

**MISCELLANEOUS CORRESPONDENCE**

**FOR THE LASALLE INITIAL EXAMINATION - MAY 2003**

Exelon Generation Company, LLC  
LaSalle County Station  
2601 North 21<sup>st</sup> Road  
Marseilles, IL 61341-9757

www.exeloncorp.com

Nuclear

cc D. Hells  
D. Peltor

September 7, 2001

10 CFR 55.5(a)

United States Nuclear Regulatory Commission  
Attention: NRC Region III Administrator  
801 Warrenville Road  
Lisle, IL 60532-4351

LaSalle County Station, Units 1 and 2  
Facility Operating License Nos. NPF-11 and NPF-18  
NRC Docket Nos. 50-373 and 50-374

Subject: Request for Initial License Examination

Pursuant to 10 CFR 55.5(a), this letter is to request an Initial License Examination for the weeks of May 19, 2003 and May 26, 2003. LaSalle County Station is beginning an initial license course in February 2002 that will contain 14 Reactor Operator and Senior Reactor Operator candidates. This course is scheduled for completion in April 2003.

Please confirm the availability of this date by contacting Mr. William Riffer, Regulatory Assurance Manager, at (815) 415-2800.

Respectfully,



Charles G. Pardee  
Site Vice President  
LaSalle County Station

cc: NRC Document Control Desk  
NRC Senior Resident Inspector - LaSalle County Station

SEP 12 2001

Exelon Generation  
4300 Winfield Road  
Warrenville, IL 60555

www.exeloncorp.com

**Exelon** SM

Nuclear

*cc: D. Shiller  
J. Hopkins  
D. Felton  
H. Peterson*

RS-01-196

September 21, 2001

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555-0001

Braidwood Station, Units 1 and 2  
Facility Operating License Nos. NPF-72 and NPF-77  
NRC Docket Nos. STN 50-456 and STN 50-457

Byron Station, Units 1 and 2  
Facility Operating License Nos. NPF-37 and NPF-66  
NRC Docket Nos. STN 50-454 and STN 50-455

Clinton Power Station, Unit 1  
Facility Operating License No. NPF-62  
NRC Docket No. 50-461

Dresden Nuclear Power Station, Units 2 and 3  
Facility Operating License Nos. DPR-19 and DPR-25  
NRC Docket Nos. 50-237 and 50-249

LaSalle County Station, Units 1 and 2  
Facility Operating License Nos. NPF-11 and NPF-18  
NRC Docket Nos. 50-373 and 50-374

Quad Cities Nuclear Power Station, Units 1 and 2  
Facility Operating License Nos. DPR-29 and DPR-30  
NRC Docket Nos. 50-254 and 50-265

Subject: Response to Regulatory Issue Summary 2001-17, "Preparation and Scheduling of Operator Licensing Examinations"

Reference: NRC Regulatory Issue Summary 2001-17, "Preparation and Scheduling of Operator Licensing Examinations," dated August 22, 2001

This letter provides our voluntary response to the request for information contained in the referenced Regulatory Issue Summary (RIS). The NRC requested that a voluntary response to the RIS be submitted within 30 days of receipt of the RIS. We received RIS 2001-17 on August 22, 2001; accordingly, we are submitting our response by September 21, 2001.

SEP 26 2001

As requested, Attachment A, "Completed NRC Form 536," contains a separate NRC Form 536 [August 2000], "Operator Licensing Examination Data," for each of our Midwest Regional Operating Group facilities addressed by the RIS. The information provided for each facility includes: the proposed examination preparation schedule for calendar years (CYs) 2002 through 2005; the estimated number of applicants and proposed dates for each of the initial operator license examinations through CY 2005; and the estimated number of candidates that plan to take the Generic Fundamentals Examination in CYs 2002 and 2003.

If you have any questions or concerns regarding this information, please contact J. A. Bauer at (630) 657-2801.

