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J. J. Linehan Section Leader Repository Projects Branch Division of Waste Management U.S. Nuclear Regulatory Commission Washington, D.C. 20555

NEVADA NUCLEAR WASTE STORAGE INVESTIGATION (NNWSI) PROJECT/NUCLEAR REGULATORY COMMISSION (NRC) WASTE PACKAGE TECHNICAL MEETING AGENDA

Enclosed is an agenda, developed by Lawrence Livermore National Laboratory (LLNL), for the subject meeting. We believe the agenda addresses the meeting objectives transmitted to you with my letter of June 11.

A list of suggested documents for NRC review prior to the meeting is also enclosed. Please let us know if you need copies of any of the publications.

Dohald L. Vieth, Director Waste Management Project Office

WMP0:JSS-1195

Enclosures: As stated

> cc w/encl: L. D. Ramspott, LLNL, Livermore, CA L. B. Ballou, LLNL, Livermore, CA N. K. Stablein, NRC, Washington, DC P. T. Prestholt, NRC, Las Vegas, NV M. A. Glora, SAIC, Las Vegas, NV V. J. Cassella, DOE/HQ (RW-22), FORSTL

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NNWSI Project/NRC Waste Package Meeting Proposed Agenda - July 23-24, 1985

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Introductions 1/4 hour NRC Participants **DOE/LLNL Participants** Others Meeting Objective & Agenda Overview 1/4 hour NRC Objectives DOE/LLNL Objective Agenda Overview ·Update on Conceptual Designs $1 \frac{1}{2}$ hours Emplacement Geometry Spent Fuel Internal Configurations WV/DHLW Containers MRS Packaging & Storage Implications Part 60 Excluded Materials 1/2 hour Discussion of NRC Intent Rationale for Implementing Criteria **Container Material Testing** 3 hours Conceptual Model for Corrosion in Tuff Environmental Conditions General & Localized Corrosion Testing Stress Corrosion Testing Planned Testing Waste Form Testing - Spent Fuel 3 hours Approach to Testing Release Rates Cladding Degradation Fuel Oxidation Results to date Comparison with Part 60 Objectives Isotopes of Concern Discussion of Fuel Population Proposed Test Matrix Adequacy of Sampling Planned Testing Modeling of Releases Waste Form Testing - Glass 11/2 hours Unsaturated Testing Procedure Development Results to Date Planned Testing Supporting Tests Modeling of Release Mechanisms **Reliability Considerations** 1 hour Discussion of NRC Approach to "substantially complete" Approach to Reliability for Containment Summary and Development of Minutes 1 hour

UCRL 89988, "Selection of Candidate Canister Materials for High-Level Nuclear Waste Containment in a Tuff Repository"

UCRL 91804, "Behavior of Stressed and Unstressed 304L Specimens in Tuff Repository Environmental Conditions"

UCID 20174, "Electrochemical Determination of the Corrosion Behavior of Candidate Alloys Proposed for Containment of High Level Nuclear Waste in Tuff"

UCRL 91257, "Laboratory Experiments Designed to Provide Limits on the Radionuclide Source Term for the NNWSI Project"

UCRL 91464 (HEDL-SA-3288), "Radionuclide Release from PWR Fuels in a Reference Tuff Repository Groundwater"

HEDL-7452, "Evaluation of the Potential for Spent Fuel Oxidation under Tuff Repository Conditions"