

10 CFR 50.90

July 22, 2003  
5928-03-20158U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555Three Mile Island, Unit 1  
Operating License No. DPR-50  
NRC Docket No. 50-289Subject: License Amendment Request No. 318  
Integrated Leak Rate Test Deferral  
Submission of Camera Ready Page

- References:
1. Letter from M. P. Gallagher (AmerGen Energy Company, LLC) to U. S. Nuclear Regulatory Commission, dated September 30, 2002
  2. Letter from T. G. Colburn (U. S. Nuclear Regulatory Commission) to J. L. Skolds (AmerGen Energy Company, LLC), dated February 28, 2003
  3. Letter from M. P. Gallagher (AmerGen Energy Company, LLC) to U. S. Nuclear Regulatory Commission, dated March 19, 2003

In the Reference 1 letter, AmerGen Energy Company (AmerGen), LLC, submitted information concerning License Amendment Request No. 318, in accordance with 10 CFR 50.90, requesting an amendment to the Technical Specifications of Operating License No. DPR-50, for Three Mile Island, Unit 1. This proposed change will revise Technical Specifications (TS) Section 6.8.5 ("Reactor Building Leakage Rate Testing Program") to reflect a one-time deferral of the scheduled performance of the next Type A Containment Integrated Leak Rate Test (ILRT) from October 2003 to no later than September 2008. Attached is an updated camera ready page associated with this change.

A017

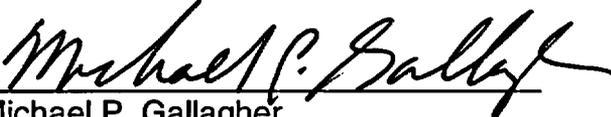
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If you have any questions or require additional information, please contact John Hufnagel at (610) 765-5507.

I declare under penalty of perjury that the foregoing is true and correct.

Respectfully,

07-22-03  
Executed on

  
Michael P. Gallagher  
Michael P. Gallagher  
Director, Licensing and Regulatory Affairs  
AmerGen Energy Company, LLC

Attachment: Page 6-11c

cc: H. J. Miller, Administrator, Region I, USNRC  
C. W. Smith, USNRC, Senior Resident Inspector, TMI-1  
D. M. Skay, USNRC, Senior Project Manager  
File No. 02079

#### 6.8.5 Reactor Building Leakage Rate Testing Program

The Reactor Building Leakage Rate Testing Program shall be established, implemented, and maintained as follows:

A program shall be established to implement the leakage rate testing of the Reactor Building as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995, as modified by the following exception to NEI 94-01, Rev. 0, "Industry Guideline for Implementing Performance-Based Option of 10 CFR Part 50, Appendix J":

- a. Section 9.2.3: The first Type A test performed after the September 1993 Type A test shall be performed no later than September 2008.

The peak calculated Reactor Building internal pressure for the design basis loss of coolant accident,  $P_{ac}$ , is 50.6 psig.

The maximum allowable Reactor Building leakage rate,  $L_a$ , shall be 0.1 weight percent of containment atmosphere per 24 hours at  $P_{ac}$ .

Reactor Building leakage rate acceptance criteria is  $\leq 1.0 L_a$ . During the first plant startup following each test performed in accordance with this program, the leakage rate acceptance criteria are  $\leq 0.60 L_a$  for the Type B and Type C tests and  $\leq 0.75 L_a$  for the Type A tests.