

July 10, 2003

Mr. Gregory M. Rueger
Senior Vice President, Generation and
Chief Nuclear Officer
Pacific Gas and Electric Company
Diablo Canyon Power Plant
P. O. Box 3
Avila Beach, CA 93424

SUBJECT: DIABLO CANYON NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2 -
ISSUANCE OF AMENDMENT RE: REVISION TO TECHNICAL
SPECIFICATIONS REGARDING SUSPENSION OF POSITIVE REACTIVITY
ADDITIONS (TAC NOS. MB4996 AND MB4997)

Dear Mr. Rueger:

The Commission has issued the enclosed Amendment No. 158 to Facility Operating License No. DPR-80 and Amendment No. 159 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant (DCPP), Unit Nos. 1 and 2, respectively. The amendments consist of changes to the technical specifications (TSs) in response to your application dated April 10, 2002.

The amendments revise several of the required actions in the DCPP TSs that require suspension of operations involving positive reactivity additions or suspension of operations involving reactor coolant system (RCS) boron concentration reductions. In addition, these amendments revise several Limiting Condition for Operation notes that preclude reductions in RCS boron concentration when a reactor coolant pump(s) and/or a residual heat removal pump(s) are removed from operation. The amendments allow small, controlled, safe insertions of positive reactivity, but limit the introduction of positive reactivity to ensure that compliance with the required shutdown margin or refueling boron concentration limits are still satisfied.

G. Rueger

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A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

/RA/

Girija S. Shukla, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-275
and 50-323

Enclosures: 1. Amendment No. 158 to DPR-80
2. Amendment No. 159 to DPR-82
3. Safety Evaluation

cc w/encls: See next page

G. Rueger

- 2 -

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

/RA/

Girija S. Shukla, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-275
and 50-323

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Enclosures: 1. Amendment No. 158 to DPR-80
2. Amendment No. 159 to DPR-82
3. Safety Evaluation

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***Memo from Reactor Systems Branch**

****See previous concurrence**

TS: ML031970286

NRR-100

PKG.: ML031900127

ACCESSION NO.: ML031900100

NRR-058

OFFICE	PDIV-1/PM	PDIV-2/PM	PDIV-1/PM	PDIV-2/LA	SRXB/SC*
NAME	MHoncharik**	JDonohew	DJaffe	EPeyton	FAkstulewicz
DATE	04/25/2003	6/23/03		7/8/03	1/31/03

OFFICE	OGC Nlo based on APH comments	PDIV-2/SC
NAME	RWeisman	SDembek
DATE	2 July, 2003	7/8/03

DOCUMENT NAME: C:\ORPCheckout\FileNET\ML031900100.wpd

OFFICIAL RECORD

COPY

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-275

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.158
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas and Electric Company (the licensee) dated April 10, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 158, are hereby incorporated in the license. Pacific Gas and Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Stephen Dembek, Chief, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: July 10, 2003

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-323

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 159
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas and Electric Company (the licensee) dated April 10, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 159, are hereby incorporated in the license. Pacific Gas and Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Stephen Dembek, Chief, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: July 10, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 158

TO FACILITY OPERATING LICENSE NO. DPR-80

AND AMENDMENT NO. 159 TO FACILITY OPERATING LICENSE NO. DPR-82

DOCKET NOS. 50-275 AND 50-323

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

3.3-3

3.3-3

3.3-3a

3.3-4

3.3-4

3.4-8

3.4-8

3.4-9

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3.8-31

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3.9-2

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3.9-4

3.9-4

3.9-6

3.9-6

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 158 TO FACILITY OPERATING LICENSE NO. DPR-80
AND AMENDMENT NO. 159 TO FACILITY OPERATING LICENSE NO. DPR-82
PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON POWER PLANT, UNITS 1 AND 2
DOCKET NOS. 50-275 AND 50-323

1.0 INTRODUCTION

By application dated April 10, 2002, Pacific Gas and Electric Company (the licensee) requested changes to the Technical Specifications (TSs), Appendix A to Facility Operating License Nos. DPR-80 and DPR-82, for the Diablo Canyon Nuclear Power Plant (DCPP), Units 1 and 2. The proposed amendment would revise several of the required actions in the DCPP TSs that require suspension of operations involving positive reactivity additions or suspension of operations involving reactor coolant system (RCS) boron concentration reductions. In addition, the proposed amendment would revise several Limiting Condition for Operation (LCO) Notes that preclude reductions in RCS boron concentration when reactor coolant pump(s) and/or residual heat removal (RHR) pump(s) are removed from operation. These proposed changes would allow small, controlled, safe insertions of positive reactivity, but limit the introduction of positive reactivity to ensure that compliance with the required shutdown margin (SDM) or refueling boron concentration limits will still be satisfied. The amendment request would allow the licensee to continue normal plant control operations while maintaining subcriticality margins.

