdown in service.
- B. Stop CCPs until PZR level is less than 70% then cycle a CCP on and off to maintain PZR level between 65 and 70%.
- C. Locally Isolate charging until PZR level is less than 70% then adjust then locally operate 1-FCV-62-93 to control charging flow.
- D. Stop CCPs until PZR level is less than 70% then start a CCP to establish seal injection flow.

Ref: WB lesson Plan 3-OT-AOI1000, obj 7
AOI-10

Distractor Analysis:

- A is incorrect, the procedure does not direct isolation of charging, but stopping.
- B is incorrect, no procedural guidance to cycle charging between 65% and 70 %
- C is incorrect, no procedural guidance to locally operate FCV
- D is correct per AOI 10, section 3,4

QUESTIONS REPORT
for WBfindraft

21. 086G2.4.25 001/T2G2/T2G2/PROCEDURE TRANSITION/M 2.9/3.4/B/WB020301/R/RLM

Given the following:

- The Unit is at 100% RTP and at steady state conditions.
- A fire is detected in the Auxiliary Building.
- The SM has determined the fire has the potential to affect equipment necessary to achieve/maintain safe shutdown.
- The US has transitioned to AOI-30.2, Fire Safe Shutdown.
- A Spurious Safety Injection occurs.

Which ONE of the following identifies the required procedure transition for the given conditions?

- A. Perform AOI-30.1, Plant Fires, concurrently with AOI-30.2, Fire Safe Shutdown.
- B. Perform AOI-30.2, Fire Safe Shutdown, while referring to ES-1.1 for SI termination actions.
- C. Perform E-O, Reactor Trip or Safety Injection, then AOI-30.2, Fire Safe Shutdown, when SI termination criteria is met.
- D. Perform AOI-30.2, Fire Safe Shutdown, while referring to other instructions as directed by AOI-30.2

Ref. WB lesson plan 3-OT-AOI3000
AOI 30.2

Distractor analysis:

A is incorrect because AOI-30.1 is a non-Appendix R fire procedure and the SM has determined it to be an Appendix R fire.

B is incorrect because AOI 30.2 takes precedence over EOP's.

C is incorrect same as B

D is correct because it is permissible to run AOI-30.2 concurrently with normal plant instructions.

QUESTIONS REPORT
for WBfindraft

22. 103A2.03 001/T2G3/T2G2/PHASE B/M 3.5/3.8/N/WB020301/R/RLM

- Unit 1 has Safety Injected
- Phase A isolation has occurred
- The crew is performing step 11 of E-0, Rx Trip or Safety Injection when the UO notes that Containment Pressure is 3.2 psig and Phase B has not actuated.

Which ONE of the following describes the action(s) required?

- A. Manually actuate either Phase B manual handswitches and manually start the Containment Spray pumps.
- B. Manually actuate both Train A and B Phase B manual handswitches and manually start the Containment Spray pumps.
- C. Manually actuate either Phase B manual handswitches and manually shut the MSIV's and bypasses.
- D. Manually actuate both Train A and B Phase B manual handswitches and manually shut the MSIV's and bypasses.

Ref: WB lesson plan 3-OT-SYS088A, obj. 9
WB dwg. 1-47W-611-88-1

Distractor analysis:

A and C are incorrect because Phase B manual actuation required BOTH switches actuated simultaneously.

B is incorrect because spray pumps start as a result of manual actuation (see logic dwg)

D is correct because both switches are actuated and MSIV's only isolate as a result of input from Containment Pressure instruments.

QUESTIONS REPORT
for WBfindraft

23. WE02EA1.2 001/T1G2/T1G1//C/A (3.4/4.0)/B/WB02301/R/EL

Given the following conditions:

- Plant is recovering from a reactor trip and safety injection due to a faulted SG.
- Faulted SG has completely blown down and as been isolated in accordance with the appropriate EOP.
- RCS pressure is 1900 psig and rising.

When terminating safety injection which ONE of the following identifies actions that result in a drop in the ECCS flow to the RCS?

- 1) Stopping the 1st charging pump.
- 2) Stopping both safety injection pumps.
- 3) Establishing normal charging and isolating BIT.
- 4) Stopping RHR pumps.

A. 1 and 3.

B. 3 and 4.

C. 1 and 2.

D. 2 and 4.

The correct answer is A.

- a. Correct - both these actions would result in reducing flow to the core since at this pressure only CCPs would inject.
- b. Incorrect - 3 would reduce flow, however RHR pumps are not injecting at this pressure.
- c. Incorrect - 1 would reduce flow, however the SI pumps are not injecting at this pressure.
- d. Incorrect - neither would reduce flow at this pressure.

Reference: SYS063A; EOP0000.18
K/A E02 EK1.1 [3.2/3.8]

QUESTIONS REPORT
for WBfindraft

24. WE08G2.4.18 001/T1G1/T1G1/PTS/M 3.4/4.2/B/WB020301/R/RLM

Step 1 of FR-P.1, "Pressurized Thermal Shock", has the operator check that RCS pressure is greater than 150 psig.

This step is based on:

- A. Preventing implementation of actions in FR-P.1 if a large break LOCA has occurred.
- B. Ensuring adequate low head safety injection cooling prior to isolating CLAs.
- C. Preventing core exit temperatures from exceeding the required temperature to place RHR in service.
- D. Ensuring RHR system is in service to provide adequate mixing in the cold leg downcomer region.

Ref: WB Lesson Plan 3-OT-FRP0001, obj. 9
WB exam bank
Answer A is correct, see attached lesson plan.

QUESTIONS REPORT
for WBfindraft

25. WE13EK3.2 001/T1G3/T1G3/OVERPRESSURE/C/A 2.9/3.3/B/WB020301/R/RLM

During the performance of FR-H.2, "Steam Generator Overpressure", operators are directed to verify T-hot less than 545°F if initial attempts to depressurize the SG(s) is unsuccessful.

Which ONE of the following is bases for this verification?

- A. Ensures that the SG level instruments (WR) will be within the required accuracy during the depressurization.
- B. Determines whether SG blowdown can be used as a method to depressurize the affected SG(s).
- C. Ensures RCS saturation pressure is below shutoff head of SIPs in the event SGs become a heat SOURCE instead of a heat SINK.
- D. Ensures excessive heat transfer from the RCS is NOT the cause for the SG overpressure.

Ref: WB Lesson Plan 3-OT-FRH0001 obj 13
WB Lesson Plan 3-OT-FRH0001 pp 44 of 56

Source WB exam bank

Distractor analysis:

Answer D is correct based on lesson plan page 44

QUESTIONS REPORT
for WBfindraft

1. 001AK3.01 001/T1G2/T1G1//C/A (3.2/3.6)/B/WB02301/C/EL

Given the following plant conditions:

- Plant at 65% power.
- Impulse pressure transmitter, 1-PT-1-73, fails high causing rods to step at 72 steps per minute.
- Rods were placed in MANUAL causing rod motion to stop 14 steps from their original position.

Which ONE of the following describes the appropriate course of action?

- A. Ensure turbine load remains constant and dilute to restore original rod position.
- B. Reduce turbine load to compensate for the rise in reactor power.
- C. Withdraw rods manually to restore Tavg to program.
- D. Insert rods manually to drop Tavg to program.

The correct answer is

- a. Incorrect - Diluting would further increase the difference between Tavg and Tref if turbine load were held constant.
- b. Incorrect - Turbine load would need to be increased to offset the increase in T-avg due to rods stepping out from the NI failure.
- c. Incorrect - withdrawing rods would further increase the difference between Tavg and Tref.
- d. Correct - Tavg would be elevated due to the rods stepping out on the PT failure. Inserting rods manually to decrease Tavg to program is appropriate.

QUESTIONS REPORT

for WBfindraft

2. 001K1.05 001/T2G1/T2G1/NI FAILURE/C/A/N/WB02301/C/KFO

Given the following:

- Unit power is 66%
- Control Rods are in Automatic
- No operator actions

Which ONE of the following would cause rods move in , then remain at the new position with a reduced T-avg.

- A. Loop A Th fails low
- B. PT 1- 73 fails high
- C. Loop A Tc fails low
- D. Pwr channel NI-41 fails high

New, REF: 3-OT-SYS85A

- a. incorrect - rod control circuit uses auct High Tave - failing low does not make this the auct hi
- b. incorrect - this channel of turbine pressure inputs the control rod logic and the Tref program is topped out at 588.2
- c. incorrect - rod control circuit uses auct High Tave - failing low does not make this the auct hi
- d. correct - Rods move in until the power mismatch rate signal decays, then remain at the new position with a reduced T-avg.

QUESTIONS REPORT
for WBfindraft

3. 002K5.10 001/T2G2/T2G2/TEMPERATURE MISMATCH/C/A 3.6/4.1/N/WB020301/C/RLM

Unit 1 is at 94% power and ramping up.

Rods are in automatic with Bank D at 210 steps

The operator has met all requirements to increase turbine load and has pressed the turbine control GO button

Turbine control valves are opening and megawatts are increasing

Tavg is in program

Which ONE of the following describes Tavg behavior assuming no dilution by the operator?

- A. Tavg and Tref will increase and continue to be matched until the control valves reach the limiter setting.
- B. Tref will increase until the control valves reach the limiter setting, but Tavg will remain constant.
- C. Tavg and Tref will remain constant and matched as the control valves reach the limiter setting.
- D. Tref will increase and Tavg will initially increase, but then Tavg will decrease as the control valves reach the limiter setting.

Ref: WB lesson plant 3-OT-SYS085A, obj 7 & 21

Distactor analysis:

D is correct because Bank D hits the C11 rod stop at 220 steps. As the governor valves continue to open, turbine power continues to rise along with Pimp (Tref). However, no more positive reactivity is added (no dilution per stem) and no rods due to C11, but power defect adds negative reactivity which drives Tavg down until Moderator temp coefficient balances at a lower Tavg.

A, B, and C are incorrect they have Tavg either rising or remaining constant.

QUESTIONS REPORT
for WBfindraft

4. 004K5.04 001/T2G1/T2G1/OXYGEN/M (2.8/3.2)/B/WB02301/C/KFO

Which one of the following is the primary reason for maintaining a hydrogen cover gas in the VCT during normal at power operations?

- A. to assure N-16 concentrations are ALARA
- B. to maintain RCS pH within TRM limits
- C. to maintain conductivity below .017 micro Mhos
- D. to maintain oxygen concentration below the TRM upper limit*

Ref: NA 2002 - validated to WB Lesson plan 3-OT-SYS062A, TRM 3.4.4 Chemistry

- a. incorrect N-16 is a bi-product of the nuclear reaction not influenced by hydrogen cover gases
- b. incorrect RCS pH is controlled by primary chemistry not with cover gases
- c. incorrect conductivity is controlled by water quality efforts not cover gases
- d. correct Hydrogen cover gases "scavenge" oxygen, thus lowering the oxygen concentration

QUESTIONS REPORT
for WBfindraft

5. 006A1.11 001/T2G2/T2G2//M (3.1/3.4)/B/WB02301/C/EL/KFO

Given the following plant conditions:

- A SGTR has occurred on SG #1.
- ES-3.1, "Post SGTR Cooldown Using Backfill", is in progress.
- Ruptured SG level is 25%.
- RCS is at 390°F.
- RCS is at 400 psig.
- Cooling down using steam dumps to condenser.
- RCP #4 in service.

