

July 17, 2003

Mr. A. Christopher Bakken III, Senior Vice President  
and Chief Nuclear Officer  
Indiana Michigan Power Company  
Nuclear Generation Group  
500 Circle Drive  
Buchanan, MI 49107

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNIT 2 - REQUEST FOR ADDITIONAL  
INFORMATION, "LICENSE AMENDMENT REQUEST TO REVISE LOW  
PRESSURIZER PRESSURE SAFETY INJECTION SETPOINT," (TAC NO.  
MB8202)

Dear Mr. Bakken:

On March 27, 2003, the Indiana Michigan Power Company (I&M) proposed to amend Appendix A, Technical Specifications (TS), of Facility Operating License DRP-74 for Donald C. Cook Nuclear Plant, Unit 2. The proposed amendment includes changes to TS Table 3.3-4, Item 1.d, and the P-11 setpoint in the Engineered Safety Features Interlock Table. Specifically, the proposed amendment would (1) revise the low pressurizer pressure safety injection (SI) trip setpoint from its current value of greater than or equal to 1900 pounds per square inch (psig), to greater than or equal to 1815 psig; (2) revise the low pressurizer pressure SI allowable value from greater than or equal to 1890 psig, to greater than or equal to 1805 psig; (3) revise the P-11 setpoint from its current value of greater than or equal to 2010 psig, to greater than or equal to 1915 psig; and (4) make format changes to the affected TS pages that improve appearance but do not affect any requirements.

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the March 27, 2003, application and determined that the additional information identified in the enclosure is needed for the NRC staff to complete its review. The items in the enclosure were discussed with Mr. Joseph Waters of your staff and a mutually agreeable target date of August 8, 2003, for your response was established. If circumstances result in the need to revise the target date, please contact me at (301) 415-2859 at the earliest opportunity.

Sincerely,

*/RA/*

Mohammed Shuaibi, Senior Project Manager, Section 1  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-316

Enclosures: As stated

cc w/encls: See next page

July 17, 2003

Mr. A. Christopher Bakken III, Senior Vice President  
and Chief Nuclear Officer  
Indiana Michigan Power Company  
Nuclear Generation Group  
500 Circle Drive  
Buchanan, MI 49107

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNIT 2 - REQUEST FOR ADDITIONAL  
INFORMATION, "LICENSE AMENDMENT REQUEST TO REVISE LOW  
PRESSURIZER PRESSURE SAFETY INJECTION SETPOINT," (TAC NO.  
MB8202)

Dear Mr. Bakken:

On March 27, 2003, the Indiana Michigan Power Company (I&M) proposed to amend Appendix A, Technical Specifications (TS), of Facility Operating License DRP-74 for Donald C. Cook Nuclear Plant, Unit 2. The proposed amendment includes changes to TS Table 3.3-4, Item 1.d, and the P-11 setpoint in the Engineered Safety Features Interlock Table. Specifically, the proposed amendment would (1) revise the low pressurizer pressure safety injection (SI) trip setpoint from its current value of greater than or equal to 1900 pounds per square inch (psig), to greater than or equal to 1815 psig; (2) revise the low pressurizer pressure SI allowable value from greater than or equal to 1890 psig, to greater than or equal to 1805 psig; (3) revise the P-11 setpoint from its current value of greater than or equal to 2010 psig, to greater than or equal to 1915 psig; and (4) make format changes to the affected TS pages that improve appearance but do not affect any requirements.

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the March 27, 2003, application and determined that the additional information identified in the enclosure is needed for the NRC staff to complete its review. The items in the enclosure were discussed with Mr. Joseph Waters of your staff and a mutually agreeable target date of August 8, 2003, for your response was established. If circumstances result in the need to revise the target date, please contact me at (301) 415-2859 at the earliest opportunity.

