# **4.0 EVALUATION OF DOSE**

# 4.1 DOSE FROM ENVIRONMENTAL MEASUREMENTS

Annual doses to maximum exposed individuals were estimated based on measured concentrations of radionuclides in 2002 MNS REMP samples. The primary purpose of estimating doses based on sample results is to allow comparison to effluent program dose estimates.

Doses based on sample results were calculated using the methodology and data presented in NRC Regulatory Guide 1.109. Measured radionuclide concentrations, averaged over the entire year for a specific radionuclide, indicator location and sample type, were used to calculate REMP-based doses. Where applicable, average background concentration at the corresponding control location was subtracted. Regulatory Guide 1.109 consumption rates for the maximum exposed individual were used in the calculations. When the guide listed "NO DATA" as the dose factor for a given radionuclide and organ, a dose factor of zero was assumed.

Maximum dose estimates (Highest Annual Mean Concentration) based on drinking water, fish, and shoreline sediment sample results are reported in Table 4.1-A. The individual critical population and pathway dose calculations are reported in Table 4.1-B.

REMP-based dose estimates are not reported for airborne radioiodine, airborne particulate, milk, or broadleaf vegetation sample types because no radionuclides other than naturally occurring K-40 and Be-7 were detected in the samples. Dose estimates are not reported for surface water because sampled surface water is not considered to be a potable drinking water source. Exposure estimates based upon REMP TLD results are discussed in Section 3.9.

The maximum environmental organ dose estimate for any single sample type (other than direct radiation from gaseous effluents) collected during 2002 was 5.84E-02 mrem to the maximum exposed child's liver, total body, thyroid, kidney, lung and GI-LLI from drinking water.

## 4.2 ESTIMATED DOSE FROM RELEASES

Throughout the year, dose estimates were calculated based on actual 2002 liquid and gaseous effluent release data. Effluent-based dose estimates were calculated using the RETDAS computer program which employs methodology and data presented in NRC Regulatory Guide 1.109. The 2002 MNS Annual Radioactive Effluent Release Report (reference 6.6) included calendar year dose estimates for the location with the highest individual organ dose from liquid and gaseous effluent releases. These reported doses are shown in Table 4.1-A along with the corresponding REMP-based dose estimates.

The effluent-based liquid release doses are summations of the dose contributions from the drinking water, fish, and shoreline pathways. The effluent-based gaseous release doses report noble gas exposure separately from iodine, particulate, and tritium exposure. For noble gas exposure there is no critical age group; as the maximum exposed individuals are assumed to receive the same doses, regardless of their age group. For iodine, particulate, and tritium exposure the effluent-based gaseous release doses are summations of the dose contributors from ground/plane, inhalation, milk and vegetation pathways.

# 4.3 <u>COMPARISON OF DOSES</u>

The environmental and effluent dose estimates given in Table 4.1-A agree reasonably well. The similarity of the doses indicate that the radioactivity levels in the environment do not differ significantly from those expected based on effluent measurements and modeling of the environmental exposure pathways. This indicates that effluent program dose estimates are both valid and reasonably conservative.

There are some differences in how effluent and environmental doses are calculated that affect the comparison. Doses calculated from environmental data are conservative because they are based on a mean that includes only samples with a net positive activity versus a mean that includes all sample results (i.e. zero results are not included in the mean). Also, airborne tritium is not measured in environmental samples but is used to calculate effluent doses.

In calculations based on liquid release pathways, drinking water and fish consumption were the predominant dose pathways based on environmental and effluent data. The maximum total organ dose based on 2002 environmental sample results was 1.09E-01 mrem to the child liver. The maximum total organ dose of 7.82E-02 mrem for liquid effluent-based estimates was to the child liver.

No environmental doses resulted from the gaseous pathway in 2002 because broadleaf vegetation, milk, and airborne radioiodines and particulates indicated no activity. The gaseous effluent dose is due to tritium on broadleaf vegetation.

Noble gas samples are not collected as part of the REMP, preventing an analogous comparison of effluent-based noble gas exposure estimates.

The doses calculated do not exceed the 40CFR190 dose commitment limits for members of the public. Doses to members of the public attributable to the operation of MNS are being maintained well within regulatory limits.

