



102

HLW

(19)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

MAR 16 1981 - 01

Mr. Greg Snipes
Systems Engineer, Fuel
Duke Power Company
P. O. Box 33189
Charlotte, North Carolina 28242

WM Record File

102

WM Project

11

Docket No.

PDR

✓

LPDR

✓

Distribution:

(Return to WM, 623-SS)

Dear Mr. Snipes:

During the flight back from the Gatlinburg meeting, I read some technical papers on waste management that I have been accumulating for some time and have just recently found time to read. I believe that one of these papers in particular emphasizes the need to approach the high-level waste management problem along the lines outlined in my presentation. I think in your capacity as Chairman of the high-level waste portion of the Utility Waste Management Group activities, you will find it interesting also.

One of the key concepts in assessing a waste management system using models is that the soils and rocks hold up radionuclides and significantly retard their movements. This idea is dealt with in systems models by using an experimentally determined factor which describes how much more slowly the nuclides move than the water which bears them. The attached technical paper describes a large-scale, very expensive test that was done at the Nevada Test Site. The surprise presented in this paper was that the radionuclide retention factors measured in the laboratory (about 1000 to 3000) are apparently wrong. This experiment showed that the nuclide in question was apparently not held up at all. Therefore, this test has thrown into doubt the laboratory tests which are conventionally used to develop the data for systems models and any predictions made by the models. I think this will be eventually resolved, however not without a great deal more expense and, more importantly, expenditure of time.

The major point I would like to make out of all this is that if something like this were to come up during the licensing review process or the licensing proceeding, it could involve months of delays, perhaps even years. This is the reason why we have gone "within" the systems approach and put some minimum requirements on key elements of the system so that the licensing process becomes more insensitive to these sorts of disruptive discoveries. I would expect that in the next few years as research programs continue that there will be a lot more things like this come up; and unless the whole process has been designed with sufficient margin to cope with these sorts of questions, I see great potential for delays and the debilitating effects to the industry that they will have. I would recommend that you discuss these ideas with the Utility Waste Management Group.

8311300393 810316
PDR WASTE
WM-11

PDR

4
NV837969

00003

Mr. Greg Snipes

- 2 -

MAR 16 1981

As I told you in Gatlinburg, I would be happy to review this further with the group or members of the group at any time. Since I also discussed this extensively with Joe Lieberman and Jay Smith at dinner Wednesday night, I am taking the liberty of sending them a copy as well. I am looking forward to discussing waste management issues further with you in the near future.

Sincerely,



John B. Martin, Director
Division of Waste Management

Enclosure: As stated

cc: Dr. Joe Lieberman
Nuclear Safety Associates, Inc.

Mr. Jay Smith

¹⁰⁶Ru MIGRATION IN A DEEP TUFFACEOUS ALLUVIUM AQUIFER, NEVADA TEST SITE

David G. Coles
Lawrence D. Ramspott

Abstract

Ruthenium-106 has been observed to migrate at the same velocity as ³H in ground water from the site of an underground nuclear explosion to a pumped satellite well. This finding contradicts laboratory sorption studies using material from this site that indicate that ¹⁰⁶Ru should migrate at a much slower rate than ³H. These field measurements raise doubts about the wisdom of relying on simple laboratory sorption measurements to predict field radionuclide migration. Field tests are needed for verification for nuclides that can exhibit complex solution chemistries(1).

Introduction

In order to assess the potential for ground-water migration of radioactivity deposited in aquifers by underground nuclear tests, a program was begun by the Department of Energy's Nevada Operations Office (NV) to study the interaction of nuclear explosion debris and ground water. This program can also shed light on what might happen in a flooded nuclear waste repository. Participants in this program are the University of Nevada's Desert Research Institute, the U. S. Geological Survey, the Los Alamos Scientific Laboratory (LASL), and the Lawrence Livermore National Laboratory (LLNL)(2). Results of the laboratory studies and a review of the migration of radionuclides in ground water have previously been published (3-5). We report here the results of field studies which show a significant discrepancy between migration rates found in the laboratory and those measured in the field.

