

July 15, 2003

Mr. L. William Pearce
Vice President
FirstEnergy Nuclear Operating Company
Beaver Valley Power Station
Post Office Box 4
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2 - ISSUANCE OF
AMENDMENT RE: CREATION OF PRESSURE-TEMPERATURE LIMITS
REPORT (TAC NOS. MB3319 AND MB3320)

Dear Mr. Pearce:

The Commission has issued the enclosed Amendment No. 256 to Facility Operating License No. DPR-66 and Amendment No. 138 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and 2), respectively. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated October 31, 2001, as supplemented by letters dated December 21, 2001, and February 4, May 31, and December 2, 2002.

These amendments relocate the pressure-temperature (P/T) limit curves and low temperature overpressure protection system limits to the Pressure and Temperature Limits Report (PTLR) in the BVPS-1 and 2 Licensing Requirements Manual and add a reference to that report in the affected TS Limiting Conditions for Operation and Bases. The changes also included the addition of the PTLR to the Definitions Section of the TSs and the addition of a new section to the reporting requirements in the Administrative Controls Section of the TSs delineating the necessary reports. The changes were based on Generic Letter 96-03, "Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits," dated January 31, 1996, and the Nuclear Regulatory Commission (NRC) staff's approval of the BVPS-1 and 2 plant-specific P/T limits methodology as documented in the letter from Richard J. Laufer, NRC, to Mark B. Bezilla, FirstEnergy Nuclear Operating Company, dated October 8, 2002.

The licensee's commitment to use only the calculated vessel fluence values when performing surveillance capsule evaluations, as documented in its December 21, 2001, letter, is reflected in the enclosed Amendments.

L. W. Pearce

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A copy of our safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Timothy G. Colburn, Senior Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosures: 1. Amendment No. 256 to DPR-66
2. Amendment No. 138 to NPF-73
3. Safety Evaluation

cc w/encls: See next page

L. W. Pearce

-2-

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Sincerely,

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Timothy G. Colburn, Senior Project Manager, Section 1
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1. Amendment No. 256 to DPR-66
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 3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION:

PUBLIC	MO'Brien	ACRS	PDI-1 R/F	TColburn	BPlatchek, RGN-I
OGC	CHaruck	RLauffer	GHill (4)	SMagruder	

ACCESSION NO. ML031960399

*No substantive changes made

OFFICE	PDI-1/PM	PDI-1/LA	RORP/SC	PDI-1/SC	OGC
NAME	TColburn	MO'Brien	RDennig	RLauffer	SCole
DATE	6/18/03	6/30/03	SE dated 3/12/03*	7/10/03	7/7/03

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PENNSYLVANIA POWER COMPANY
OHIO EDISON COMPANY
FIRSTENERGY NUCLEAR OPERATING COMPANY
DOCKET NO. 50-334
BEAVER VALLEY POWER STATION, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 256
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by FirstEnergy Nuclear Operating Company, et al. (the licensee), dated October 31, 2001, as supplemented by letters dated December 21, 2001, and February 4, May 31, and December 2, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 256, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The following is a condition of this amendment: the licensee has committed to use only the calculated vessel fluence values when performing future surveillance capsule evaluations, as documented in its December 21, 2001, letter.
4. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Richard J. Laufer, Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: July 15, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 256

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

II
XV
XIX
1-8
3/4 4-2c
3/4 4-6
3/4 4-22
3/4 4-23
3/4 4-24
3/4 4-25
3/4 4-27a
3/4 4-27b
3/4 5-6
3/4 5-7a
3/4 10-4
6-20 (2sheets)
--

Insert

II
XV
XIX
1-8
3/4 4-2c
3/4 4-6
3/4 4-22
3/4 4-23
--
--
3/4 4-27a
3/4 4-27b
3/4 5-6
3/4 5-7a
3/4 10-4
6-20
6-21

