II. EXPLORATORY STUDIES FACILITY (ESF)

The design status of the ESF is as follows:

- Title I design completed and submitted to DOE Headquarters.
- Approval received to begin limited Title II design prior to ESAAB approval of project baseline.
- ESF designers, Raytheon Services Nevada (RSN), have ٠ completed a readiness review to start Title II design.
- RSN's Title II engineering plan approved.
- ♦ Title II design has begun on road and pads for first access.

The following milestones are in place for FY 92:

• :	ESAAB approval of the Title I cost and schedule	-	Dec. 91
٠	Complete RFP for construction contractor	-,	Feb. 92
٠	Receipt of data from the soil and rock propertie	5	
	surface-based testing program	-	Feb. 92
٠	50% ESF design review (roads and pads)	-	March 92
٠	90% ESF design review (roads and pads)	-	July 92

NRC staff will be invited to observe the 50% and the 90% design review.

The Title II design packages that are scheduled for FY 92 completion are:

- ♦ Site preparation of surface and portal first access (WBS) 1.2.6.2.1.1.
- First access ramp highwall, portal pad and portal (WBS) 1.2.6.3.1.1.
- Surface utilities and communications systems (WBS) 1.2.6.2.2.1.) is scheduled for partial completion in FY 92.

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A chart showing <u>PROPOSED</u> (emphasis DOE's) ESF design/ construction activities for FY 92 and 93 is insert (2). All planned activities are dependent on sufficiant funds forthcoming in the FY 93 and FY 94 budgets.

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III. EARLY SITE SUITABILITY EVALUATION (ESSE)

The final ESSE report will be submitted to the Project Office in early January 1992 for approval to release. It is planned that the peer review comments/response package will be printed as a separate document.

The preliminary conclusions of the scientists who conducted the early site suitability evaluation are:

A. Qualifying conditions of DOE siting guidelines:

- 9 of 24 qualifying conditions are present and new information is unlikely to change this conclusion.
- 15 of 24 qualifying conditions are likely to be present but further information is needed.
- B. Disqualifying conditions of DOE siting guidelines:
- 14 of 18 disqualifying conditions are not present and new information is unlikely to change this conclusion.
- 4 of 18 disqualifying conditions are not likely to be present but further information is needed.
 Inserted are the preliminary conclusions described above.
 (3)

The planned DOE actions after receipt of the final report are:

- Final ESSE report and peer review report are due to DOE in late January, 1992.
- Public will be given the opportunity to comment on the final report in early 1992.
- ♦ All comments will receive written responses.

Inserted are the names, organization and area of expertise of the ESSE core team (4) and the peer review panel (5). Also attached is a time chart for the ESSE activity. (6)

4

DOE Siting Guideline

(Insert 3-a)

Conclusion

Postclosure Guidelines						
Postclosure system: EPA & NRC standards can be met	Postclosure system: EPA & NRC standards can be met Condition is likely to be present					
Geohydrology		•				
QC: Compatible with waste containment & isolation DC: <1000 year ground-water travel time		Condition is likely to be present Condition is not likely to be present				
Geochemistry		· · ·				
QC: Compatible with waste containment and isolation		Condition is likely to be present				
Rock Characteristics						
QC: Accommodate thermal, chemical, mechanical stresses		Condition is likely to be present				
Climatic Changes						
QC: No unacceptable releases due to climate change		Condition is likely to be present				
Erosion						
QC: No unacceptable releases due to erosion		Condition present: new information unlikely to change conclusion				
DC: Bunai cannot be >200m Dissolution		Condition not present: new information unlikely to change conclusion				
QC: No unacceptable releases due to dissolution		Condition present: new information unlikely to change conclusion				
DC: Loss of isolation due to dissolution expected	· ·	Condition not present: new information unlikely to change conclusion				
Tectonics		· · ·				
QC: No unacceptable releases due to tectonics		Condition is likely to be present				
DC. Faux movement expected to cause bass of waste isolation		Condition not present. new information uninkely to change conclusion				
Human Interference: Natural Resources						
QC: Interference due to resources will not lead to unacceptable releases		Condition is likely to be present				
DC1: Significant pathways exist from previous mining		Condition not present: new information unlikely to change conclusion				
DC2: Mining activities expected to lead to loss of waste isolation		Condition not present: new information unlikely to change conclusion				
Human Interference: Site Ownership and Control						
QC: DOE can obtain land ownership and rights	1 m	Condition present: new information unlikely to change conclusion				

DOE Siting Guideline

Conclusion

	Preclosure Guid	delines:	Radiological Safety
System: n	adiological safety standards can be met	-	Condition is likely to be present
Population QC1: QC2: DC1: DC2: DC3:	n Density Doses to highly populated areas meet limits Doses to public in unrestricted areas meet limits Population density too high Adjacent area with >1,000 population DOE cannot develop emergency preparedness program		Condition present: new information unlikely to change conclusion Condition present: new information unlikely to change conclusion Condition not present: new information unlikely to change conclusion Condition not present: new information unlikely to change conclusion Condition not present: new information unlikely to change conclusion
Site Owne QC:	ership and Control DOE can obtain land ownership and rights		Condition present: new information unlikely to change conclusion
Meteorolo	gy Oge ditions with not load to unsee stable selectors		
QC:	Conditions will not lead to unacceptable releases	•	Condition present: new information unlikely to change conclusion
Offsite Ins QC:	stallations and Operations Offsite facilities will not lead to unacceptable releases		Condition is likely to be present
DC:	Irreconcilable conflicts expected with atomic energy defense activities		Condition not present: new information unlikely to change conclusion
		- 	

ESSEJY5P.125.NWTRB/11-4-91

DOE Siting Guideline

Conclusion

Preclosure Guidelines: Environment-Socioeconomic Impacts-Transportation

System Guideline: Public and environment can be protected

Environmental Quality

- QC: Environmental quality adequately protected
- DC1: Environment cannot be protected and impacts cannot be mitigated
- DC2: Facilities located in federally protected areas
- DC3: Irreconcilable conflicts with protected areas expected

Socioeconomic Impacts

- QC: Impacts can be offset by reasonable mitigation or compensation
- DC: Water quality/quantity expected to be significantly impacted

Transportation

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- QC1: No conflicts due to location of access routes
- QC2: Technology adequate to develop system
- QC3: Extreme performance standards not required
- QC4: No unacceptable risks or environmental impacts

Condition is likely to be present

Condition is likely to be present Condition is not likely to be present

Condition not present: new information unlikely to change conclusion Condition is not likely to be present

Condition is likely to be present

Condition is not likely to be present

Condition is likely to be present Condition is likely to be present Condition is likely to be present Condition is likely to be present

DOE Siting Guideline

Conclusion

	Ease and Cost of Siting, Construction, Operation, and Closure						
System G	uideline: Technology available to accommodate site conditions		Condition is likely to be present				
Surface C	haracteristics						
QC:	Technology available for terrain & flood control		Condition present: new information unlikely to change conclusion				
Rock Cha	racteristics						
OC1:	Adequate rock thickness and lateral extent		Condition is likely to be present				
QC2:	Conditions will cause no undue hazards to personnel	· · · · ·	Condition present: new information unlikely to change conclusion				
QC3:	Technology available to accommodate conditions	· ·	Condition present: new information unlikely to change conclusion				
DC:	Significant risk to health and safety expected		Condition not present: new information unlikely to change conclusion				
Hvdroloav							
QC1:	Conditions allow repository development		Condition present: new information unlikely to change conclusion				
QC2:	Liners and seals will function as intended		Condition present: new information unlikely to change conclusion				
QC3:	Technology available to accommodate hydrology		Condition present: new information unlikely to change conclusion				
DC:	Technology not available for ground-water conditions expected		Condition not present: new information unlikely to change conclusion				
Tectonics							
QC:	Technology adequate for expected conditions		Condition is likely to be present				
DC:	Technology not available to accommodate expected		Condition not present: new information unlikely to change conclusion				
	fault movement or ground motion						
			· ·				
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ESSEJY5P.125.NWTRB/11-4-91

ESSE CORE TEAM

JEAN L. YOUNKER ROBERT C. MURRAY

WILLIAM B. ANDREWS LYNDON B. BALLOU JAN A. DOCKA **ARTHUR A. DUCHARME** WILLIAM W. DUDLEY **GREGORY A. FASANO RICHARD J. HERBST DWIGHT T. HOXIE STEVEN R. MATTSON** MICHAEL A. REVELLI LAWRENCE D. RICKERTSEN LES E. SHEPHARD **BRUCE R. JUDD**

JANE R. STOCKEY JEREMY M. BOAK

T&MSS T&MSS T&MSS LLNL WESTON SNL USGS T&MSS LANL USGS T&MSS LLNL WESTON SNL **DECISION ANALYSIS** COMPANY **DOE/HQ** DOE/YMP

TASK MANAGER **DEPUTY TASK MANAGER &** PEER REVIEW CHAIRMAN TRANSPORTATION **ROCK PROPERTIES. ENGR SYSTEMS** PETROLOGY **RISK ASSESSMENT/SEISMIC HAZARD** SURFACE CHARACTERISTICS **ENVIRONMENTAL QUALITY** GEOCHEMISTRY CLIMATE NATURAL RESOURCES **ENGINEERED SYSTEMS** TOTAL SYSTEM PERFORMANCE **HYDROLOGY CONSENSUS BUILDING, EXPERT ASSESSMENTS TECHNICAL MONITOR TECHNICAL MONITOR**

ESINT5P.125/10-7-91

PEER REVIEW PANEL FOR THE EARLY SITE SUITABILTIY EVALUATION

<u>NAME</u>

Stan L. Albrecht Walter J. Arabasz John H. Bell

F. William Cambray

Steven W. Carothers

James Drever Marco T. Einaudi Donald E. French Kip V. Hodges Robert H. Jones David K. Kreamer William G. Pariseau

Thomas A. Vogel Thompson Webb, III

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ORGANIZATION

Brigham Young University University of Utah University of Nevada, Las Vegas

Michigan State University

Southwest Environmental Consultants, Inc. University of Wyoming Stanford University Private Consultant MIT

Private Consultant University of Nevada, Las Vegas University of Utah

Michigan State University Brown University

SPECIALTY

Socioeconomic Impacts

Tectonics/Seismic Hazards Health Physics & Radiological Safety

Structural Geology & Tectonics

Environmental Quality

Geochemistry Economic Geology Petroleum Geology Tectonics - General Transportation Impacts Hydrology Rock Characteristics -Engineering Geology Tectonics - Volcanology Climatic Change

EARLY SITE SUITABILITY EVALUATION (ESSE) TASK



IV. A DRILLING AND CORING SYSTEM FOR OBTAINING NATURAL STATE SAMPLES FROM THE UNSATURATED ZONE

The need to obtain natural state core from the unsaturated zone at Yucca Mountain from depths greater than 1000 feet was a problem to the DOE Yucca Mountain Project because the technology to do this did not exist. The normal method of coring using water (drilling mud) was not satisfactory for site characterization. To solve this problem, DOE contracted Lang Manufacturing of Salt Lake City, Utah, to design the dry core drilling system and build a drill rig to use it. The result is the LM-300 drilling rig that will be used to drill and core UZ-16 to the water table.