The nuclides we measured were produced in the Cambric test, which was detonated in NTS tuffaceous alluvium on May 14, 1965, with a yield of 0.75 kt. At the Frenchman Flat explosion site the static water level was 220 m below the

land surface. The explosion point was 294 m deep. Figure 1 is a cross section of the experiment site. In order to determine the concentration of radioactivity in the ground water at the explosion site, water was sampled from a well that intersected the explosion cavity (6). The only radionuclides measured at levels above MPC(7) were ^3H and ^{90}Sr , even though the water had been in direct contact with nuclear debris for more than 15 years. The concentration of ^{106}Ru was 120 pCi/liter.

During April 1974 a satellite well was drilled 91 m south of the explosion point coordinates to enable the study of migration rates of the radionuclides present in the cavity water. A pump was placed in the well at the same depth as the explosion point. Pumping induced an artificial ground water gradient between the explosion point and the well. If one assumes that ^3H is not retarded, the relative movement of observed radionuclides can be compared to ^3H movement, and an in situ retardation factor can be determined.

No radioactivity was observed in the satellite well until ^3H was detected after $1.44 \times 10^6 \text{ m}^3$ of water had been pumped (8,9). With further pumping, the ^3H concentration continues to increase in the satellite well and has decreased fifty-fold in the cavity well after more than two years of pumping(8).

Measurement Program

After establishing that cavity water had been drawn to the satellite well, we wondered whether it would be possible to measure other radionuclides besides ^3H . From calculations based on the known relative concentrations of radionuclides, dilution factors, and the "volume pumped curve" of Hoffman (8), we concluded that the activities of the radionuclides would be below the limits of analytical detection. The calculated activity of ^{106}Ru was 0.02 pCi/liter.

To solve this problem we collected 200-liter water samples, and evaporated them in the laboratory. The resultant salts (approximately 150 g) were counted on an ultra-low background Ge(Li) Compton suppressed gamma-ray spectrometer (10,11). This special counting equipment was necessary because the activity level for these samples was very low. Normal counting equipment would have been unable to detect the low levels of ^{106}Ru observed. In order

UCRL- 85320
PREPRINT

^{106}Ru MIGRATION IN A DEEP TUFFACEOUS
ALLUVIUM AQUIFER, NEVADA TEST SITE

David G. Coles
Lawrence D. Ramspott

This paper was prepared for submittal to
Science

February 5, 1981



This is a preprint of a paper intended for publication in a journal or proceedings. Since changes may be made before publication, this preprint is made available with the understanding that it will not be cited or reproduced without the permission of the author.

original source ground water and the satellite-well ground water. Since ^3H exchanges rapidly with the hydrogen in water, most of the ^3H produced in the explosion is probably in the ground water. However, since ^{106}Ru can become immobilized in the melt glass (4) or be unavailable due to sorption or precipitation, only about 1% of the total produced became a mobile species in the ground water. We also analyzed a 400-liter sample from the cavity well four years after the pumping was begun in the satellite well. While 120 pCi/liter of ^{106}Ru was observed in water from the cavity well before pumping began at the satellite well, in the much larger sample taken four years later no ^{106}Ru was observed above the detection limit of 0.01 pCi/liter for that sample. This indicates that the mobile species was swept from the cavity and not replenished by leaching of melt glass or by reversible reaction from a sorbed or precipitated form.

Having concluded that our field observation was valid, we then considered its relationship to previous data, both field and laboratory. Many reports (3, 5, 13, 14) show ruthenium to have K_d values ranging from 10 to 8000, which should cause it to migrate at a significantly slower rate than ^3H . We note here that a K_d of zero defines a nonsorbed species; any K_d greater than 1 would result in retardation sufficient to eliminate the ruthenium from our samples. We conclude that all indications from laboratory studies were that we should not have detected ruthenium in the water.

* Very little data for ruthenium geochemistry exist, and only a few field investigations on the migration of ruthenium have been reported. Ames and Rai (14) report that ruthenium is considered to exist only as complex ions in solution. Based on simplified Eh (redox potential)-pH diagrams for the range of Eh and pH values expected for ground waters, Brookins (15, 16) indicates that three species could exist, depending on Eh conditions and assuming that certain anions were present. For reducing conditions, RuS_2 is a potential stable phase. For more oxidizing conditions, RuO_2 is a potential stable phase. At still stronger oxidizing conditions, a stability field exists for the complex ion RuO_4^{2-} . Ames and Rai also point out that the most stable aqueous ions in an oxidizing soil environment would be RuO_4^- and RuO_4^{2-} . Since little thermodynamic data exist for ruthenium, these chemical species are speculative but they do point out some possible species as well as demonstrate the confused state of understanding of aqueous ruthenium geochemistry.