ES-3.1 step 13 requires a return to step 3 if RCS temperature is greater than 200°F.
Step 3 requires the operator to ensure adequate shutdown margin.

Why is it necessary to reverify adequate shutdown margin at this point in the procedure?

- A. The RCS temperature change during cooldown will cause significant boron concentration changes due to PZR outsurge.
- B. Charging to maintain PZR level during cooldown will cause significant boron concentration changes.
- C. The secondary fluid in the ruptured SG will cause significant boron concentration changes.
- D. The auxiliary spray will cause significant boron concentration changes.

The correct answer is C

QUESTIONS REPORT

for WBfindraft

6. 006K1.07 001/T2G2/T2G2//C/A (2.6/2.9)/B/WB02301/C/EL/KFO

Given the following conditions:

- The plant is operating at 100% power.
- Inadvertent Safety Injection occurred
- #1 SG level transmitter that controls MDAFW pump LCV failed low.
- #4 SG PORV opened momentarily after the reactor trip and developed a large packing leak.

Which ONE of the following would cause the initial main feedwater isolation during this transient?

- A. The #1 SG level reached 83%.
- B. The safety injection actuation signal.
- C. Tavg dropping to 564°F following the reactor trip.
- D. When the south valve vault level had risen to 4 inches due to the PORV packing leak.

The correct answer is B.

- a. Incorrect - level may reach 83% but FWI would already have been actuated by the SI.
- b. Correct - SI causes immediate FWI.
- c. Incorrect - would normally actuate FWI following a reactor trip however the SI initiated the FWI immediately.
- d. Incorrect - PORV leak would cause increase in level in the vault room however FWI would have already been actuated by the SI.

QUESTIONS REPORT
for WBfindraft

7. 008AK1.01 001/T1G2/T1G2//C/A (3.2/3.7)/B/WB02301/C/KFO

Given the following:

- Unit 1 is stable at 100% power
- A pressurizer safety valve opens and fails to reseal and the Unit trips

Which of the following indications would the operator expect to see as a result of this event over the next 30 min?

- A. Safety tailpipe temperature would increase to greater than 600 F and then slowly decrease.
- B. Safety tailpipe temperature would increase to greater than 600 F and then slowly increase.
- C. Safety tailpipe temperature would increase to between 220 F and 340 F and then slowly decrease and stabilize.
- D. Safety tailpipe temperature would increase to between 220 F and 340 F and then slowly increase and stabilize.

Ref- Farley 2000, validated for WB -

- a. incorrect - the temperature is correct for pressures of 2240
- b. incorrect - the temperature is correct for pressures of 2240
- c. correct - since it relieves to the PRT, the pressure will increase until the rupture disc relieves (100 psig) and then the pressure (and temperature) will decrease and eventually stabilize
- d. incorrect - will not continue to increase once the PRT rupture disc relieves.

QUESTIONS REPORT
for WBfindraft

8. 010K3.01 001/T2G2/T2G2/PRESS CONTROL/C/A 3.8/3.9/B/WB020301/C/RLM

Given the following:

- Pressurizer pressure is 2230 psig and rising.
- The variable heaters are energized.
- The spray valves are closed.
- The Master Pressure Controller, 1-PIC-68-340A, fails to a constant output equivalent to 2230 psig.

Which ONE of the following describes the response of the pressure control system if the operator takes no further action?

- A. Pressure will rise until PORV's PCV-68-334 and PCV-68-340A open to control pressure.
- B. ✓ Pressure will rise until relief valve PCV-68-334 opens to control pressure.
- C. Pressure will rise until the spray valves open to control pressure.
- D. Pressure will cycle on the variable heaters at a higher setpoint.

Ref: WB lesson plan 3-OT-SYS068C

Distractor analysis:

A is incorrect because PORV continues to receive a constant input and therefore remains closed.

B is correct because PORV 334 receives input from PT 323 which is seeing the actual pressure rise.

C is incorrect because the spray valves remain closed because their input is not changing from the master controller and is spray open setpoint.

D is incorrect because the input to the variable heaters is constant at a value less than their shutoff point.

QUESTIONS REPORT

for WBfindraft

9. 011A3.03 001/T2G2/T2G2/PZR LEVEL/C/A 3.2/3.3/B/WB020301/C/EL/RLM

Given the following:

- Unit is at 50% power.
- All automatic control systems are in their normal lineup.
- Controlling pressurizer program level fails to an output corresponding to 50% load.
- Assume no operator action is taken.

Which of the following describes the effect on charging flow and PZR level as the plant load is increased to 100%?

- A. Charging flow increases and actual PZR level remains constant.
- B. Charging flow decreases and actual PZR level decreases.
- C. Charging flow decreases and actual PZR level increases.
- D. Charging flow remains constant and actual PZR level increases.

Reference: WB Lesson Plan 3-OT-SYS068C Obj. #14

Distractor analysis:

A is incorrect because Tav increases as power increases which will make PZR actual level increase.

B is incorrect, same reason as A

C is correct because as PZR level rises with coolant expansion due to Tav increase, with LT-339 output at 50%, an error is generated that PZR level is too high, causing charging flow to decrease.

D is incorrect, same as A for PZR level, charging flow increases, see C.

Note, WB exam bank had answer C as "Charging flow rises and actual PZR level rises." Need to check this with them.

QUESTIONS REPORT

for WBfindraft

10. 011EA1.04 001/T1G2/T1G1//C/A (4.4/4.4)/B/WB02301/C/EL

Given the following plant conditions:

- Unit 1 tripped due to a Large Break LOCA.
- Containment pressure = 12.5 psid.
- RWST level = 20%.
- Containment Emergency Sump level = 15%.
- RHR Swapover to the Containment Sump could not be performed.
- The operating crew has transitioned to ECA - 1.1, "Loss of RHR Sump Recirculation."
- The crew is performing step 3 of ECA - 1.1, "Loss of RHR Sump Recirculation", to determine the proper Containment Spray pump alignment and operation.

Which ONE of the following actions will result in the Containment Spray pumps being in the proper alignment under the existing plant conditions?

- A. Leave both Containment Spray pumps running until RWST level drops to 8%.
- B. Stop both Containment Spray pumps and place handswitches in "pull-to-lock."
- C. Stop one Containment Spray pump and allow the remaining pump to take suction from the RWST.
- D. Stop both Containment Spray pumps, until suction can be aligned to the Containment Sump, then restart one pump.

QUESTIONS REPORT
for WBfindraft

11. 011EK2.02 001/T1G2/T1G1//C/A (2.6/2.7)/B/WB02301/C/EL

Given the following conditions:

- A large break LOCA occurred
- Operators have just completed swapover to Containment Sump
- A loss of offsite power occurs

Which ONE of the following describes the actions required for this condition?

- A. Pull to lock SI pumps and CCPs until the RHR pumps have been restarted after the shutdown boards are reenergized.
- B. ✓ Pull to lock the CCPs until the RHR pumps are restarted after the shutdown boards are reenergized.
- C. Ensure both RHR pumps are started by the blackout sequencer after the diesel generators reenergize the shutdown boards then restart the SI pumps.
- D. Ensure all ECCS pumps are started by the blackout sequencer when the diesel generators reenergize the shutdown boards.

The correct answer is B

QUESTIONS REPORT

for WBfindraft

12. 012K6.10 001/T2G2/T2T2/P-7/M3.3/3.5/N/WB020301/C/RLM

Unit 1 is at 100% power

Permissive 70-D, P-7 LO POWER TRIPS BLOCKED illuminates

Which ONE of the following describes the effects on RPS?

- A. The reactor will not trip on Pressurizer High Pressure.
- B. The reactor will not trip on Pressurizer Low Water Level.
- C✓ The reactor will not trip on Pressurizer Low Pressure.
- D. The reactor will not trip on Loss of Flow in one loop.

Ref: WB Lesson Plan 3-OT-SYS099A, obj. 17

Distractor analysis:

A is incorrect because P-7 is not an input to the trip.

B is incorrect because PZR Lo Water Level is not a trip.

C is correct, because P-7 blocks it when P-7 is off (light on).

D is incorrect because P-8 is unaffected and trips Rx on 1/4 logic.

QUESTIONS REPORT
for WBfindraft

13. 013K2.01 001/T2G1/T2G1/PUMP TRIP LOGIC/M (3.6/3.8)/B/WB02301/C/KFO

Given the following:

- 1A-A and 1B-B SI pump breakers are "racked in"
- A fuse blows in the NORMAL DC Trip Circuit for the 1A-A SI pump
- A safety injection (SI) acutation occurs

Which of the following describes the response of the SI pumps to the SI signal?

- A. 1B-B SI pump will start, but 1A-A SI pump will not auto start until the control power supply is transferred.
- B. 1B-B SI pump will start, but 1A-A SI pump will not auto start and must be started from MCR handswitch.
- C. Both SI pumps will auto start, but the 1A-A SI pump can not be stopped from the MCR.
- D. Both SI pumps will auto start, but the 1A-A SI pump can not be stopped from the mechanically at the breaker.

Watts Bar exam bank - NEED CONFIRMATION ON BREAKER LOGIC

QUESTIONS REPORT

for WBfindraft

14. 014A4.01 001/T2G2/T2G1/STARTUP/C/A (3.3/3.4)/B/WB02301/C/KFO

Given the following:

- Operators are preparing for a reactor startup
- Both shutdown banks were withdrawn
- All control banks are still fully inserted
- The rod control startup reset switch is manipulated in error

Which of the following describes the required actions to proceed with the startup?

- A. Restore the P/A converter to 230 steps
- B. ✓ Restore the shutdown group step counters to 230 steps
- C. Restore the bank overlap unit to 230 steps
- D. Reinsert all shutdown banks

INPO bank - validated for WB lesson plan 3-OT-SYS85A

- a. *incorrect* - P/A converter does not need to be reset because the control banks are fully inserted
- b. *correct* - all group step counters are reset to 0 by the reset switch
- c. *incorrect* - bank overlap counters are at 0 because the control rods are fully inserted
- d. the shutdown group counters can be manually reset - reinsertion is not required

QUESTIONS REPORT
for WBfindraft

15. 015K4.06 001/T2G1/T2G1//M/B/WB02301/C/KFO

While operating at 90% power, one power range channel of nuclear instrumentation Power Range High Flux Reactor Trip is placed in bypass.