DCPP Units 1 and 2 have adopted the Standard Technical Specifications (STS). Generic changes to the STS have been developed by the Technical Specifications Task Force (TSTF). The amendment request is based on TSTF-286, Revision 2 (TSTF-286), which was approved by the NRC staff by letter dated July 6, 2000. TSTF-286 revised TS Action requirements that suspended operations involving positive reactivity additions or RCS boron concentration reductions to allow positive reactivity additions due to temperature changes and boron concentration changes as long as SDM and refueling boron concentration are maintained. As described in the SDM LCO 3.11 Bases, a sufficient SDM ensures that: (1) the reactor can be made subcritical from all operating conditions, transients, and design basis events; (2) the reactivity transients associated with postulated accident conditions are controllable within acceptable limits; and (3) the reactor will be maintained sufficiently subcritical to preclude inadvertent criticality in the shutdown condition. The Bases for LCO 3.9.1 refueling boron concentration similarly indicate that the limitations on reactivity conditions during refueling ensure that the reactor will remain subcritical during Mode 6. Since the proposed changes will not alter the limits established in these specifications, there will be no effect on the ability to shut down and maintain the reactor in a subcritical condition.

DCPP employs two independent reactivity control systems: one uses the movable control and shutdown rod cluster control assemblies (RCCAs), and the other uses the chemical and volume control system (CVCS) to adjust the soluble boron concentration. In MODES 1 and 2, both systems are used to compensate for the reactivity effects from the fuel and coolant temperature changes in the RCS during power operation from full load to no load conditions. In MODES 3, 4, and 5, the CVCS is used to compensate for the reactivity effects from temperature and Xenon changes. In MODE 6, the CVCS is used to maintain the boron concentration within the required limits. The boron concentration of the RCS inventory addition may be lower than the RCS boron concentration. However, the makeup inventory source boron concentration will be no lower than that needed to meet the boron concentration of LCO 3.9.1.

In the application, the licensee proposed changes to the Notes to Limiting Conditions for Operation (LCOs) 3.4.5, 3.4.6, 3.4.7, 3.4.8, and 3.9.5, and to Required Actions 3.4.5.D.2, 3.4.6.B.1, 3.4.7.B.1, 3.4.8.B.1, 3.9.3.A.2, 3.9.5.A.1, and 3.9.6.B.1. The licensee had proposed the same wording that is in TSTF-286. In reviewing the proposed changes, the staff has decided that the proposed wording could be made clearer, as explained below.

The proposed Notes for the above LCOs, except for LCO 3.9.5, state no "operations are permitted that would cause introduction into the RCS, coolant with boron concentration less than required to meet the SDM of LCO 3.1.1." The staff concludes that the Notes should read that no "operations are permitted that would cause introduction **of coolant into the RCS** with boron concentration less than required to meet the SDM of LCO 3.1.1." The difference is shown underlined in the application and in **bold** in the staff-requested wording.

For LCO 3.9.5, the Note states no "operations are permitted that would cause introduction into the Reactor Coolant System, coolant with boron concentration less than that required to meet the minimum required boron concentration of LCO 3.9.1." The staff concludes that the Note should read no "operations are permitted that would cause introduction **of coolant into the Reactor Coolant System** with boron concentration less than that required to meet the minimum required boron concentration of LCO 3.9.1." The difference is shown underlined and in **bold**.

The above proposed Required Actions (except for 3.9.3.A.2, 3.9.5.A.1, and 3.9.6.B.1 below) state "Suspend operations that would cause introduction into the RCS, coolant with boron concentrations less than required to meet SDM of LCO 3.1.1." The staff concludes that the required Actions should read "Suspend operations that would cause introduction **of coolant into the RCS** with boron concentrations less than required to meet **the** SDM of LCO 3.1.1." Again, the difference is shown underlined and in **bold**.

