

NOTICE OF VIOLATION

PECO Nuclear
Limerick Generating Station

Docket Nos. 50-352; 50-353
License Nos. NPF-39; NPF-85

During an NRC inspection conducted from July 6 through 10, 1998, a violation of NRC requirements was identified. In accordance with the "General Statement of Policy and Procedures for NRC Enforcement Actions," NUREG 1600, Revision 1, the violation is delineated below:

10 CFR 50.65(a)(1) requires, in part, that holders of an operating license shall monitor the performance or condition of structures, systems and components (SSCs), within the scope of the monitoring program, as defined by 10 CFR 50.65(b), against licensee established goals, in a manner sufficient to provide reasonable assurance that such SSCs are capable of fulfilling their intended functions. Such goals shall be established commensurate with safety and, where practical, take into account industry-wide operating experience. When the performance or condition of a SSC does not meet established goals, appropriate corrective action shall be taken.

10 CFR 50.65(a)(2), requires, in part, that the monitoring as specified in 10 CFR 50.65(a)(1) is not required where it has been demonstrated that the performance or condition of a SSC is being effectively controlled by performing appropriate preventive maintenance, such that the SSC remains capable of performing its intended function.

Contrary to 10 CFR 50.65(a)(2), as of July 10, 1996, (the time that the licensee elected to not monitor the performance or condition of the Safety Relief Valves and the Spent Fuel Pool Cooling system against licensee established goals pursuant to the requirements of 10 CFR 50.65 (a)(1)) the licensee failed to demonstrate that the performance or condition of these systems, within the scope of 10 CFR 50.65, had been effectively controlled through the performance of appropriate preventive maintenance in accordance with the requirements of 10 CFR 50.65(a)(2).

Specifically, the licensee failed to adequately evaluate the appropriateness of the performance of preventive maintenance on these SSCs prior to placing SSCs under Paragraph (a)(2). The licensee failed to establish adequate performance measures for the SSCs and was therefore unable to effectively demonstrate that the SSCs remained capable of performing their intended function as listed below.

- Safety Relief Valves(SRV) -Reliability performance measures, in part, were no failures of any grouping of SRVs to lift before the ASME code safety limit for maximum reactor coolant system pressure. This measure would allow multiple SRVs to exceed their technical specification set point limits and thus would not ensure that the SRVs remain capable of performing their intended function.

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- Spent fuel pool cooling system -Reliability performance measure was no more than two system failures within a two year operating cycle that would result in a loss of spent fuel cooling. This measure allowed repetitive failures of individual operating and standby trains without placing the system in an (a)(1) status. Specifically, as of July 10, 1996 using this measure, the system was placed in the (a)(2) category despite two complete system failures and multiple individual train failures.

This is a severity level IV violation, NUREG-1600, Revision 1, Supplement I

Pursuant to the provisions of 10 CFR 2.201, PECO Nuclear is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the Regional Administrator, Region I, and a copy to the NRC Resident Inspector at the facility that is the subject of this Notice, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. Your response may reference or include previous docketed correspondence, if the correspondence adequately addresses the required response. If an adequate reply is not received within the time specified in this Notice, an order or a Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

If you contest this enforcement action, you should also provide a copy of your response to the Director, Office of Enforcement, United States Nuclear Commission, Washington, DC 20555-0001.

Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without redaction. If personal privacy or proprietary information is necessary to provide an acceptable response, then please provide a bracketed copy of your response that identifies the information that should be protected and a redacted copy of your response that deletes such information. If you request withholding of such material, you must specifically identify the portions of your response that you seek to have withheld and provide in detail the bases for your claim of withholding (e.g., explain why the disclosure of information will create an unwarranted invasion of personal privacy or provide the information required by 10 CFR 2.790(b) to support a request for withholding confidential commercial or financial information). If safeguards information is necessary to provide an acceptable response, please provide the level of protection described in 10 CFR 73.21.

Dated at King of Prussia, Pennsylvania
this 26th day of August, 1998