What is the coincidence for a NIS reactor trip?

- A. 2 out of 4
- B. 1 out of 3
- C. 2 out of 4
- D. 1 out of 3

INPO bank

- a. incorrect - coincidence changes to 1 out of three when in bypass
- b. correct - coincidence changes to 1 out of 3
- c. incorrect - wrong logic and bypass limited to 4 hours
- d. incorrect - bypass status limited to 4 hours

QUESTIONS REPORT

for WBfindraft

16. 017AK1.04 001/T1G1/T1G1/LOSS OF FLOW/C/A (3.7/4.1)/B/WB02301/C/RLM

Given the following plant conditions:

- Unit is operating at 30% power.
- All control systems are in AUTO.
- Loop 3 Reactor Coolant Pump has just tripped.

Which ONE of the following is the overall plant response?

- A. Reactor trips on HIGH steam generator level when the #3 steam generator level "swells".
- B. ✓ Unit power remains the same with steam flow rising on the other steam generators.
- C. Unit power is reduced to approximately 22% power (3/4 of original power level).
- D. The reactor trips on a Low RCS Flow condition.

WB bank

QUESTIONS REPORT

for WBfindraft

17. 017K3.01 001/T2G1/T2G1/FAILED CORE EXIT/M/B/WB02301/C/KFO

If all core-exit thermocouples are inoperable during an event in which the RCPs were tripped, what indication(s) may be used to verify that natural circulation cooling is occurring?

- A. RCS hot leg temperatures only
- B. RCS cold leg temperatures only
- C. Both RCS cold leg and hot leg temperatures
- D. There are no direct indications to verify natural circulation for this condition.

Bank (NA'02) - validated for Watts Bar, ES-01, "Reactor Trip Response

- a. incorrect - insufficient information
- b. incorrect - insufficient information
- c. correct - the difference in temperatures will be adequate to determine if natural circulation has been established
- d. incorrect, using both hot and cold legs is adequate

QUESTIONS REPORT

for WBfindraft

18. 022AG2.1.32 001/T1G1/T1G1/LOW PRESSURE LETDOWN/M 3.4/3.8/N/WB020301/C/RLM

During water Solid operations with letdown from RHR, SOI-62.01 requires that FCV-62-83, RHR LETDOWN FLOW CONTROL be full open.

Which ONE of the following describes the basis for this precaution?

- A. To ensure maximum letdown flow rate for purification.
- B. To ensure VCT level can be maintained under all charging flow conditions.
- C. To ensure PCV-62-81, Letdown Pressure Control can control pressure transients.
- D. To ensure RCS to RHR Supply Line Relief Valve (74-505) isn't challenged.

No specific learning objective found.

Distractor analysis:

Answer A is incorrect because charging flow controls letdown flow.

Answer B is incorrect because balancing charging and letdown controls VCT level.

Answer C is correct because with FCV-62-83 less than full open, it can in effect limit flow and prevent pressure reduction when PCV-62-81 fully opens in response to a high pressure transient.

Answer D is incorrect because the suction relief can be challenged by other factors (eg. pump starts) even with FCV-62-83 full open.

QUESTIONS REPORT

for WBfindraft

19. 022G2.1.10 001/T2G1/T2G1/CTMT AIR TEMP/M 2.7/3.9/M/WB020301/C/RLM

Which ONE of the following correctly identifies the minimum and maximum allowable temperatures per Tech Spec for the Containment upper compartment temperature in Mode 2.

- A. Between 85°F and 120°F.
- B. Between 60°F and 110°F.
- C. Between 60°F and 120°F.
- D. Between 85°F and 110°F.

Ref: WB lesson plan 3-OT-SYS030C, obj 3

WB TS 3.6.5

Modified from WB exam bank which asked MODE 1 limits.

Distractor analysis:

See attached TS

QUESTIONS REPORT
for WBfindraft

20. 024AK2.01 001/T1G1/T1G1//M (2.7/2.7)/B/WB02301/C/EL

A condition has occurred which warrants entry into AOI-34, "Immediate Boration". While attempting to borate using the immediate emergency boration flowpath, the aligned boric acid pump fails to start.

Which ONE of the following Immediate Boration flowpaths should be used in this condition?

- A. To the suction of the CCPs through Emergency Borate valve 1-FCV-62-138 after re-aligning the boric acid pump on tank recirc.
- B. To the suction of the CCPs through Manual Boration valve 1-ISV-62-929 after re-aligning the boric acid pump on tank recirc.
- C. To the suction of the CCPs through 1-FCV-62-135 and 136 using the RWST as the boration source.
- D. To the discharge of the CCPs through 1-FCV-63-125 and 126 using the BIT as the boration source.

The correct answer is C

QUESTIONS REPORT

for WBfindraft

21. 025A3.02 001/T2G1/T2G1//C/A/B/WB02301/C/KFO

Given the following:

- Large Break LOCA has occurred causing a Reactor Trip and Safety Injection
- OAC observes annunciator "Glycol Exp Tank Level HI/HI-Hi" is lit

Which of the following describes the most likely reason the annunciator is illuminated?

- A. Phase A closes the glycol containment isolation valves; glycol inside containment heats up and relief valves on the auxiliary building side of the isolation valves relieve glycol into the glycol expansion tank.
- B. ✓ Phase A closes the glycol containment isolation valves; glycol inside containment heats up and expands into the glycol expansion tank.
- C. Phase B closes the glycol containment isolation valves; glycol inside containment heats up and relief valves on the auxiliary building side of the isolation valves relieve glycol into the glycol expansion tank.
- D. Phase B closes the glycol containment isolation valves; glycol inside containment heats up and expands into the glycol expansion tank.

Bank, lesson plan OT-SYS061A, drawing 1-47-814-2

In the event of a LOCA, the glycol heats up from approximately -5 F to the containment accident temperature and expands harmlessly into the expansion tank. The liquid trapped between a pair of isolation valves is relieved around the inner isolation valve through a bypass line via a small check valve. The bypass line also contains test connections for periodic leak testing of the isolation valves and check valve.

- a. incorrect - no relief valves feedback to the expansion tank
- b. correct
- c. incorrect - no relief valves feedback to the expansion tank
- d. incorrect - phase B does not close these valves

QUESTIONS REPORT

for WBfindraft

22. 025AA2.07 001/T1G2/T1G2/MIDLOOP/C/A (3.2/3.2)/B/WB02301/C/KFO

Given the following:

- RCS temperature is 118 F
- Reactor Vessel head is removed
- Reactor Upper Internals are installed in the reactor vessel
- Refueling Level is 716.8 ft.
- RCS draining is in process at 10 gpm
- RHR pump A is running with indicated flow of 2200 gpm
- RHR pump A begins to exhibit indications of cavitation

The cavitation and subsequent loss of RHR heat removal is occurring due to

- A. draining with the upper internals in place, which reduced the RHR suction pressure.
- B. steam binding of the RHR pump, caused by low recirculation flow.
- C. air entrapment at the RHR suction inlet, caused by the high flow conditions.
- D. draining with the upper internals in place, which reduced the RHR discharge pressure.

Ref. Bank - WB verification: SOI-74.01, AOI-14, GO.10

- a. incorrect, upper internal installation will not effect RHR suction pressure.
- b. incorrect, recirculation flow valve shuts at 14000gpm
- c. correct, air entrapment occurs at higher flow rates, normal midloop flow rate is 2000
- d. incorrect, upper internals installed will not effect RHR discharge pressure to this extent

QUESTIONS REPORT
for WBfindraft

23. 025G2.1.27 001/T2G1/T2G1/PURPOSE OF ICE COND/M 2.8/2.9/M/WB020301/C/RLM

Which ONE of the following is a purpose of the Ice Condenser System?

- A. Helps limit peak clad temperature of the fuel by maintaining a minimum back pressure on the RCS during blowdown phase.
- B. Uses sodium tetraborate to remove elemental hydrogen from the containment atmosphere.
- C. Absorbs (thermal) energy released during LOCA to control the peak pressure in containment.
- D. Uses sodium hydroxide to remove elemental iodine from the containment atmosphere.

Ref: WB lesson plan 3-OT-SYS061A, obj 2

Distractor analysis:

QUESTIONS REPORT
for WBfindraft

24. 026A1.01 001/T2G2/T2G1/DESIGN/C/A/M/WB02301/C/RLM

A large break LOCA has occurred

- Containment Pressure is 1.6 psig and rising
- One High Containment Pressure bistable has illuminated

Which ONE of the following describes the correct operator response to prevent Containment Pressure from exceeding design limits?

- A. Continue to closely monitor Containment Pressure and the High Containment Pressure bistable status.
- B. Manually actuate HS-30-64A, Phase B Actuation Switch on M-6, and HS-30-68A, Phase B Actuation Switch on M-5, simultaneously.
- C. Manually start and align Train A and Train B Containment Spray Systems.
- D. Manually actuate HS-30-64A, Phase B Actuation Switch on M-6, and HS-30-64B, Phase B Actuation Switch on M-6, simultaneously.

WB bank-modified

QUESTIONS REPORT
for WBfindraft

25. 026AK3.03 001/T1G1/T1G1/AOI 15/C/A (4.0/4.2)/B/WB02301/C/RLM

Given the following:

- Unit is at 100% power.
- "A" Train CCS Surge tank level is dropping due to a leak.
- US has entered AOI-15, Loss of Component Cooling Water.

Which ONE of the following describes required actions in the event "A" Train CCS surge tank level cannot be maintained.

- A. Transfer "A" Train CCS suction to the "B" Train side of the surge tank, isolate non ESF header.
- B. Shift Thermal Barrier Booster pump suction to "B" train, shutdown affected "A" Train components, start "B" Train components as necessary.
- C. Stop the thermal barrier booster pumps, trip the Reactor and stop all RCPs.
- D. Shutdown running "A" Train components and place in auto, start "B" Train components as necessary.

WB Bank

QUESTIONS REPORT

for WBfindraft

26. 027AK3.03 001/T1G1/T1G2/PORV/C/A 3.7/4.1/N/WB020301/C/RLM

Unit 1 is at 100% power.

Annunciator 89A, PZR PORV LINE TEMP HI, illuminates.

Both PORV's indicate closed.

Pressurizer pressure is normal.

In accordance with AOI-18, Malfunction of Pressurizer Pressure Control System, which ONE of the following is the correct action and reason?

- A. Close the associated block valve because a vapor-space leak causes PZR level to increase.
- B. Close both block valves because a vapor-space leak causes PZR level to increase.
- C✓ Close the associated block valve because the PRT will rupture.
- D. Close both block valves and remove power because the PRT will rupture.

Ref: WB lesson plan 3-OT-AOI1800, obj. 1 & 5

WB procedures AOI-18, AOI-6, ARI 89-A

Disaster analysis:

Answer A and B are incorrect because PZR level rise is only associated with large leaks that affect PZR pressure.