### TABLE 4.1-A

# MCGUIRE NUCLEAR STATION 2002 ENVIRONMENTAL AND EFFLUENT DOSE COMPARISON

### LIQUID RELEASE PATHWAY

Organ	Environmental or Effluent Data	Critical Age <sup>(1)</sup>	Critical Pathway <sup>(2)</sup>	Location	Maximum Dose <sup>(3)</sup> (mrem)
Skin	Environmental	Teen	Shoreline Sediment	130 (0 5 mi SW)	2 05E-04
Skin	Effluent	Teen	Shoreline Sediment	0.5 mi. ENE	5.22E-04
Bone	Environmental	Child	Fish	129 (0.5 mi ENE)	5.26E-02
Bone	Effluent	Child	Fish	0.5 mi. ENE	1.09E-02
Liver	Environmental	Child	Drinking Water	101 (3.3 mi E)	1.09E-01
Liver	Effluent	Child	Drinking Water	0.5 mi. ENE	7.82E-02
T. Body	Environmental	Adult	Drinking Water	101 (3.3 mi E)	7.93E-02
T. Body	Effluent	Child	Drinking Water	0.5 mi. ENE	6.94E-02
Thyroid	Environmental	Child	Drinking Water	101 (3.3 mi E)	5.91E-02
Thyroid	Effluent	Child	Drinking Water	0.5 mi. ENE	6.76E-02
Kidney	Environmental	Child	Drinking Water	101 (3.3 mi E)	7.55E-02
Kidney	Effluent	Child	Drinking Water	0.5 mi. ENE	7.10E-02
Lung	Environmental	Child	Drinking Water	101 (3.3 mi E)	6.50E-02
Lung	Effluent	Child	Drinking Water	0.5 mi. ENE	6.88E-02
GI-LLI	Environmental	Child	Drinking Water	101 (3.3 mi E)	5.95E-02
GI-LLI	Effluent	Child	Drinking Water	0.5 mi. ENE	6.97E-02

(1) Critical Age is the highest total dose (all pathways) to an age group.

(2) Critial Pathway is the highest individual dose within the identified Critical Age group.

(3) Maximum dose is a summation of the fish, drinking water and shoreline sediment pathways.

## GASEOUS RELEASE PATHWAY

### IODINE, PARTICULATE, and TRITIUM

Organ	Environmental or Effluent Data	Critical Age <sup>(1)</sup>	Critical Pathway <sup>(2)</sup>	Location	Maximum Dose <sup>(3)</sup> (mrem)
Skin	Environmental	-	-	-	0.00E+00
Skin	Effluent	All	Ground Plane	0.5 mi. E	7.70E-07
Bone	Environmental	-	-	-	0.00E+00
Bone	Effluent	Child	Vegetation	0.5 mi. E	8.87E-07
Liver	Environmental	-	-	-	0.00E+00
Liver	Effluent	Child	Vegetation	0.5 mi. E	7.42E-01
T. Body	Environmental	-	-	-	0.00E+00
T. Body	Effluent	Child	Vegetation	0.5 mi. E	7.42E-01
Thyroid	Environmental	-	-	-	0.00E+00
Thyroid	Effluent	Child	Vegetation	0.5 mi. E	7.42E-01
Kidney	Environmental	-	-	-	0.00E+00
Kidney	Effluent	Child	Vegetation	0.5 mi. E	7.42E-01
Lung	Environmental	-	-	-	0.00E+00
Lung	Effluent	Child	Vegetation	0.5 mi. E	7.42E-01
GI-LLI	Environmental	-	-	-	0.00E+00
GI-LLI	Effluent	Child	Vegetation	0.5 mi. E	7.42E-01

(1) Critical Age is the highest total dose (all pathways) to an age group.

(2) Critial Pathway is the highest individual dose within the identified Critical Age group.

(3) Maximum dose is a summation of the ground/plane, inhalation, milk and vegetation pathways.

### NOBLE GAS

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T	Air	<b>Environmental or</b>	Critical	Critical		Maximum Dose
[	Dose	Effluent Data	Age	Pathway	Location	(mrad)
~[						
	Beta	Environmental	-	-	-	Not Sampled
	Beta	Effluent	N/A	Noble Gas	0.5 mi.NNE	2.58E-02
	Gamma	Environmental	-	-	-	Not Sampled
~	Gamma	Effluent	N/A	Noble Gas	0.5 mi.NNE	7.11E-02

# TABLE 4.1-B

Maximum Individual Dose for 2002 based on Environmental Measurements (mrem) for McGuire Nuclear Station