The laboratory experiments discussed by Ames and Rai are varied and confusing. Values for K_d on Hanford soil ranged from 40 to 752 within the

range for ground-water pH [7-9] values. Depending on the solution variables, they state that ruthenium can simultaneously exist as a colloid, cationic form, and anionic form.

A few other field observations of ruthenium migration have been made, mostly from leaking storage tanks or waste pits near the ground surface. In most of the cases discussed by Ames and Rai, ruthenium showed significant mobility and in one case was observed to migrate with tritium and technetium (17). Gancarz et al. (18) have inferred limited ruthenium migration at the OKLO uranium deposit in Gabon, Africa. A review by Onishi et al. (19) concluded that ruthenium migration in the field has been well-documented, while on the other hand laboratory tracer experiments have shown high K_d values for ruthenium. These authors concluded that anionic forms of ruthenium are generally nonsorptive, that nitrate complexing accounted for the migration from nitrate-rich waste tanks, and that accurate thermodynamic data for ruthenium complexes did not exist. Knowledge of ruthenium speciation is necessary before its behavior can be understood.

The character of ground water in the tuffaceous alluvial aquifer of Frenchman Flat has been well-documented by chemical analysis from several wells near the Cambric site (4, 5, 6, 20). The ground-water composition of the satellite well is considered here as a typical Frenchman Flat ground water, although some variability exists in the composition of the other nearby well samples, which are as much as 6.6 km apart. An understanding of the oxidation-reduction (redox) conditions is critical to interpreting radionuclide behavior in ground water. Knowledge of valence distribution is required for predicting chemical speciation of elements in aqueous systems, and this speciation information provides insight to the mechanisms that caused the observed behavior of the radionuclide in the ground water. It has often been assumed that deep aquifers (>250 m) are reducing since they are isolated from the atmosphere. However, recent studies by Winograd (21) have shown that many deep aquifers have nearly saturated contents of dissolved oxygen (DO).

Because he has studied the NTS ground water also and observed nearly saturated levels of DO there, we conclude that the ground water in this study is oxidizing. Wolfsberg (5) measured the platinum electrode Eh at the satellite well and observed mildly oxidizing conditions (+330 mV). Based on the DO content in these waters and the Eh-pH diagrams for ruthenium (15, 16), we would expect some migration of ruthenium at the Cambric site, probably as RuO_4^{2-} .

Laboratory K_d measurements (both sorption and desorption) were done on samples of tuffaceous alluvium from the cavity well using satellite water. A range of K_d values for ruthenium of 976 to 3390 was reported (5). These laboratory measurements were designed to be specifically relevant to the Cambric study; they were done on the same medium with the Cambric ground water. The conclusions, based solely on the laboratory work, are contrary to observations in the field. The laboratory "batch" K_d methodology employed in the Cambric study (5) is similar to that currently utilized as a primary K_d data-gathering source for use in assessing the quality of various geologic media for nuclear waste disposal repositories (13, 22, 23). This technique is attractive because it is inexpensive, simple to carry out, and allows for high sample throughput. These features are important to the task of screening many different candidate media. However, the present observation of ^{106}Ru migration raises the question of whether this type of laboratory evaluation of radionuclide sorption behavior can be extrapolated to the actual repository conditions, at least for nuclides with complex solution chemistry.

The fact that ^{106}Ru has been confirmed to migrate with ^3H in a tuffaceous alluvial aquifer at the NTS does not indicate a hazard from potential ground-water migration from nuclear tests to off-site wells or springs. First, the ^{106}Ru initial concentration in the chimney itself is well below MPC for drinking water (6). Second, the ^{106}Ru half-life is so short (1.01 years) that it would never reach distant wells before it had completely decayed away. Only 1% of the total ^{106}Ru was mobile and it was not replenished after the pumping swept the initial amount from the cavity region.

