

Fort Calhoun Station
P.O. Box 550, Highway 75
Fort Calhoun, NE 68023-0550

August 7, 2003
LIC-03-0112

U. S. Nuclear Regulatory Commission
ATTN.: Document Control Desk
Washington, DC 20555

References: 1. Docket No. 50-285
2. Letter from OPPD (R. L. Phelps) to NRC (Document Control Desk) dated July 7, 2003 (LIC-03-0089)

SUBJECT: License Renewal Safety Evaluation Report for Fort Calhoun Station, Unit 1- Clarification of Responses to Open Items

The Reference 2 letter transmitted the Omaha Public Power District (OPPD) comments and responses to the Open Items and Confirmatory Items contained in the License Renewal Safety Evaluation Report for Fort Calhoun Station, Unit 1. Based on NRC Staff review of the Open Items responses, additional clarifying information from OPPD for two Open Items is required. This letter provides that information.

Open Item 4.7-1

Open Item 4.7-1 concerns the pressurizer lower shell j-weld. On Page 18 of Attachment 1 of the Reference 2 letter, OPPD provided a new Section 4.7.4 of the Fort Calhoun Station License Renewal Application (LRA) to document the Time-Limited Aging Analysis for the j-weld. NRC Staff review determined that a corresponding Updated Safety Analysis Supplement is also required. Thus, OPPD provides the following new section for Appendix A of the LRA:

A.3.6.4 Pressurizer Alloy 600 J-Weld Left in Place

Omaha Public Power District (OPPD) has evaluated the crack of the temperature nozzle in the pressurizer lower shell, and any potential future growth of the crack, and determined it does not impact the structural integrity of the vessel for the current licensed 40 year life. OPPD has elected to defer completion of the evaluation that demonstrates that the crack and any potential future growth of the crack does not impact the structural integrity of the vessel for the period of extended operation. OPPD will perform the fracture mechanics evaluation of the small-bore instrument nozzle j-weld region at the repaired instrument nozzle in the side of the pressurizer lower shell. This evaluation will include bounding the flaw size by the size of the j-weld itself, and addressing the possibility of corrosion in the presence of a flaw. The fracture mechanics evaluation of the small-bore instrument nozzle j-weld region at the repaired instrument nozzle shall be performed using the methodology and criteria described in the OPPD letter dated July 7, 2003. Proposed corrective actions have been identified to provide

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reasonable assurance that the component in question will perform its intended function when called upon, or will not be outside of its design basis established by the plant's current license basis. Such corrective actions include assuring that the pressure at any temperature should not be any higher than the higher of the following two limits: [1] The saturation pressure plus 200 psi, or [2] 350 psi and maximum rate of temperature decrease is 200°F/hr.

OPPD will submit a License Amendment Request, containing the fracture mechanics and corrosion evaluations of the small-bore instrument nozzle j-weld region at the repaired instrument nozzle described above, to the NRC before the period of extended operation.

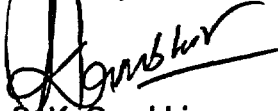
Open Item 3.0-1

The Reference 2 letter included a Summary of Revisions to FCS LRA Tables matrix. In Item 79 of this matrix, OPPD clarified that the One Time Inspection Program is credited for the reactor coolant pump (RCP) oil collection system (AMR links 3.3.2.73 and 3.3.2.75). Although this is not consistent with NUREG-1801, the NRC Staff found it acceptable because some components of the RCP oil collection system at FCS are made from stainless steel (e.g., drip pans, collection tank and drain piping attached to the drip pans), instead of carbon steel as is reflected in NUREG-1801. However, that clarification generated an inconsistency within AMR link 3.3.1.06 in Table 3.3.1 of the LRA. This is the NUREG-1801 item for the RCP oil collection system, and it contains recommendations for further evaluation of the aging effects. Upon further evaluation of matrix Item 79, OPPD has concluded that AMR link 3.3.1.06 was inadvertently deleted from LRA Table 2.3.3.14-1. The FCS RCP oil collection system does contain copper drain piping from the drip pans to the collection tank. As a result, the AMR link to 3.3.1.06 is correct, including the associated AMP of One Time Inspection. Therefore, the change in matrix Item 79 to delete AMR link 3.3.1.06 has been rescinded.

No new or additional commitments to the NRC are included in this letter. If you have any questions or require additional information, please contact T. C. Matthews at (402) 533-6938.

I declare under penalty of perjury that the foregoing is true and correct. (Executed on August 7, 2003)

Sincerely,



S.K. Gambhir
Division Manager
Nuclear Projects

TCM/tcm

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