

August 4, 2003

Mr. Michael Kansler
President
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 - ISSUANCE OF
AMENDMENT RE: CITY WATER TANK (TAC NO. MB5506)

Dear Mr. Kansler:

The Commission has issued the enclosed Amendment No. 218 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated June 24, 2002, as supplemented on June 23, 2003.

The amendment revises TS Surveillance Requirement (SR) 3.7.7.2 to require all city water header isolation valves be open rather than only the one header supply isolation valve. In its June 23, 2003, letter, Entergy Nuclear Operations, Inc. withdrew its request for changes to SR 3.7.7.1 pertaining to the city water tank volume.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

/RA/

Patrick D. Milano, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosures: 1. Amendment No. 218 to DPR-64
2. Safety Evaluation

cc w/encls: See next page

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Accession Number: ML032160069

*See previous concurrence

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DATE	07/16/03	07/17/03	07/14/03	07/22/03	08/1/03

Official Record Copy

DATED: August 4, 2003

AMENDMENT NO. 218 TO FACILITY OPERATING LICENSE NO. DPR-64 INDIAN POINT
UNIT 3

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ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 218
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Nuclear Operations, Inc. (the licensee) dated June 24, 2002, as supplemented on June 23, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 218, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Richard J. Laufer, Chief, Section I
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 4, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 218

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page

3.7.7-2

Insert Page

3.7.7-2

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 218 TO FACILITY OPERATING LICENSE NO. DPR-64
ENTERGY NUCLEAR OPERATIONS, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

1.0 INTRODUCTION

By letter dated June 24, 2002, as supplemented on June 23, 2003, Entergy Nuclear Operations, Inc. (the licensee) submitted a request for changes to the Indian Point Nuclear Generating Unit No. 3 (IP3) Technical Specifications (TSs). The requested changes would revise the TS surveillance requirements (SRs) for the city water system. The June 23, 2003, letter provided clarifying information that did not enlarge the scope of the original *Federal Register* notice or change the initial proposed no significant hazards consideration determination.

2.0 REGULATORY EVALUATION

The U.S. Nuclear Regulatory Commission (NRC) staff finds that the licensee in its June 24, 2002, application addressed the applicable regulatory requirements. Title 10 of the *Code of Federal Regulations* (10 CFR) establishes the fundamental regulatory requirements. The regulatory requirements upon which the staff based its review of the application are as follows:

1. Criterion 3 of 10 CFR 50.36(c)(2)(ii) requires that a TS limiting condition for operation (LCO) be established for a structure, system or component that is part of the primary success path and which functions or actuates to mitigate a design-basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
2. The requirements include several General Design Criteria (GDC) in Appendix A, "General Design Criteria for Nuclear Power Plants," to 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities," namely:
 - a. GDC-2, "Design bases for protection against natural phenomena," states, in part, that structures, systems, and components important to safety shall be designed to withstand the effects of natural phenomena.
 - b. GDC-3, "Fire protection," states, in part, that firefighting systems of appropriate capacity and capability shall be designed and provided to minimize the adverse affects of fires on structures, systems, and components important to safety.

- c. GDC-4, "Environmental and missile design bases," states, in part, that the system be capable of withstanding the effects of external missiles.
 - d. GDC-44, "Cooling water," requires, in part, that the system has the capability to transfer heat loads from the reactor system to a heat sink under both normal operating and accident conditions. It should also have component redundancy so that safety functions can be performed assuming a single active failure, and be capable to isolate components and piping if required to prevent loss of safety function.
3. IP3 Final Safety Analysis Report (FSAR) Section 10.2.6, "Condensate and Feedwater System," states that the water supply source for the auxiliary feedwater system is redundant. In this regard, an alternate supply is provided by a connection to the City Water System. Also, FSAR Section 9.6.2.3, "Fire Suppression Systems," describes the city water system as the supply of water to the fire water storage tanks.

3.0 TECHNICAL EVALUATION

3.1 Background

Auxiliary Feedwater System

In addition to the redundancy afforded by the multiple pumping systems with independent motive power sources, the water supply source for the auxiliary feedwater system is also redundant. The main water source is provided by gravity feed from the condensate storage tank (CST). The minimum water maintained in the CST is sufficient to remove residual heat generation for 24 hours at hot shutdown condition following a trip from full power. Since the CST was not designed to withstand the effects of a tornado-generated missile, an alternative supply is available through a connection to the city water main and the 1.5 million gallon city water tank. The city water supply will be used only if the CST supply is exhausted or otherwise not available.

Fire Suppression System

The supply of water to the fire suppression system and to the two 350,000 gallon fire water storage tanks is from the city water system. The city water supply to the fire water storage tanks is automatically controlled to maintain a minimum volume in each tank. The licensee's modification to add isolation valves does not change the configuration and control of the city water supply for fire protection.

3.2 Licensee's Proposed TS Change

In its letter dated June 24, 2002, the licensee proposed revisions to TS SR 3.7.7.1 and SR 3.7.7.2. Specifically, the licensee proposed changes to SR 3.7.7.1 to require the verification of the city water tank volume rather than city water header pressure and to increase the SR frequency from 12 hours to 24 hours. The licensee also proposed changes to SR 3.7.7.2 to require all city water header isolation valves are open rather than only the one header supply isolation valve.

By letter dated June 23, 2003, the licensee stated that it had re-evaluated the proposed change to the SR for city water tank volume. The intent of the change was to provide added assurance that adequate volume of water was available. However, the licensee has concluded that the intent can be met by including this information in the FSAR. Thus, the licensee withdrew the proposed change to SR 3.7.7.1.

3.3 Staff Evaluation

In its June 24, 2002, application, the licensee indicated that, as part of a back-flow preventer modification, isolation valves were added in the flow path to the auxiliary feedwater pump suction from the city water tank. The modification included the addition of two manual isolation valves, CT-1300 and CT-1302. The current TS Bases for SR 3.7.7.2 states the valve, CT-49, located in the Unit 1 Utility Tunnel, is periodically verified to be locked open. Thus, the licensee proposed to add isolation valves CT-1300 and CT-1302 to the TS Bases and to revise SR 3.7.7.2 to verify that all of these city water header supply isolation valves are open.

The NRC staff finds that the isolation valves CT-1300 and CT-1302 need to be open along with CT-49 in order for the city water to be available in the event it is needed as a backup to the CST as a supply for the auxiliary feedwater system. Therefore, the staff finds the proposed change to SR 3.7.7.2 is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a surveillance requirement. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (67 FR 50952). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Milano

Date: August 4, 2003