Answer C is correct because it follows the ARI guidance and small leaks can raise PRT pressure to the rupture disc setpoint.

Answer D is incorrect because power should not be removed.

QUESTIONS REPORT
for WBfindraft

27. 029EA1.13 001/T1G2/T1G1/TURBINE TRIP/M 4.1/3.9/B/WB020301/C/RLM

Which ONE of the following is the NEXT action the operator is required to take if the main turbine does NOT trip automatically and CANNOT be tripped from the MCR: per FR-S.1, "RESPONSE TO NUCLEAR POWER GENERATION/ATWS"?

- A. Place both EHC pumps control switches in P-T-L.
- B. Trip the turbine locally at the front standard.
- C. Manually RUNBACK the turbine.
- D. Shut the MSIV's.

WB lesson plan 3-OT-FRS-0001, Obj. 3
WB exam bank

Distractor analysis:

A is incorrect because it is a followup action later in the procedure, step 9 RNO.

B is incorrect because it is a local action if MCR actions fail, step 9 RNO.

C is correct per RNO step 2

D is also in RNO step 2, but only occurs if runback doesn't work.

QUESTIONS REPORT
for WBfindraft

28. 029K1.02 001/T2G2/T2G2/RAD MONITOR/M 3.3/3.6/N/WB020301/C/RLM

Which ONE of the following describes the sample path of RE-90-130, Cntmt Purge Exhaust, and effect of the alarm exceeding the alarm setpoint?

- A. RE-90-130 samples the Cntmt Purge Exhaust discharge and initiates a Containment Purge trip upon alarm.
- B. RE-90-130 samples the Cntmt Purge Exhaust suction and initiates a Containment Vent Isolation upon alarm.
- C. RE-90-130 samples the Cntmt Purge Exhaust discharge and initiates a Containment Vent Isolation upon alarm.
- D. RE-90-130 samples the Cntmt Purge Exhaust suction and initiates a Containment Purge trip upon alarm.

Ref: WB Lesson Plan 3-OT-SYS030C, obj. 10
WB Lesson Plan 3-OT-SYS090A, pp. 63
SOI 88.1, Checklist #3
Dwg. 1-47W866-1

QUESTIONS REPORT
for WBfindraft

29. 032AK1.01 001/T1G2/T1G2/POWER FAILURE/C/A/M/WB02301/C/KFO

Given the following conditions:

- Reactor Startup in progress
- Shutdown Banks withdrawn
- Control Bank withdrawal is imminent
- SRM N132 indicates approximately 1000 cps
- SRM N131 is in bypass

Which ONE of the following will occur if the control power fuse for SRM N131 blows?

- A. Lose indication for SRM N131 on Main Control Board and NIS cabinets
- B. Both SRM drawers deenergizes and "non-operate" alarm acuates
- C. Reactor Trips
- D. Rod withdrawal is blocked

Modified from WB bank - rewrote question stem and changed distractor. Validated in Lesson Plan SYS092A

- a. incorrect-not all indicatipon lost since instrument power is available
- b. incorrect-SRM N132 unaffected by loss of control power to SRM N131
- c. correct- loss of control power deenergizes bistables and initiates trip signal (1/2 logic)
- d. incorrect - Source range low does not initiate rod stop

QUESTIONS REPORT

for WBfindraft

30. 034K4.01 001/T2G3/T2G2/LOSS OF AIR/M (2.6/3.4)/B/WB02301/C/KFO

Which one of the following describes a feature of the Refueling Machine designed to prevent the accidental release of a fuel assembly?

- A. The Gripper is mechanically engaged and disengaged by a remote operating handle on the bridge and requires no power or air to operate.
- B.✓ The gripper requires air to disengage, however, a mechanical latch prevents gripper release under load even if air is supplied.
- C. The gripper disengages upon loss of air, however, a mechanical latch prevents gripper release under load even if air is removed.
- D. When the gripper is engaged, operators mechanically lock gripper in place with extension shaft which must be unlocked before the gripper can release.

Ref: lesson 3-OT-SYS079 (LO 11)

- a. incorrect - air required to disengage
- b. correct - mechanical latch on gripper works under load
- c. incorrect - engages on loss of air
- d. incorrect - no operator action required for gripper mechanical latch to operate

QUESTIONS REPORT

for WBfindraft

31. 035K1.09 001/T2G2/T2G2/TRANSIENT RESPONSE/C/A 3.8/4.0/B/WB020301/C/EL/RLM

Given the following plant conditions:

- The reactor is operating at 50% power.
- Rod control is in MANUAL.
- Turbine control is in IMP-IN.
- #3 S/G PORV fails OPEN.

Which ONE of the following describes the resulting steady-state conditions?
(Assume no reactor trip or operator action and turbine power remains constant)

- A. Final $T_{avg} < \text{initial } T_{avg}$ and final power $>$ initial power.
- B. Final $T_{avg} < \text{initial } T_{avg}$ and final power = initial power.
- C. Final $T_{avg} = \text{initial } T_{avg}$ and final power $>$ initial power.
- D. Final $T_{avg} = \text{initial } T_{avg}$ and final power = initial power.

Reference: TAA011, General theory section

Distractor analysis:

A is correct steam loss through PORV causes T_{avg} decrease which adds positive reactivity which causes power to rise. T_{avg} will remain less than initial T_{avg} because some of the reactivity is used to overcome power defect associated with power rise.

B, C, and D are incorrect because they conflict with the above correct answer.

Note: Since IMP-IN is given, do we need to tell them that turbine power remains constant?

QUESTIONS REPORT
for WBfindraft

32. 037AK3.07 001/T1G2/T1G2/TUBE LEAK/M 4.2/4.4/B/WB020301/C/RLM

Given the following:

- A SG tube leak of approximately 30 gpm has been identified on SG #3.
- The operating crew has entered AOI-33, Steam Generator Tube Leak.
- Operators have completed Step 21 of the AOI and have isolated feed flow to SG #3 after verifying level greater than 10%.

Which ONE of the following is a basis for ensuring the affected SG level greater than 10%?

- A. To ensure that the pressure and temperature limits of the SG shell are maintained.
- B. To prevent the RCS cooldown from causing depressurization of the affected SG.
- C. To prevent SG overfill.
- D. To prevent thermal shock to the tubes during RCS cooldown.

Ref: WB Lesson Plan 3-OT-AOI3300, obj. 8

A is incorrect in that these limits apply to CSD conditions

B is correct because the insulating layer of water above the tubesheet helps trap pressure in the S/G and minimize tube d/p during cooldown.

C is incorrect because level control is not an issue.

D is incorrect because there is no sudden introduction of cold water after the level is attained.

QUESTIONS REPORT
for WBfindraft

33. 039A1.03 001/T2G2/T2G2/MS WARMUP/C/A 2.6/2.7/B/WB020301/C/EL/RLM

Given the following plant conditions:

- Startup in progress.
- Operators are warming the main steam lines using the MSIV bypasses.
- The OAC observes that the RCS has cooled down 108°F in the past hour.
- The CRO observes that the main steam lines have heated up 102°F in the past hour.

Which ONE of the following indicates the actions that should be taken by the operators and why?

- A. Close the MSIV bypass valves; RCS cooldown limit was exceeded.
- B. Close the MSIV bypass valves; main steam line heat-up limit was exceeded.
- C. Close the MSIV bypass valves; both RCS and main steam line limits were exceeded.
- D. Leave MSIV bypass valves open; NO RCS or main steam line limits were exceeded.

Ref: WB Exam Bank
WB Lesson Plan 3-OT-SYS001A , obj 24

- a. Correct - RCS cooldown limit of 100°F/hr was exceeded; main steam line limit of 200°F/hr was NOT exceeded. Closing the bypass valves would terminate the cooldown
- b. Incorrect - main steam line limit was not exceeded. Examinee could confuse the RCS and main steam line limits.
- c. Incorrect - only the RCS cooldown limit was exceeded. Examinee may believe the limits were the same. Closing the bypass valve would terminate the cooldown.
- d. Incorrect - examinee could confuse the limits and believe that neither limit was exceeded, however the RCS cooldown limit was exceeded and closing the bypass valve would terminate the cooldown.

QUESTIONS REPORT

for WBfindraft

34. 040AG2.4.6 001/T1G1/T1G1//C/A (4.0/4.3)/B/WB02301/C/EL

Given the following plant conditions:

- Unit was at 100% power.
- A main steam line break occurred in the Turbine Building.
- Operators were unable to close the MSIVs and transitioned to ECA-2.1, Uncontrolled Depressurization of All Steam Generators.
- SI termination steps are in progress.
- Loop 3 MSIV is closed locally.
- The CRO observes the #3 SG pressure rising slowly.

Which ONE of the following actions should be performed?

- A. Transition to E-2, "Faulted SG Isolation".
- B. Transition to ES-1.1, "SI Termination".
- C. Remain in ECA-2.1 until RHR is in service.
- D. ✓ Remain in ECA-2.1 until SI is terminated.

The correct answer is D

QUESTIONS REPORT
for WBfindraft

35. 041K3.04 001/T2G3/T2G3/STEAM DUMP RX POWER/C/A3.5/3.4/B/WB020301/C/EL/RLM

Given the following conditions:

- Unit at 100% power, EOL conditions.
- Turbine operating in IMP OUT.
- A steam dump valve inadvertently comes full open.
- All other control systems normal.

Which ONE of the following correctly describes the plant conditions, when plant stabilizes, and assuming NO operator action?

- A. Megawatts electrical same as initial; reactor power rises.
- B. Megawatts electrical same as initial; reactor power drops.
- C. Megawatts electrical drops; reactor power rises.
- D. Megawatts electrical drops; reactor power drops.

Ref: WB Exam Bank

WB Lesson Plan 3-OT-SYS001B, obj 27

WB Lesson Plan 3-OT-SYS001B, pp. 24

QUESTIONS REPORT

for WBfindraft

36. 045A3.05 001/T2G3/T2G3/GOVERNOR VALVE/C/A3.5/3.6/B/WB020301/C/EL/RLM

Given the following plant conditions:

- Plant operating at 100% power.
- All systems aligned normal.
- Turbine EHC control is in IMP OUT.

Which ONE of the following describes turbine governor valve operation while in IMP OUT mode?

- A. Governor valve position is adjusted to maintain generator load constant and will NOT respond to changes in system frequency.
- B. Governor valve position is adjusted to maintain generator load constant and will respond to changes in system frequency.
- C. Maintains governor valves at a set reference position and will NOT respond to changes in system frequency.
- D. Maintains governor valves at a set reference position but will respond to changes in system frequency.

Ref: WB Exam Bank

WB Lesson Plan OT-3- SYS047A Obj. 2

Lesson plan pp. 12 and 18

Info on pp. 12 and 18 seem to conflict, need to follow up w/ site.

