AmerenUE Callaway Plant PO Box 620 Fulton, MO 65251

June 9, 2003

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Mail Stop P1-137 Washington, DC 20555-0001

ULNRC04862

Ladies and Gentlemen:

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DOCKET NUMBER 50-483 Callaway PLANT UNIT 1 UNION ELECTRIC CO. FACILITY OPERATING LICENSE NPF-30 LICENSEE EVENT REPORT 2003-004-00 Boron Dilution Mitigation System blocked in Mode 3.

The enclosed licensee event report is submitted in accordance with 10CFR50.73(a)(2)(ii)(B) and 10CFR50.73(a)(2)(v)(D), to report events where the Boron Dilution Mitigation System was blocked while in Mode 3. This action is inconsistent with Final Safety Analysis Report accident analysis.

Very truly yours, Warren A. aut

Warren A. Witt Manager, Callaway Plant

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Enclosure



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 cc: Mr. Thomas P. Gwynn Acting Regional Administrator U.S. Nuclear Regulatory Commission Region IV 611 Ryan Plaza Drive, Suite 400 Arlington, TX 76011-4005

> Senior Resident Inspector Callaway Resident Office U.S. Nuclear Regulatory Commission 8201 NRC Road Steedman, MO 65077

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Manager, Electric Department Missouri Public Service Commission PO Box 360 Jefferson City, MO 65102

Records Center Institute of Nuclear Power Operations 700 Galleria Parkway Atlanta, GA 30339

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| 4. TITLE | | | | | | | | | | | | | | | -T | |
| Boron dilution n | nitigation s | system b | lock | ed in | Mode 3 whic | h not e | consiste | nt with <u>I</u> | FSAR acci | den | t analysis. | | | | | |
| 5. EVE | INT DATE | | | 6. | LER NUMBER | | 7. F | REPORT | DATE | | | OTHER F | ACILITIES I | NVOLVED | | |
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| Mark A. Reid | dmeyer | | | | | | | | | | | | <u>3) 676-43</u> | 106 | | |
| | | <u>13. CO</u> | MPL | ETE (| ONE LINE FO | OR E | ACH CO | MPONI | ENT FAIL | JRE | DESCRIBED | IN THIS | REPORT | | | |
| | | | | | MANU- | RE | PORTABLI | E | | | | | | MANU- | REPORTABLE | |
| CAUSE | SYSTEM | | MPON | ENT | FACTURER | | TO EPIX | _ _ | CAUSE | _ | SYSTEM | COMPO | | CTURER | TO EPIX | |
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| | 14. | SUPPL | EME | ENTA | L REPORT E | EXPEC | TED | | | | 15. EXPE | CTED | MONTH | DAY | YEAR | |
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| 16. ABSTRACT | | | | | | | | | | line | | | <u> </u> | <u></u> | | |
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On 4/11/03, while at 100 percent power, it was discovered that a note contained in Technical Specification (T/S) 3.3.9 for the Boron Dilution Mitigation System (BDMS), had been inappropriately applied during past reactor startups. This had been interpreted to allow blocking BDMS while withdrawing Shutdown (S/D) Bank rods in Mode 3. This action is not allowed in Mode 3 per Final Safety Analysis Report (FSAR) accident analysis Section 15.4.6.2 where BDMS is credited for automatically terminating a dilution event while in Mode 3.

Wording of T/S 3.3.9 and T/S 3.3.9 Bases did not provide clear guidance as to what constitutes "reactor startup". The Bases indicate BDMS could be blocked prior to withdrawing "rods" for startup. These words do not delineate between control banks and shutdown banks. Based on this unclear guidance, procedure OTG-ZZ-0001A was incorrectly revised allowing the blocking of BDMS prior to withdrawing shutdown banks. The discovery of the unclear T/S wording was the result of requested procedure enhancements to clarify when it was allowable to block BDMS.