Respectfully,

  
for K. A. Ainger  
Director – Licensing  
Mid-West Regional Operating Group

Attachment

cc: Regional Administrator – NRC Region III  
NRC Senior Resident Inspector – Braidwood Station  
NRC Senior Resident Inspector – Byron Station  
NRC Senior Resident Inspector – Clinton Power Station  
NRC Senior Resident Inspector – Dresden Nuclear Power Station  
NRC Senior Resident Inspector – LaSalle County Station  
NRC Senior Resident Inspector – Quad Cities Nuclear Power Station

**OPERATOR LICENSING  
EXAMINATION DATA**

Estimated burden per response to comply with this voluntary information collection request: 1 hour. This information collection is used to plan budgets and resources for operator examinations. Send comments regarding burden estimate to the Records Management Branch (T-6 E6) U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001 or by internet e-mail to bjs1@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0131), Office of Management and Budget Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY

LaSalle County Station, Units 1 and 2  
Facility Operating License Nos. NPF-11 and NPF-18  
NRC Docket Nos. 50-373 and 50-374

NRC REGION

III

**A. PROPOSED EXAMINATION PREPARATION SCHEDULE**

PROPOSED NUMBER	CY <u>02</u>	CY <u>03</u>	CY <u>04</u>	CY <u>05</u>
ESTIMATED NUMBER OF LICENSEE-PREPARED EXAMINATIONS	0	1	1	1
ESTIMATED NUMBER OF NRC-PREPARED EXAMINATIONS	1	0	0	0

**B. INITIAL OPERATOR LICENSING EXAMINATIONS**

PROPOSED NUMBER	CY <u>02</u>	CY <u>03</u>	CY <u>04</u>	CY <u>05</u>
NUMBER OF REACTOR OPERATORS	9	6	6	6
NUMBER OF SENIOR REACTOR OPERATORS-INSTANT	10	6	6	6
NUMBER OF SENIOR REACTOR OPERATORS-UPGRADE	2	2	0	0
NUMBER OF SENIOR REACTOR OPERATORS-LIMITED	0	0	0	0
<b>PROPOSED DATES</b>				
PRIMARY DATE	04/08/02 – 04/19/02	05/19/03 – 05/30/03	05/17/04	May
ALTERNATE DATE	May	June	June	June

**C. PROPOSED GENERIC FUNDAMENTALS EXAMINATION (GFE) SCHEDULE**

PROPOSED NUMBER	CY <u>02</u>			CY <u>03</u>		
	FIRST	SECOND	THIRD	FIRST	SECOND	THIRD
ESTIMATED NUMBER OF CANDIDATES	0	12	0	0	12	0



Exelon Generation  
4300 Winfield Road  
Warrenville, IL 60555

www.exeloncorp.com

Nuclear

cc: H. Peterson  
D. Feltner

RS-02-123

July 18, 2002

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555-0001

Braidwood Station, Units 1 and 2  
Facility Operating License Nos. NPF-72 and NPF-77  
NRC Docket Nos. STN 50-456 and STN 50-457

Byron Station, Units 1 and 2  
Facility Operating License Nos. NPF-37 and NPF-66  
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NRC Docket Nos. 50-373 and 50-374

Quad Cities Nuclear Power Station, Units 1 and 2  
Facility Operating License Nos. DPR-29 and DPR-30  
NRC Docket Nos. 50-254 and 50-265

Subject: Response to Regulatory Issue Summary 2002-09, "Preparation and Scheduling of Operator Licensing Examinations"

Reference: NRC Regulatory Issue Summary 2002-09, "Preparation and Scheduling of Operator Licensing Examinations," dated June 6, 2002

This letter provides our voluntary response to the request for information contained in the referenced Regulatory Issue Summary (RIS).