- a. Incorrect - in the IMP OUT mode governor valves adjust to maintain load and system frequency.
- b. Incorrect - in the IMP OUT mode governor valves adjust to maintain load and system frequency.
- c. Incorrect - governor valves are set at a reference set in by the operator, but WILL change in response to system frequency changes
- d. Correct - governor valves are set at a reference set in by the operator and WILL change in response to system frequency changes.

QUESTIONS REPORT

for WBfindraft

37. 054AG2.4.48 001/T1G2/T1G2/SG LEVEL/C/A 3.5/3.8/N/WB020301/C/RLM

Unit 1 is at 100% power.

Annunciator 63-F, SG LEVEL DEVIATION, alarms and the operator notices S/G #1 level rising along with MFP speed and the remaining S/G levels constant.

Which ONE of the follow describes the (1) cause, (2) required action and (3) consequence of no operator action?

- A. (1) #1 S/G FRV is opening, (2) return #1 S/G level to program, (3) Auto Turbine Runback initiated
- B. (1) #1 S/G FRV is opening, (2) return #1 S/G level to program, (3) Feedwater Isolation initiated
- C. (1) MFP master controller failing high, (2) control MFP speed using manual, (3) Auto Turbine Trip initiated
- D. (1) MFP master controller failing high, (2) manually trip turbine, (3) Feedwater Isolation initiated

Ref: WB 3-AOI-1600

Note: electronic copy of AOI 16 corrupt

Distractor analysis:

A is incorrect because the consequence is a P-14, Hi-Hi SG level which initiates a FW isolation.

B is correct because only one SG is rising, hence a FRV is the cause, action is from AOI-16 for FRV failure and FW isolation occurs with no operator action at P-14.

C is incorrect because a problem with the MFP controller would affect all SG's the same.

D is incorrect same as C.

QUESTIONS REPORT
for WBfindraft

38. 055EK3.02 001/T1G1/T1G1//M (4.3/4.6)/B/WB02301/C/EL/KFO

Which ONE of the following is a purpose for depressurizing all intact SGs to 300 psig during the performance of ECA-0.0, "Loss of Shutdown Power"?

- A. Reduces DP across SG U-tubes to minimize possibility of tube rupture.
- B.✓ Reduces DP across RCP seals to minimize leakage and loss of RCS inventory.
- C. Maximizes Natural Circulation flow before Reflux cooling begins as the RCS becomes saturated.
- D. Maximizes Natural Circulation flow to allow reactor vessel head to cool since CRDM are unavailable.

WB bank, validated with ECA-0.0

QUESTIONS REPORT
for WBfindraft

39. 055K3.01 001/T2G2/T2G2/VACUUM PUMP/C/A 2.5/2.7/B/WB020301/C/RLM

Given the following conditions:

- Reactor power is steady-state at 100%.
- Rod control is in automatic.
- Condenser Vacuum Pumps 1A and 1B are running with 1C pump out of service for maintenance.

Which ONE of the following conditions could result if NO operator action is taken in response to a COND VACUUM PUMP 1B SEAL WTR PRESS LOW alarm?

- A. Rising megawatt output and rising steam seal header pressure.
- B. Rising megawatt output and rising condenser hotwell level.
- C. Dropping megawatt output and dropping condensate temperature.
- D. Dropping megawatt output and rising condensate temperature.

WB Lesson plan 3-OT-SYS1100, obj 4

Distractor analysis:

Degraded vacuum causes megawatt output to decrease due to reduce delta H. Therefore, A and B are incorrect.

Condenser is a saturated system, so higher P_{sat} means higher T_{sat} . Therefore D is correct and C is incorrect.

QUESTIONS REPORT

for WBfindraft

40. 056K1.03 001/T2G1/T2G1/FEED PUMP TRIP/C/A 2.6/2.6/N/WB020301/C/RLM

Unit 1 is at 72% power. A control problem causes 2-FCV-210, Condensate Inlet to 1A MFPT Condenser to close.

Which ONE of the following describes the effect on continued plant operation?

- A. 1A MFPT rolls to idle, the Standby MFP starts and Rx power can remain 72%
- B. ✓ 1A MFPT trips, the Standby MFP starts and Rx power can remain 72%.
- C. 1A MFPT trips, the Standby MFP does not start and Rx power must be reduced.
- D. 1A MFPT rolls to idle, the Standby MFP does not start and Rx power must be reduced.

Ref: 3-OT-SYS002A, obj. 15
3-OT-SYS003A, obj. 5 & 7

Distractor analysis:

A and D are incorrect, because the 1A MFP trips on low condenser vacuum

C is incorrect because the standby feed pump auto starts

B is correct, because the standby feed pump starts and is able to carry ~15% turbine power and the 1B MFP can carry 67%, the sum of which is above 72%.

QUESTIONS REPORT

for WBfindraft

41. 059AA1.01 001/T1G2/T1G1/RAD FLUID LEAKAGE/M 3.5/3.5/N/WB020301/C/RLM

-Unit 1 is operating at 50% power

-Annunciator 178-A, SG BLDN 1-RM-120/121 LIQ RAD HI alarms

Which ONE of the following correctly describes plant response and the actions of the control room operators?

- A. No automatic action and operators verify alarm validity.
- B. Operators verify SG Blowdown is isolated to Cooling Tower Blowdown and aligned to Condensate Demin.
- C. Operators verify Auxiliary Bldg Rad Waste discharge header is isolated from the blowdown line.
- D. Operators verify Cooling Tower blowdown flow is bypassed to the 35 acre pond.

Ref: WB Lesson Plan 3-OT-AOI31000, obj. 2 & 3
ARI 178-A

Distractor analysis:

A is incorrect because RM 120/121 have auto actions

B is correct base on ARI 178-A

C is incorrect the Aux Bldg in not involved in SGBD stream

D is incorrect because this action would not prevent an offsite release.

QUESTIONS REPORT
for WBfindraft

42. 059K6.09 001/T2G1/T2G1/SPEED CONTROL/C/A 2.4/2.6/N/WB020301/C/RLM

Unit 1 is at 60% power with both 1A and 1B MFP's operating in AUTO.

PT-1-33, Steam Header Pressure, output begins to slowly drift low.
Assume no operator action.

Which ONE of the following describes the effect on the Main Feed Water System?

- A. Feed Water Reg valves begin to close and then both MFP's discharge pressure begins to increase.
- B. Both MFP's discharge pressure begins to increase and all then Feed Water Reg valves begin to open.
- C. Both MFP's discharge pressure begins to decrease and all then Feed Water Reg valves begin to open.
- D. Feed Water Reg valves begin to close and then both MFP's discharge pressure begins to decrease.

Ref: WB Lesson Plan 3-OT-SYS0003A, obj. 6
WB Dwg. 47W611-03-02

Distractor analysis:

C is correct because input from d/p program remains constant because steam flow remains constant. However, PT-1-33 failing low, causes the d/p actual (as sensed) to increase above program. This in turn causes the speed summer to decrease its output to the speed control station which will reduce feedpump speed and its discharge pressure. Flow rate will decrease and the FRV's will open to increase flow to the S/G's.

All other answer's are incorrect because they are variations of the answer with one parameter going in the wrong direction.

QUESTIONS REPORT
for WBfindraft

43. 061A1.04 001/T2G1/T2G1/CST/C/A/M/WB02301/C/KFO

Which ONE of the following describes the base assumptions for minimum CST volume for a Loss of Offsite Power?

- A. 2 hours in Mode 3 followed by 50 °F/hr cooldown rate to Mode 5.
- B. Immediately cooldown at 50 °F/hr cooldown rate to Mode 4 .
- C. 2 hours in Mode 3 followed by a 50 °F/hr cooldown rate to Mode 4.
- D. Immediately cooldown at 50 °F/hr cooldown rate to Mode 5.

Modified from Byron - validated from WB AFW System description N3-3B-4002

- a. incorrect - capacity is for a 2 hour hold then cool to mode 4
- b. incorrect - capacity for 2 hours and then cool to mode 4
- c. correct - stand pipe in CST ensures 2 hours hold and then 50 F/hr cooldown to mode 4
- d. incorrect - capacity to hold for 2 hours with 50 F/hr cooldown to mode 4

QUESTIONS REPORT
for WBfindraft

44. 061AA1.01 001/T1G2/T1G2//M/N/WB02301/C/KFO

Which ONE of the following area radiation monitors initiate a Containment Ventillation Isolation?

- A. Waste Disposal System Gas Effluent Monitor, 0-RM-90-118
- B. Containment Purge Rad Monitor, 1RM-90-130
- C. Fuel Pool Area Monitor, 0-RM-90-102
- D. Personnel Access Area Monitor, RM-90-2

New - Lesson plan SYS90A -

- a. incorrect - hi rad closes the vent header isol
- b. correct
- c. incorrect - isolates the aux building, does not generate an ABI
- d. incorrect - no auto actions

QUESTIONS REPORT
for WBfindraft

45. 062A3.05 001/T2G2/T2G2/LOSP LOOP/C/A 3.5/3.6/B/WB020301/C/RLM

Given that the following occurred in sequence:

- A small break LOCA occurred which resulted in a reactor trip and SI.
- The SI signal was reset during the performance of E-1, "Loss of Reactor or Secondary Coolant."
- A loss of offsite power (LOOP) occurred and the diesel generators loaded as designed.

Assuming no operator actions, which ONE of the following would be the status of the loads on the 6.9kV SD boards?

- A. All equipment powered from the SD boards with the control board switch in automatic will be restarted.
- B. No 6.9kV SD board loads are automatically restarted.
- C. Equipment normally started during a LOSP will be automatically restarted; SI and RHR pumps remain OFF.
- D. All equipment that was operating prior to the LOSP will be automatically restarted; All running ESF equipment will be reenergized

Reference: WB Exam Bank (inadequate info to verify, missing lesson plans on EDG loading.)

QUESTIONS REPORT
for WBfindraft

46. 062AG2.4.24 001/T1G1/T1G1//M/B/WB02301/C/EL/KFO

Which one of the following correctly identifies the emergency cooling water supply to the control and station service air compressors, per AOI-13 "Loss of ERCW", if normal ERCW supply is lost?

- A. Raw Cooling Water header
- B. Raw Service Water header
- C. Unit 1 Primary Water header
- D. Demin Water header

Ref - AOI-13, 47w850-1, 47w845-5

- a. incorrect - RCW does not support the compressors
- b. correct - RSW / Fire header connection in emergency
- c. incorrect - no ERCW connection to support compressors
- d. incorrect -

QUESTIONS REPORT
for WBfindraft

47. 063K3.02 001/T2G2/T2G1/CONTROL POWER/C/A 3.5/3.7/N/WB020301/C/RLM

Unit 1 was at 100% power

DG1A was running parallel to the 6.9kV Shutdown Board for surveillance.