X The significance of this work, rather, is the disagreement between complementary laboratory and field studies. Careful, relevant laboratory sorption measurements predicted essentially no ground-water migration of ^{106}Ru . Under field conditions ^{106}Ru was observed to migrate with essentially no retardation relative to ^3H . While these field observations provide insufficient data for invalidating "batch" K_d measurements, they do demonstrate that the batch K_d values must be used with caution and verified with in situ radionuclide migration studies. Certainly there is a need for additional work to interpret and understand the field observations. In particular, an understanding of chemical speciation of multi-valence elements like ruthenium (e.g., Tc and actinides) is needed in order to grasp their behavior in a ground-water environment. For predominantly low-valence elements (e.g., Sr and Cs), the batch K_d measurements may be quite adequate.

Table 1. Radioactivity (pCi/liter)* observed in salts from evaporated⁺ large volume water samples from satellite well, Cambria Site, NTS.

Radionuclides	Sample/(sample volume in parenthesis)						
	Blank ⁺ (200-1)	11/29/78 ⁺ (200-1)	2/7/79 (200-1)	3/14/79 (200-1)	8/29/79 (710-1)	4/17/80 [#] (740-1)	7/23/80 (860-1)
⁴⁰ K	8.6 ± 2%	10. ± 2%	7.2 ± 3%	7.7 ± 2%	10. ± 2%	12. ± 1%	8.6 ± 1%
¹⁰⁶ Ru	≤0.02	0.04 ± 45%	0.04 ± 40%	0.04 ± 30%	0.07 ± 7%	0.08 ± 45%	0.08 ± 21%
²³⁵ U	0.07 ± 6%	0.09 ± 6%	0.05 ± 7%	0.08 ± 5%	0.04 ± 4%	0.12 ± 7%	0.09 ± 2%
²³⁸ U	1.9 ± 5%	2.5 ± 5%	1.0 ± 35%	1.6 ± 26%	0.90 ± 6%	3.0 ± 16%	2.2 ± 4%

* Data recomputed to 15 years after original shot time so that all values are closer to current observed levels in water. Errors are 1σ counting statistics only.

⁺ We have been collecting various 200-1 samples from the satellite well since April 1977. Some have not been evaporated, and a few others were contaminated.

⁺ These samples were supplied by LASL. The blank was taken from NTS well 5B which is located 2.25 km south of the Cambria site. It was unknown to us at the time of the analysis that this was a blank sample. It is the water supply well for Mercury, NV, but from the same aquifer as the satellite well.

[#] This sample was counted on a Ge(Li) gamma-ray spectrometer different from that used to count the other samples. Since that spectrometer had a higher background, the ¹⁰⁶Ru was more difficult to observe. This is shown by the high counting statistics error for ¹⁰⁶Ru as well as for both ²³⁵U and ²³⁸U.

Table 2. $^{106}\text{Ru}/^3\text{H}$ activity ratios[†] from various Cambrian water samples.

Original source [*]	Cavity samples [†]		Satellite well samples					
	Zone II	Zone III	11/29/78	2/7/79	3/14/79	8/29/79	4/17/80	7/23/80
4×10^{-6}	5×10^{-8} , 3×10^{-8}	2×10^{-8}	5×10^{-8}	4×10^{-8}	4×10^{-8}	4×10^{-8}	3×10^{-8}	3×10^{-8}

[†] Because ^3H and ^{106}Ru decay with different half-lives, all these ratios were calculated for 15 years after the detonation.

^{*} This is the ratio calculated for the total ^3H and ^{106}Ru produced by the explosion. If all of the produced activity were soluble in the ground water, then the pumped water should have an identical ratio. Since the calculated source ratio is approximately a factor of 10^2 higher, less than 1% of ^{106}Ru produced is in the ground water.

[†] Reference 6.