The proposed Required Actions 3.9.3.A.2, 3.9.5.A.1, and 3.9.6.B.1 state "Suspend operations that would cause introduction into the RCS, coolant with boron concentrations less than required to meet the boron concentration of LCO 3.9.1." The staff concludes that the Required Action should read "Suspend operations that would cause introduction **of coolant into the RCS** with boron concentrations less than required to meet the boron concentration of LCO 3.9.1." The difference is again underlined and in **bold**.

The staff believes that the above changes make the statements in the Notes and Required Actions clearer than the words in the application from TSTF-286, Revision 2. This prevents unacceptable reactivity additions to the core.

The staff requested by e-mail that the licensee agree to having the revised wording added to the TSs as part of its proposed license amendment request. The licensee agreed to the revised wording (see ADAMS Accession No. ML031600318). The revised wording for the above Notes to LCOs and Required Actions accomplish the intended goal of TSTF-286, and the technical reasoning supporting TSTF-286, which is set forth in the following sections of this safety evaluation, also supports the revised wording.

2.0 REGULATORY EVALUATION

The NRC staff finds that the licensee in Section 4.0 of its submittal identified the applicable regulatory requirements. The regulatory requirement on which the NRC staff based its acceptance is General Design Criterion (GDC) 26, "Reactivity control system redundancy and capability." Because the proposed amendment does not change the fuel to be in the core, or the reactivity instrumentation and controls for the core, the relevant regulatory requirement for the proposed amendment is that, per GDC 26, the SDM for the core must be such that the core, with the RCS boron concentration, can be made subcritical under cold conditions with the existing control systems.

3.0 TECHNICAL EVALUATION

The staff has reviewed the licensee's technical and regulatory analyses in support of its license amendment which are described in Sections 5.0 and 6.0 of the licensee's submittal. The detailed evaluation below will support the conclusion that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

3.1 Deviations from TSTF-286, Revision 2

TS 3.3.1, "RTS Instrumentation," Required Actions G.1 and I.1, and their respective Bases, are deviations from TSTF-286. The Bases refer to the Core Operating Limits Report since that document specifies the SDM. Actions G.1 and I.1 are described below.

3.1.1 TS 3.3.1, "RTS Instrumentation," Required Action G.1

TS 3.3.1, Condition G is applicable when there are two inoperable intermediate range neutron flux channels while in MODE 1 below P-10 interlocks (power range neutron flux) or MODE 2 above the P-6 interlocks (intermediate range neutron flux). The intermediate range reactor trip which limits reactivity excursions during startup will be unavailable in this condition. Currently Required Action G.1 states: "Suspend operations involving positive reactivity additions." The proposed Required Action G.1 would immediately suspend operations involving positive reactivity additions with the added note that limited boron concentration changes associated with RCS inventory control or limited plant temperature changes are allowed. The proposed

change to Required Action G.1 differs from TSTF-286 in that it does not include "...provided the change is accounted for in the calculated SDM."

During MODES 1 and 2 with $K_{\text{eff}} \geq 1.0$, the SDM is not a calculated value but is assured by operation within rod insertion limits of LCO 3.1.5, "Shutdown Bank Insertion Limits," and LCO 3.1.6, "Control Bank Insertion Limits," and by operating the plant in accordance with the requirements of LCO 3.4.2, "RCS Minimum Temperature for Criticality." The licensee clarified that during MODES 1 and 2, SDM is ensured by the rod control system using the RCCAs and the boration system, or the CVCS during power operation and is capable of making the core subcritical rapidly enough to prevent exceeding acceptable fuel design limits, assuming that the rod of highest reactivity worth remains fully withdrawn.

Since the rod insertion limits and minimum temperature for criticality ensure SDM, the potential temperature fluctuations and boron concentration variation must be accounted for when determining these limits in the reload analysis. The licensee stated that the limits on shutdown or control rod alignments ensure that the assumptions in the safety analysis will remain valid. Since the fluctuations are accounted for by compliance with the TSs, the staff finds this acceptable.