ML 022860082

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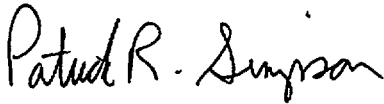
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July 18, 2002  
U. S. Nuclear Regulatory Commission  
Page 2

As requested, Attachment A, "Completed NRC Form 536," contains a separate NRC Form 536 [August 2000], "Operator Licensing Examination Data," for each of our Mid-West Regional Operating Group facilities addressed by the RIS. The information provided for each facility includes: the proposed examination preparation schedule for calendar years (CYs) 2003 through 2006; the estimated number of applicants and proposed dates for each of the initial operator license examinations through CY 2006; and the estimated number of candidates that plan to take the Generic Fundamentals Examination in CYs 2003 and 2004.

If you have any questions or concerns regarding this information, please contact Mr. Brian Sweeney at (630) 657-2836.

Respectfully,



Patrick R. Simpson  
Manager – Licensing, Dresden and Quad Cities Nuclear Power Stations  
Mid-West Regional Operating Group

Attachment

cc: Regional Administrator – NRC Region III  
NRC Senior Resident Inspector – Braidwood Station  
NRC Senior Resident Inspector – Byron Station  
NRC Senior Resident Inspector – Clinton Power Station  
NRC Senior Resident Inspector – Dresden Nuclear Power Station  
NRC Senior Resident Inspector – LaSalle County Station  
NRC Senior Resident Inspector – Quad Cities Nuclear Power Station

NRC FORM 536.  
(8-2000)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO 3150-0131

EXPIRES 07/31/2002

## OPERATOR LICENSING EXAMINATION DATA

Estimated burden per response to comply with this voluntary information collection request: 1 hour. This information collection is used to plan budgets and resources for operator examinations. Send comments regarding burden estimate to the Records Management Branch (T-6 E6) U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001 or by internet e-mail to [bis1@nrc.gov](mailto:bis1@nrc.gov), and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0131), Office of Management and Budget Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY

LaSalle County Station, Units 1 and 2  
Facility Operating License Nos. NPF-11 and NPF-18  
NRC Docket Nos. 50-373 and 50-374

NRC REGION

III

### A. PROPOSED EXAMINATION PREPARATION SCHEDULE

PROPOSED NUMBER	CY <u>03</u>	CY <u>04</u>	CY <u>05</u>	CY <u>06</u>
ESTIMATED NUMBER OF LICENSEE-PREPARED EXAMINATIONS	1	1	1	0
ESTIMATED NUMBER OF NRC-PREPARED EXAMINATIONS	0	0	0	0

### B. INITIAL OPERATOR LICENSING EXAMINATIONS

PROPOSED NUMBER	CY <u>03</u>	CY <u>04</u>	CY <u>05</u>	CY <u>06</u>
NUMBER OF REACTOR OPERATORS	5	0	0	0
NUMBER OF SENIOR REACTOR OPERATORS-INSTANT	5	6	6	0
NUMBER OF SENIOR REACTOR OPERATORS-UPGRADE	1	0	0	0
NUMBER OF SENIOR REACTOR OPERATORS-LIMITED	0	0	0	0
<b>PROPOSED DATES</b>				
PRIMARY DATE	05/19/03 – 05/30/03	05/17/04 – 05/28/04	05/02/05 – 05/13/05	N/A
ALTERNATE DATE	06/03	06/04	06/05	N/A

### C. PROPOSED GENERIC FUNDAMENTALS EXAMINATION (GFE) SCHEDULE

PROPOSED NUMBER	CY <u>03</u>			CY <u>04</u>		
	FIRST	SECOND	THIRD	FIRST	SECOND	THIRD
ESTIMATED NUMBER OF CANDIDATES	0	6	0	0	6	0



February 12, 2002

Mr. Oliver D. Kingsley, President  
Exelon Nuclear  
Exelon Generation Company, LLC  
4300 Winfield Road  
Warrenville, IL 60555

Dear Mr. Kingsley:

In response to your facility management letters dated September 7 and September 21, 2001, and as augmented by discussions with Michael Entwistle of your staff on January 28, 2002, we have tentatively scheduled an initial licensing examination for your operator license applicants at the LaSalle County Station during the weeks of May 19 and May 26, 2003. Validation of the examination will occur at the station during the week of April 28, 2003. In the unlikely event that we are unable to support the examination during the scheduled weeks, we will inform you immediately upon discovery of such conditions and make arrangements to administer the examination at a mutually acceptable date.