On October 27, 1990, Dr. Uel S. Clanton, Chief Site Investigations Branch, DOE-YMP prepared, with Roy C. Long, a Drilling Engineer on Dr. Clanton's staff, a presentation showing how this drilling and coring system works. The presentation is in the form of viewgraphs that are self explanatory.

Attached are the viewgraphs showing the LM-300 drill rig and the dry coring system that was developed for this project. (7)

V. MEETINGS

Three meetings were attended during the month of November, 1991. These were:

- Secretary of Energy Advisory Board Task Nov. 6-7, 1991
 Force on Civilian Radioactive Waste Oakland, CA
 Management
- Workshop on Ground Water Travel Time in Nov. 13-14, 1991 the Saturated Zone Tucson, AZ
- Total System Performance Assessment Nov. 18-19-20, 1991
 Review
 Las Vegas, NV

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A DRILLING AND CORING SYSTEM FOR OBTAINING NATURAL STATE SAMPLES FROM THE UNSATURATED ZONE

AUTHORS:

DR. UEL S. CLANTON, ROY C. LONG

PRESENTED BY:

DR. UEL S. CLANTON CHIEF, SITE INVESTIGATIONS BRANCH YUCCA MOUNTAIN PROJECT U.S. DEPARTMENT OF ENERGY, LAS VEGAS, NEVADA

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(Insert 7-a)

OCTOBER 29, 1990

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(Insert 7-b)



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THE CORE ROD IS RUN IN THE HOLE INSIDE THE DUAL WALL PIPE. THE DRILLPIPE ACTS AS A PROTECTIVE CASING TO PROTECT THE CORE ROD FROM THE FORMATION AND TO PROTECT THE FORMATION FROM THE HIGH PRESSURE AIR AND CUTTINGS PRODUCED BY THE CORING OPERATION. ARROWS INSIDE AND ADJACENT TO CORING ASSEMBLY INDICATE DIRECTION OF AIR FLOW DURING CORING OPERATIONS.



ert 7-d

CORING OPERATIONS ARE COMMENCED AND THE CORE ROD IS ADVANCED 40 FEET AHEAD OF THE DUAL WALL **PIPE IN 10 FOOT INCREMENTS** (10 FOOT CORES). THE CORES **ARE RETRIEVED BY CONVENTIONAL WIRELINE** WHILE THE CORE ROD IS LEFT IN THE HOLE FOR THE **DURATION OF THE 40 FOOT** CORE RUN. THE 40 FOOT LIMIT **IS USED TO PREVENT THE** MORE FLEXIBLE CORE ROD FROM INITIATING A DEVIATION IN THE BOREHOLE AND **CAUSING THE DRILLPIPE TO** FOLLOW A DEVIATED PATH **RESULTING IN BINDING OF THE DUAL WALL PIPE.**



AT THE END OF EACH 10 FOOT CORED INTERVAL THE CORE ROD IS PICKED UP SLIGHTLY AND THE CORE IS BROKEN BY THE CORE CATCHER JUST ABOVE THE CORE BIT. THE CATCHER IS A DEVICE WHICH ALLOWS THE CORE TO ENTER THE INNER BARREL BUT PREVENTS IT FROM BACKING OUT. A WIRELINE LATCH (OVERSHOT) IS THEN RUN INSIDE THE CORE ROD AND THE TOP OF THE INNER BARREL IS "CAUGHT" WITH THE WIRELINE.



SRPDDP3P.A03/3-26-90

AFTER THE CORE IS BROKEN, THE INNER BARREL (WITH CORE HELD IN BY THE CORE CATCHER) IS PULLED OUT OF THE HOLE BY WIRELINE. A NEW (EMPTY) INNER BARREL IS THEN RUN IN HOLE, LATCHED INTO THE OUTER BARREL, AND THE WIRELINE IS REMOVED. THIS SEQUENCE IS REPEATED EACH TIME THE CORE TRACK IS ADVANCED 10 FEET.



SRPDDP3P A03/3-26-90

THE CORING STRING IS PULLED OUT OF THE HOLE AT THE END OF THE 40 FOOT CORE RUN IN PREPARATION FOR REAMING DOWN THE CORE TRACK WITH THE DUAL WALL PIPE.



(Insert 7-h

ONCE THE CORING ASSEMBLY IS OUT OF THE BOREHOLE, IT IS DRILLED/REAMED WITH THE DUAL WALL DRILL STRING TO THE **BOTTOM OF THE CORE TRACK.** THE FORMATION IS PROTECTED FROM CONTAMINATION NORMALLY **ASSOCIATED WITH DRILLING BY CIRCULATING THE CUTTINGS UP** THE CENTER OF THE DUAL WALL PIPE. CONTAMINATED FORMATION **CAUSED BY THE CORING OPERATION IS REMOVED WHEN** THE CORE TRACK IS REAMED **DOWN. THE BOLD ARROWS** INDICATE THE DIRECTION OF AIR FLOW DURING REAMING.



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A. <u>Secretary of Energy Advisory Board Task Force on Civilian</u> <u>Radioactive Waste Management</u>

In May, 1991, Secretary of Energy James D. Watkins established the Task Force on Civilian Radioactive Waste Management under the auspices of the Secretary of Energy Review Board. The Task Force has been asked to:

- Identify the factors that affect the level of public trust and confidence in its programs.
- Assess the effectiveness of alternative financial, organizational, legal, and regulatory arrangements in promoting public trust and confidence.
- Consider the effects on other programatic objectives, such as cost and timely acceptance of waste, or those alternative arrangements.
- Provide the Secretary with recommendations and guidance for implementing those recommendations.

In September, 1991, the Secretary expanded the Task Force's scope to include the program to manage wastes from the defense program (the above from the Advisory Board's meeting notice).

This was a public meeting held in the DOE office in Oakland, California.

Mr. Daniel Metlay, DOE Task Force Director invited a group of DOE and local government organizations to speak on the problems and concerns of the public and and how those problems and concerns are being addressed. Inserted is the agenda for the meeting and the Task Force membership roster. (8)

Since Mr. Metlay invited DOE representatives from Rocky Flats, OakRidge, Richland and the Nevada Operations Office to describe the programs these activities have emplaced to address public concerns, I was surprised and very disappointed that the Yucca Mountain Site Characterization Project Office was excluded. As is well known, YMPO has a very active public interaction program. As can be seen from the agenda, the State of Nevada and

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AGENDA SECOND MEETING SECRETARY OF ENERGY ADVISORY BOARD TASK FORCE ON CIVILIAN RADIOACTIVE WASTE MANAGEMENT OAKLAND, CALIFORNIA

November	6,	1991		
8:30 AM		11:45	AM	<pre>Task Force discussion Activities overview Commissioned papers Mr. Craig Thomas Dr. Jack Citrin National Academy of Sciences workshop National Academy of Public Administration workshop Case studies Description of waste management organizations</pre>
11:45 AM 12:00 PM 1:00 PM		12:00 1:00 4:45	PM PM PM	Public comments Lunch break Presentations by representatives of Nevada state and local governments and groups Mr. Robert Loux, Nevada Nuclear Waste Project Office Mr. Elgie Holstein and Mr. Phillip Niedzielski- Eichner, Nye County Mr. Dennis Bechtel, Clark County Mr. Vernon Poe, Mineral County Ms. Judy Treichel Nevada Nuclear Waste Task Force
4:45 PM 5:00 PM		5:00	PM	Public comments Adjourn
November	7,	1991		
8:00 AM		12:00	РМ	 Presentations by representatives of the Department of Energy Field Offices Mr. Robert Nelson, Jr., Manager, Rocky Flats Mr. William Adams, Assistant Manager for Environment and Waste Management, Oak Ridge Mr. Ron Izatt, Program Manager, Environmental Assurance, Policy, and Permit Program, Richland Mr. Bruce Church, Assistant Manager for Environment. Safety and Health. Nevada
12:00 PM 1:00 PM		1:00 3:00	PM PM	Lunch break Presentations by Nevada local governments and groups Mr. Mike Baughman, Lincoln County Mr. Hugh Anderson, Nevada Nuclear Waste Study Committee
3:00 PM 3:15 PM 4:30 PM	-,- -	3:15 4:30	PM ⁻ PM	Public comments Task force discussion Adjourn



Secretary of Energy Advisory Board Washington, DC 20585

MEMBERS OF THE SECRETARY OF ENERGY ADVISORY BOARD TASK FORCE ON CIVILIAN RADIOACTIVE WASTE MANAGEMENT

Ms. Barbara Barry Director Rocky Flats Program Colorado Department of Health Denver, CO 80220

Dr. William Bishop Vice President Desert Research Institute Las Vegas, NV 89120

Mr. William Eichbaum Vice President World Wildlife Fund Washington, DC 20037

Mr. Robert Fri President Resources for the Future Washington, DC 20036

Ms. Kristine Gebbie Secretary of Health State of Washington Olympia, WA 53706

Dr. Donald Kettl Professor, Political Science University of Wisconsin Madison, WI 53706

Dr. John Landis Senior Vice President Stone and Webster Engineering Corporation Boston, MA 02325 Dr. Todd La Porte, Chair Professor, Political Science University of California Berkeley, CA 94720

Mr. David Lester Executive Director Council of Energy Resource Tribes Denver, CO 80202

Mr. Gene Lucero Partner Sidley & Austin Los Angeles, CA 90071

Dr. Alfred Schneider Professor, Nuclear Engineering Georgia Institute of Technology Atlanta, GA 30338

Mr. Mason Willrich Chief Executive Officer PG&E Enterprises San Francisco, CA 94106

Mr. Michael Wilson Member Florida Public Service Commission Tallahassee, FL 32399

Dr. Mayer Zald Professor of Sociology and Social Work University of Michigan Ann Arbor, MI 48109

(Insert 8-b)

local Government bodys were included and gave their views. Mr. Church from DOE NVO only spoke on the weapons program at the NTS. Also, the only speaker whose remarks were cut short by the Task Force Moderator was the representative of the Nevada Nuclear Waste Study Committee, the only public organization that spoke in favor of the Yucca Mountain Study.