PENNSYLVANIA POWER COMPANY
OHIO EDISON COMPANY
THE CLEVELAND ELECTRIC ILLUMINATING COMPANY
THE TOLEDO EDISON COMPANY
FIRSTENERGY NUCLEAR OPERATING COMPANY
DOCKET NO. 50-412
BEAVER VALLEY POWER STATION, UNIT 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 138
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by FirstEnergy Nuclear Operating Company, et al. (the licensee), dated October 31, 2001, as supplemented by letters dated December 21, 2001, and February 4, May 31, and December 2, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 138, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. FENOC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The following is a condition of this amendment: the licensee has committed to use only the calculated vessel fluence values when performing future surveillance capsule evaluations, as documented in its December 21, 2001, letter.
4. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Richard J. Laufer, Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: July 15, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 138

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
II	II
XIV	XIV
XV	XV
--	1-7a
3/4 4-2	3/4 4-2
3/4 4-3	3/4 4-3
3/4 4-9	3/4 4-9
3/4 4-30	3/4 4-30
3/4 4-30a	3/4 4-30a
3/4 4-31	--
3/4 4-32	--
3/4 4-32a	--
3/4 4-32b	--
3/4 4-32c	--
3/4 4-32d	--
3/4 4-35	3/4 4-35
3/4 4-36	3/4 4-36
3/4 4-36a	3/4 4-36a
3/4 4-37	--
3/4 5-3	3/4 5-3
3/4 5-6	3/4 5-6
6-21 (2 sheets)	6-21
6-22	6-22
--	6-23

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 256 AND 138 TO FACILITY OPERATING
LICENSE NOS. DPR-66 AND NPF-73
PENNSYLVANIA POWER COMPANY
OHIO EDISON COMPANY
THE CLEVELAND ELECTRIC ILLUMINATING COMPANY
THE TOLEDO EDISON COMPANY
FIRSTENERGY NUCLEAR OPERATING COMPANY
BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-334 AND 50-412

1.0 INTRODUCTION

By application dated October 31, 2001, as supplemented by letters dated December 21, 2001, and February 4, May 31, and December 2, 2002, the FirstEnergy Nuclear Operating Company (FENOC, the licensee), requested changes to the Technical Specifications (TSs) for Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and 2). The supplements dated December 21, 2001, and February 4, May 31, and December 2, 2002, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the Federal Register on December 26, 2001 (66 FR 66465).

The proposed changes would relocate the pressure-temperature (P/T) limit curves and low temperature overpressure protection (LTOP) system limits to the Pressure and Temperature Limits Report (PTLR) in the BVPS-1 and 2 Licensing Requirements Manual and add a reference to that report in the affected TS limiting conditions for operation (LCOs) and Bases. The proposed changes also included the addition of the PTLR to the Definitions Section of the TSs and the addition of a new section to the reporting requirements in the Administrative Controls Section of the TSs delineating the necessary reports. The proposed changes were based on Generic Letter (GL) 96-03, "Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits," dated January 31, 1996, and the NRC staff's approval of the BVPS-1 and 2 plant-specific P/T limits methodology documented in the letter from Richard J. Laufer, NRC, to Mark B. Bezilla, FENOC, dated October 8, 2002.

2.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act (the Act) requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36. That regulation requires that the TSs include requirements in five specific categories: (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls, and states also that the Commission may include such additional TSs as it finds to be appropriate. However, the regulation does not specify the particular requirements to be included in a plant's TSs.

The Commission has provided guidance for the contents of TSs in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (Final Policy Statement), 58 FR 39132 (July 22, 1993), in which the Commission indicated that compliance with the Final Policy Statement satisfies Section 182a of the Act. In particular, the Commission indicated that certain items could be relocated from the TSs to licensee-controlled documents, consistent with the standard enunciated in Portland General Electric Co. (Trojan Nuclear Plant), ASLAB-531, 9 NRC 263, 273 (1979). In that case, the Atomic Safety and Licensing Appeal Board indicated that "technical specifications are to be reserved for those matters as to which the imposition of rigid conditions or limitations upon reactor operation is deemed necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety."

