

June 26, 2003

Mr. Roy A. Anderson
President & Chief Nuclear Officer
PSEG Nuclear, LLC - X04
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - HOPE CREEK GENERATING
STATION - RELIEF REQUEST HC-RR-B12 (TAC NO. MB8407)

Dear Mr. Anderson:

By letter dated April 14, 2003, PSEG Nuclear, LLC, requested relief from the required volumetric examination required by the American Society of Mechanical Engineers Code, Section XI, Table IWB-2500-1, Examination Category B-D, Item B3.100. The relief was requested pursuant to Title 10 of the *Code of Federal Regulations* Section 50.55a(a)(3)(ii). The U.S. Nuclear Regulatory Commission staff is reviewing your submittal and has determined that the information contained in the enclosed request for additional information will be needed for the staff to complete its review.

Sincerely,

/RA/

Richard B. Ennis, Senior Project Manager, Section 2
Project Directorate I
Division of Licensing and Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosure: Request for Additional Information

cc w/encl: See next page

Hope Creek Generating Station

cc:

Mr. Timothy J. O'Connor
Vice President - Operations
PSEG Nuclear - X15
P.O. Box 236
Hancocks Bridge, NJ 08038

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Vice President - Engineering
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P.O. Box 236
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Mr. David F. Garchow
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Manager - Nuclear Safety and Licensing
PSEG Nuclear - N21
P.O. Box 236
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Jeffrie J. Keenan, Esquire
PSEG Nuclear - N21
P.O. Box 236
Hancocks Bridge, NJ 08038

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Joint Owner Affairs
PECO Energy Company
Nuclear Group Headquarters KSA1-E
200 Exelon Way
Kennett Square, PA 19348

Lower Alloways Creek Township
c/o Mary O. Henderson, Clerk
Municipal Building, P.O. Box 157
Hancocks Bridge, NJ 08038

Dr. Jill Lipoti, Asst. Director
Radiation Protection Programs
NJ Department of Environmental
Protection and Energy
CN 415
Trenton, NJ 08625-0415

Brian Beam
Board of Public Utilities
2 Gateway Center, Tenth Floor
Newark, NJ 07102

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector
Hope Creek Generating Station
U.S. Nuclear Regulatory Commission
Drawer 0509
Hancocks Bridge, NJ 08038

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DISTRIBUTION:

PUBLIC	PDI-2 R/F	JClifford	REnnis	GMiller	CRaynor
OGC	WHeld	TChan	GMeyer, RGN 1		

ADAMS Accession Number: ML031710391

OFFICE	PDI-2/PM	PDI-2/PM	PDI-2/LA	PDI-2/SC
NAME	GMiller	REnnis	CRaynor	REnnis for JClifford
DATE	6/24/2003	6/24/03	06/25/03	6/25/03

Official Record Copy

REQUEST FOR ADDITIONAL INFORMATION
BY THE OFFICE OF NUCLEAR REACTOR REGULATION
REGARDING RELIEF REQUEST HC-RR-B12
HOPE CREEK GENERATING STATION
DOCKET NUMBER 50-354

By letter dated April 14, 2003, PSEG Nuclear, LLC (the licensee), requested relief from the required volumetric examination required by the American Society of Mechanical Engineers Code, Section XI, Table IWB-2500-1, Examination Category B-D, Item B3.100. The relief was requested pursuant to Title 10 of the *Code of Federal Regulations* Section 50.55a(a)(3)(ii). The NRC staff is reviewing your submittal and has determined that the following information will be needed for the staff to complete its review.

1. Table 1 in the submittal includes the summary number and component identification for each nozzle to be examined. The type of nozzle to be examined is also important because nozzles subjected to large thermal gradients have a past history of thermal cracking. Describe the type (i.e., feedwater, recirculation inlets, jet pump instrumentation, etc.), nominal inside diameter and material (i.e., carbon, nickel, etc.) of the nozzles included in the relief request.
2. The licensee states the estimated coverage of each of the nozzles in Table 1 of its submittal. This list fails to state specific information about which area of the nozzle will be inaccessible to examination (e.g., top, side, bottom). The severity of thermal fatigue is dependant upon the circumferential location, thermal gradient and thermal sleeve leakage. Describe the portion of each nozzle that will be inaccessible to examination, providing a sketch if necessary.
3. In the alternative, the licensee states that the resolution sensitivity will be established using a 1-mil diameter wire standard that is similar to that used for other reactor pressure vessel internal examinations intended to detect cracking, without stating which standards are being referenced. Explain the qualification process that will be used to demonstrate the 1-mil width sensitivity and the equipment used for the examination.

Enclosure