3.1.2 TS 3.3.1, "RTS Instrumentation," Required Action I.1

TS 3.3.1, Condition I is applicable when one of the two required source range neutron flux channels is inoperable in MODE 2 below P-6 interlocks (intermediate range neutron flux). Currently, Required Action I.1 states: "Suspend operations involving positive reactivity additions." A note will be added which states, "Limited boron concentration changes associated with RCS inventory control or limited plant temperature changes are allowed."

The proposed change differs from TSTF-286 in that it does not include "...provided the change is accounted for in the calculated SDM."

This phrase was omitted for the same reasons explained above in Section 3.1.1 of this safety evaluation; therefore, the staff finds this acceptable.

3.2 Changes Identical to TSTF-286, Revision 2

TSTF-286 limits the introduction of reactivity more positive than that required to meet the required SDM or refueling boron concentration. For the following proposed changes, the licensee limits plant operations to those agreed to by the staff when the staff approved TSTF-286. Therefore, the staff finds the changes acceptable. The individual plant changes are described below.

3.2.1 TS 3.3.1, "RTS Instrumentation," Required Action L.1

TS 3.3.1, Condition L is applicable in MODES 3, 4, and 5 when the required source range neutron flux channel is inoperable with the reactor trip breakers open or all rods fully inserted and incapable of withdrawal. In this condition, the source range function does not provide reactor trip, but does provide indication.

Currently, Required Action L.1 states: "Suspend operations involving positive reactivity additions."

A note will allow plant temperature changes "provided the temperature change is accounted for in the calculated SDM."

3.2.2 TS 3.4.5, "RCS Loops, MODE 3"; TS 3.4.6, "RCS Loops, Mode 4"; TS 3.4.7, "RCS Loops, MODE 5, Loops Filled"; and TS 3.4.8, "RCS Loops, MODE 5, Loops Not Filled"

A note will be added to the above four TSs that states: "No operations are permitted that would cause introduction of coolant into the RCS with boron concentration less than required to meet the SDM of LCO 3.1.1."

Currently, the notes state: "No operations are permitted that would cause reduction of the RCS boron concentration," and are intended to preclude dilution of the RCS when no forced mixing is taking place.

The proposed changes would allow dilution of the RCS, but the source of the inventory makeup is required to contain a soluble boron concentration greater than that required to meet the SDM of LCO 3.1.1. The proposed changes are corrected versions of the changes in TSTF-286.

3.2.3 TS 3.4.5, Required Action D.2; TS 3.4.6, Required Action B.1; TS 3.4.7, Required Action B.1; and TS 3.4.8, Required Action B.1

The required actions would be modified to state: "Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet the SDM of LCO 3.1.1."

Currently, the required actions state: "Suspend all operations involving a reduction of RCS boron concentration," and are intended to preclude dilution of the RCS when no forced mixing is taking place.

The proposed changes would allow dilution of the RCS, but the source of the inventory makeup is required to contain a soluble boron concentration greater than that required to meet the SDM of LCO 3.1.1. The proposed changes are corrected versions of the changes in TSTF-286.

3.2.4 TS 3.8.2, "AC Sources - Shutdown," Required Actions A.2.3 and B.3; TS 3.8.5, "DC Sources - Shutdown," Required Action A.2.3; TS 3.8.8, "Inverters - Shutdown," Required Action A.2.3; and TS 3.8.10, "Distribution Systems - Shutdown," Required Action A.2.3

For the same reasons as stated previously in Section 3.2.3 of this safety evaluation, and to allow temperature changes that could add positive reactivity provided the reactivity insertions do not result in a loss of SDM or required refueling boron concentration, the Required Actions will be changed to state: "Suspend operations involving positive reactivity additions that could result in a loss of required SDM or boron concentration."

Currently, the Required Actions state: "Initiate action to suspend operations involving positive reactivity additions," and are intended to initiate suspension of operations involving positive reactivity additions based on the loss of required electrical sources and distribution equipment.

3.2.5 TS 3.9.3, "Nuclear Instrumentation," Required Action A.2

The required action would be revised to state: "Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet the boron concentration of LCO 3.9.1."

Currently, the required action states: "Suspend positive reactivity additions except for latching control rod drive shafts and friction testing of individual control rods," and is intended to initiate suspension of operations involving positive reactivity additions when there is a loss of one required source range neutron flux monitor, thereby rendering inoperable the redundant channel for monitoring core reactivity.