Consistent with this approach, the Final Policy Statement identified four criteria to be used in determining whether a particular matter is required to be included in the TSs, as follows: (1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety. The Commission promulgated a change to 10 CFR 50.36, which amends the rule to codify and incorporate these criteria. See Final Rule, "Technical Specifications," 60 FR 36953 (July 19, 1995). As a result, existing TS requirements which fall within or satisfy any of the four criteria in 10 CFR 50.36(c)(2)(ii) must be retained in the TSs, while those TS requirements which do not fall within or satisfy these criteria may be relocated to other licensee-controlled documents.

Appendix G to 10 CFR Part 50, "Fracture Toughness Requirements," specifies fracture toughness requirements for ferritic materials of pressure-retaining components of the reactor coolant pressure boundary to provide adequate margins of safety during any condition of normal operation, including anticipated operational occurrences and system hydrostatic tests, to which the pressure boundary may be subjected over its service lifetime. The P/T methodology used to develop the explanations, figures, values, and parameters to be maintained in a PTLR must conform to the requirements of Appendix G and approved exemptions.

In addition to the requirements of 10 CFR 50.36 and Appendix G, the NRC staff based the review of the licensee's proposed PTLR and associated TS changes on the guidance in GL 96-03. This guidance, which applies to the proposed changes, was developed by the NRC on the basis of a proposal by the owners groups during the development of the improved standard technical specifications (STs), and was provided to all power reactor licensees and applicants. According to this guidance, the applicant should have obtained NRC review and written approval of the P/T methodology and the proposed PTLR before the NRC approves TS changes associated with establishing a PTLR. These associated changes affect the definitions, LCOs, and administrative controls sections of the TSs. Specifically, the applicant should modify its plant TSs by adding:

- in the definitions section, the definition of a named formal report (PTLR or a similar document) that would contain the explanations, figures, values, and parameters (currently contained in TSs) derived in accordance with an NRC-approved methodology and consistent with all of the design assumptions and stress limits for cyclic operation;
- in affected LCOs, references to the PTLR that require maintaining the P/T limits within the limits specified in the PTLR, in place of the existing P/T limits explanations, figures, values, and parameters; and
- in the administrative controls section, a reporting requirement to submit the PTLR to the NRC, when it is issued, for each reactor vessel fluence period. The PTLR administrative controls specification should reference the document from the NRC that approved the supporting P/T methodology.

3.0 EVALUATION

All components of the reactor coolant system (RCS) are designed to withstand the effects of cyclic loads resulting from system pressure and temperature changes. These loads are introduced by heatup and cooldown operations, power transients, and reactor trips. In accordance with Appendix G to 10 CFR Part 50, TSs limit the pressure and temperature changes during RCS heatup and cooldown within the design assumptions and the stress limits for cyclic operation. These limits are defined by P/T limit curves for RCS heatup, cooldown, LTOP, and inservice leak and hydrostatic testing. Each curve defines an acceptable region for normal operation. The curves are used for operational guidance during RCS heatup and cooldown evolutions, when pressure and temperature indications are monitored and compared to the applicable curve to determine that operation is within the allowable region.

In addition to following Appendix G to determine the P/T limits for BVPS-1 and 2, the licensee used the American Society of Mechanical Engineers Boiler and Pressure Vessel Code Case N-640, which required an exemption from Appendix G. In its exemption request, the licensee justified why BVPS-1 and 2 cannot meet Appendix G and demonstrated that a specific exemption was warranted under 10 CFR 50.12. The NRC granted the exemptions on February 19, 2002, for BVPS-1 and on September 6, 2000, for BVPS-2.

The LTOP system controls RCS pressure at low temperatures so that the integrity of the reactor coolant pressure boundary is not compromised by violating Appendix G. The BVPS-1 and 2 systems for pressure relief at low RCS temperatures (the over pressure protection system, or OPPS) consists of the pressurizer power-operated relief valves (PORVs) and OPPS

unit-specific setpoints and enable temperatures. The OPPS is reevaluated each time the P/T limit curves are revised to ensure that it is capable of performing its intended function.