Age	Sample Medium	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	· Skin
Infant	Airborne	0.00E+00							
	Drinking Water	0.00E+00	5.73E-02	5.73E-02	5.73E-02	5.73E-02	5.73E-02	5.73E-02	0.00E+00
	Milk	0.00E+00							
	TOTAL	0.00E+00	5.73E-02	5.73E-02	5.73E-02	5.73E-02	5.73E-02	5.73E-02	0.00E+00
Child	Airborne	0.00E+00							
	Drinking Water	0.00E+00	5.84E-02	5.84E-02	5.84E-02	5.84E-02	5.84E-02	5.84E-02	0.00E+00
	Milk	0.00E+00							
	Broadleaf Vegetation	0.00E+00							
	Fish	5.26E-02	5.10E-02	8.14E-03	7.13E-04	1.71E-02	6.61E-03	1.03E-03	0.00E+00
	Shoreline Sediment	3.67E-05	4.29E-05						
	TOTAL	5.26E-02	1.09E-01	6.66E-02	5.91E-02	7.55E-02	6.50E-02	5.95E-02	4.29E-05
Teen	Airborne	0.00E+00							
	Drinking Water	0.00E+00	3.05E-02	3.05E-02	3.05E-02	3.05E-02	3.05E-02	3.05E-02	0.00E+00
	Milk	0.00E+00							
	Broadleaf Vegetation	0.00E+00							
	Fish	4.18E-02	5.64E-02	2.02E-02	8.63E-04	1.98E-02	8.21E-03	1.65E-03	0.00E+00
	Shoreline Sediment	1.75E-04	2.05E-04						
	TOTAL	4.20E-02	8.71E-02	5.09E-02	3.15E-02	5.05E-02	3.89E-02	3.23E-02	2.05E-04
Adult	Airborne	0.00E+00							
	Drinking Water	0.00E+00	4.32E-02	4.32E-02	4.32E-02	4.32E-02	4.32E-02	4.32E-02	0.00E+00
	Milk	0.00E+00							
	<b>Broadleaf Vegetation</b>	0.00E+00							
	Fish	3.90E-02	5.45E-02	3.61E-02	1.12E-03	1.92E-02	7.14E-03	2.15E-03	0.00E+00
	Shoreline Sediment	3.14E-05	3.67E-05						
	TOTAL		9.77E-02	7.93E-02	4.44E-02	6.24E-02	5.04E-02	4.54E-02	3.67E-05

Note: Dose tables are provided for sample media displaying positive nuclide occurrence.

# McGuire Nuclear Station Dose from Drinking Water Pathway for 2002 Data Maximum Exposed Infant

**Highest Annual** 

#### Infant Dose from Drinking Water Pathway (mrem) = Usage (l) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year) = 330 1

								Net N	fean							
				Ingestion	n Dose Fa	<u>actor</u>		<u>Concen</u>	tration				Dose (m	<u>rem)</u>		
Radionuclide	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator Location	Water (pCi/l)	Bone	Liver	T. Body	Thyrold	Kidney	Lung	GI-LLI
Mn-54	NO DATA	1.99E-05	4.51E-06	NO DATA	4.41E-06	NO DATA	7.31E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	3.60E-06	8.98E-06	NO DATA	NO DATA	NO DATA	8.97E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	3.08E-05	5.38E-05	2.12E-05	NO DATA	NO DATA	1.59E-05	2.57E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	1.08E-05	2.55E-05	NO DATA	NO DATA	NO DATA	2.57E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	1.84E-05	6.31E-05	2.91E-05	NO DATA	3.06E-05	NO DATA	5.33E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	4.20E-08	1.73E-08	1.00E-08	NO DATA	1.24E-08	NO DATA	1.46E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	2.06E-07	5.02E-08	3.56E-08	NO DATA	5.41E-08	NO DATA	2.50E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	3.59E-05	4.23E-05	1.86E-05	1.39E-02	4.94E-05	NO DATA	1.51E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	3.77E-04	7.03E-04	7.10E-05	NO DATA	1.81E-04	7.42E-05	1.91E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	5.22E-04	6.11E-04	4.33E-05	NO DATA	1.64E-04	6.64E-05	1.91E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BaLa-140	1.71E-04	1.71E-07	8.81E-06	NO DATA	4.06E-08	1.05E-07	4.20E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
11-3	NO DATA	3.08E-07	3.08E-07	3.08E-07	3.08E-07	3.08E-07	3.08E-07	101	564.00	0.00E+00	5.73E-02	5.73E-02	5.73E-02	5.73E-02	5.73E-02	5.73E-02

Dose Commitment (mrem) =

0.00E+00 5.73E-02 5.73E-02 5.73E-02 5.73E-02 5.73E-02 5.73E-02

# McGuire Nuclear Station Dose from Drinking Water Pathway for 2002 Data Maximum Exposed Child

#### Child Dose from Drinking Water Pathway (mrem) = Usage (I) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year)= 510 1