The major criticism of this meeting as expressed by the non-DOE participants was that the meeting was held outside the State of Nevada in a DOE facility, thus limiting public participation. I notice that the next meeting of the Task Force is in Washington, D.C.

B. Workshop on Ground-Water Travel Time in the Saturated Zone

Enclosed is the agenda and a partial list of participants.

The important message, at least to me, that came out of this workshop is that the saturated zone is an important component of the hydrologic system at Yucca Mountain and that the story will not be complete unless the saturated zone is understood and incorporated into the total system. Until now, the emphasis has been almost totally on the unsaturated zone.

As the understanding of the saturated zone matures, there will be additional workshops on this subject.

C. Total System Performance Assessment Review

At the beginning of this meeting, Abe Van Luik (Intera-M&O) cautioned all present that the various models that would be discussed were not mature enough to yield answers. Further, the participants were told that they would not be held accountable for the content of their presentations. Only Weston and EPRI furnished handouts.

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Abe Van Luik will prepare a summary of the meeting to be available in January and a full report will be ready by summer; 1992.

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Two points were apparent to me after listening for three days:

- The models presented are still a long way from describing earth processes in a useable way. The participants seem confident that these problems will be resolved.
- The second point relates strongly to the first there still does not seem to be any real cooperation between the modelers and the data gatherers and interpreters. This meeting was attended primarily by statisticians, mathematicians and computer scientists. Very few attendees work on site characterization. It seems logical that for a model to do its job, the modeler must know what the interpreter needs and the data gatherer must know what the modeler needs. This is undoubtedly an over simplification but it is the impression that meetings such as this leave with an observer.

There are no new issues that this office has identified that have not been brought to management's attention.

cc: w/encs.: K. Stablein, 4H3, J.E. Latz cc: w/o encs.: J. Roberts, C.P. Gertz, R.E. Loux, C. Pflum, J. Martin, G. Cook, D.M. Kunihiro, D. Weigel, J. Linehan, 4H3; B.J. Youngblood, 4H3; R. Bernero, 6A4; H. Thompson, 17G21; H. Denton, 17F2; S. Gagner, 2G5; E. O'Donnell, NLS 260; J. Holonich, 17G21

Encs: Total System Performance Assessment Review Meeting Agenda, 11/18-20/91; Performance Assessments Supporting the Early Site-Suitability Evaluation, 11/18/91, (Rickertsen); EPRI HLW Performance Assessment Model, 11/18-20/91, (Shaw); EPRI High-Level Waste Performance Assessment Projects, Phase 2, 11/18/91, (McGuire); Attendance lists; Agenda, 11/13/91 (Tucson, AZ); Saturated Zone Workshop (Tucson, AZ list); Terms of Reference, Secretary of Energy Advisory Board, Task Force on Civilian Radioactive Waste Management, Oakland, CA w/Biography of Panel Members; Impact of Repository-Heat-Driven Hydrothermal Flow on Repository Performance: Benefits from High Repository Thermal Loading, TPO, 12/13/91; FY92 November Status Surface Based Testing Program

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AGENDA

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101 Convention Center Drive, Phase 2 Yucca Mountain Project Office Large Conference Room Las Vegas, 18-20 November 1991

Day 1

Topic	<u>Speaker</u>	Duration	<u>Start</u>
Introduction - Participants - Purpose of meeting	Dyer/Boak	20 min	1:30
Overview & Discussion of Golder work	Miller	40 min	1:50
Overview & Discussion of EPRI work	Shaw	40 min	2:30
Overview & Discussion of the "ESSE" total system evaluation	Rickertson	40 min	3:10
Overview & Discussion of LLNL system model	Halsey	40 min	3:50
Overview of Days 2 and 3 - Overview of SNL work - Overview of PNL work	Bingham Eslinger	15 min 15 min	4:30 4:45
Adjournment	Day 2		5:00
Topic	Speaker	Duration	<u>Start</u>
Welcome for additional participants	Boak	15 min	8:45

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Day 2 - continued

Topi	<u>c</u>	Speaker	Duration	<u>Start</u>
Overview - 6-step method, SCP process - reasons for simpli- fication, the pyramid - CCDF construction method		Bingham	40 min	9:00
Brea	k		.20 min	9:40
Analy	ysis Setup and Results			
-	Basic data set (domain, boundary conditions, geohydrol. parameters, geochem.)	Dockery	15 min	10:00
-	Development of Geohydro- logical parameter distributions	Kaplan	15 min	10:15
-	Source term			
	1) LLNL assumptions of source-term model	O'Connell	20 min	10:30
	2) SNL source-term implementation	Wilson	20 min	10:50
	3) PNL source-term implementation	Engel	20 min	11:10
-	Geohydrologic & gas transport and results			
	1) PNL detailed anal-	Nichols	40 min	11:30
	2) PNL detailed anal- ysis (gas)	White	20 min	12:10
LUNCI	E		1 hour	12:30
-	Geohydrologic & gas			
	1) SNL abstract anal-	ONTINUED) Wilson	30 min	1:30
	ysis (geohydrology)			2.30
	2) SNL abstract anal- ysis (gas)	Wilson	30 min	2:00

AGENDA			PAGE 3/4
Day	<u>2</u> - continued		
Topic	Speaker	Duration	<u>Start</u>
- Human-intrusion compone and results	ent		
1) PNL - detailed	Eslinger	30 min	.2:30
2) SNL - abstract	Barnard	30 min	3:00
 Volcanism component and results 1) SNL 2) PNL 	l Dockery Murphy	25 min 15 min	3:30 3:55
BREAK		20 min	4:10
 Tectonics component and results (PNL) 	Rohay	20 min	4:30
- Complete CCDF 1) Construction of a combined CCDF (SN	Wilson IL)	20 min	4:50
2) Construction of a combined CCDF (PN	Eslinger IL)	20 min	5:10
Adjournment			5:30
	Day 3		
Topic	<u>Speaker</u>	Duration	<u>Start</u>
Analysis and Comparison			
- SNL sensitivity studies	Dockery	30 min	8:30
- PNL sensitivity studies	Eslinger	30 min	9:00
Comparison of abstract and detailed calculational results	Barnard/ Group	45 min	9:30

BREAK

-

15 min

10:15

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AGENDA					PAGE 4/4
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Day 3 - continued

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Topic	<u>Speaker</u>	Duration	<u>Start</u>
Comparison of abstract and detailed calculational results (continued)	Barnard/ Group	30 min	10:30
Dose Calculations			
- Methods & results (PNL)	Miley	1 hour	11:00
 Comparison with NRC & EPA methods (SNL) 	Wilson	20 min	12:00
LUNCH		1 hour	12:20
- Lessons learned from abstraction (SNL)	Wilson	40 min	1:20
- Lessons learned from modeling done (PNL)	Eslinger	20 min	2:00
BREAK		20 min	2:20
Discussion and Summary			
- Alternate conceptual models	Andrews/Group	1 hour	2:40
- Structuring a total system assessment	Pahwa/Group	1 hour	3:40
Adjournment			4:40



Larry D. Rickertsen Weston Technical Support Team

18 November 1991



- OVERVIEW OF THE EARLY SITE-SUITABILITY EVALUATION (ESSE)
- OVERVIEW OF PERFORMANCE ASSESSMENT MODEL
- ESSE PA ANALYSES



WHEN ARE SITE-SUITABILITY CRITERIA MET?





- is considered to be met.
- Selection depends upon regulatory criteria and other constraints.



- High confidence means that additional work unlikely to reduce probability below suitability threshold.
- Low confidence means that additional work unlikely to increase confidence above suitability threshold
- Thresholds depend on decision-maker's values for relative importance of performance, cost, schedule etc.

ESSE SITING CRITERIA ARE SITING GUIDELINES OF 10 CFR PART 960

- Postclosure guidelines
- Preclosure guidelines
- Implementation guidelines


POSTCLOSURE GUIDELINES

- System guidelines Performance requirements from regulations
 - System performance (EPA standards)
 - EBS performance
- Technical Guidelines Detailed geologic considerations that can disqualify site
 - Geohydrology
 - Geochemistry
 - Climate changes
 - Erosion
 - Dissolution
 - Tectonics
 - Human Interference

EARLY SITE-SUITABILITY EVALUATION

- Focus on features or conditions that indicate site unsuitable for repository development
- Examine current information to identify key issues for each guideline
- Evaluate importance of issues relative to system performance



AGENDA

- OVERVIEW OF THE EARLY SITE-SUITABILITY EVALUATION (ESSE)
- OVERVIEW OF PERFORMANCE ASSESSMENT MODEL
- ESSE PA ANALYSES



ESSE SYSTEM PERFORMANCE ASSESSMENT

ESSE AQUEOUS RELEASES UNDISTURBED PERFORMANCE



ESSE GASEOUS RELEASES UNDISTURBED PERFORMANCE



.