The licensee-proposed changes to the TSs are in accordance with the guidance in GL 96-03, as follows:

- (1) The definitions section of the TS is modified to include a definition of the PTLR to which the figures, values, and parameters for P/T and OPPS limits will be relocated on a unit-specific basis. These figures, values, and parameters are established in accordance with an NRC-approved methodology that maintains the P/T acceptance limits and the P/T limits of the safety analysis. As noted in the definition, plant operation within these limits is addressed by individual specifications. The definition (TS 1.38) is as follows:

The PTLR is the unit specific document that provides the reactor vessel pressure and temperature (P/T) limits, including heatup and cooldown rates and Overpressure Protection System setpoint and enable temperature, for the current reactor vessel fluence period. These pressure and temperature limits shall be determined for each fluence period in accordance with Specification 6.9.6. Plant operation within these operating limits is addressed in Specification 3.4.9.1, Reactor Coolant System Pressure/Temperature Limits”, and Specification 3.4.9.3, “Reactor Coolant System Overpressure Protection System.”

- (2) The following specifications are revised to replace the numerical values of the P/T and OPPS limits with a reference to the PTLR that provides these values:

Unit 1 Specifications:

- 3.1.2.4 Reactivity Control Systems — Charging Pumps — Operating
- 3.4.1.3 Reactor Coolant System — Shutdown
- 3.4.9.1 Reactor Coolant System — Pressure/Temperature Limits
(including heatup/cooldown P/T curves in Figures 3.4-2 and 3.4-3)
- 3.4.9.3 Overpressure Protection Systems (OPPS)
- 3.5.2 ECCS Subsystems — $T_{avg} \geq 350^{\circ}\text{F}$
- 3.5.3 ECCS Subsystems — $T_{avg} < 350^{\circ}\text{F}$

Unit 2 Specifications:

- 3.1.2.2 Reactivity Control Systems — Flow Paths — Operating
- 3.1.2.4 Reactivity Control Systems — Charging Pumps — Operating
- 3.4.1.3 Reactor Coolant System — Shutdown
- 3.4.9.1 Reactor Coolant System — Pressure/Temperature Limits
(including heatup/cooldown P/T curves in Figures 3.4-2, 3.4-3, and 3.4-4)
- 3.4.9.3 Overpressure Protection Systems (OPPS)
- 3.5.2 ECCS Subsystems — $T_{avg} \geq 350^{\circ}\text{F}$
- 3.5.3 ECCS Subsystems — $T_{avg} < 350^{\circ}\text{F}$

- (3) Specification 6.9.6, "Pressure and Temperature Limits Report," is added to the reporting requirements of the administrative controls section of the TSs. This specification requires the licensee to submit the PTLR, upon issuance, to the NRC Document Control Desk with copies to the regional administrator and resident inspector. The report provides the explanations, figures, values, and parameters of the P/T and OPPS limits for the applicable effective fluence period. Furthermore, this specification requires the figures, values, and parameters to be (a) established using the BVPS-1 and 2 plant-specific methodology approved by the NRC for this purpose in the letter from Richard J. Laufer, NRC, to Mark B. Bezilla, FENOC, dated October 8, 2002, and (b) consistent with all applicable acceptance limits and the limits of the BVPS safety analyses. Finally, this specification requires the licensee to document in the PTLR all changes in the values of these limits each effective fluence period and submit to the NRC the revised PTLR upon its issuance.

Moving the P/T curves and the values of the LTOP setpoints and enable temperatures from TSs to a PTLR does not eliminate the regulatory requirement to operate the plant in accordance with the limits specified in Appendix G to 10 CFR Part 50. Once a PTLR is established, the plant's TSs will require and control operation within the limits in the PTLR. Only the figures, values, and parameters associated with the P/T limits and LTOP system limits are to be moved to the PTLR. In order to move its TS P/T curves and the values of the LTOP system setpoints and enable temperatures to a PTLR, a licensee should obtain NRC prior review and approval of a methodology for their development. A licensee should develop its methodology for NRC approval in accordance with GL 96-03. This GL provides guidance on both the methodology and the PTLR itself, including, but not limited to, the requirements of Appendix G to 10 CFR Part 50, and approved exemptions. Subsequent to establishment of a plant-specific PTLR, the specified PTLR change review process will ensure that changes to the methodology receive prior approval by the NRC. Further, when changes are made to the limits contained in the PTLR, in accordance with the NRC-approved methodology, the licensee is required to update the PTLR and submit the PTLR to the NRC upon its issuance.