								Net N	lean							
				Investio	n Dose F	'actor		Concen	tration				Dose (m	rem)		
				ingestio	II DOSC I	actor		Indicator	Water				Dose (m			
Radionuclide	Bone	Liver	T. Body	Thyroid	Kidney	Lung	<b>GI-LLI</b>	Location	(pCi/l)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	1.07E-05	2.85E-06	NO DATA	3.00E-06	NO DATA	8.98E-06	ALL	0.00	0.00E+00						
Co-58	NO DATA	1.80E-06	5.51E-06	ΝΟ DATA	NO DATA	NO DATA	1.05E-05	ALL	0.00	0.00E+00						
Fe-59	1.65E-05	2.67E-05	1.33E-05	NO DATA	NO DATA	7.74E-06	2.78E-05	ALL	0.00	0.00E+00						
C0-60	NO DATA	5.29E-06	1.56E-05	NO DATA	NO DATA	NO DATA	2.93E-05	ALL	0.00	0.00E+00						
Zn-65	1.37E-05	3.65E-05	2.27E-05	NO DATA	2.30E-05	ΝΟ DATA	6.41E-06	ALL	0.00	0.00E+00						
Nb-95	2.25E-08	8.76E-09	6.26E-09	NO DATA	8.23E-09	NO DATA	1.62E-05	ALL	0.00	0.00E+00						
Zr-95	1.16E-07	2.55E-08	2.27E-08	NO DATA	3.65E-08	NO DATA	2.66E-05	ALL	0.00	0.00E+00						
I-131	1.72E-05	1.73E-05	9.83E-06	5.72E-03	2.84E-05	NO DATA	1.54E-06	ALL	0.00	0.00E+00						
Cs-134	2.34E-04	3.84E-04	8.10E-05	NO DATA	1.19E-04	4.27E-05	2.07E-06	ALL	0.00	0.00E+00						
Cs-137	3.27E-04	3.13E-04	4.62E-05	NO DATA	1.02E-04	3.67E-05	1.96E-06	ALL	0.00	0.00E+00						
BaLa-140	8.31E-05	7.28E-08	4.85E-06	NO DATA	2.37E-08	4.34E-08	4.21E-05	ALL	0.00	0.00E+00						
н-з	NO DATA	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	101	564.00	0.00E+00	5.84E-02	5.84E-02	5.84E-02	5.84E-02	5.84E-02	5.84E-02

Highest Annual

0.00E+00 5.84E-02 5.84E-02 5.84E-02 5.84E-02 5.84E-02 5.84E-02

Dose Commitment (mrem) =

McGuire Nuclear Station Dose from Fish Pathway for 2002 Data Maximum Exposed Child

Child Dose from Fish Pathway (mrem) = Usage (kg) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/kg) H-3 Concentration in Fish = Surface Water pCi/l x Bioaccumulation Factor 0.9 pCi/kg per pCi/l = 565 pCi/l x 0.9 = 509 pCi/kg Usage (intake in one year) = 6.9 kg

								Highest	Annual							
								Net N	lean							
				Ingestion	n Dose Fa	actor		Concen	tration				Dose (m)	<u>rem)</u>		
						_		Indicator	Fish							
Radionuclide	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Location	(pCi/kg)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	1.07E-05	2.85E-06	NO DATA	3.00E-06	NO DATA	8.98E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	1.80E-06	5.51E-06	NO DATA	NO DATA	NO DATA	1.05E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	1.65E-05	2.67E-05	1.33E-05	NO DATA	NO DATA	7.74E-06	2.78E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
C0-60	NO DATA	5.29E-06	1.56E-05	NO DATA	NO DATA	NO DATA	2.93E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	1.37E-05	3.65E-05	2.27E-05	NO DATA	2.30E-05	NO DATA	6.41E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	2.34E-04	3.84E-04	8.10E-05	NO DATA	1.19E-04	4.27E-05	2.07E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	3.27E-04	3.13E-04	4.62E-05	NO DATA	1.02E-04	3.67E-05	1.96E-06	129	23.30	5.26E-02	5.03E-02	7.43E-03	0.00E+00	1.64E-02	5.90E-03	3.15E-04
Н-3	NO DATA	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	2.03E-07	128	509.00	0.00E+00	7.13E-04	7.13E-04	7.13E-04	7.13E-04	7.13E-04	7.13E-04

Dose Commitment (mrem) = 5.26E-02 5.10E-02 8.14E-03 7.13E-04 1.71E-02 6.61E-03 1.03E-03