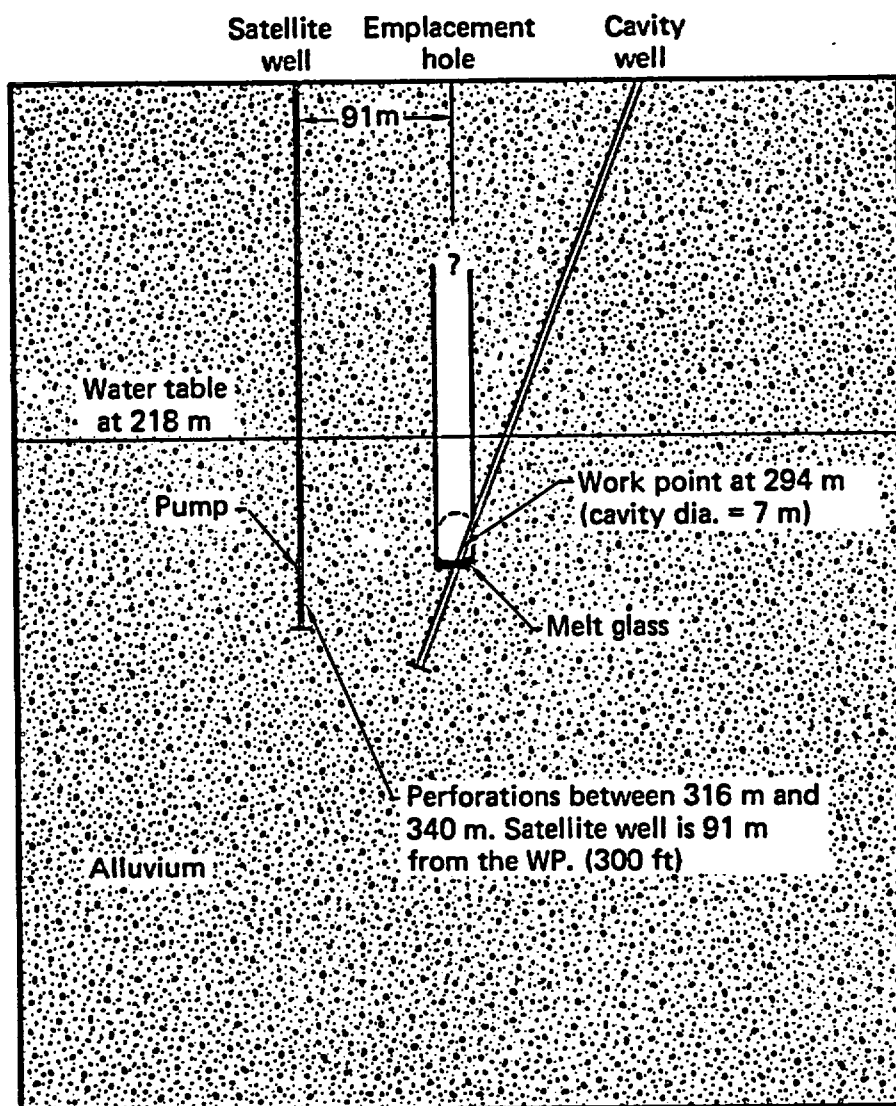


Fig. 1. Cross section of Cambric Experiment Site.

References and Notes

1. Work performed under the auspices of the U.S. Department of Energy by the Lawrence Livermore National Laboratory under contract number W-7405-ENG-48.
2. The authors express special thanks to a variety of colleagues for reviewing the manuscript and making many valuable suggestions. These were E. Douthett, NV, W. Daniels, LASL, W. Wilson and H. Claassen, USGS-Denver, I. Winograd, USGS-Reston, D. Bookins, University of New Mexico, and J. Serne, Battelle-Pacific Northwest Laboratory. Those at LLNL who reviewed the manuscript were R. Ide, D. Isherwood, T. Wolery, R. Schock, and J. Tewhey. J. Schweiger, F. Bazan, and J. Rego provided field sampling support. J. Bazan and L. Gazlay counted the samples. P. Beringer typed the manuscript.
3. I. Y. Borg, R. Stone, H. B. Levy, L. D. Ramspott, "Information Pertinent to the Migration of Radionuclides in Ground Water at the Nevada Test Site," Lawrence Livermore Laboratory, Rept. UCRL-52078, Part 1: "Review and Analysis of Existing Information," (May 25, 1976); Part 2: "Annotated Bibliography," (August 31, 1976).
4. D. G. Coles, H. C. Weed, J. S. Schweiger, "Single-Pass Leaching of Nuclear Melt Glass by Groundwater," in Radioactive Waste in Geologic Storage, S. Fried, Ed. (ACS Symposium Series 100, Washington, D.C., 1978), p. 93.
5. K. Wolfsberg, "Sorption-desorption Studies of Nevada Test Site Alluvium and Leaching Studies of Nuclear Explosion Debris," Los Alamos Scientific Laboratory, Rept. LA-7216-MS (1978).
6. D. C. Hoffman, R. Stone, W. W. Dudley, Jr., "Radioactivity in the Underground Environment of the Cambrian Nuclear Explosion at the Nevada Test Site," Los Alamos Scientific Laboratory, Rept. LA-6788-MS (1977).