A review of reactor startups within the last 3 years indicated that BDMS was inappropriately blocked on three separate startups. The first occurred on 11/24/02, the second on 12/17/02, and the third on 4/2/03. Plant procedures governing reactor startup were revised to remove statements allowing blocking BDMS while withdrawing S/D Bank rods in Mode 3.

| FACILITY NAME (1) | DOCKET (2) NUMBER (2) | LER NUMBER (6 |) | | PAGE (3) |
|--|--|---|---|---|---|
| Callaway Plant Unit 1 | 05000483 | YEAR SEQUENTIAL NUMBER 2003 - 004 | REVISION NUMBER - 00 | 2 | OF |
| VE (If more space is required, use additional o | opies of NRC Form 366 | A) (17) | | | |
| DESCRIPTION OF THE REPORTABL | LE EVENT | | | | |
| A. REPORTABLE EVENT CLASSIFI | CATION | | | | |
| This event has been determined to be rep 10CFR50.73(a)(2)(v)(D), as a condition consequences of an accident. | | | | | |
| B. PLANT OPERATING CONDITION | NS PRIOR TO THE I | EVENT | | | |
| Mode 1 at 100 percent power. | | | | | |
| C. STATUS OF STRUCTURES, SYST OF THE EVENT AND THAT CON | | | PERABLE | AT THI | E START |
| There were no components inoperable the | hat contributed to this | event. | | | |
| D. NARRATIVE SUMMARY OF THI | E EVENT, INCLUDI | NG DATES AND APPRO | XIMATE T | IMES | |
| for the Boron Dilution Mitigation System note states: "The boron dilution flux mu Range Neutron Flux) interlock) and 3 du | ultiplication signal ma | y be blocked in Modes 2 (| below P-6 (| Interme | diate BDMS |
| Report (FSAR) accident analysis. | | ion is not allowed in Mode | | Safety | Analysis |
| | ord (AOR) is discuss ere is sufficient time a generates a neutron flu IS also initiates signa om Refueling Water S ak outlet isolation val- ltiplication condition. | ion is not allowed in Mode ed in FSAR Section 15.4.6 available for automatic mit ex-multiplication alarm that is to automatically open va storage Tank) to initiate bo we) to terminate the dilution This occurs prior to the lo | 3 per Final 2. The acce igation prior t indicates a lves BN-LC ration and th h. Re-borations of shutdo | ptance of to the of n inadvo V-112I nen to ci | criterion f complete ertent D/E lose valve urs within |
| Report (FSAR) accident analysis. The Callaway Mode 3 Accident Of Reca a Mode 3 boron dilution event is that the loss of shutdown margin. The BDMS g boron dilution is in progress. The BDM (Centrifugal Charging Pump Suction fro BG-LCV-112B/C (Volume Control Tan 3.14 minutes after reaching the flux-mu | in Mode 3. This act ord (AOR) is discuss- ere is sufficient time is also initiates signa om Refueling Water S ak outlet isolation val- ltiplication condition, cation condition, had tion Event AOR has is shutdown margin, the refore, the T/S 3.3.9 r plies to blocking BDI hutdown (S/D) Bank | ion is not allowed in Mode ed in FSAR Section 15.4.6. available for automatic mit ux-multiplication alarm that is to automatically open va btorage Tank) to initiate bo we) to terminate the dilution This occurs prior to the lo the condition gone unmitig nsufficient time available to BDMS must be credited to to the which allows blocking MS just prior to withdrawin rods without blocking all p | 3 per Final 2. The acce- igation prior t indicates a lves BN-LC ration and th n. Re-borations of shutdor pated. to credit ope o mitigate that the boron do ng Control E potential dilu | ptance of r to the on n inadve V-112E nen to clion occu own main rator acce conse ilution for Bank root ttion south | criterion f complete ertent D/E lose valve urs within rgin at 6.2 ctions to equences of flux ds. BDM urces |

