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THE FOLLOWING CHANGES HAVE OCCURRED TO THE HARDCOPY OR ELECTRONIC MANUAL ASSIGNED TO YOU:

111 - 111 - TSC LEAD ENGINEER

REMOVE MANUAL TABLE OF CONTENTS DATE: 07/02/2003

ADD MANUAL TABLE OF CONTENTS DATE: 07/30/2003

CATEGORY: PROCEDURES TYPE: EP

ID: EP-PS-111

ADD: PCAF 2003-1551 REV: N/A

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A045

PROCEDURE CHANGE PROCESS FORM

1. PCAF NO. 2003-1551 | 2. PAGE 2 OF 3 | 3. PROC. NO. EP-PS-111 REV. 4

11. This question documents the outcome of the 50.59 and 72.48 Review required by NDAP-QA-0726. Either 11a, b, c or d must be checked "YES" and the appropriate form attached or referenced.
- a. This change is an Administrative Correction for which 50.59 and 72.48 are not applicable. YES N/A
- b. This change is a change to any surveillance, maintenance or administrative procedure for which 50.59 and 72.48 are not applicable. YES N/A
- c. This change is bounded by a 50.59/72.48 Screen/Evaluation, therefore, no new 50.59/72.48 Evaluation is required. YES N/A
Screen/Evaluation No. _____
- d. 50.59 and/or 72.48 are applicable to this change and a 50.59/72.48 Screen/Evaluation is attached. YES N/A
12. This change is consistent with the FSAR or an FSAR change is required. YES
Change Request No. _____
13. Should this change be reviewed for potential effects on Training Needs or Material? YES NO
If YES, enter an Action Item @ NIMS/Action/Gen Work Mech/PICN _____
14. Is a Surveillance Procedure Review Checklist required per NDAP-QA-0722? YES NO
15. Is a Special, Infrequent or Complex Test/Evolution Analysis Form required per NDAP-QA-0320? (SICT/E form does not need to be attached.) YES NO

16. Reviews may be documented below or by attaching Document Review Forms NDAP-QA-0101-1.

REVIEW	REVIEWED BY WITH NO COMMENTS	DATE
QADR	_____	_____
TECHNICAL REVIEW	_____	_____
REACTOR ENGINEERING/NUCLEAR FUELS *	_____	_____
IST **	_____	_____
OPERATIONS	_____	_____
NUCLEAR SYSTEMS ENGINEERING	_____	_____
NUCLEAR MODIFICATIONS	_____	_____
MAINTENANCE	_____	_____
HEALTH PHYSICS	_____	_____
NUCLEAR TECHNOLOGY	_____	_____
CHEMISTRY	_____	_____
OTHER <u>10 CFR 50.54Q</u>	<u><i>J. Dalry</i></u>	<u>7/25/03</u>

* Required for changes that affect, or have potential for affecting core reactivity, nuclear fuel, core power level indication or impact the thermal power heat balance. ⁽⁵⁸⁾

** Required for changes to Section XI Inservice Test Acceptance Criteria.

SUPPORTING INFORMATION:

TAB:

Emergency Telephone Instructions	TAB 1
Emergency Organization	TAB 2
Logkeeping	TAB 3
TSC Library Floor Plan for Work Stations	TAB 4
Emergency Facility Form Flow	TAB 5
Emergency Classification	TAB 6
Big Picture Status	TAB 7
Emergency Forms o Emergency Notification Report	TAB 8
Anticipated Question List	TAB 9
Intentionally Blank	TAB 10
<u>Core Damage Estimate I</u>	<u>Tab 11</u>
<u>Core Damage Estimate II</u>	<u>Tab 12</u>
<u>Fuel Damage Worksheet</u>	<u>Tab 13</u>

PCAF

REFERENCES:

SSES Emergency Plan

Calculation #M-RAF-024, Rev. 0 "COTTAP Analysis - Post DBA Reactor Building Temperature Issue"

NUREG-0654, Planning Standards and Evaluation Criteria

NUREG-0731, Guidelines for Utility Management Structure and Technical Resources, 1980

SEA-ME-096, Appendix R Study

EWR #M70777, "Post Accident - Reactor Building Temperature Issue"

SEA-EE-063, Rev. 0 "Post LOCA DBA Reactor Building Temperatures - Electrical Heat Loads"

ANTICIPATED QUESTION LIST

The following questions are intended to provide an overview of the event that precipitated entry into the SSES EMERGENCY PLAN. They will be used by managers in the emergency response organization to better understand the situation and answer questions posed by offsite agencies and regulators.

QUESTION #1: (TECHNICAL SUPPORT COORDINATOR/ENGINEERING SUPPORT SUPERVISOR)

What is the status of all three fission product boundaries? Indicate what data supports each determination.

- a. Fuel cladding
- b. Reactor coolant pressure boundary
- c. Primary containment

QUESTION #2: (TECHNICAL SUPPORT COORDINATOR/ENGINEERING SUPPORT SUPERVISOR)

For intact barriers, what threats exist to their continued integrity? For degraded barriers, what is the potential for further degradation? Indicate what data supports each determination.

- a. Fuel cladding
- b. Reactor coolant pressure boundary
- c. Primary containment

ANTICIPATED QUESTION LIST

QUESTION #3: (TECHNICAL SUPPORT COORDINATOR/ENGINEERING SUPPORT SUPERVISOR)

- a. How is the core being cooled?
- b. How do you know that the cooling system(s) in service are adequately removing heat from the core?
- c. What backup cooling systems are available?

QUESTION #4: (TECHNICAL SUPPORT COORDINATOR/ENGINEERING SUPPORT SUPERVISOR)

- a. How do you know that the core is in a coolable configuration?
- b. What is the current estimate of core damage?
- c. What is the prognosis for further degradation?

QUESTION #5: (TECH SUPPORT COORDINATOR)

- a. How do you correlate the in-plant radiological data with the in-plant system parameter data to support your understanding of the situation?
- b. What is the status of off-site radiological releases?

ANTICIPATED QUESTION LIST

- c. If releases are occurring:
 - 1) What is the release path?
 - 2) Is the release monitored?
 - 3) Is the release filtered?
 - 4) What is the potential for increased release levels? When?
 - 5) What is the potential for termination of the release? When?

- c. If releases are not currently occurring:
 - 1) What are the potentially releasable source terms?
 - 2) What is the status of SBGTS?

QUESTION #6: (DOSE ASSESSMENT SUPERVISOR/RADIATION PROTECTION COORDINATOR)

- a. If releases are occurring, what are the off-site release consequences?

- b. What is the potential for the release to change and what would be the off-site consequences of the postulated release?