

MEMORANDUM TO: Pao-Tsin Kuo, Program Director
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

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Region I

FROM: Stuart A. Richards, Chief
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Wayne D. Lanning, Director
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SUBJECT: R. E. GINNA LICENSE RENEWAL INSPECTIONS

Attached is the final version of the R. E. Ginna Nuclear Power Plant License Renewal Inspection Plan. The plan, which was developed jointly by NRR and Region I, is hereby approved. You are directed to use this plan to prepare and conduct the license renewal inspections at the R. E. Ginna Nuclear Power Plant.

/Original signed by/
Stuart A. Richards, Chief
Inspection Program Branch
Division of Inspection Program Management
Office of Nuclear Reactor Regulation

Date: 5/21/03

/Original signed by PTKuo for/
Wayne D. Lanning, Director
Division of Reactor Safety
Region I

Date: 5/21/03

Attachments: Ginna License Renewal Inspection Plan

Distribution: See next page

*See previous concurrence

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R. E. Ginna Nuclear Power Plant LICENSE RENEWAL INSPECTION PLAN

I PURPOSE

This inspection plan specifies methods for implementing Manual Chapter 2516 requirements for activities relating to 10 CFR Part 54 (herein after referred to as “the rule”) and the R. E. Ginna Nuclear Power Plant (Ginna Station) license renewal inspection program. This plan defines the scope of the inspections planned to verify that Ginna Station’s license renewal program is in compliance with the requirements of the rule and is consistent with Rochester Gas and Electric Corporation’s (RG&E’s) license renewal application (LRA) and the staff’s safety evaluation of RG&E’s LRA. The plan also provides guidance for inspection scheduling, inspector training, inspection activities, and resource requirements.

RG&E’s LRA identified the systems and structures that RG&E determined were within the scope of the rule. Page 4 of this document lists the systems and structures selected for this inspection. The inspection team chose the items, after reviewing the scoping results provided in Ginna Station’s LRA, on the basis of their risk significance, uniqueness to Ginna Station, and current issues. The scope and depth of inspections of these systems and structures may vary.

II OBJECTIVES

The overall objective of this plan is to provide guidance for inspecting the implementation and effectiveness of the programs and activities associated with RG&E’s license renewal program. The inspection will verify that there is reasonable assurance that the effects of aging will be adequately managed so that the intended function(s) of structures, and components (SCs), for which an aging management review is required, will be maintained consistent with the current licensing basis (CLB) during the period of extended operation. Region I will implement the license renewal inspection plan (LRIP) at Ginna Station before NRR approves RG&E’s LRA to verify that RG&E meets the requirements of the rule and has implemented license renewal programs and activities consistent with the rule, the LRA, and the staff’s safety evaluation report (SER) on the LRA.

III INSPECTION ACTIVITIES

Inspection Procedure (IP) 71002, “License Renewal Inspections,” will be the primary procedure used to inspect RG&E’s implementation of the requirements of the rule. The latest revision of IP 71002 can be reviewed by accessing <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/ip71002.pdf>

1. The systems and structures groups to be inspected are identified in Page 4 of this plan. The selection of these systems and structures is based on risk significance and the importance of the safety function performed. The inspection team will verify that RG&E has implemented the scoping and screening process consistent with the rule and RG&E’s methodology, as described in the LRA submitted by letter dated July 30, 2002. The inspection team will also inspect a sample of the systems and structures that RG&E

concluded were not within the scope of the rule, in order to verify that there is reasonable assurance that all systems and structures within the scope of 10 CFR 54.4 have been identified.