As stated in your letter, your staff will develop the examination. To support the examination administration date, we have tentatively scheduled the date of March 31, 2003, to begin our review of your submitted examination.

Your letter indicated you are training approximately 14 candidates for the examination. Please inform us if the number of candidates declines below 10 as this will impact the examination schedule. Please also inform us at your earliest opportunity if you discover you are unable to support the examination on the scheduled dates.

**Once your staff has determined a schedule for examination development, please have them contact the Chief Examiner to arrange for a suitable examination outline submittal date for NRC review. The intent is for the examination outline to be submitted early in the examination development process. This is to preclude the need to make significant changes to developed examination material as a result of the NRC review of the outline.** David L. Pelton has been tentatively assigned as the Chief Examiner and can be reached at 630-829-9732.

A supplementary letter will be sent to the training department approximately 120 days prior to the examination outlining examination security expectations, listing the materials required by the NRC to conduct the examination, reconfirming the examination dates, and reconfirming the number of candidates you have in the training program. If you have any questions concerning this information, please contact David L. Pelton of my staff at 630-829-9732.

ML 020460385

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Sincerely,



David E. Hills, Chief  
Operations Branch  
Division of Reactor Safety

Docket Nos. 50-373; 50-374  
License Nos. NPF-11; NPF-18

cc: W. Bohlke, Senior Vice President, Nuclear Services  
C. Crane, Senior Vice President - Mid-West Regional  
J. Cotton, Senior Vice President - Operations Support  
J. Benjamin, Vice President - Licensing and Regulatory Affairs  
R. Hovey, Operations Vice President  
J. Skolds, Chief Operating Officer  
K. Jury, Director - Licensing  
R. Helfrich, Senior Counsel, Nuclear  
DCD - Licensing  
G. Barnes, Site Vice President  
M. Schiavoni, Station Manager  
W. Riffer, Regulatory Assurance Supervisor  
M. Aguilar, Assistant Attorney General  
Illinois Department of Nuclear Safety  
State Liaison Officer  
Chairman, Illinois Commerce Commission  
G. Kaegi, Training Department

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M. A. Bies, DRS



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

REGION III  
801 WARRENVILLE ROAD  
LISLE, ILLINOIS 60532-4351

February 13, 2003

Mr. John L. Skolds, President  
Exelon Nuclear  
Exelon Generation Company, LLC  
4300 Winfield Road  
Warrenville, IL 60555

Dear Mr. Skolds:

In a telephone conversation on February 12, 2003, between Mr. P. Leheney, Exam Lead, and Mr. M. Bielby, Chief Examiner, arrangements were made for the administration of licensing examinations at the LaSalle County Station, Units 1 and 2, the weeks of May 19 and May 26, 2003. In addition, the NRC will make an examination validation visit to your facility the week of April 28, 2003.

As agreed during the telephone conversation, your staff will prepare the examinations based on the guidelines in Revision 8, Supplement 1, of NUREG-1021, "Operator Licensing Examination Standards for Power Reactors." The NRC regional office will discuss with your staff any changes that might be necessary before the examinations are administered.

To meet the above schedule, it will be necessary for your staff to furnish the examination outlines by March 5, 2003. The written examinations, operating tests, and the supporting reference materials identified in Attachment 2 of ES-201 will be due by March 31, 2003. Pursuant to 10 CFR 55.40(b)(3), an authorized representative of the facility licensee shall approve the outlines, examinations, and tests before they are submitted to the NRC for review and approval. All materials shall be complete and ready to use. Any delay in receiving the required examination and reference materials, or the submittal of inadequate or incomplete materials, may cause the examinations to be rescheduled.