On this basis, the NRC staff concludes that the licensee has proposed, consistent with GL 96-03, an acceptable means of maintaining the detailed values of the current P/T limit curves and OPPS limits, and making changes to these limits, as needed, in the future. Further, the requirements of Appendix G to 10 CFR Part 50 and the existing TSs will continue to limit plant operation in accordance with the PTLR values of the P/T limit curves and OPPS limits on the TS-required parameters. And these values will be established using an NRC-approved methodology. Therefore, moving the values of P/T limits and OPPS limits to the PTLR will not impact plant safety.

The information discussed above relating to the P/T limits and OPPS limits is not itself required to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety. The previously listed associated LCOs, which do satisfy one or more of the four criteria in 10 CFR 50.36(c)(2)(ii), will remain in TSs. These LCOs, consistent with Appendix G P/T requirements, will continue to require operating the plant in accordance with the PTLR P/T limits and OPPS limits. These limits will be maintained and revised using the NRC-approved methodology, as required by TS 6.9.6, or NRC prior approval of a license amendment to revise P/T limits and methodology must be obtained. In addition, the licensee has committed to use only the calculated vessel fluence values vice best-estimate fluence values when performing future capsule surveillance evaluations. This commitment has been

included as a condition of the license amendments and will remain in effect until the NRC staff approves an alternate methodology to perform these evaluations. Accordingly, the NRC staff concludes that the detailed values of the current P/T limit curves and OPPS limits may be removed from the TSs and maintained in the PTLR. Therefore, the proposed PTLR and associated TS changes are acceptable.

Along with the above changes, the licensee also proposed appropriate changes to the TS table of contents (index and figure index) and TS bases, including moving bases Table B 3/4.4-1 and Figure B 3/4.4-2 for both units to the PTLR. These changes are administrative and are, therefore, acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (66 FR 66465). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. Harbuck

Date: July 15, 2003

Beaver Valley Power Station, Units 1 and 2

Mary O'Reilly, Attorney
FirstEnergy Nuclear Operating Company
FirstEnergy Corporation
76 South Main Street
Akron, OH 44308

FirstEnergy Nuclear Operating Company
Regulatory Affairs/Performance
Improvement
Larry R. Freeland, Manager
Beaver Valley Power Station
Post Office Box 4, BV-A
Shippingport, PA 15077

Commissioner James R. Lewis
West Virginia Division of Labor
749-B, Building No. 6
Capitol Complex
Charleston, WV 25305

Director, Utilities Department
Public Utilities Commission
180 East Broad Street
Columbus, OH 43266-0573

Director, Pennsylvania Emergency
Management Agency
2605 Interstate Dr.
Harrisburg, PA 17110-9364

Ohio EPA-DERR
ATTN: Zack A. Clayton
Post Office Box 1049
Columbus, OH 43266-0149

Dr. Judith Johnsrud
National Energy Committee
Sierra Club
433 Orlando Avenue
State College, PA 16803

J. H. Lash, Plant Manager (BV-IPAB)
FirstEnergy Nuclear Operating Company
Beaver Valley Power Station
Post Office Box 4
Shippingport, PA 15077

Rich Janati, Chief
Division of Nuclear Safety
Bureau of Radiation Protection
Department of Environmental Protection
Rachel Carson State Office Building
P.O. Box 8469
Harrisburg, PA 17105-8469

Mayor of the Borough of
Shippingport
P O Box 3
Shippingport, PA 15077

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 298
Shippingport, PA 15077

FirstEnergy Nuclear Operating Company
Beaver Valley Power Station
ATTN: M. P. Pearson, Director
Services and Projects (BV-IPAB)
Post Office Box 4
Shippingport, PA 15077

FirstEnergy Nuclear Operating Company
Beaver Valley Power Station
Mr. B. F. Sepelak
Post Office Box 4, BV-A
Shippingport, PA 15077