Sincerely,  
/RA/

Mohammed Shuaibi, Senior Project Manager, Section 1  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-316  
Enclosures: As stated  
cc w/encls: See next page  
DISTRIBUTION

WBeckner  
PUBLIC PD 3-1 r/f OGC  
AVegel, RIII ACRS MBarrilla JUhle

**ADAMS ACCESSION NUMBER: ML031960668** \*By email

<b>OFFICE</b>	<b>PM:PD3-1</b>	<b>LA:PD3-1*</b>	<b>SC:SRXB</b>	<b>SC:PD3-1</b>
<b>NAME</b>	<b>MShuaibi</b>	<b>THarris</b>	<b>LLois for JUhle</b>	<b>LRaghavan</b>
<b>DATE</b>	<b>07/16/03</b>	<b>07/16 /03</b>	<b>07/17/03</b>	<b>07/17/03</b>

OFFICIAL RECORD COPY

Donald C. Cook Nuclear Plant, Units 1 and 2

cc:

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
801 Warrenville Road  
Lisle, IL 60532-4351

Attorney General  
Department of Attorney General  
525 West Ottawa Street  
Lansing, MI 48913

Township Supervisor  
Lake Township Hall  
P.O. Box 818  
Bridgman, MI 49106

U.S. Nuclear Regulatory Commission  
Resident Inspector's Office  
7700 Red Arrow Highway  
Stevensville, MI 49127

David W. Jenkins, Esquire  
Indiana Michigan Power Company  
One Cook Place  
Bridgman, MI 49106

Mayor, City of Bridgman  
P.O. Box 366  
Bridgman, MI 49106

Special Assistant to the Governor  
Room 1 - State Capitol  
Lansing, MI 48909

Drinking Water and Radiological  
Project Division  
Michigan Department of  
Environmental Quality  
3423 N. Martin Luther King Jr. Blvd.  
P. O. Box 30630, CPH Mailroom  
Lansing, MI 48909-8130

Scot A. Greenlee  
Director, Nuclear Technical Services  
Indiana Michigan Power Company  
Nuclear Generation Group  
500 Circle Drive  
Buchanan, MI 49107

David A. Lochbaum  
Union of Concerned Scientists  
1616 P Street NW, Suite 310  
Washington, DC 20036-1495

Michael J. Finissi  
Plant Manager  
Indiana Michigan Power Company  
Nuclear Generation Group  
One Cook Place  
Bridgman, MI 49106

Joseph E. Pollock  
Site Vice President  
Indiana Michigan Power Company  
Nuclear Generation Group  
One Cook Place  
Bridgman, MI 49106

DONALD C. COOK NUCLEAR PLANT, UNIT 2  
REQUEST FOR ADDITIONAL INFORMATION  
CHANGES TO LOW PRESSURIZER PRESSURE SAFETY INJECTION SETPOINT

1. The licensee states, "Approval of these changes will alleviate an operator concern that a safety injection (SI) actuation is imminent following a reactor trip." The staff understands this is proposed to reduce the number of SI signals that result from transients that do not require SI and to reduce necessary operator actions. Please justify how the change in the setpoint will allow the plant to successfully mitigate transients that require SI.
2. By changing the low pressurizer pressure SI trip setpoint, a time delay is introduced from the time the reactor trips to the time SI is actuated. Are there any effects on the plant due to this time delay? Will there be changes to the emergency operating procedures with respect to the new SI setpoint? What kind of training will operators receive to these procedural changes?
3. Describe the methodology used to determine the new SI setpoint and the analysis which demonstrates the new setpoint is still bounded by the uncertainty margin.
4. The licensee states there was a design change that provides a 3.5 second delay to the auxiliary feedwater flow retention circuit which exacerbates the operator's concern about reactor coolant system cooling. What is the purpose of the 3.5 second delay? What would be the consequence of eliminating the 3.5 second delay?
5. Which licensing basis transient and accident analyses have been reevaluated to confirm lowering the SI setpoint is acceptable? Are there any effects on departure from nucleate boiling or fuel design limits due to this change?
6. In your application you indicated that a loss-of-coolant accident (LOCA), a feedwater line break, and an inadvertent depressurization of the main steam system are affected by the low pressurizer pressure SI setpoint. Please confirm that no other transients are affected by the low pressurizer pressure SI setpoint. Describe in detail all transients, including but not limited to LOCA, feedwater line break, and inadvertent depressurization of the main steam system, that are affected by this modification and demonstrate that the new SI setpoint doesn't hamper the system's ability to successfully mitigate them.

ENCLOSURE