In addition to the expected post trip annunciators, the operator notes annunciators:

17-A 125 DC VITAL CHGR/BATT I ABNORMAL

17-B 125 DC VITAL BATT BD I ABNORMAL CKTS
ISOLATED

Which ONE of the following describes the response to this event by DG1A?

- A. DG1A trips and its output breaker opens
- B. DG1A continues to run and it's output breaker opens
- C. DG1A trips and its output breaker remains closed
- D. DG1A continues to run and its output breaker remains closed

Ref: WB lesson plan 3-OT-AOI2100, obj. 1 & 2
3-OT-SYS057P, obj. 8

WB Dwg's
1-45W724-1
1-45W727
1-45W760-82-2
1-45W760-211-4

Distractor analysis:

A is incorrect because the DG can run using its own DC control power, but the output breaker has not control power to trip.

B is incorrect because the output breaker has no control power to trip.

C is incorrect because the DG does no trip

D is correct because the DG has its own control power and the output breaker remains closed due to no control power.

QUESTIONS REPORT
for WBfindraft

48. 064K2.03 001/T2G2/T2G2/D/G CONTROL POWER/M 3.2/3.6/N/WB020301/C/RLM

Unit 1 is at 100% power
All Diesel Generators are in standby
Annunciator 281-A, DG CONTROL POWER FAILURE for DG1A

Which ONE of the following describes the status of the Diesel Generators AFTER the annunciator alarms?

- A. DG1A starts and is operable.
- B. DG1A does not start and is inoperable.
- C. All four diesel generators start and are operable.
- D. All four diesel generators do not start and are inoperable.

Ref: WB lesson plan 3-OT-SYS082B, obj. 10

ARI 281-A

Dwg 1-45W727

1-45W760-82-2

1-45W760-82-4

1-45W760-82-6

Distractor analysis:

A is incorrect because the starting air solenoids need power from the Diesel 125V DC Distribution Panel 1-DPL-82-A-A which is deenergized as evidenced by the annunciator 281-A.
B is correct because the starting air solenoids have no power and cannot respond to an auto start signal
C and D are incorrect because they continue to have control power and all 125V vital battery boards are still energized.

QUESTIONS REPORT

for WBfindraft

49. 065AA1.01 001/T1G3/T1G2//C/A/N/WB02301/C/RLM

Unit 1 is at 100% power

- Annunciator 42D, C&SS AIR COMPR SEQUENCER UNDERVOLTAGE
- Control Air pressure is slowly decreasing

Which ONE of the following describes actions for locally controlling air pressure?

- A. Take manual control of the Foxboro 43AP pneumatic controller
- B. Shift AC control power
- C. Start and stop Control Air Compressors
- D. Take manual control at the local control station using the solenoid manual valve

New - ref: Watts Bar ARI 42-E and lesson plan 3-OT-SYS032A, obj 11

Distractor analysis:

A incorrect because controller has no power

B incorrect because control power is DC, but shifting control power takes precedence over taking manual control of unloader

C is incorrect due to no procedural guidance and undesirable from equipment standpoint

D is correct IAW ARI

QUESTIONS REPORT
for WBfindraft

50. 067AK1.02 001/T1G1/T1G1/FIRE FIGHTING/M (3.1/3.9)/M/WB02301/C/EL

Which one of the following is the most effective method of fire fighting in the 5th DG Oil Pump Room?

- A. Auto actuation of sprinkler system
- B. Auto actuation of fog system
- C. Auto actuation of foam system
- D. Auto actuation of CO2 system

REF: Modified from Watts Bar Exam Bank SYS026A.21

QUESTIONS REPORT
for WBfindraft

51. 068AK3.18 001/T1G1/T1G1/EVACUATION/M/N/WB02301/C/KFO

Evacuation of the Control Room is required due to a control room fire. The actions of AOI-30.2, "Main Control Room Inaccessibility", prior to evacuating the control room include which of the following?

- A. Tripping the feedwater pumps
- B. Deenergizing the RCP start busses and Electrical Control Board
- C. Initiating main steam isolation
- D. Tripping unrequired diesel generators

Ref: AOI-30.2 Fire Safe Shutdown, appendix C steps 1 - 8,

- a. incorrect, feedwater pump status is not addressed prior to evacuation
- c. incorrect, ensure the MSIVs are closed but does not direct initiating a main steam isolation signal
- d. incorrect, addresses ensuring the 6.9 bus energized but does not address tripping any unnecessary DGs

QUESTIONS REPORT
for WBfindraft

52. 068K1.07 001/T2G1/T2G1/RAD WASTE/M 2.7/2.9/N/WB020301/C/RLM

Which one of the following describe how Incore Intrumentation Seal Table leakage enters the Liquid Radwaste System?

- A. Drains to the Reactor Coolant Drain Tank and pumped to the Tritiated Drain Collector Tank.
- B. ✓ Drains to the Reactor Building Floor and Equipment Drain Pocket Sump and pumps to the Reactor Building Floor and pumped via Reactor Building Floor and Equipment Drain Sump to the Tritiated Drain Collector Tank.
- C. Drains to the Containment Pit Sump and pumped via the Reactor Building Floor and Equipment Drain Sump to the Floor Drain Collector Tank.
- D. Drains to the Reactor Building Floor and Equipment Drain Sump and pumped to the Floor Drain Collector Tank.

Ref: WB lesson plan 3-OT-SYS077A, obj.
Dwg. 1-47W851-1

Distractor analysis:

See attached drawing

Note: From the drawing, it appears that the Reactor Building Floor and Equipment Drain Pocket Sump has its own pumps, but can overflow into the Reactor Building Floor and Equipment Drain Sump. If this becomes an issue, we can substitute the Passive Sump for distractor D.

QUESTIONS REPORT
for WBfindraft

53. 069AK2.03 001/T1G1/T1G1/CTMT INTEGRITY/C/A 2.8/2.9/N/WB020301/C/RLM

Unit 1 is at 100% power.

Annunciator 158-A, UPR CNTMT AIRLOCK INNER/OUTER alarmed for no apparent reason on the previous shift. Maintenance personnel have been dispatched to investigate the alarm. They report that they have opened the outer door and found the inner door ajar.

Which ONE of the following describes the correct actions required by AOI 12, "Loss of Containment Integrity?"

- A. Verify the outer door closed within one hour and restore the air lock to operable status within 24 hours.
 - B. Repair the inner door within one hour and lock the outer door within 24 hours.
 - C. Close the outer door within one hour and lock the outer door within 24 hours.
 - D. Commence plant shutdown within one hour and be in Mode 5 within 24 hours.
- Ref: WB 3-OT-AOI1200 , obj. 4
WB 3-OT-AOI1200, pp. 5
WB Tech Specs

Distractor analysis:

A is incorrect because the air lock is not inoperable because the outer door is operable. These actions are for an inoperable air lock.

B is incorrect because there is no requirement to repair the inner door within one hour.

C is correct per Tech Specs 6.2 condition A.

D is incorrect because there is no requirement per AOI 12 to commence shutdown within 1 hour.

QUESTIONS REPORT
for WBfindraft

54. 071K3.05 001/T2G1/T2G1/ARM PRM/C/A 3.2/3.2/M/WB020301/C/RLM

Given the following plant conditions:

- A Gas Decay Tank is being released to the atmosphere using the normal release path.
- The tank being released has HIGH gaseous activity.
- PCV-77-117, Plant Vent Pressure Control Valve fails toward the open position.

Which ONE of the following radiation monitor pairs will alarm to alert the operators and secure the release, respectively?

- A. Shield Building Ventilation Monitor [RE-90-400] and Aux Building Vent Monitor [0-RE-90-101].
- B. Auxiliary Building Ventilation Monitor [RE-90-101A] and Shield Building Ventilation Monitor [RE-90-400].
- C. Shield Building Ventilation Monitor [RE-90-400] and WDS Gas Effluent Monitor [0-RE-90-118].
- D. Auxiliary Building Ventilation Monitor [RE-90-101A] and WDS Gas Effluent Monitor [0-RE-90-118].

Ref: WB lesson plan 3-OT-SYS077B, obj. 9 and 3-OT-SYS090A, obj. 7

r analysis

Distracto

A is incorrect because the AB monitor is unrelated to WDS release

B is incorrect same as A

C is correct because the Shield Bldg monitor will upscale and the WDS effluent monitor will trip FCV-77-119 shut and secure release.

D is incorrect same as B

QUESTIONS REPORT
for WBfindraft

55. 072G2.4.31 001/T2G1/T2G1/ARI RESPONSE/M 3.3/3.4/N/WB020301/C/RLM
-Unit 1 is in Mode 6
-Fuel Movement is in progress
-Annunciator 186-B MCR AREA 0-RM-135 RAD HI alarms

Which ONE of the following correctly describes the actions of the control room operators?

- A. Verify Control Building Isolation occurs.
- B. Notify fuel handling crew to stop all fuel movement.
- C. Notify RADCON to investigate the alarm.
- D. Verify emergency control building air cleanup and pressurizing fans RUNNING.

Ref: WB Lesson Plan 3-OT-090A, obj. 9
ARI 186-B

Distractor analysis:

A and D are incorrect because they are auto actions which occur if the MCR Intake rad monitors 125 or 126 alarm

B is incorrect because there would be no relation between Control Room area rad monitor and refueling operations (and no other rad monitor alarms are in)

C is correct per ARI guidance

QUESTIONS REPORT
for WBfindraft

56. 075A4.01 001/T2G2/T2G2//M (3.2/3.2)/B/WB02301/C/KFO

Given the following:

- Reactor trip and Safety Injection occurred while the plant was operating at 100% power
- Four ERCW pumps were running in their normal alignment before the Si occurred.

Which of the following identifies ERCW pump status after the SI and the effect it has on the Condenser Circulating Water (CCW) system make-up?

- A. Four ERCW pumps running; CCW make-up is provided only from ERCW since RCW bypass strainer is isolated.
- B. Four ERCW pumps running; CCW make-up is provided only from ERCW since RCW bypass strainer is routed through the overflow structure.
- C. Four ERCW pumps running; CCW make-up is provided from both ERCW and RCW.
- D. Four ERCW pumps running; CCW make-up is not required since the Unit is tripped.

has a bypass that ensures continuous make-up to CCWREF: lesson plans 3-OT-SYS027A (LO 2) and 067A (LO 13)

- a. incorrect - RCW has a bypass that ensures continuous make-up to CCW
- b. incorrect - RCW has a bypass that ensures continuous make-up to CCW
- c. correct - 2 pumps per train start on SI and RCW has a bypass that ensures continuous make-up to CCW
- d. incorrect - ERCW provides make-up during SI

QUESTIONS REPORT

for WBfindraft

57. 076K2.08 001/T2G3/T2G3/POWER SUPPLY/M 3.1/3.3/N/WB020301/C/RLM

Which ONE of the following describes the power supply for 1-FCV-67-91B, Lower Containment 1C Cooler Supply, and the normal feeder to that power supply?

