

August 22, 2003

MEMORANDUM TO: Laura Dudes, Section Chief
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New, Research and Test Reactors Program
Division of Regulatory Improvement Programs, NRR

FROM: Joseph Colaccino, Senior Project Manager */RA/*
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New, Research and Test Reactors Program
Division of Regulatory Improvement Programs, NRR

SUBJECT: JUNE 26, 2003, AP1000 TELEPHONE CONFERENCE CALL SUMMARY

On Thursday, June 26, 2003, a telephone conference call was held with Westinghouse Electric Company (Westinghouse) representatives and Nuclear Regulatory Commission (NRC) staff to discuss AP1000 draft safety evaluation report open items (DSER OIs) 2.3.4-1, 14.3.4-1, 15.3-1, 15.3-2, and 15.3.6-1. The call participants are listed in Attachment 1. A summary of the conference call is included below.

OI 2.3.4-1:

This OI is related to NRC Regulatory Guide (RG) 1.194, "Atmospheric Relative Concentrations for Control Room Radiological Habitability Assessments at Nuclear Power Plants," which was issued in June 2003 (ADAMS Accession Number ML031530505). Westinghouse stated that the calculations related to this OI were performed in 1997 for the AP600. The NRC staff stated that Westinghouse should compare the inputs and assumptions used in its calculations of the X/Q values used in the control room dose analysis with those in RG 1.194. Westinghouse agreed to address this OI by reviewing their calculations and addressing and justifying the differences between their calculation methodology and the methodology discussed in RG 1.194.

Westinghouse and the NRC staff also discussed Table 1 in the Westinghouse revision 1 response to NRC request for additional information number 451.006.

OI 14.3.4-1:

Westinghouse stated that the control room values were not a Tier 1 site parameter in any other advanced light water reactor certification and there was no need to deviate for the AP1000. The NRC staff stated that offsite χ/Q s are Tier 1. The NRC staff plans to evaluate this issue upon resolution of OI 2.3.4-1.

OI 15.3-1:

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The NRC staff had not completed their independent evaluation of the use of the AP600 aerosol removal coefficients at the time of the phone call.

OI 15.3-2:

The NRC staff pointed out that this OI, which involves an evaluation of the hypothetical reference control room atmospheric dispersion factors, is dependent on the successful resolution of OIs 2.3.4-1 and 6.4-1. The NRC staff noted that preliminary analyses are acceptable. However, control room dose results for the steam generator tube rupture and main steam line break accidents may be impacted if the control room χ/Qs were to increase.

OI 15.3.6-1:

The NRC staff had not completed their independent evaluation of the loss-of-coolant accident radiological consequences analysis at the time of the phone call.

Docket No. 52-006

Attachment: As stated

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OI 15.3-2:

The NRC staff pointed out that this OI, which involves an evaluation of the hypothetical reference control room atmospheric dispersion factors, is dependent on the successful resolution of OIs 2.3.4-1 and 6.4-1. The NRC staff noted that preliminary analyses are acceptable. However, control room dose results for the steam generator tube rupture and main steam line break accidents may be impacted if the control room χ/Qs were to increase.

OI 15.3.6-1:

The NRC staff had not completed their independent evaluation of the loss-of-coolant accident radiological consequences analysis at the time of the phone call.

Docket No. 52-006

Attachment: As stated

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JUNE 26, 2003
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