



444 South 16th Street Mall  
Omaha NE 68102-2247

August 5, 2003  
LIC-03-0110

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

- References:
1. Docket No. 50-285
  2. Letter from OPPD (D. J. Bannister) to NRC (Document Control Desk) dated October 8, 2002, Fort Calhoun Station Unit No. 1 License Amendment Request, "Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR)" (LIC-02-0109)
  3. Letter from OPPD (R. T. Ridenoure) to NRC (Document Control Desk) dated April 10, 2003, Response to Request For Additional Information, Pressure-Temperature Limit Report Amendment Request (LIC-03-0052)
  4. Letter from OPPD (R. T. Ridenoure) to NRC (Document Control Desk) dated May 20, 2003, Response to Request for Additional Information, Pressure-Temperature Limits Report Amendment Request; Low Temperature Over Pressure (LIC-03-0081)
  5. Letter from OPPD (R. T. Ridenoure) to NRC (Document Control Desk) dated July 31, 2003, Fort Calhoun Station Unit No. 1, Technical Specifications Updated for NRC Comments on Pending Amendment Request on the Pressure-Temperature Limits Report (LIC-03-0104)

**SUBJECT: Fort Calhoun Station Unit No. 1 "Technical Specifications Updated for NRC Comments on Pending Amendment Request on the Pressure-Temperature Limits Report"**

In support of the license amendment request, "Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR)" (Reference 2), the Omaha Public Power District (OPPD) provided responses to the Nuclear Regulatory Commission (NRC) Requests for Additional Information in Reference 3 and 4. This transmittal re-submits Page 5-10a of the proposed Technical Specifications which were originally submitted by Reference 5. The attached proposed Technical Specifications have been updated to include an additional reference to Technical Specification 5.9.6 requested by the NRC during their review of the amendment request.

I declare under penalty of perjury that the forgoing is true and correct (Executed on August 5, 2003). No commitments are made to the NRC in this letter.

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If you have any questions or require additional information, please contact Dr. R. L. Jaworski of the Fort Calhoun Station Licensing staff at (402) 533-6833.

Sincerely,

A handwritten signature in black ink, appearing to read 'R. P. Clemens', with a long horizontal line extending to the right.

R. P. Clemens  
Division Manager  
Nuclear Assessments

RPC/rrl

Attachments: 1. Marked-Up - Technical Specifications Updated for Pending Amendment of the Pressure-Temperature Limits Report - Replacement Page  
2. Clean Copy - Technical Specifications Updated for Pending Amendment of the Pressure-Temperature Limits Report - Replacement Page

c: T. P. Gwynn, Acting Regional Administrator, NRC Region IV  
A. B. Wang, NRC Project Manager  
J. G. Kramer, NRC Senior Resident Inspector  
Division Administrator, Public Health Assurance, State of Nebraska

**Attachment 1**

**Marked-Up  
Technical Specifications  
Updated for Pending Amendment  
of the  
Pressure-Temperature Limits Report  
Replacement Page**

## TECHNICAL SPECIFICATIONS

### 5.0 ADMINISTRATIVE CONTROLS

#### 5.9.6 Reactor Coolant System (RCS) Pressure - Temperature Limits Report (PTLR)

- a. RCS pressure and temperature limits for heatup, cooldown, low temperature overpressure protection, criticality, and hydrostatic testing as well as heatup and cooldown rates shall be established and documented in the PTLR for Technical Specifications 2.1.1 and 2.1.2.
- b. The analytical methods used in the PTLR shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:
  1. CE NPSD-683-A, Revision 6, "Development of a RCS Pressure and Temperature Limits Report for the Removal of P-T Limits and LTOP Requirements from the Technical Specifications," April 2001.
  2. WCAP-15443, Revision 0, "Fast Neutron Fluence Evaluations for the Fort Calhoun Unit 1 Reactor Pressure Vessel," July 2000.
  3. Safety Evaluation by the Office of Nuclear Reactor Regulation Related to Amendment Number 199 to Facility Operating License DPR-40 Omaha Public Power District Fort Calhoun Station, Unit Number 1, dated June 7, 2001.
  4. CEN-636, Revision 2, "Evaluation of Reactor Vessel Surveillance Data Pertinent to the Fort Calhoun Reactor Vessel Beltline Materials, dated July 2000.
  5. FC06876, Revision 0, "Performance of Low Temperature Overpressure Protection System Analyses Using RELAP5: Methodology Paper."
  6. FC06877, "Low Temperature Overpressure Protection (LTOP) Analysis, Revision 1."
  7. Safety Evaluation by the Office of Nuclear Reactor Regulation Related to Amendment Number 207 to Facility Operating License Number DPR-40 Omaha Public Power District Fort Calhoun Station, Unit Number 1, dated April 22, 2002.
  8. Letter LTR-CI-01-25, Revision 0 from Westinghouse Electric Company (S.T. Byrne) to OPPD (J. Jensen), "Assessment of Extended Beltline Limit for Fort Calhoun Station Reactor Pressure Vessel," dated December 18, 2001.
  9. WCAP-15741, Revision 0, "Reactor Vessel Surveillance Program Withdrawal Schedule Modifications," dated September 2001.
  10. Letter from NRC (A. B. Wang) to Omaha Public Power District (R. T. Ridenoure), Fort Calhoun Station - Unit 1, Exemption from the Requirements of Appendix G to 10 CFR Part 50 (TAC No. MB8237), dated July 30, 2003.
  11. Letter from Information Systems Laboratories (William Arcieri) to OPPD (J. Jensen), "WCA-09-2002: Transmittal of RELAP5/MOD3.2d," dated August 2, 2002.
- c. The PTLR shall be provided to the NRC upon issuance for each reactor vessel fluence period (i.e., the number of EFPY used in the P-T limit/LTOP analysis) and for any revision or supplement thereto.

**Attachment 2**

**Clean Copy**  
**Technical Specifications**  
**Updated for Pending Amendment**  
**of the**  
**Pressure-Temperature Limits Report**  
**Replacement Page**

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