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| | FACILITY NAME (1) | DOCKET (2) NUMBER (2) | | LER | UMBE | R (6) | | | PAGE (| 3) |
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| | | NOMBER (2) | | | EQUENTI | | REVISION | 1 | | <u>.</u> |
| | Callaway Plant Unit 1 | 05000400 | <u>YEAR</u> 2003 | | NUMBEF | <u>२</u> | NUMBER 00 | | | |
| RRA | TIVE (If more space is required, use additional | 05000483 copies of NRC Form 366 | | | | | | 3 | OF | - |
| _ | | • • • • • • • • | | | | • | <u> </u> | | | •• |
| | A historical review of reactor startups withdrawing S/D Bank rods in Mode 3. | | | | | | | | | 11e |
| | 12/17/03, and the third on $4/2/03$. | , •••• •••• •••• | upor rite | ILD. | | u on | 11/2 02, | , 110 000 | | |
| | An immediate corrective action was to | issue OTG-77-0001A | revision 6 | Thi | s r evisi | on re | emoved a | sten whi | ich state | d |
| | that BDMS could be blocked. | | | | | 0111 | | btop till | | - |
| | E. METHOD OF DISCOVERY OF E | ACH COMPONENT, | SYSTEM | FAIL | URE, C | OR P | ROCEDI | JRAL E | RROR | |
| | This problem was discovered while res | | | | | | | | | |
| | SHUTDOWN BANK WITHDRAWA | | plant oper | aung | proceu | uie (| JIU-22-0 | 0001A, | | |
| 11, | EVENT DRIVEN INFORMATION | | | | | | | | | |
| | A. SAFETY SYSTEMS THAT RESP | | | | | | | | | |
| | | | | | | | | | | |
| | Not applicable at the time of discovery | on 4/11/03. | | | | | | | | |
| | B. DURATION OF SAFETY SYSTE | M INOPERABILITY | | | | | | | | |
| | Not applicable. | | | | | | | | | |
| | C. SAFETY CONSEQUENCES AND | IMPLICATIONS OF | THE EVE | NT. | | | | | | |
| | The source range or intermediate range trained to immediately enter Emergence "Subcriticality" is one of the critical sat power does not exceed 5 percent. These coolant system if neutron flux is not de damage or radiological release from the impact on the health and safety of the p | y Operating Procedure fety functions continuous se procedures require the creasing. These proce e plant. Therefore, this | s (EOP) fo usly monit te licensed dural contr | llowi tored oper ols w | ng any through ators to ould en | indie hout imn hsure | cation of a the EOPs nediately that there | a reactor to ensur borate the is no re | trip. re reacto he reacto esulting | er er |
| III. | CAUSE OF THE EVENT | | | | | | | | | |
| | Wording of T/S 3.3.9 and T/S 3.3.9 Ba Bases indicate BDMS could be blocked between control banks and shutdown b incorrectly revised allowing the blocking | d prior to withdrawing anks. Based on this ur | "rods" for clear guid | startı ance, | ip. The proced | ese w lure (| vords do n OTG-ZZ-(| ot delin | eate | Гhe |
| IV. | CORRECTIVE ACTIONS | | | | | | | | | |
| | T/S 3.3.9 Bases will be revised to clear reactor startup. | rly specify that BDMS | can be blo | cked | prior to | with | hdrawing | control 1 | rods for | |

| | FACILITY | (NAME (1) | DOCKET (2) NUMBER (2) | 1 | ER NUMBER (6) | | | PAGE (3) | |
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| | Callaway | Plant Unit 1 | 05000483 | <u>year</u> 2003 | SEQUENTIAL NUMBER | REVISION NUMBER 00 | 4 | OF | 4 |
| ARRA | TIVE (If more space | is required, use additional | | A) (17) | | | | | |
| V. | PREVIOUS | SIMILAR EVENTS | ····· | <u></u> | | | | | |
| | system identified | a no other related even | ts beyond the three eve | ents reported | I in this LER. | | | | |
| VI. | A review of Cal | laway LERs from 2000 AL INFORMATION component codes liste |) to present confirmed | that there w | ere no past LEF | | - | | ŀ |
| VI. | A review of Cal <u>ADDITION</u> The system and | laway LERs from 2000 AL INFORMATION component codes liste |) to present confirmed | that there w | ere no past LEF | | - | | Ļ |