SITE SUITABILITY FOR FIRST CRITERION P(R<1)



SITE SUITABILITY FOR SECOND CRITERION P(R<10)



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ESSE PERFORMANCE ASSESSMENT MODEL





ESSE AQUEOUS RELEASE MODEL

ESSEQRM

ESSE GASEOUS RELEASE MODEL



ESSE PERFORMANCE ASSESSMENT MODEL

Aqueous Release

Waste package breach time Congruent leach rate Non congruent leach rate Rn solubility limit EBS exposure UZ GWTT Effective UZ retardation SZ GWTT Repository dilution

Gaseous Releases

Rapid release fraction Gradual release fraction (off- gassing) Gas travel time Off gas travel time

Basis

Design goal Estimated from flux and UO2 solubility Data for UO2 Current information on radionuclides Fraction of flux in fractures in TS Calculated in 1D from site properties Estimated from matrix or fracture properties Estimated from analyses of SZ Calculated from geometry and SZ GWTT

Data for spent fuel Estimated from partitioning coefficients Estimates by Ross et al considering heat Gas travel time and small fraction of GWTT

PARAMETERS FOR UNDISTURBED CASE-

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<u>Inventories</u>

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All inventories are specified in terms of an initial (t-0) inventory multiplied by an appropriate exponential decay term.

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Aqueous Source Term

WP loading	2.3 MTHM
Awp, WP area	0.33 m2
q, flux past WP	Exponential distribution with mean = 1 mm/yr
Ks	Exponential distribution with mean = .62 mm/yr
Fracture porosity	0.00041
P(fracture flow)	0 if q < Ks, 1 if q > Ks
Twp, breach time	100,000 years if no flow in fractures
•	Normal distribution with mean = 1000 yrs, s.d.=
	350 yrs if flow in fractures
Usl, UO2 sol. lt.	Log is uniformly distributed from -4 to -3
Noncongruent 1ch rt	Log of normal distribution with mean $= -7$ and s.d
-	0405

Gaseous Source Term

Rapid rls fraction	Log of normal distribution with mean of -3.7 and
	s.d. of 1.1
Offgassed fraction	Uniform distribution from .05 to 0.1

Element Properties

Element Log S.L. Log Rslow	Rfast
U Uniform(-4,-3) Uniform(.7,.15) 1
Am Uniform(-9.7,-6.7) Uniform(2,4)	1.4
C 0 0	1
I 0 0	1
Np Uniform(-4.7,-3) Uniform(.7,2)	1
Pu Uniform(-7.3,-5.5) Uniform(2,3.2)	1.1
Tc 0 Uniform(0,1)	1.1

Travel Time

Tsz, SZ GWTT Repos dilution	Log Tsz distributed uniformly between 2.1 and 3.6 0.5/Tsz
Tuz, UZ GWTT	Tslow with prob. 0.82. Tfast with Pfast = 0.18
Tslow ,	Lognormal distr. with mean-20K yrs and s.d10K
	yrs .
Tfast	Exponential distribution with mean of 1 year
Tgas	Uniform distribution from 2000 to 6000 years
Toffgas	Tuz*Uniform(0,.1)*(1+Uniform(0,1))+Tgas

PARAMETERS FOR ISSUES

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Increased flux	
q Tslow Pfest	Exponential distribution with mean $= 1.5 \text{ mm/yr}$ Lognormal with mean $= 12\text{K}$ yrs and s.d. $= 6\text{K}$ yrs 0.21
<u>Water table rise</u>	
Tslow	Lognormal with mean - 16K yrs and s.d 10K yrs
Flux concentration	
q, flux	Exponential distribution with mean - 10 mm/y for 5 % of the repository area and exponential distribution with mean = 0.5 mm/yr for 95 % of the area
Tuz Tl	T1 with a prob = 0.05 and T2 with a prob = 0.95 Tslow is lognormal with mean = 4K yrs and s.d. = 2K yrs and Pfast is 0.75
T2	Tslow is lognormal with mean - 28K yrs and s.d 14K yrs and Pfast is 0.15
<u>Colloidal plutonium</u>	
R(plutonium)	Log of R distributed uniformly between3 and 3.2
Ground-water modeling	uncertainty
Tslow	Log of mean distributed uniformly between 3.3 and 5.3, log of s.d distributed uniformly between 3 and 5
Pfast	Log distributed uniformly between -1.36 and12
Source-term modeling u	incertainty
Flux, q Pwetting UO2 S.L. N.C. lch rate	Log of mean uniformly distibuted between -5 and -1 Uniformly distributed between 0 and 1 Log uniformly distributed between -6 and -1 Logs of mean and s.d. uniformly distributed 2 units above and below the reference values
<u>Human intrusion</u>	•
Distribution for for drilling thr 1991)	c undisturbed releases plus releases calculated by Wilson cough repository (Memorandum by Bernard et al. of June 23,

EFFECTS OF ESSE MODEL FEATURES EBS CONTAINMENT



EFFECTS OF ESSE MODEL FEATURES SATURATED ZONE AND RETARDATION MODELS





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COMPARISON OF ESSE AND EA ASSESSMENTS

 $\omega \in \mathcal{O}(\omega) \to \omega$



ISSUES IDENTIFIED IN TECHNICAL GUIDELINE EVALUATION

Geohydrology

- Effect of "fast paths" (e.g. heterogeneity; episodic infiltration; topography; perching)
- Modeling uncertainty (e.g. 2D vs 1D modeling)

Geochemistry

Colloidal transport of plutonium

Climate Changes

- Potential water table rise
- Potential increase in flux

Postclosure tectonics

- Occurrence of faults
- Local concentration of flux due to new paths
- Modification to water table

Human Interference

Potential for future exploration

Others

- Potential for gaseous release



EVALUATION OF TECHNICAL GUIDELINE ISSUES



EVALUATION OF TECHNICAL GUIDELINE ISSUES



EVALUATION OF TECHNICAL GUIDELINE ISSUES



SITE SUITABILITY FOR FIRST CRITERION P(R<1)





EPRI

EPRI HLW Performance Assessment Model

> Total System Assessment Review Meeting

November 18-20, 1991 Las Vegas, NV

> Robert A. Shaw EPRI

EPRI HIGH-LEVEL WASTE PERFORMANCE ASSESSMENT PROJECT PHASE 2

Presentation of

PRELIMINARY MODEL DESCRIPTION AND RESULTS

by

ROBIN K. MCGUIRE

Risk Engineering Inc.

 \mathbf{at}

DOE – Las Vegas, NV

November 18, 1991

Risk Engineering, Inc. (RKM)

11-18-91



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EPRI High Level Waste Project Methodology Development Team Attiliation Expertise

<u>Name</u>

EPRI/NPD

Michael J. Apted Daniel B. Bullen Stuart Childs **Neville Cook** Kevin Coppersmith Ralph L. Keeney John M. Kemeny Austin Long Robin K. McGuire F. Joseph Pearson, Jr. **Benjamin Ross** Frank W. Schwartz Michael Sheridan Robert A. Shaw J. Carl Stepp Robert F. Williams **Robert Youngs** Delbert S. Barth **Russ Dyer**

Intera Sciences Georgia Tech Cascade Earth Sciences, Ltd. Univ. of Calif, Berkeley Geomatrix Consultants Univ. of Southern California University of Arizona University of Arizona **Risk Engineering** Consultant Disposal Safety, Inc. **Ohio State University** State Univ. of NY, Buffaio EPRI EPRI EPRI Geomatrix Consultants UNLV/ERC Department of Energy

Waste Package Infiltration **Rock Mechanics** Seismic Geology Risk/Decision Analysis **Rock Mechanics** Climatology **Risk Analysis** Geochemistry Gaseous Transport Hydrology Volcanology Project Manager Seismology & Geophysics **HLW Sciences** Geotechnical Engineering Observer Observer - HLW&SFS

Port.Asses Rev./RAS 1918-3091 2







1. FLUX

2. LATERAL REDISTRIBUTION OF FLUX

3. $\Delta WT \mid FLUX$

4. FRACTURE-MATRIX COUPLING

5. SATURATED FLOW VELOCITY

6. MATRIX RETARDATION

7. VOLCANOES

8. $\Delta WT \mid VOLCANOES$

9. EARTHQUAKES

10. $\triangle WT \mid EARTHQUAKES$

11. HEAT PULSE

12. BOREHOLE FRACTURES

| HEAT PULSE

13. CONTAINER PERFORMANCE

| HEAT PULSE

14. WASTE AND ELEMENT SOLUBILITY

15. HUMAN INTRUSION



SE79 --- FULL INPUT, 14 LEVELS, 30240 END BRANCHE

(ey issues for describing net infiltration are:

- 1. Variability of Climate. Climate input to infiltration calculations are external but direct. Austin Long supplied probability distributions for:
 - Annual precipitation
 - Fraction of precipitation falling in the summer months (May October)
 - Annual air temperature.
- 2. Spatial variability of net infiltration. This issue was addressed in two steps:
 - Development of hydrologic/soil/topographic land units
 - Procedure for integrating net infiltration fluxes over the project area
- 3. Physical/biological processes for vertical flow. These were primarily addressed in an offline model of water flow:
 - Unsaturated zone water flow through the soil matrix
 - Plant canopy water uptake and surface water runoff
 - Plant canopy growth
 - Development of daily climate regimes from annual characterizations.