In order to conduct the requested written examinations and operating tests, it will be necessary for your staff to provide adequate space and accommodations in accordance with ES-402, and to make the simulation facility available on the dates noted above. In accordance with ES-302, your staff should retain the original simulator performance data (e.g., system pressures, temperatures, and levels) generated during the dynamic operating tests until the examination results are final.

Appendix E of NUREG-1021 contains a number of NRC policies and guidelines that will be in effect while the written examinations and operating tests are being administered.

ML030450548

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

*/RA/*

Roger D. Lanksbury, Chief  
Operations Branch  
Division of Reactor Safety

Docket Nos. 50-373; 50-374  
License Nos. NPF-11; NPF-18

cc: Site Vice President - LaSalle County Station  
LaSalle County Station Plant Manager  
Regulatory Assurance Manager - LaSalle  
Chief Operating Officer  
Senior Vice President - Nuclear Services  
Senior Vice President - Mid-West Regional  
Operating Group  
Vice President - Mid-West Operations Support  
Vice President - Licensing and Regulatory Affairs  
Director Licensing - Mid-West Regional  
Operating Group  
Manager Licensing - Clinton and LaSalle  
Senior Counsel, Nuclear, Mid-West Regional  
Operating Group  
Document Control Desk - Licensing  
M. Aguilar, Assistant Attorney General  
Illinois Department of Nuclear Safety  
State Liaison Officer  
Chairman, Illinois Commerce Commission  
J. Burns-Muntz, Training Department

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M. A. Bies, DRS

Exelon Generation Company, LLC  
LaSalle County Station  
2601 North 21<sup>st</sup> Road  
Marseilles, IL 61341-9757

www.exeloncorp.com

March 31, 2003

10 CFR 55.40

United States Nuclear Regulatory Commission  
Attention: NRC Region III Administrator  
801 Warrenville Road  
Lisle, IL 60532-4351

LaSalle County Station, Units 1 and 2  
Facility Operating License Nos. NPF-11 and NPF-18  
NRC Docket Nos. 50-373 and 50-374

Subject: Submittal of Integrated Initial Operator Licensing Examination Materials

Enclosed are the examination materials, which LaSalle County Station is submitting in support of the Initial License Examination scheduled for the period of May 19, 2003, through May 30, 2003 at LaSalle County Station.

This submittal includes the Senior Reactor Operator and Reactor Operator Written Examinations, Job Performance Measures, and Integrated Plant Operation Scenario Guides.

These examination materials have been developed in accordance with NUREG –1021, "Operator Licensing Examination Standards," Revision 8, Supplement 1. Please note that reference materials are attached to each individual examination question or item.

Some minor modifications have been made to the Integrated Operating and Written Examination Outline with regards to the operational scenarios in order to improve balance and content. These changes improve examination quality and are in compliance with NUREG –1021, Revision 8, Supplement 1.

Some modifications or adjustments to the examination material may be required due to procedural changes.

March 31, 2003  
U.S. Nuclear Regulatory Commission  
Page 2

In accordance with NUREG -1021, Revision 8, Supplement 1, Section ES-201, "Initial Operator Licensing Examination Process," please ensure that these materials are withheld from public disclosure until after the examinations are complete.

Should you have any questions concerning this letter, please contact Mr. Glen Kaegi, Regulatory Assurance Manager, at (815) 415-2800. For questions concerning examination outlines, please contact Patrick Leheney at (815) 415-2534.