2. The implementation of the screening activities required under 10 CFR 54.21(a)(1) will be inspected by reviewing system boundaries on plant drawings, intended functions, and the active/passive and short-/long-lived characteristics of the SCs listed in Page 4, within the scope of RG&E's aging management review.
3. The inspection team will also walk down accessible portions of the systems and structures to identify any observable inconsistencies in the scoping and screening activities and any aging effects on the systems and structures that are not covered in the LRA. Aging effects identified by RG&E will be reviewed and evaluated during the NRR technical review. The inspection team will perform a sample audit of related maintenance records of the systems and structures listed in Page 4 to attempt to identify any previously unrecognized aging.
4. The inspection team will inspect the aging management programs (AMPs), including AMPs where RG&E claims they are consistent with the Generic Aging Lessons Learned (GALL) report, for approximately half of the aging effects in each of the systems and structures listed in Enclosure. The inspection team will examine records for existing aging management programs to evaluate the programs' effectiveness and will review plans for new aging management programs. The inspection team will then document its findings on the effectiveness of the aging management programs to maintain the systems' and structures' intended function(s) consistent with the CLB for the period of extended operation.

IV INSPECTION SCOPE

The Ginna Station license renewal inspection activities will be implemented through two or more site inspections.

1. The first inspection will last one week, or longer if necessary, and focus on the scoping and screening processes to verify that they have been implemented consistent with the rule, RG&E's methodology, and the staff's safety evaluation of RG&E's methodology. This inspection should be performed after the staff has completed its safety evaluation of the scoping and screening methodology, but before the SER is issued. The inspection will verify that there is reasonable assurance that RG&E's scoping and screening processes have identified all of the systems, structures, and components for which an aging management review is required consistent with the requirements of the rule.
2. In the second inspection, the team will spend one week at the site, return to the region for one week to review documents, begin to write the inspection report, adjust the inspection plan, if needed, and go back to the site for another week. This inspection will examine existing and proposed aging management programs and past results. To support the NRR review process, the report for the second inspection will be issued before the "SER with open items" is issued.

2. If the regional administrator decides that the open inspection items from the first two inspections warrant a third inspection, the team will followup on previous inspection activities and inspect RG&E actions on any SER open items requested by NRR. This inspection will also focus on any portion of the LRA updated by the applicant as a result of recent plant modifications. The third inspection report will document the need for any future follow-up inspections.

V INSPECTION RESOURCES

The inspection will need the following inspection resources:

1. Inspectors

- One team leader
- Three inspectors from the region
- One or more participants from the program office

2. Skills

The inspection team needs a cross-section of skills, including mechanical, material, civil/structural, and electrical engineering skills.

The scope of the third inspection (and, thus, the resources) will depend on how many open issues remain from the previous inspection activities.

SYSTEMS AND STRUCTURES SELECTED FOR INSPECTION

SYSTEMS OR STRUCTURES	WITHIN SCOPE
Mechanical Systems	
Reactor Coolant, Core, and Internals	Yes
Non-Class 1 RCS Components	Yes
Containment Hydrogen Detectors and Recombiners	Yes
Containment Isolation Components	Yes
Containment Spray	Yes
Residual Heat Removal	Yes
Safety Injection	Yes
Chemical and Volume Control	Yes
Chilled Water	No
Component Cooling Water	Yes
Containment Ventilation Systems	Yes
Emergency Power	Yes
Fire Protection	Yes
Fuel Handling	No
Plant Air Systems	No
Service Water	Yes
Auxiliary Feedwater	Yes
Feedwater and Condensate	Yes
Main and Auxiliary Steam	Yes
Structures and Structural Components	
Auxiliary Building	Yes
Cable Tunnel	Yes
Component Supports Commodity Group	Yes
Containment Structures	Yes
Control Building	Yes
Diesel Building	Yes
Essential Buildings and Yard Structures	Yes
Essential Yard Structures	Yes
Intermediate Building	Yes
Non-Essential Buildings and Yard Structures	No
Screen House Building	Yes
Service Building	Yes
Standby Auxiliary Feedwater Building	Yes
Turbine Building	Yes
Electrical Components	
120 VAC Vital Instrument Buses	Yes
125 VDC Power	Yes
4160 VAC Power	Yes
480 VAC Power	Yes
Control Rod Drive and Process Instruments	Yes
Engineered Safety Features Actuation	Yes
Offsite Power	Yes
Plant Security	No
Reactor Protection	Yes