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SCHEMATIC OF EBS RELEASE MODES (O'Connell and Drach, 1986, UCRL-53761)



Assumptions/ Simplifications

- Two Groups of Radioelements Identified;
 - "Insoluble"/ Solubility-Limited Radioelements (e.g., Cs, Sn, U, Np, Pu, Am),
 - "Soluble"/ Reaction-Rate Limited Radioelements (e.g., Se, Tc, I, C),
- Initial "Gap" Portions ~ 2% of Total Inventory,
- "Wet-Drip", "Moist/ Wet-Continuous" and "Dry" Modes

"Wet-Drip" Mode Assumes:

- Entire Water Flux Directed into Waste Packages,
- Filled Bathtub Geometry

"Moist/ Wet-Continuous" Mode Includes:

- Radioactive Decay in Waste Form and During Migration (Decay-Chain Ingrowth Excluded),
- Sorption by Tuff,
- Diffusion or Convection-Diffusion in Porous Tuff,
- Degree of Hydrologic Saturation (Moist or Wet),
- Calculate Steady-State Release Rates (No Transients),
 - Attenuation from Radioactive Decay + Sorption,
 - No Sorption Delay to Reach Final Release Rates,
 - Current Yucca Mountain Waste-Package Design Has No Buffer/Backfill Barrier for Sorption,
 - Relatively Short Pathway (3 cm),
 - Uncertain Aggregate Properties of Crushed Tuff,
- Geometry Simplification (Equivalent Sphere),
- No Credit for Partially Failed Containment.

"Dry" Mode - Only Gaseous C-14 Can Escape.

INTERN

Proposed Three Cases

- Hot most canisters at Nitao temperatures
- Warm most canisters reach boiling point only; some hotter
- Cold maximum temperature 87°C



Calculated Path Lines



100× permeability contrast, repository temperature 57°C



100× permeability contrast, repository unheated

Histogram of ¹⁴C Travel Times



100× permeability contrast, repository temperature 57°C





Logic Tree For Groundwater System • groundwater system defined from the repository down to the water table and laterally to the accessible environment 5 km away (1) Groundwater Flux at Repository 235.0 m 20.0 m 1500FEE BEDION (2) Fracture/Matrix Coupling (3) Matrix Sorption Calico Hills unit 0.0 m (4) Saturated Flow Velocity

* depth to water table and timing of flux changes

determined upstream in the logic tree

- * depth to the water table changes tomorrow
- * flux can be changed once during simulation
- * diverted flow along faults does not encounter waste



transport model uses a moving particle approach to calculate mass outflow at the accessible environment
accounts for transport of a single constituent subject to advection, sorption, radioactive decay, loading
each particle defined by z-position and attached mass or activity of nuclide

 \cdot model provides ψ at a series of locations 10m apart

(c) Strong Coupling - Infiltration Rate 0.5 m/ma





Figure 2. Model for inadvertent Human Intrusion at Yucca Mountain (note: the first number of boreholes is for the 1000 year period until the year 3000; the number in parentheses indicate number of boreholes from 3000 to 12000 A.D.)



Normalized Releases

Probability



NOTMATIZED KG



Normalized Releases

Probability



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Normalized Releases



Probabi





Normalized Releases

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17. Tom	Buscheck L	LNL 510-42	-9390 3-9390	42 R	OGER STAF	HE 61	2482 9493
18.5.5	SARGEN TH	U (203)934-7	1625 H-T778	43. Mi	chael O.Clo	ninger 702	- 794-784
<u>19. Da</u>	vid Stahl Bel	<u>V (MBO) (702) 74</u>	<u>14-7778</u>	44. JO	ne Stocke Miller	<u>y 202-5</u>	86-
21. Rob	in N. Dalta, MA	0/wsfs (702) 7	94-183 2	46.			1203-0777
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24. Kg/	ston Barna	<u>vil SN2 (605)</u>	845-8403	49.			
D. hai	UTENCE J. LAS	11N, 5NL (50	100797001	<u>20</u>	<u></u>		

TOTAL SUSTEM ASSESSMENT Meetina: ·YMP/LSV Rm. 202 Location: Time: 8:45 AM Date: 11 Распе Attendee Сопралу Attendae Phone Company 702794-7441 25. ROBIN K. MEGUIRE REL 303-278-9800 1. AREVANLUIK (INTERA/MAD) FTS 544-7778 2 Mark White (509)572-008627. David Stah BRO (MRO) FUL (702) 794-7778 PTS 444-1247 3. WILL NICHOLS Duquid INTERA(MO) (PNL) 934-2431 in (504) 376 - 8247 4. ALGIN BRANDSTETTER (LAIC) FTS S44-7279 29. JOHN FLUECK UNLV 700-597-4124 30. Robin N. Datta M&O/WEFS (702) 794-1832 IN TERA/MAN E. Raha 512-346-2000 31. S. SAREFN 1- LNL 510-422-8789 6. Ell C. Commell TEW (703) 939-7625 U. Num #6124829493 7. JIN Gunse yer LLNL 510 422 7553 STHATE 32 33. Tom Plaford Univ. Calif. SNU Bleines 505-844-9160 510-642-1469 NORONHA/WESTON/202.646-6768 34. 9. CLIFF anastatt AVE DDE-RL (509) 10. Felton Binginam, SNL (505) 844-8816 35. SNL Michael son (505)846 -9865 36. PAHWA MÍQ 1 7021744 -1889 11. 702-798-7083 COOTER(206)883-0777 N DH 251 37. 12 Lany Rickertsin, wester/RAEL 202-6460 33: 301 Inal+ 13. NRL-L11 702 3886:25 14. Raleton Barnard SNL (505) 845 .840339. Terri Miley PNL 1529) 375-2601 Eng.Q. Pls(15. NL (510/8979 40. 11.90 (প্লেপ 75-210 AVL505 6675200 Lagen 1505 844 CM -1756 16. SNL 41. DOE/4MPD 702-794-7588 42 Bill HAUSEY LLNL 510-423-1133 17. Jerry Bon FTS 776-5019 505 544-178643. Derraht: Hoxie US6S 13031286-5019 18. 2. Cloke SHC- 6007794-782 501)376-8337-44. Poul MARK MURPHY. 19. inger PNL (509) 376-2797 45. Midledt, CLONIVART 50E (362) 79 20. 565) 844-0397 45. Claudio LESCATORE BNUL (516) 282-2731 1-Sun Park .47. SAIC 702-794 -7643 · · · . Kuss Unon 702-744-7586 23. Dre 48. LAL 570-481-0 Joseph Work 49. 415-955-2026 Dob Shaws EPRI 50. A.T. 12

Meeting: TSPA Day 3 Location: YMP/LSV Rm. 202 Time: 8:3074 Date: _____1/10/91 Phone Соптралу Phone Attendee Attendee Company 702 1. ABE VANLUIK, INTERA MAYO, TOI 794-7441 25 Paul Presthort NIRC-LV 3886125 505-844-910 27. JIM Gansemer 14NL 5104227553 2 Tom Blaiwers SNL (505) B44-8816 22 Laurence Costin SNL 505 849-039 3. Felton Bingham, SNL 4. Ian Miller Golder (206) 883-0777 25. JOHN FLUECK UNLV 702-597-4124 5. Russ Dyer Kaplan Doe (702) 794-7586 30. Marl SNL 505 BYY-1786 423-113 E. Lorn Ricketsen WESTON/PAGE 646-6760 31. Bill HALSEY LLNL 510 7. Holly Dockery 32 Paul L. Cloke SNL 505 - 844-1756 SAIC 702-794-7823 8. Jerry Boak DOE/414PO 702-794-7588 33. Mark White PNL 509 372 0086 S. W Barnard SNL 505. 545-5403 34. Krl Andrus INTERA/MID 512-346-2000 10. Kauliw. Eslinger PNL 309-326-279735. AUTIN BRANDIFETTER SAIC 702-794-7279 Lewis LLNL 510-422-8949 Michael Wilson SNL (505) 8+6-986836. LYNN 11. PNL 509 325-320 TEN (703) 934-7625 37. Dave Engel S. SAFEEN 12. Intuil 140 754.7686 38. Terri Miley (SO9) 375-2601 PNL 13. SURE SH PAHWA FTS 494 - \$247 702-794-764-3 39. Hue Nicitars PNL U-Sun Park SAIC - 376- X297 15. Tom Pigtord Univ. Calif. 510-642-14-920. MARK MURPM PNL (509)376-8337 570-107-6753 47. 15. Joseph Wang LBL EPRI 415 855 2026 22 BOR SHAW 17. 18. ROBIN K. MCGUIRE REI 303-278-9800 43. 19. Robin N. Datta Mzo/wers (702)794-1832 44. 20. Jim Duguid INTERA/MED 703-939-243145. 21. Dave Langstaff DOE-RL 509-376-1669 45. 22. Bill Nelson INTER IMED (702) 794-1880 47. KALPH HAAK RSN 702-794-7083 48. 24. Everett Springer LANL 505-667-9836 49. 25. CLIFF NORONHA /WESTON/202.646.6768 50. and the second State Barriston and

AGENDA

Yucca Mountain Site-Characterization Project Hydrologic Integration Task Force (HITF)

Workshop on Ground-Water Travel Time in the Saturated Zone

Radisson Suite Hotel Tucson 6555 East Speedway Tucson, Arizona 85710 (602) 721-7100

November 13, 1991 Salon A

8:00 A.M.	Welcome and introduction						
	Dwight Hoxie, U.S. Geological Survey (USGS)						

Opening remarks John Czarnecki, USGS, Chairman, HITF

Conceptual Models for the Site Saturated-Zone Geohydrologic Session I: System

Moderator: George Barr, Sandia National Laboratories (SNL)

8:30 A.M. Factors that may affect ground-water flow direction and magnitude at Yucca Mountain, Nevada John Czarnecki, USGS

9:15 A.M. A geologic hypothesis for the large hydraulic gradient at Yucca Mountain. Nevada Chris Fridrich, U.S. Department of Energy (DOE)

BREAK 10:00 A.M. -

Carl

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10:30 A.M. A conceptual model for the large hydraulic gradient, Yucca Mountain, Nevada **Richard Luckey, USGS**