Respectfully,

*Susan R. Sandahl for*

George P. Barnes  
Site Vice President  
LaSalle County Station

Enclosures: (Hand delivered to Michael Bielby, Chief Examiner, NRC Region III)

Administrative Topics Outline (Form ES-301-1)  
Control Room Systems and Facility Walk-Through Test Outline (Form ES-301-2)  
Transient and Event Checklist (Form ES-301-5)  
Scenario Outlines (Form ES-D-1)  
BWR SRO Examination Outline (Forms ES-401-1 and ES-401-5)  
BWR RO Examination Outline (Forms ES-401-2 and ES-401-5)  
RO/SRO Composite Examination with references attached  
Control Room Systems and Facility Walk-Through Job Performance Measures with references attached  
Administrative Topic Job Performance Measures with references attached  
Integrated Plant Operation Scenario Guides  
Examination Security Agreements (Form ES-201-3)(Only the first four (4) columns)  
Operating Test Quality Checklist (Form ES-301-3)  
Simulator Scenario Quality Checklist (Form ES-301-4)  
Competencies Checklist (Form ES-301-6)  
Written Exam Quality Checklist (Form ES-401-7)  
Record of Rejected K/As (Form ES-401-10)

cc: Chief, NRC Operator Licensing Branch (w/o enclosures)  
Senior Resident Inspector - LaSalle County Station (w/o enclosures)



Exelon Generation Company, LLC  
LaSalle County Station  
2601 North 21<sup>st</sup> Road  
Marseilles, IL 61341-9757

www.exeloncorp.com

May 30, 2002

United States Nuclear Regulatory Commission  
Attention: NRC Region III Administrator  
801 Warrenville Road  
Lisle, IL 60532-4351

LaSalle County Station, Units 1 and 2  
Facility Operating License Nos. NPF-11 and NPF-18  
NRC Docket Nos. 50-373 and 50-374

Subject: 2003 LaSalle NRC Initial License Exam

In accordance with NUREG 1021, ES 402 Section E, LaSalle County Station has completed a facility review of the 2003 LaSalle NRC Initial License Exam, administered between May 19, 2003 and May 23, 2003.

During the review, facility representatives identified the issues listed in Attachment 1. Please consider these comments in the grading of the written examinations.

Should you have any questions concerning this letter, please contact Mr. Glen T. Kaegi, Regulatory Assurance Manager, at (815) 415-2800.

Respectfully,



George P. Barnes  
Site Vice President  
LaSalle County Station

Attachment

cc: NRC Document Control Desk  
NRC Senior Resident Inspector - LaSalle County Station

## **Attachment 1**

1. The following question was identified as having the incorrect answer identified.

### **RO/SRO Question 56**

The answer key stated the correct answer was "C". The correct answer is "D".  
The explanation in the answer key supports "D" as the correct answer.

2. The following question was identified as having an incorrect answer identified after review with the examinees and discussion with the facility representative.

### **SRO Question 80**

The question asked the minimum water level in the fuel pool that would still allow fuel movement. Validation of the exam looked strictly at the Technical Specification requirement ( $\geq 21.4$  feet) and not the EAL threshold value of 23 feet as stated in MU11 of the LaSalle Annex. This level would require an Unusual Event classification and support the action of stopping fuel moves.

The correct answer was changed from "C" to "D".

**RECOGNITION CATEGORY  
SYSTEM MALFUNCTIONS****MU11****INITIATING CONDITION**

Unplanned decrease in Spent Fuel Pool level.

**EAL THRESHOLD VALUE**

1. Spent Fuel Pool water level  $\leq 23$  ft. (reference level 841' 8").

**MODE APPLICABILITY**

ALL

**BASIS: (References)**

Classification of an Unusual Event for the threshold values is warranted as a precursor to more serious events and thus is a potential degradation of the level of safety of the plant. The events described as threshold values tend to have a long lead time relative to a radiological release and thus the threat to public health and safety is very low. The levels provided are the minimum allowed by Technical Specifications.

Level taken from procedures LOS-AA-W1 & LOP-SF-06.