7. MPC is maximum permissible concentration of a radionuclide for unrestricted public use of drinking water.
8. D. C. Hoffman, "A Field Study of Radionuclide Migration," in Radioactive Waste in Geologic Storage, S. Fried, Ed. (ASC Symposium Series 100, Washington, D. C. (1978), p. 149.
9. W. R. Daniels, private communication (August 15, 1980).
10. D. C. Camp, C. Gatrousis, L. A. Maynard, "Low Background Ge(Li) Detector Systems for Radioenvironmental Studies," Nucl. Instrum. Methods 117, 189 (1974).
11. D. G. Coles, J. W. T. Meadows, C. K. Lindeken, "The Direct Measurement of ppm Levels of Uranium in Soils Using High-Resolution (Ge(Li) Gamma-Ray Spectroscopy," Sci. Total. Environ. 5, 171 (1976).
12. The predicted value is a value previously calculated from cavity well concentration and tritium dilution factors.
13. R. J. Serne, D. Rai, M. J. Mason, M. A. Molecke, "Batch K_d Measurements of Nuclides to Estimate Migration Potential at the Proposed Waste Isolation Pilot Plant in New Mexico," Battelle-Pacific Northwest Laboratory Rept. PNL-2448 (1977).
14. L. L. Ames, D. Rai, "Radionuclide Interactions with Soil and Rock Media, Vol. 1: Processes Influencing Radionuclide Mobility and Retention Element Chemistry and Geochemistry--Conclusions and Evaluations," U.S. Environmental Protection Agency, Washington, D. C., EPA 520/6-78-007 (1978).
15. D. G. Brookins, "Eh-pH Diagrams for Elements from $Z = 40$ to $Z = 52$; Application to the OKLO Natural Reactor, Gabon," Chemical Geology 23, 325 (1978).

16. D. G. Brookins, private communication (March 1980).
17. D. J. Brown, "Migration Characteristics of Radionuclides Through Sediments Underlying the Hanford Reservation," Isochem Inc., Richland, Wash., ISO-SA-32 (1967) (CONF-670512-2).
18. A. J. Gancarz, G. A. Cowan, D. B. Curtis, "⁹⁹Tc, Pb, and Ru Migration Around the OKLO Natural Fission Reactors," in Proceedings of the International Symposium on the Scientific Basis for Nuclear Waste Management, November 26-29, 1979, Cambridge, Mass., in press.
19. Y. Onishi, R. J. Serne, E. M. Arnold, C. E. Cowan, F. L. Thompson, "Critical Review: Radionuclide Transport, Sediment Transport and Water Quality Mathematical Modeling, Radionuclide Sorption/Desorption Mechanisms," NUREG/Cr-0657, Battelle-Pacific Northwest Laboratory, Rept. PNL-2901 (1979).
20. H. C. Claassen, "Water Quality and Physical Characteristics of Nevada Test Site Water--Supply Wells," United States Geological Survey Rept. USGS-474-158 (1973).
21. I. J. Winograd, private communication (January 23, 1980).
22. J. F. Relyea, R. J. Serne, "Controlled Sample Program Publication Number 2: Interlaboratory Comparison of Batch K_d Values," Battelle-Pacific Northwest Laboratory Rept. PNL-2872 (1979).
23. A. Brandstetter, M. A. Harwell, "The Waste Isolation Safety Assessment Program," Battelle-Pacific Northwest Laboratory Rept. PNL-2A-7243 (1979).

DISCLAIMER

This document was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Government nor the University of California nor any of their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial products, process, or service by trade name, trademark, manufacturer, or otherwise, does not necessarily constitute or imply its endorsement, recommendation, or favoring by the United States Government or the University of California. The views and opinions of authors expressed herein do not necessarily state or reflect those of the United States Government thereof, and shall not be used for advertising or product endorsement purposes.

Technical Information Department • Lawrence Livermore Laboratory
University of California • Livermore, California 94550