11:00 A.M.	Developing framework-based conceptual and numerical models for ground-water systems Claudia Faunt and E.D. Gutentag, USGS
11:30 A.M.	Geologic considerations for modeling potential flow pathways in the saturated zone Richard Spengler, USGS
12:00 P.M.	LUNCH - Salon C
Session II:	Geohydrologic Data Needs and Availability Moderator: Claudia Newbury, DOE
1:00 P.M.	Flow and transport in fractured rocks: field tests and modeling Kenzi Karasaki, Lawrence Berkeley Laboratory (LBL)
1:30 P.M.	Fracture data needs for ground-water flow modeling at Yucca Mountain Elisabeth Ervin, USGS
2:00 P.M.	Pumping-test analysis in dual-porosity aquifers Nick Saines, Harza Engineering
2:30 P.M.	Saturated-zone hydrochemistry and data needs Bill Steinkampf, USGS
3:00 P.M.	BREAK
3:30 P.M.	Sr isotopes in ground waters, southern Nevada Zell Peterman, USGS
4:00 P.M.	DISCUSSION
5:00 P.M.	ADJOURN
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November 14, 1991 Garden Court

- Session III:Ground-Water Travel Time (GWTT) in the Site Saturated Zone
Moderator: Dwight Hoxie, USGS8:00 A.M.GWTT in the saturated zone: results from the Environmental
Assessment (1986)
Dwight Hoxie, USGS
- 8:30 A.M. **GWTT in the unsaturated zone: lessons learned** Paul Kaplan, SNL
- 9:00 A.M. GWTT and tortuosity: a parameter for scaling field and laboratory tests Todd Rasmussen, University of Arizona
- 9:30 A.M. Saturated-zone heterogeneity and GWTT Dwayne Chesnut, Lawrence Livermore National Laboratory (LLNL)
- 10:00 A.M. BREAK
- 10:15 A.M. Impacts of the conceptual models-on data development and GWTT calculations
- 12:00 P.M. SUMMARIZE AND ADJOURN

ONSITE C	UES.	FIONAIRE	-H	YDROLOGY	INTE	G R/	TION	TASK	FORC	Ξ(HITF)
WORKSHOP	P ON	GROUNDWA	TER	TRAVEL	TIME	IN	THE	SATUR	\TED	ZON	ΙE

1. Was this workshop a worthwhile effort? yes____ no____ sort-of____

2. What did you gain from it?

3. Would additional interactions on this same topic be beneficial? yes _____ no _____

4. Why did you attend?

5. How could it be improved?

6. What other topics would you like to see addressed in this sort of format?

7. Are there other types of interaction that you would prefer? yes______ no, this was just fine______ none, this was a waste of time______ If yes, what kinds of interactions would you prefer?

8. The HITF charter is attached along with a list of the membership; what can we do for you?

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9. Additional comments?

CURRENT HITF MEMBERSHIP

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John Czarnecki, Chairman - USGS Bill Dudley, USGS George Barr - SNL Tom Buscheck - LINL Everett Springer - LANL Levi Kroitoru - Weston (for OGD/Analysis & Verification Division) Chris Fridrich - YMP/RSED Claudia Newbury - YMP/RSED (WBS manager)

CHARTER FOR THE HYDROLOGY INTEGRATION TASK FORCE (HITF)

The HITF is chartered to provide a Project forum for the consideration of issues relative to both saturated and unsaturated zone hydrologic investigations, model development, and performance assessment. Issues related to:

- o prioritization of activities and studies based on regulatory requirements;
- o consistency between near- and far-field and saturated and unsaturated models;
- o support of the test and evaluation process;
- o prioritization of tasks, funding and deliverables;
- o support of peer reviews;
- o credibility of performance models;
- o sharing of information or data;
- o combining of resources; and
- o concerns of individual hydrology Principal Investigators will be brought to the HITF. Through consideration of those issues, the HITF will develop recommendations to Regulatory and Site Evaluation Division/Technical Analysis Branch for synthesis of the existing hydrologic program, or possible redirection or reprioritization of hydrologic investigations.

The HITF will consist of a representative from:

- each of those project participants who conduct research in the field of hydrology, including both site investigations and performance assessment;
- o the Management and Operation integrator for hydrology;
- o the Office of Geologic Disposal Yucca Mountain Site Characterization Project Office, and others who may be appointed by the Project Manager.

The chairmanship of the HITF will rotate on an annual basis, beginning with the representative of the U.S. Geological Survey.

The HITF will submit its recommendations to the U.S. Department of Energy representative appointed as the Project Work Breakdown Structure Manager for the hydrology task.

The HITF will meet on at least a quarterly basis.

Carl P. Gertz, Project Manager Yucca Mountain Site Characterization Project

*0*C J. Russell Dyer, Director (Acting) Regulatory and Site Evaluation Division

Claudia M. Newbury

Work Breakdown Structure Manager

bert W.

Ar John P. Czarnecki, Chairman

Saturated Zone Workshop Tucson, Arizona

G. E. Barr Sandia National Laboratories P.O. Box 5800 Kirtland AFB Albuquerque, NM 87185

Harold Bentley Hydro Geo Chem 1430 North 6th Avenue Tucson, AZ 85705

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Kay Birdsell Group EES-5, M/S F665 Los Alamos National Laboratory Los Alamos, NM 87544

Albin Brandstetter Science Applications International Corporation 101 Convention Center Drive, Suite 407 Las Vegas, NV 89109

William F. Chambers Organization 6312 Sandia National Laboratories Albuquerque, NM 87185

Dwayne Chesnut Lawrence Livermore National Laboratory, L-202 P.O. Box 808 Livermore, CA 94550

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Elisabeth Ervin U.S. Geological Survey Box 25046, M/S 421 Denver Federal Center Denver, CO 80225

June Fabryka-Martin Los Alamos National Laboratory, M/S J514 Los Alamos, NM 87545

Claudia Faunt U.S. Geological Survey Box 25046, M/S 421 Denver Federal Center Denver, Co 80225 Saturated tome workshop

Tucson, Avizona

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E. D. Gutentag U.S. Geological Survey Box 25046, M/S 421 Denver Federal Center Denver, CO 80225

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TERMS OF REFERENCE SECRETARY OF ENERGY ADVISORY BOARD TASK FORCE ON CIVILIAN RADIOACTIVE WASTE MANAGEMENT

INTRODUCTION

The Department of Energy recognizes that the resolution of outstanding institutional issues, such as access to sites, social and economic impacts, and organizational design, is as critical to the ultimate success of the civilian radioactive waste management program as the resolution of outstanding technical issues. No institutional issue commands as much attention and is as widely regarded as pivotal and far-reaching as the question of public trust and confidence. It is, for example, a common theme in reviews by organizations such as the National Academy of Sciences, the Congressional Office of Technology Assessment, and the Nuclear Waste Technical Review Board.

Although numerous oversight and advisory bodies are examining the technical foundations of the program, there is currently little systematic analysis and guidance on developing the institutional framework for managing radioactive waste in a manner that ensures public trust and confidence. Such analysis and guidance would be helpful not only to the existing policy-making organizations that are conducting many of the program's immediate activities but also in the on-going creation and design of the technical development and operating organizations that will play increasingly critical roles in the program's future. The objective of the Secretary of Energy Advisory Board (SEAB) Task Force of Civilian Radioactive Waste Management is to begin to undertake those institutional analyses and to suggest approaches for establishing public trustworthiness so as to facilitate progress toward the Department's satisfaction of its statutory obligations.

As detailed below, the Task Force should examine what is meant by "public trust and confidence" and describe the conditions that are important for ensuring it. The group should explore what additional steps the program might take to strengthen public trust and confidence in efforts to dispose of radioactive waste. The Task Force should investigate whether attempts to increase public trust and confidence affect other objectives such as timely waste acceptance and cost-effectiveness. Finally, the group should consider how its recommendations and guidance might be implemented.

STUDY OBJECTIVES

The Meaning and Development of Public Trust and Confidence

The phrase "public trust and confidence" is frequently used, but its meaning is rarely articulated with precision. Consequently, misunderstandings among parties with an interest in those ends may arise, and accusations of bad faith may be leveled, leading ironically to reduced trust and confidence. The Task Force should strive to develop a clear understanding of what it means for the radioactive waste management program to have public trust and confidence extended or withheld. The group should then analyze the factors and processes that cause it to be gained, maintained, lost, and reestablished. Among the questions the Task Force should address are:

Whose trust and confidence is most critical? Why? -

What are the most important factors affecting the level of public trust and confidence in the program? 200 Barris Contraction Contraction

1 What lessons has the program learned from the past? What can be done to build on past successes and avoid past failures? . . .

Opportunities for Ensuring Public Trust and Confidence

The management of radioactive waste poses a number of challenges, which, in combination, may make the establishment and maintenance of public trust and confidence problematic. Hazardous materials must be processed and transported; the benefits of nuclear power are widely distributed, but many of the costs of waste management are geographically concentrated; political and technical accountability must be sustained over extended periods; a relatively large-scale technological system with a complex institutional infrastructure must be created; some errors may only arise in the far future, and others may be hard to detect. Based on the understanding and insights developed in the first phase of the study and through other means, the Task Force should consider questions such as these:

- 7 N How can the challenges that tend to make public trust and confidence in the radioactive waste management program problematic be addressed?
- Ĩ Under what circumstances, if any, can alternative financial, organizational, and regulatory arrangements for the program promote public trust and confidence?
- 1 Can the organizational structures and processes adopted for similar programs in other nations provide models for increasing the perceived trustworthiness of the U.S. program?

Consequences of Ensuring Public Trust and Confidence

Actions taken to ensure a significant reservoir of public trust and confidence may affect other program objectives such as the timely acceptance of waste, cost-effectiveness, and confidence in the program's schedule. Those other factors must be taken into account as any long-term implementation plan is developed. If trade-offs between conflicting goals have to be made, it is important that the stakes be clarified and the balancing of advantages and disadvantages of various approaches be done explicitly. To inform choices that will have to be made, the Task Force should investigate these questions:

- To what degree would additional efforts to foster public trust and confidence disrupt established program routines and organizational interactions?
- 1 How would efforts to ensure high levels of public trust and confidence

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influence the timeliness and the cost of the radioactive waste management program?

To what extent would initiatives to increase public trust and confidence affect or be affected by the regulatory regime for developing and licensing a repository?