- A. 480 V Reactor MOV Board 1B2-B
- B. 480 V Shutdown Board 1B2-B
- C. 480 V Reactor MOV Board 2B1-B
- D. 480 V Shutdown Board 1B1-B

Ref: WB Lesson Plan 3-OT-SYS067A, no specific learning objective

Dwgs: 1-45W760-67-5

1-45W751-11

1-45W749-4

1-45-W724-2

Distractor Analysis:

A is correct based on attached dwg's.

B and D are incorrect because the 480 V shutdown board does not directly feed any MOV's

C is incorrect because they have unit 2 designators.

QUESTIONS REPORT
for WBfindraft

58. 079K4.01 001/T2G2/T2G2/SAS CROSS CONNECT/M 2.9/3.2/B/WB020301/C/EL/RLM

If Station Control and Service Air System air pressure is dropping, the Service Air System automatically isolates from the Control Air System when pressure drops below which ONE of the following?

- A. 80 psig.
- B. 79.5 psig.
- C. 78 psig.
- D. 70 psig.

Ref: WB Exam Bank
WB Lesson Plan 3-OT-SYS032A, obj. 16

Distractor analysis:

A is correct, see lesson plan pp. 33 (attached)

B is incorrect, corresponds to isolation setpoint of Aux Air

C is incorrect, doesn't correspond to any auto event

D is incorrect, corresponds to essential and non-essential air to the Rx Bldg isolation

QUESTIONS REPORT
for WBfindraft

59. G2.1.2 001/T3/T3/PROCEDURE COMPLIANCE/M 3.0/4.0/N/WB020301/C/RLM

Operators may take reasonable action that departs from a license condition or Technical Specifications under which ONE of the following circumstances?

- A. In the event of a national security emergency and no action consistent with the license condition or TS that can meet the national security objectives is immediately apparent and the action is immediately needed to implement national security objectives as designated by the national command authority through the NRC.
- B. In an emergency when this action is immediately needed to protect the public health and safety and no action consistent with the license conditions and TS that can provide adequate or equivalent protection is immediately apparent.
- C. In the event of a national security emergency and no action consistent with the license condition or TS that can meet the national security objectives is immediately apparent and such action is approved by a licensed SRO.
- D. In an emergency when this action is immediately needed to protect risk significant equipment and no action consistent with the license conditions and TS that can provide adequate or equivalent protection is immediately apparent.

Ref: WB procedure OPDP-1, sec. 3.4.3, TS compliance

A is correct, see attached

B is incorrect, lacks SRO approval statement

C is incorrect, lacks national command authority requirement

D is incorrect, refers to risk significant equipment

QUESTIONS REPORT
for WBfindraft

60. G2.1.3 001/T3/T3/SHIFT TURNOVER/M 3.7/3.8/B/WB020301/C/RLM

A Unit 1 UO is preparing to relieve the on-shift UO for a lunch break during a normal shift.

Which one of the following describes their required minimum actions when this occurs?

- A. A review of the Operating Log since last held shift or 3 days, whichever is less, Standing Orders, LCO(s) in actions
- B. A review of the Rounds sheets/Abnormal readings, Standing Orders, any SI/Test in progress/planned
- C. Abnormal or unusual plant conditions existing, any evolutions in progress, actions anticipated
- D. Radiological changes in plant during shift, a walkdown of the MCR control boards, SI/Test in progress/planned

Ref: Surry exam 2002

OPDP-1, pp. 24 and Shift Turnover Checklist

A, B, and D are items from the Shift Turnover Checklist which is NOT required for a temporary relief per page 24 of OPDP-1, pp.24 and are therefore incorrect.

C are the items listed on pp. 24 as required for temporary turnover.

QUESTIONS REPORT

for WBfindraft

61. G2.1.32 001/T3/T3/COMPONENT COOLING WA/M 3.4/3.8/N/WB020301/C/RLM

Flowrates in the Component Cooling Water System is limited to 12000 gpm shell side flow.

Which ONE of the following explains the basis for this precaution?

- A. It limits tube d/p and Heat Exchanger loading.
- B. It limits pump runout and tube vibration.
- C. It limits tube vibration and Heat Exchanger loading.
- D. It limits pump runout and tube erosion.

Ref: WB lesson plan 3-OT-SYS070A, obj. 1

SOI-70.01, section 3, P&L's

Distractor analysis:

C is correct, see P&L

A,B, and D are incorrect due to either erosion or pump runout which are not mentioned but is plausible at high flowrates.

QUESTIONS REPORT
for WBfindraft

62. G2.2.12 001/T3/T3/INVENTORY BALANCE/C/A 3.0/3.4/N/WB020301/C/RLM

Unit 1 is at 100%

SI-68-32, Reactor Coolant System Water Inventory Balance is in progress

1 hour has elapsed since commencing data collection

Final readings from the RCSWIB indicate that the total RCS leakage rate is -0.08 gpm

Which ONE of the following describes the correct action to be taken?

- A. Sign the surveillance as satisfactory
- B. Continue data collection for an additional hour
- C. Void the surveillance and perform at a later time
- D. Investigate to determine the source of inleakage to the RCS

Ref: WB procedure SI-68-32, pp. 16 & 17

A is incorrect because data must be collected for at least an additional hour before the surveillance can be termed complete

B is correct, see SI-68-32, pp. 16 & 17

C is incorrect because it is contrary to procedure guidance

D is incorrect unless the leakage continues to be negative by more than -0.10 gpm after the second data collection

QUESTIONS REPORT
for WBfindraft

63. G2.2.13 001/T3/T3/TAGGING/M 3.6/3.8/N/WB020301/C/RLM

Which one of the following is NOT a reason to tag a device, component, or piece of equipment per SSP-10.2, Clearance Program?

- A. Design deficiency
- B. DCN
- C. Test activity
- D. Open Work Orders

WB procedure SPP-10.2, pp. 9

Distractor analysis:
See attached

QUESTIONS REPORT
for WBfindraft

64. G2.2.22 001/T3/T3/SAFETY LIMITS/M 3.4/4.1/M/WB020301/C/RLM

Which one of the following sets of conditions represents a violation of a technical specification safety limit and required action?

- A. Power = 10%, Pressure = 2400 psig, Tavg = 655°F, be in Mode 3 within 1 hour
- B. ✓ Power = 80%, Pressure = 2250 psig, Tavg = 640°F, be in Mode 3 within 1 hour
- C. Power = 10%, Pressure = 2400 psig, Tavg = 655°F, restore to within limits in = 2 hours
- D. Power = 80%, Pressure = 2250 psig, Tavg = 640°F, restore to within limits in = 2 hours

Ref: Modified from TP 2000 by adding required action to pick up LCO part of KA.
WB TS Section 2, Safety limits

Distractor analysis:

A and C are incorrect because they are within limits and no action required

D is incorrect, because even though it is out of limits, it must be restore within 1 hour, 2 hours is the DNB parameter spec.

B is correct because it is both out of limits and has the correct required action per TS

QUESTIONS REPORT
for WBfindraft

65. G2.3.1 001/T3/T3/DOSE EQUIVILENTS/M 2.6/3.0/B/WB020301/C/RLM

Which one of the following dose components are included in a Radiation Worker's Occupational Dose?

- A. Total Effective Dose Equivalent and Planned Special Exposures.
- B. Planned Special Exposures and Committed Effective Dose Equivalent.
- C. Deep Dose Equivalent and Planned Special Exposure
- D. Deep Dose Equivalent and Committed Effective Dose Equivalent.

Ref: Surry exam 2002

WB Rad con procedure SPP-5.1, section 5, pp. 32

WB lesson plan 3-OT-RAD0003, pp.10

QUESTIONS REPORT
for WBfindraft

66. G2.3.4 001/T3/T3/EXPOSURE LIMITS/M 2.5/3.1/B/WB020301/C/RLM

Given the following plant conditions:

- A LOCA has occurred and a SAE has been declared.
- The TSC and OSC have been activated.
- To prevent core damage it is recommended that entry be made into Safety Injection Pump Room 1A.
- Projected dose rate in the pump room is 1.16×10^5 mr/hr.
- Duration of the exposure is expected to be 3 minutes.

Which ONE of the following must authorize this exposure?

- A. Site Vice President
- B. Plant Manager
- C. Site Emergency Director
- D. Radcon Manager

REFERENCES:

EPIP-15 p.3, SPP-5.1 p.10

WBN Exam Bank

- d. incorrect - per EPIP-15 of the Radiological Emergency plan the Site Emergency Director must authorize emergency dose limits in excess of TVA admin limits and 10 CFR 20. Examinee may select since the Radcon Manager is responsible for completing the paperwork and he must authorize exceeding the limit during non-emergency conditions.
- c. correct - With a projected dose of 1.16×10^5 mr/hr, the total dose to respond to this emergency condition is 5.8 Rem. (1.16×10^5 mR/hr / 60 minutes x 3 minutes). Per EPIP-15 of the Radiological Emergency plan the Site Emergency Director must authorize emergency dose limits in excess of TVA admin limits and 10 CFR 20.
- b. incorrect - per EPIP-15 of the Radiological Emergency plan the Site Emergency Director must authorize emergency dose limits in excess of TVA admin limits and 10 CFR 20. Examinee may select since Plant Manager must authorize exceeding 5 R during non-emergency conditions.
- a. incorrect - per EPIP-15 of the Radiological Emergency plan the Site Emergency Director must authorize emergency dose limits in excess of TVA admin limits and 10 CFR 20. Examinee may select since Plant Manager must authorize exceeding 5 R during non-emergency conditions.

QUESTIONS REPORT
for WBfindraft

67. G2.3.9 001/T3/T3/PURGE/M 2.5/3.4/N/WB020301/C/RLM

- Unit 1 is in Mode 3 following a refueling outage
- Both Containment Purge Systems are in operation
- The Shift Manager has directed that the Containment Purge System be secured in accordance with SOI-30.02, CONTAINMENT PURGE SYSTEM.

Reference Provided

Which ONE of the following should be considered prior to securing Containment Purge?

- A. Outside air temperature
- B. Outside atmospheric pressure
- C. Radiological implications
- D. Containment Purge HEPA and Charcoal filter DP

Ref: WB lesson plan 3-OT-SYS30C, obj. 12
WB procedure SOI-30.02

Distractor analysis:

- A and D are incorrect because they only apply while the system is in service
- B is incorrect, because it only applies when placing the system in service
- C is correct because changes in ventilation, particularly reduction in ventilation can have adverse effects on radiation levels. This is also a precaution in the procedure related to securing CTMT purge. (see below)

CAUTION Radiological implications (e.g.: causing an area to become airborne or raising the humidity in contaminated areas) should be considered any time Containment Purge is secured.