The proposed change would allow dilution of the RCS, but the source of the inventory makeup is required to contain a soluble boron concentration greater than that required to meet the minimum refueling boron concentration requirement of LCO 3.9.1. The proposed change is a corrected version of the change in TSTF-286.

3.2.6 TS 3.9.5, "RHR and Coolant Circulation - High Water Level"

The first LCO note for TS 3.9.5 would be revised to state: "The required RHR loop may be removed from operation for less than or equal to 1 hour per 8 hour period, provided no operations are permitted that would cause introduction of coolant into the RCS with boron concentration less than that required to meet the minimum required boron concentration of LCO 3.9.1."

Currently, the LCO note states: "The required RHR loop may be removed from operation for less than or equal to 1 hour per 8 hour period, provided no operations are permitted that would cause reduction of the RCS boron concentration," and is intended to preclude dilution of the RCS when no forced mixing is taking place.

The proposed change would allow boron dilution of the RCS, but the source of the inventory makeup is required to contain a soluble boron concentration greater than that required to meet the minimum refueling boron concentration of LCO 3.9.1. The proposed change is a corrected version of the change in TSTF-286.

3.2.7 TS 3.9.5, "RHR and Coolant Circulation - High Water Level," Required Action A.1 and TS 3.9.6, "RHR and Coolant Circulation - Low Water Level," Required Action B.1

The required actions would be revised to state: "Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet the boron concentration of LCO 3.9.1."

Currently, the required actions state: "Suspend operations involving a reduction in reactor coolant boron concentration," and is intended to preclude dilution of the RCS when no forced mixing is taking place.

The proposed changes would allow dilution of the RCS, but the source of the inventory makeup is required to contain a soluble boron concentration greater than that required to meet the minimum refueling boron concentration requirement of LCO 3.9.1. The proposed changes are corrected versions of the changes in TSTF-286.

3.3 Summary

Based on the review of the licensee's application and supporting documentation, the NRC staff concludes that the proposed changes to the DCPD TSs will continue to ensure that the required minimum SDM and refueling boron concentration are met, which will limit any potential reactivity additions to acceptable levels. Based on this, the NRC staff concludes that the proposed amendments meet the requirements of GDC 26 and, therefore, are acceptable. In addition, the proposed changes are consistent with TSTF-286.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (67 FR 40024). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Colpo

Date: July 10, 2003

Diablo Canyon Power Plant, Units 1 and 2

cc:

NRC Resident Inspector
Diablo Canyon Power Plant
c/o U.S. Nuclear Regulatory Commission
P.O. Box 369
Avila Beach, CA 93424

Mr. Pete Wagner
Sierra Club California
2650 Maple Avenue
Morro Bay, California 93442

Ms. Nancy Culver
San Luis Obispo
Mothers for Peace
P.O. Box 164
Pismo Beach, CA 93448

Chairman
San Luis Obispo County Board of
Supervisors
Room 370
County Government Center
San Luis Obispo, CA 93408

Mr. Truman Burns
Mr. Robert Kinoshian
California Public Utilities Commission
505 Van Ness, Room 4102
San Francisco, CA 94102

Diablo Canyon Independent Safety
Committee
ATTN: Robert R. Wellington, Esq.
Legal Counsel
857 Cass Street, Suite D
Monterey, CA 93940

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
Harris Tower & Pavillion
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

Richard F. Locke, Esq.
Pacific Gas & Electric Company
P.O. Box 7442
San Francisco, CA 94120

Mr. David H. Oatley, Vice President
and General Manager
Diablo Canyon Power Plant
P.O. Box 56
Avila Beach, CA 93424

City Editor
The Tribune
3825 South Higuera Street
P.O. Box 112
San Luis Obispo, CA 93406-0112

Mr. Ed Bailey, Radiation Program Director
Radiologic Health Branch
State Department of Health Services
P.O. Box 942732 (MS 178)
Sacramento, CA 94234-7320

Mr. James D. Boyd, Commissioner
California Energy Commission
1516 Ninth Street (MS 31)
Sacramento, CA 95814

Mr. James R. Becker, Vice President
Diablo Canyon Operations
and Station Director
Diablo Canyon Power Plant
P.O. Box 3
Avila Beach, CA 93424