Recommendations

Having assessed alternative approaches for ensuring public trust and confidence and having considered in general terms what the central advantages and disadvantages of each might be, the Task Force should present recommendations to the Secretary of Energy. Included in those recommendations should be guidance on what steps can be taken to implement them. In particular, the Task Force should note which actions can be taken under authority already vested in the Department, which actions require new authority, and which actions depend on the cooperation of other governmental and non-governmental entities.

In pursuing these objectives, the Task Force can

- Obtain the advice of recognized experts in organizational design;
- Examine program decisions and policies over the last decade that have strongly contributed to the current level of public trust and confidence;
- Solicit the views of informed and interested individuals both inside and outside of government;
- Secure information from DOE program offices and contractors that helps identify the characteristics of the policy-making, technical design and development, and operating organizations of the radioactive waste management system.

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DONALD F. KETTL is a professor in the Department of Political Science and the Robert M. La Follette Institute of Public Affairs at the University of Wisconsin-Madison. He has held academic appointments at Vanderbilt University, the University of Virginia, and Columbia University. Dr. Kettl is the author of numerous articles and books, including <u>Private Markets and Public Management</u> (forthcoming), <u>Deficit Politics: Public Budgeting in</u> <u>Institutional and Historical Context</u>, and <u>Government by Proxy: (Mis?)Managing Federal</u> <u>Programs</u>. He is an active member of the American Political Science Association, whose Committee on Publications he will chair in 1992. He has consulted for a number of federal bodies, including the Committee on the Budget of the U.S. House of Representatives, the Food and Drug Administration, and the Securities and Exchange Commission. Dr. Kettl is a member of Phi Beta Kappa and currently serves on the editorial boards of <u>Public</u> <u>Administration Review</u>, Journal of Public Administration Research and Theory, and <u>American</u> <u>Review of Public Administration</u>.

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TODD R. LA PORTE is professor of political science at the University of California, Berkeley, CA. He formerly served as associate director of the University's Institute of Governmental Studies. He was a Woodrow Wilson fellow at the International Center for Scholars at the Smithsonian Institution. He was a member of the National Academy of Sciences' panel on the agenda for Research on Human Factors in Commercial Nuclear Power Plant Operations, and he served on the Academy's Board on Radioactive Waste Management. He is a member of the National Academy of Public Administration.

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Impact of Repository-Heat-Driven Hydrothermal Flow on Repository Performance: Benefits from High Repository Thermal Loading

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Yucca Mountain Project - Project Mananger's/TPO Meeting December 13, 1991

Impact of thermal loading on repository performance at Yucca Mountain

- Overview of Yucca Mountain hydrology
- Hydrothermal flow at the repository horizon
- Temperature profiles as a function of thermal load
- Impact of hydrothermal flow on temperature distribution
- Impact of thermal load on repository performance
- Impact of thermal load on hydrogeologic uncertainties
- Conclusions
- Appendix

Overview of Yucca Mountain hydrology

- The key consideration is the impact of thermal load on fracture-dominated flow
 - Matrix-dominated flow will not result in significant vertical transport of radionuclides
 - Field evidence indicates fracture-dominated flow can occur to considerable depth
 - Fracture-dominated flow is only credible mechanism bringing water to waste packages and transporting radionuclides
- Boiling and dry-out greatly enhance fracture flow attenuation
 - These effects can reduce the impact of uncertainties

Factors mitigating liquid flow along preferential fracture pathways

- Discontinuity in fracture networks
- Liquid-phase dispersion in fracture networks
- Fracture-matrix interaction
 - For low APD's, only matrix imbibition
 - For high APD's, boiling effects and enhanced imbibition due to dry-out

5-TB-15 (10-3-01) PM







Above the repository, the travel time for liquid flow down a preferential fracture pathway is dominated by matrix flow into the vitric nonwelded Paintbrush tuff unit (PTn)

Dimensionless liquid saturation in the matrix resulting from a wetting event down a 100µm fracture



The travel time for a fracture front to penetrate a hydrostratigraphic unit is extremely sensitive to fracture aperture

 $T < T_b$

D_m = matrix capillary diffusivity

- $\phi_m = matrix porosity$
- S_i = initial matrix. saturation
- S_s = maximum matrix saturation
 - τ = tortuosity of fracture pathway
 - t'= travel time

Flow period I $t \sim \frac{\tau L}{h^2}$

Flow period II

$$t \sim \frac{D_m[\tau L \phi_m (S_s - S_i)]^2}{b^6}$$

The travel time for liquid flow down a preferential fracture pathway is extremely sensitive to the hydraulic aperture of the fracture

Dimensionless liquid saturation in the matrix resulting from a wetting event down (a) a 1000µm fracture, (b) a 200µm fracture, and (c) a 100µm fracture



Below the repository, the travel time for liquid flow down a preferential fracture pathway is dominated by matrix flow into the vitric nonwelded Calico Hills unit (CHnv)

Dimensionless liquid saturation in the matrix resulting from a wetting event down a 100µm fracture



Below the repository, the travel time for liquid flow down a preferential fracture pathway is dominated by the thickness of the vitric nonwelded Calico Hills unit (CHnv)

Dimensionless liquid saturation in the matrix resulting from a wetting event down a 100µm fracture for cases where the CHnv is (a) absent, (b) 4.6-m-thick, and (c) 40-m-thick



Hydrothermal flow at the repository horizon

- Unsaturated, fractured tuff promotes rock dry-out by boiling
- Volume of dry-out zone dominated by thermal load and thermal properties
- Fracture-matrix properties of host rock promote rapid condensate drainage
- Volume of dry-out zone can be enhanced by alternative emplacement configurations
- The numerical models used in this study are very conservative in predicting the dry-out volume

Under hydrothermally perturbed conditions, boiling will mitigate episodic fracture flow from reaching the waste package (for up to 1000 years for a repository heat loading rate of 57 *kw/acre*) (Buscheck and Nitao, 1991)



The bulk permeability data for Topopah Spring tuff (TSw2) at the repository horizon is much greater than the threshold bulk permeability for significant rock dry-out

Dimensionless liquid saturation for 30-yr-old fuel, an APD of 57 kW/acre, a drift spacing of 38.4 m, and a recharge flux of 0.0 mm/yr at t = 60 yr (the boiling point isotherm, T_b is shown in yellow)



ES-TB-26 (10-7-01) IL

The existing bulk permeability data for Topopah Spring tuff (TSw2) at the repository horizon is much greater than the threshold bulk permeability for significant rock dry-out

Liquid saturation history at drift wall for drift emplacement for an APD of 57 kW/acre, 30-yr-old fuel, and a recharge flux of 0.0 mm/yr



ES-TB-24 (10-7-91) IL

Near-field temperatures are relatively insensitive to the bulk permeability of the fractured host rock

Temperature history at drift wall for drift emplacement for an APD of 57 kW/acre, 30-yr-old fuel, and a recharge flux of 0.0 mm/yr



A "hydrothermal umbrella" is established along each of the emplacement drifts due to condensate being shed off of the sides of the boiling zone



The shedding of condensate between emplacement drifts will continue until the boiling zones coalesce approximately 80 years after emplacement

Dimensionless liquid saturation for 30-yr-old fuel, an APD of 57 kW/acre, a drift spacing of 38.4 m, and a recharge flux of 0.0 mm/yr



After 1000 years, boiling has resulted in a 100-m-thick dry-out zone, surrounded by a condensation zone, with condensation drainage extending to the water table

Dimensionless liquid saturation for 30-yr-old fuel, an APD of 57 kW/acre, a drift spacing of 38.4 m, and a recharge flux of 0.0 mm/yr



ES-TB-3 (9-21-91) IL

Although boiling ceased after 1800 years, most of the repository remains dry 5000 years after emplacement

Dimensionless liquid saturation for 30-yr-old fuel, an APD of 57 kW/acre, a drift spacing of 38.4 m, and a recharge flux of 0.0 mm/yr



Temperature profiles as a function of thermal load

- Thermal disturbance reaches ground surface and water table within 300 years
- For given fuel age, temperature rise is linear in APD
- Repository temperatures are uniform within the inner two-thirds of repository area
- The emplacement drift-scale model (which accounts for local thermal load distribution) predicts temperatures similar to those in the inner two-thirds of the repository-scale model (which averages the thermal load)

ES-TB-5 (10-1-91) PM

Temperature profile is flattened at boiling zone (~ 96°C) and the temperature disturbance reaches ground surface 300 years after emplacement

Temperature profile along repository centerline for 30-year-old fuel, an APD of 57 kW/acre, and a recharge flux of 0.000 mm/yr



For a given age fuel, temperature rise is proportional to APD

Temperature history at repository center for 30-yr-old fuel and a recharge flux of 0.0 mm/yr



ES-TB-17 (9-14-91) IL

Repository temperatures are uniform within the inner two-thirds of repository



Radial temperature profile at repository horizon for 30-year-old fuel, and an APD of 57 kW/acre, and a recharge flux of 0.0 mm/yr

Impact of hydrothermal flow on temperature field

- For 30-year-old fuel and APDs up to 100 kW/acre, heat flow around the repository is dominated by heat conduction
- Temperatures in the vicinity of the waste packages decrease modestly with increasing recharge flux
- Boiling results in lower temperatures in the vicinity of the waste packages
- Heat conduction models yield
 - conservatively high temperatures in the vicinity of the waste packages
 - conservatively low temperatures with respect to the extent of the boiling zone
- Hydrothermal models predict higher temperatures in the Calico Hills units (CHnv and CHnz)

ES-TB-6 (10-1-01) PM



ES-TB-26 (9-23-91) IL
Ratio of heat conduction flux to total heat flux along repository centerline for 30-year-old fuel, an APD of 57 kW/acre, and a recharge flux of 0.0 mm/yr at t = 1000 yr



ES-TB-27 (10-7-91) IL

The heat conduction model yields conservatively high temperatures near the waste packages and conservatively low temperatures with respect to the extent of boiling