QUESTIONS REPORT
for WBfindraft

68. G2.4.1 001/T3/T3/IMMEDIATE OP ACTION/C/A 4.3/4.6/N/WB020301/C/RLM

- The Unit is at 100% power.
- A Loss of Shutdown Power occurs

Which ONE of the following describes the correct procedure and immediate operator action.

- A. E-0, Rx Trip or Safety Injection, ensure Rx is tripped by Rod all bottom lights lit
- B. ECA-0.0, Loss of Shutdown Power, ensure Rx tripped by neutron flux dropping
- C. FR-S.1, Nuclear Power Generation/ATWS, ensure Rx is tripped by neutron flux dropping
- D. ECA-0.0, Loss of Shutdown Power, ensure Rx tripped by rod bottom lights lit

Ref: WB lesson plan 3-OT-PAI1204, obj 1
WB procedures
PAI-1204
E-0
ECA-0.0

Need to check with site to verify that rod bottom lights have no power when shutdown boards deenergized.

Distractor analysis:

- A is incorrect because rod bottom lights will not have power
- B is correct because emergency procedures can be entered directly from a Loss of Shutdown power and NI's will still be powered from battery via inverter.
- C is incorrect because FR-S.1 is not an allowable procedure to enter EOP's
- D is incorrect because rod bottom lights will not have power

QUESTIONS REPORT

for WBfindraft

69. G2.4.11 001/T3/T3/SG LEAK/C/A 3.4/3.6/M/WB020301/C/RLM

Unit 1 is at 100% power with known leakage in SG 2 of 0.05 gpm

The Chem Lab reports that today's leakage is 0.06, up from yesterday's reading of 0.05 gpm

Which ONE of the following should be performed?

Reference provided

A. Be in Mode 3 within 24 hours

B. Reduce load to hot standby in accordance with AOI-39, then cooldown and depressurize the RCS.

C. Trip the reactor; enter E-0, then transition to E-3.

D. Convene PORC to evaluate continued operation.

WB lesson plan 3-OT-AOI300, obj. 8

Procedure AOI-33, appendix B and C

Distractor analysis:

A is correct per table in App. B

B is incorrect because it the action for greater than 600 per day thru all or 150 per day thru one

C is incorrect because 0.6 is well within charging capability which is the decision point for E-0

D is incorrect because it is the action for leaks less than 0.05 per day

QUESTIONS REPORT

for WBfindraft

70. G2.4.16 001/T3/T3/GO TO/M 3.0/3.9/B/WB020301/C/RLM

While in the Emergency Response procedures the team is directed to "Go To" another procedure, which one of the following is the correct implementation of this action?

- A. The "GO TO" implies the procedure in use is no longer applicable, and any tasks in progress need not be completed.
- B. Tasks still in progress must be completed prior to the transition directed by the "GO TO" step.
- C. The "GO TO" implies the procedure in use is no longer applicable, but any tasks in progress should be completed.
- D. Tasks still in progress need not be completed prior to the transition directed by the "GO TO" step, unless preceded by a double asterisk.

WB lesson plan 3-OT-PAI-1204, obj. 13 & 19

WB procedure PAI-1204

Bank: From Surry 2002

Distractor analysis:

Answer C is correct, see below paragraph from PAI-1204

Answer A and B are incorrect due to timing or required completion

Answer D is incorrect because double astericks relates to high and low level steps vice transitions.

From PAI-1204:

Transition to another step or instruction may be completed before completion of the task in progress, provided that completion of the task is verified at a later time.

Transitions which are not high level action steps are identified with a double asterisk preceding the words GO TO and are in bold face type.

QUESTIONS REPORT
for WBfindraft

71. G2.4.8 001/T3/T3/PROCEDURE TRANSITION/C/A 3.0/3.7/B/WB020301/C/RLM

Given the following condition:

- The crew has entered ECA-0.0 due to a loss of both SD Bds.
- The Load Dispatcher has recovered one 161KV line from Watts Bar Hydro.
- The crew has implemented AOI-35.

Which ONE of the following actions should the crew take?

- A. Immediately transition to ECA-0.1, Recovery from Loss of Shutdown Power without SI required.
- B. Immediately transition to ECA-0.2, Recovery from Loss of Shutdown Power with SI required.
- C. Manually actuate SI and return to E-0, Rx Trip or Safety Injection.
- D. Continue with ECA-0.0, Loss of Shutdown Power while concurrently performing AOI-35, Loss of Offsite Power.

Ref: WB procedure ECA-0.0 and AOI-35

WB bank question, changed two obviously incorrect distractors

QUESTIONS REPORT
for WBfindraft

72. WE03EK2.2 001/T1G2/T1G2/HEAT REMOVAL/C/A 3.7/4.0/B/WB020301/C/RLM
Given the following plant conditions:

- A small break LOCA has occurred.
- RCPs have been tripped.
- Appropriate actions in accordance with E-0 and E-1 have been completed.
- RCS pressure is stable at 1525 psig.
- ECCS is operating in cold leg injection mode.

Which ONE of the following statements describes the primary method of decay heat removal at this time?

- A. Heat transfer between the RCS and the S/Gs due to natural circulation flow.
- B. Heat transfer between the RCS and CCS via the RHR Heat Exchangers.
- C. Heat transfer from the injection of water from the RWST and the removal of steam/water out of the break.
- D. Heat transfer from Reflux boiling in the S/Gs.

Ref: WB Lesson Plan 3-OT-EOP0100, no specific learning objective found
WB lesson plan 3-OT-SYS063A, no specific learning objective found
WB exam bank

Distractor Analysis:

A is correct because due to the thermodynamic ΔH between the primary and secondary and resultant heat transfer rate will exceed the heat transfer into the injection flow water.

B is incorrect because CCS flow is isolated from the RHR Hx's at this point in the accident.

C is incorrect same as A

D is incorrect because at this point in the accident the S/G U-tubes are still filled with water.

QUESTIONS REPORT
for WBfindraft

73. WE05EA1.3 001/T1G2/T1G2/FEED AND BLEED/C/A 3.8/4.2/B/WB020301/C/RLM

Given the following plant conditions:

- Unit is operating at 100% power at EOL.
- Total loss of feedwater occurs and operators implement FR-H.1, "Loss of Secondary Heat Sink".
- No means of feedwater addition is available and the operators have initiated feed and bleed.
- Manual Safety Injection was initiated and when the operator attempted to open the pressurizer PORVs, PCV-68-340 failed to open.

Which ONE of the following describes the correct operator mitigation strategy to respond to this problem?

- A. Stop one Centrifugal Charging Pump to reduce loss of inventory through PORVs.
- B. Close any open Pzr PORV to conserve RCS inventory and return to the steps to re-establish Main Feedwater.
- C. Open the reactor head vents to reduce RCS pressure since one pressurizer PORV may not provide sufficient heat removal capacity.
- D. Verify PORV, 1-PCV-68-334, and it's block valve open to reduce RCS pressure since 1 Pzr PORV provides adequate heat removal capacity for a loss of heat sink.

Ref: WB lesson plan 3-OT-FRH0001, obj. 9 & 10

Distractor analysis:

- a. Incorrect - one PORV is not sufficient to provide adequate heat removal.
- b. Correct - increases the bleed path capability and reduce pressure to ensure the core remains cooled.
- c. Incorrect - procedure directs bleed and feed not depressurizing a SG which would be a less effective cooling method.
- d. Incorrect - more bleed path capacity is needed to ensure pressure reduction and cooling capability, not less injection.

QUESTIONS REPORT
for WBfindraft

74. WE10EA2.2 001/T1G1/T1G1//C/A (3.4/3.8)/B/WB02301/C/EL

Given the following plant conditions:

- Reactor trip occurred with subsequent loss of RCPs.
- Operators have implemented ES-0.2, "Natural Circulation Cooldown".
- A cooldown rate of 25°F/hour has been established.
- RCS depressurization has been initiated while maintaining subcooling > 165°F.
- Operators are monitoring PZR level and RVLIS for void formation.
- The OAC observes that loss of inventory in the Condensate Storage Tank is imminent.

Which ONE of the following describes the appropriate procedural actions?

- A. Stop the cooldown and remain in ES-0.2.
- B. Raise the cooldown rate and remain in ES-0.2.
- C. Transition to ES-0.3, "Natural Circulation Cooldown With Steam Voids in Vessel (With RVLIS) and lower the cooldown rate.
- D. ✓ Transition to ES-0.3, "Natural Circulation Cooldown With Steam Voids in Vessel (With RVLIS) and raise the cooldown rate.

The correct answer is D.

- a. Incorrect - loss of CST inventory should cue the examinee that transition to ES-0.3 is appropriate. Examinee may believe stopping cooldown is appropriate to conserve inventory.
- b. Incorrect - loss of CST inventory should cue the examinee that transition to ES-0.3 is appropriate. Examinee may believe raising cooldown rate would be appropriate in order to reach RHR conditions sooner however ES-0.2 does not provide instruction to do this.
- c. Incorrect - loss of CST inventory is an appropriate condition to require transition to ES-0.3 since more rapid cooldown rate is allowed while addressing voids in the RCS. Examinee may believe it is necessary to lower cooldown rate to conserve inventory.
- d. Correct - loss of CST inventory is an appropriate condition to require transition to ES-0.3 since more rapid cooldown rate is allowed while addressing voids in the RCS.

QUESTIONS REPORT
for WBfindraft

75. WE11EA1.3 001/T1G2/T1G2/RECIRCULATION/C/A 3.7/4.2/B/WB020301/C/RLM

Given the following plant conditions:

- Reactor trip and SI occurred on Unit 1 due to a small break LOCA.
- Crew has transitioned from ES-1.3, "Transfer to RHR Containment Sump", to ECA-1.1, "Loss of RHR Sump Recirculation", due to the failure of both RHR sump suction valves to open.
- Crew has reduced ECCS flow to 1 CCP per Figure 1 of ECA-1.1.
- Crew is performing Step 23 of ECA-1.1 to check makeup flow adequate and observes the following indications:
 - Loop 2 RCP running
 - RVLIS = 60% and slowly dropping

Which one of the following lists the correct operator action for this condition?

- A. Ensure additional makeup source to RWST has been aligned.
- B. Control charging to raise makeup flow.
- C. Place RHR shutdown cooling in service.
- D. Slowly depressurize RCS to inject CLAs.

Ref: WB lesson plan 3-OT-ECA0101, obj. 1
WB procedure ECA-1.1

Distractor analysis:

A is incorrect because this is a continuing action from earlier in the procedure, not a hold point.

B is correct per step 23 RNO because RVLIS <63% w/ RCP running.

C is incorrect because stem does not indicate that pressure-temp conditions would allow.

D is incorrect because the depressurization step is later in the procedure assuming adequate subcooling and PZR level which are not indicated in the stem.

In original bank question the following was a distractor:

"Place at least one train of ECCS pumps in service until RVLIS > 65%.". It has been replaced because step 23 RNO also has "OR restart ECCS pumps as necessary".