GENERAL EMERGENCY		SITE AREA EMERGENCY		ALERT		UNUSUAL EVENT	
SYSTEM MALFUNCTIONS (cont.)							
Decay Heat Removal / Coolant Activity		<b>MS7</b> Loss of decay heat removal and core uncover <div><div></div><div></div><div></div><div>4</div><div>5</div></div> <div><u>EAL Threshold Value:</u> 1. Reactor core is or will be uncovered <b>AND</b> 2. Any of the Following:<ul style="list-style-type: none"><li>Reactor coolant temperature &gt; 200 °F</li><li><b>OR</b></li><li>Uncontrolled temperature rise approaching 200 °F</li></ul></div>		<b>MA7</b> Inability to establish or maintain Mode 4 <div><div></div><div></div><div></div><div>3</div><div>4</div><div>5</div></div> <div><u>EAL Threshold Value:</u> 1. Inability to establish Mode 4 when required. <b>OR</b> 2. Inability to maintain Mode 4 conditions as indicated by EITHER:<ul style="list-style-type: none"><li>Temperature increase that exceeds 200 °F.</li><li><b>OR</b></li><li>Uncontrolled temperature rise approaching 200 °F.</li></ul></div>		<b>MU7</b> High coolant activity <div><div></div><div></div><div></div><div>1</div><div>2</div><div>3</div><div>4</div><div>5</div><div>D</div></div> <div><u>EAL Threshold Value:</u> 1. Offgas system isolation due to valid Offgas radiation monitor high trip <b>OR</b> 2. Coolant activity ≥ 4.0 μCi/g I-131 dose equivalent</div>	
RSC Leakage	<div><div>Table M1 Onsite Communications Equipment</div><div><ul style="list-style-type: none"><li>Plant radio system</li><li>Plant paging system</li><li>Sound power phones</li><li>In-plant telephones</li></ul></div></div>	<div><div>Table M2 Offsite Communications Equipment</div><div><ul style="list-style-type: none"><li>All telephone lines (commercial and microwave)</li><li>NARS</li><li>ENS</li><li>HPN</li><li>Cellular Phones</li></ul></div></div>				<b>MU8</b> RCS leakage <div><div></div><div></div><div></div><div>1</div><div>2</div><div>3</div><div></div><div></div></div> <div><u>EAL Threshold Value:</u> 1. Unidentified RCS leakage into the primary containment &gt; 10 gpm. <b>OR</b> 2. Total RCS leakage (identified + unidentified) into primary containment &gt; 35 gpm</div>	
Loss of Communications						<b>MU9</b> Unplanned loss of all onsite and offsite communications capabilities <div><div></div><div></div><div></div><div>1</div><div>2</div><div>3</div><div>4</div><div>5</div><div>D</div></div> <div><u>EAL Threshold Value:</u> 1. Loss of all onsite communications equipment, Table M1 <b>OR</b> 2. Loss of all offsite communications capability, Table M2</div>	
Technical Specs						<b>MU10</b> Inability to reach required mode within Technical Specification time limits <div><div></div><div></div><div></div><div>1</div><div>2</div><div>3</div><div></div><div></div></div> <div><u>EAL Threshold Value:</u> Plant is not brought to required mode per T.S. Action Statement in the required time.</div>	
Spent Fuel Events				<b>MA11</b> Major fuel damage or fuel uncover in the Spent Fuel Pool <div><div></div><div></div><div></div><div>1</div><div>2</div><div>3</div><div>4</div><div>5</div><div>D</div></div> <div><u>EAL Threshold Value:</u> 1. &gt; 10 R/hr on Refuel Floor Radiation Monitors 0D21-K604A <b>OR</b> 2. Report of visual observation of a rapid decrease of water level such that irradiated fuel is predicted to become uncovered</div>		<b>MU11</b> Unplanned decrease in Spent Fuel Pool level <div><div></div><div></div><div></div><div>1</div><div>2</div><div>3</div><div>4</div><div>5</div><div>D</div></div> <div><u>EAL Threshold Value:</u> Spent Fuel Pool water level ≤ 23 ft. (level 841' 8")</div>	
Defueling				<b>MA12</b> Unplanned loss of Refueling Cavity Volume <div><div></div><div></div><div></div><div></div><div></div><div>5</div></div> <div><u>EAL Threshold Values:</u> 1. Report of a rapid decrease of water level such that irradiated fuel will become uncovered <b>OR</b> 2. Reactor vessel level ≤ -161" (top of Active Fuel)</div>		<b>MU12</b> Unplanned loss of Refueling Cavity Level <div><div></div><div></div><div></div><div></div><div></div><div></div><div>5</div></div> <div><u>EAL Threshold Value:</u> Inability to maintain Refueling Cavity Level<ul style="list-style-type: none"><li>≥ 22 ft. (level 841' 8") above the reactor vessel flange.</li><li><b>OR</b></li><li>≥ 55" on Shutdown Level Range (reference level 845 feet - 7 inches)</li></ul></div>	