Temperature profile along repository centerline for 30-yr-old fuel, and APD of 57 kW/acre predicted by the hydrothermal and heat conduction models at t = 100 yr



ES-TB-24 (9-23-01) IL

Impact of thermal load on repository performance

- The threshold for significant rock dry-out benefits occurs between 36 and 57 kW/acre for 30-yr-old fuel
- For low-to-medium APD's (20 to 40 kW/acre for 30-yr-old fuel) performance considerations remain with no dry-out benefits
- Substantial boiling and dry-out benefits occur for high APD's
 - Dry steam boiling conditions persist at the waste package for thousands of years
 - Rock dry-out benefits remain thousands of years after boiling ceases
- For drift emplacement, substantial dry-out benefits are obtained with minimal impact on waste package temperatures
- Even high APD's result in minimal temperature disturbance at ground surface
- Boiling conditions and rock dry-out greatly enhance fracture flow attenuation

For 30-yr-old fuel, the threshold APD for significant dry-out by boiling lies between 36 and 57 kW/acre

Dry-out volume of liquid water vs. time for 30-yr-old fuel, and a recharge flux of 0.0 mm/yr



For a given APD, dry-out benefits can be substantially increased Using older age fuel Dry-out volume of liquid water vs. time for an APD of 57 kW/acre, and a recharge flux of 0.0 mm/yr



ES-TB-20 (9-21-91) IL

For 30-yr-old fuel, the threshold APD for significant dry-out by boiling lies between 36 and 57 kW/acre

Liquid saturation history at drift wall for drift emplacement for 30-year-old fuel and a recharge flux of 0.0 mm/yr



Although boiling ceases after 10,000 yr, the re-wetting of the dry-out zone to ambient saturation conditions takes over 200,000 yr; during re-wetting, matrix-dominated flow will be directed towards the repository

Dimensionless liquid saturation contours for 60-yr-old fuel, an APD of 114 kW/acre, a repository area of 1747 acres, and a recharge flux of 0.0 mm/yr



Although boiling ceases after 10,000 yr, the re-wetting of the dry-out zone to ambient saturation conditions takes over 200,000 yr; during re-wetting, matrix-dominated flow will be directed towards the repository

Dimensionless liquid saturation contours for 60-yr-old fuel, an APD of 114 kW/acre, a repository area of 348 acres, and a recharge flux of 0.0 mm/yr



Although boiling ceases after 10,000 years, a large dry-out zone remains for over 100,000 years

Liquid saturation profiles along repository centerline for 60-yr-old fuel, an APD of 114 kW/acre, and a recharge flux of 0.0 mm/yr



ES-TB-Babc (12-7-01) IL

The duration of dry steam boiling conditions is relatively insensitive to repository size

Liquid saturation history for 60-yr-old fuel, an APD of 114 kW/acre and a recharge flux of 0.0 mm/yr



ES-TB-7ab (12-7-91) L

The duration of repository dry-out is relatively insensitive to repository size

Liquid saturation history for 60-yr-old fuel, an APD of 114 kW/acre and a recharge flux of 0.0 mm/yr



ES-TB-6ab (12-7-91) IL

Ground surface temperature effects

- For 30-year-old fuel and APDs up to 100 kW/acre, heat flux at the ground surface never exceeds 1.5 W/m²
 - Therefore, the temperature rise at the ground surface should never exceed 1°C

Above the repository horizon, the attenuation of fracture flow will be much greater for boiling conditions than for sub-boiling conditions



Impact of thermal load on hydrogeologic uncertainties

- For APD's as low as 20 kW/acre, the flow and transport properties of potential radionuclide pathways may be significantly altered
- The hydrologic performance of the repository is much less sensitive to hydrogeologic uncertainty at high APD's than at low APD's

Although boiling and dry-out benefits are negligible, condensation drainage extends all the way to the water table

Dimensionless liquid saturation for 30-year-old fuel, an APD of 20 kW/acre, and a recharge flux of 0.0 mm/yr



For a given fuel age, temperature rise at the top of the Calico Hills (CHnv) is proportional to APD

Temperature history at top of the CHnv, 60 m below the repository horizon for 30-yr-old fuel and a recharge flux of 0.0 mm/yr



ES-TB-15 (9-14-91) K

The spatial extent of the boiling front is relatively insensitive to a wide range in effective infiltration flux and initial saturation

Vertical temperature profiles along repository centerline for 30-yr-old fuel and an APD of 114 kW/acre



ES-TB-11ab (12-7-91) IL

Although boiling ceases after 5000 years, re-wetting of the repository to ambient saturation takes 100,000 years even for very high initial saturation

Liquid saturation profiles along repository centerline for 30-yr-old fuel, an APD of 114 kW/acre, and a recharge flux of 0.132 mm/yr



The duration of dry steam boiling conditions is relatively insensitive to a wide range in effective infiltration flux and initial saturation

Temperature history for 30-yr-old fuel and an APD of 114 kW/acre



ES-TB-10ab (12-7-91) IL

Key hydrogeologic/geochemistry uncertainty considerations

- Zeolitization of the vitric nonwelded CHnv even at low APD's
- Alteration of flow and transport properties of fracture pathways in the zeolitized nonwelded CHnz even at low APD's
 - Impact on performance may be significant for low-to-medium APD's
 - Impact on performance is much less significant for high APD's

Key hydrogeologic/geomechanical uncertainty considerations

- Thermally-induced macro-fracturing near openings
 - may result in additional preferential pathways
 - may also result in increased liquid-phase dispersion in fracture networks
- Thermally-induced micro-fracturing out to the boiling front
 - may increase matrix capillary diffusivity, enhancing the impact of matrix imbibition on fracture flow attenuation
- Both macro- and micro-fracturing may enhance rock dry-out rate due to boiling

Conclusions

Impact of repository-heat-driven hydrothermal flow on site suitability/characterization

- Repository-heat-driven flow of vapor and liquid in fractures will dominate the ambient hydrological system
- Unsaturated, fracturedtuff promotes rock dry-out by boiling and rapid condensate drainage
- Repository-heat-driven alteration of the properties of the natural barriers is possible even at low APD's
- Potentially negative impact of nonequilibrium fracture-matrix flow on site suitability is mitigated by boiling and rock dry-out effects
 - Nonequilibrium fracture-matrix flow is beneficial to condensate drainage and rock dry-out at higher APDs
- Site suitability findings may vary significantly depending on repository thermal loading

Conclusions

Impact of repository-heat-driven hydrothermal flow on total systems performance

- For higher APD's and older age fuel, boiling benefits persist for thousands of years; rock dry-out benefits persist for 100,000 years or longer
 - Promoting more favorable waste package conditions
 - Greatly enhancing fracture flow attenuation
 - Reversing the direction of matrix-dominated flow back towards the repository
- Duration of boiling conditions and dry-out effects are dominated by thermal load and thermal properties
- Repository performance at higher APD's is much less sensitive to hydrogeologic variability/uncertainty
- Potential performance problems remain at lower APD's with minimal dry-out benefits

Conclusions

Impact of repository-heat-driven hydrothermal flow on ESF testing

- Critical performance issues cannot be entirely resolved by ambient property measurements
- Questions concerning the potential for repository-heat-driven alteration of the natural barriers can be addressed by heater tests at multiple hydrostratigraphic horizons
- Hypothesis testing of the dominance of heat conduction on repository boiling and dry-out performance can be addressed by relatively large-scale heater tests at the repository horizon
- Testing under boiling conditions provides a better experimental basis for model validation
- More likely to adequately resolve uncertainties associated with high APDs than with low APDs

Appendix

ES-TB-17 (6-15-91) PM

l (areas are	Normalized area function of ini e divided by are	a required tial APD a ea for the	l for repos Ind spent reference	itory as a fuel age SCP-CDF	R design)
	20	36	57	80	100

	20 kW/acre	36 kW/acre	57 kW/acre	80 kW/acre	100 kW/acre
10-yr-old fuel	2.85	1.57	1.00	0.71	0.57
30-yr-old fuel	1.81	1.00	0.64	0.45	0.36
60-yr-old fuel	1.14	0.63	0.40	0.29	0.23
100-yr-old fuel	0.73	0.40	0.25	0.18	0.15

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YUCCA MOUNTAIN SITE CHARACTERIZATION PROJECT REGULATORY AND SITE EVALUATION DIVISION

FY92 NOVEMBER STATUS SURFACE BASED TESTING PROGRAM

JR Dyer, Acting Division Director GD Roberson, Acting Deputy Division Director





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QUATERNARY FAULTING REGION					
Planned start date:	March 1992				
Status:	Awaiting study plan from USGS				
Concerns:	Study plan is not complete				
Solutions:	Ensure completion of study plan				

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UNSATURATED ZONE NATURAL INFILTRATION

Planned start date:

Status:

Concerns:

Solutions:

September 1991

Test planning and job packages completed

Drilling temporarily suspended until early January

Completed N54, N55

Penetration rate slower than expected

Improved bits are in use

Procurement of new rig in progress

UNSATURATED ZONE PERCOLATION UZ - 16 (VSP2)

Planned start date:

Status:

Concerns:

TBD - early CY92

TPOs input to TPP and JP requested by 12/24

Completion of technical and administrative prerequisites to continue drilling activities at the site

Solutions:

Task force assembled to concentrate on above concerns

ENVIRONMENTAL MONITORING JF - 3 Planned start date: Initiated site prep in November 1991 NTS rig on site and Status: work began on schedule Drilling progress Concerns: behind schedule Solutions: Planning to start two shifts per day

SOIL AND ROCK PROPERTIES RAMP BOREHOLES

Planned start date:

February 1992

Status:

Concerns:

Solutions:

Study plan currently being reviewed by NRC

Test planning and job packages being developed

Design of drill pad and access road

Combine management and technical reviews

Develop standard design products


GEOPHYSICAL REFLECTION SURVEY

Planned start date:

February 1992

Status:

Concerns:

Solutions:

Test Planning Package to be initiated 12/91

All data collection to be performed by subcontractor through RFP

Ensure compliance with environmental requirements

Effective coordination