Plant Modes (white boxes indicate applicable modes)

1 Power Operations
 2 Startup
 3 Hot Shutdown
 4 Cold Shutdown
 5 Refueling
 D Defueled

Post Examination Comments and Resolution  
(continued)

Which of the following is the minimum water level that would meet the requirements to perform this evolution?

\_\_\_\_\_ above the spent fuel seated in the fuel pool.

- A. 20 feet
- B. 21 feet
- C. 22 feet
- D. 23 feet

Comment: The question asked the minimum water level in the fuel pool that would still allow fuel movement. Validation of the exam looked strictly at the Technical Specification requirement ( $\geq 21.4$  feet) and not the EAL threshold value of 23 feet as stated in the MU11 of the LaSalle Annex. This level would require an Unusual Event classification and support the action of stopping fuel moves. The correct answer was changed from "C" to "D."

Resolution: NRC examiner, licensee training, and an operations representative agreed during the facility written examination review that the question term "minimum water level that would meet the requirements..." was sufficient to illicit the water level identified in Technical Specification 3.7.8. The EAL (Emergency Action Level) MU11 is based on an "unplanned decrease in Spent Fuel Pool level" which was not part of the initial conditions. The question basically asked what was the minimum water level requirement for spent fuel movement, and that minimum water level is specified by Technical Specifications. There was no reference documentation submitted that would prevent operators from performing a planned decrease of spent fuel pool water level (ie, for pool clarity) below the MU11 level of 23 feet as long as they did not exceed the minimum Technical Specification level of 21 feet 4 inches. There were no applicant concerns recorded for this question during the written examination administration. As a result, the reference supports the original answer "C" not "D."

Post Examination Comments and Resolution

Question #56 on the Reactor Operator and Senior Reactor Operator examination was reviewed:

QUESTION: 056 (1.00)

Unit 1 Primary Containment Chillers A & C are off.  
Unit 1 Primary Containment Chiller "B" trips.

Which below describes...

- (1) the status of containment cooling, AND
  - (2) the expected IMMEDIATE (within one minute) effect on Unit 1 Drywell pressure?
- A. (1) All cooling is lost.  
(2) Drywell pressure will rise.
  - B. (1) All cooling is lost.  
(2) Drywell pressure will remain constant.
  - C. (1) Limited cooling is still maintained.  
(2) Drywell pressure will rise.
  - D. (1) Limited cooling is still maintained.  
(2) Drywell pressure will remain constant.

Comment: The answer key stated the correct answer was "C." The correct answer is "D." The explanation in the answer key supports "D" as the correct answer.

Resolution: The lesson plan reference submitted (with applicable paragraph circled) for the question was entitled "096 Primary Containment Cooling," page 16 of 56, paragraph "7. Chiller Unit Trip," LIC Objective 096.00.12. The reference does not support question answer "D." As stated in the reference, the Holdup Tank provides enough cooling to allow starting standby equipment; however, the same paragraph reference also states that Drywell air temperature and pressure will begin to rise when the chiller unit trips. As a result, the reference supports the original answer "C."

Question #80 on the Senior Reactor Operator examination was reviewed:

QUESTION: 080 (1.00)

Unit 1 is Refuel.

Spent fuel movements within the Unit 1 Spent fuel pool are in progress.