# McGuire Nuclear Station Dose from Shoreline Sediment Pathway for 2002 Data Maximum Exposed Child

Shoreline Recreation =	14	hr (in one year)
Shore Width Factor =	0.3	(lake shore - location 129)
Shore Width Factor =	0.2	(river shoreline - location 130)
Sediment Surface Mass =	40	kg/m <sup>2</sup>

Child Dose from Shoreline Sediment Pathway (mrem) = Shoreline Recreation (hr) x External Dose Factor (mrem/hr per pCi/m<sup>2</sup>) x Shore Width Factor x Sediment Surface Mass (kg/n<sup>2</sup>) x Sediment Concentration (pCi/kg)

External <u>on Con</u>	Dose Fact taminated	tor Standing <u>Ground</u>	Highest A <u>Mean Co</u> i	nnual Net ncentration	<u>Dose</u>				
Radionuclide	(mrem/hr T. Body	per pCi/m²) . Skin	Indicator Location	Sediment (pCi/kg)	(mi T. Body	rem) Skin			
Mn-54	5.80E-09	6.80E-09	130	22.40	1.46E-05	1.71E-05			
Co-58	7.00E-09	8.20E-09	ALL	0.00	0.00E+00	0.00E+00			
Co-60	1.70E-08	2.00E-08	ALL	0.00	0.00E+00	0.00E+00			
Cs-134	1.20E-08	1.40E-08	ALL	0.00	0.00E+00	0.00E+00			
Cs-137	4.20E-09	4.90E-09	130	47.00	2.21E-05	2.58E-05			
		Dose Commitm	ent (mrem) =		3.67E-05	4.29E-05			

# McGuire Nuclear Station Dose from Drinking Water Pathway for 2002 Data Maximum Exposed Teen

#### Teen Dose from Drinking Water Pathway (mrem) = Usage (I) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year)= 510 l

								IIighest .	Annual							
								Net N	lean							
				Ingestio	n Dose F	<u>'actor</u>		Concent	tration	ion Dose (mrem)						
								Indicator	Water							
Radionuclide	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Location	(рСИ)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	ΝΟ DATA	5.90E-06	1.17E-06	NO DATA	1.76E-06	NO DATA	1.21E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	9.72E-07	2.24E-06	NO DATA	NO DATA	ΝΟ DATA	1.34E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	5.87E-06	1.37E-05	5.29E-06	NO DATA	NO DATA	4.32E-06	3.24E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	2.81E-06	6.33E-06	NO DATA	NO DATA	ΝΟ DATA	3.66E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	5.76E-06	2.00E-05	9.33E-06	NO DATA	1.28E-05	NO DATA	8.47E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	8.22E-09	4.56E-09	2.51E-09	NO DATA	4.42E-09	NO DATA	1.95E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	4.12E-08	1.30E-08	8.94E-09	NO DATA	1.91E-08	NO DATA	3.00E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	5.85E-06	8.19E-06	4.40E-06	2.39E-03	1.41E-05	NO DATA	1.62E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	8.37E-05	1.97E-04	9.14E-05	NO DATA	6.26E-05	2.39E-05	2.45E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	1.12E-04	1.49E-04	5.19E-05	NO DATA	5.07E-05	1.97E-05	2.12E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BaLa-140	2.84E-05	3.48E-08	1.83E-06	NO DATA	1.18E-08	2.34E-08	4.38E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
11-3	NO DATA	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	101	564.00	0.00E+00	3.05E-02	3.05E-02	3.05E-02	3.05E-02	3.05E-02	3.05E-02

Dose Commitment (mrem)=

0.00E+00 3.05E-02 3.05E-02 3.05E-02 3.05E-02 3.05E-02 3.05E-02

McGuire Nuclear Station Dose from Fish Pathway for 2002 Data Maximum Exposed Teen

Teen Dose from Fish Pathway (mrem) = Usage (kg) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/kg) H-3 Concentration in Fish = Surface Water pCi/l x Bioaccumulation Factor 0.9 pCi/kg per pCi/l = 565 pCi/l x 0.9 = 509 pCi/kg Usage (intake in one year) = 16 kg

								Highest	Annual							
				Ingestio	n Dose F	<u>actor</u>		Net N	Mean				Dose (m	<u>rem)</u>		
								<u>Concen</u>	<u>itration</u>							
Radionuclide	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Location	(pCi/kg)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	5.90E-06	1.17E-06	NO DATA	1.76E-06	NO DATA	1.21E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	9.72E-07	2.24E-06	ΝΟ DATA	NO DATA	NO DATA	1.34E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	5.87E-06	1.37E-05	5.29E-06	NO DATA	NO DATA	4.32E-06	3.24E-05	лll	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	2.81E-06	6.33E-06	NO DATA	NO DATA	NO DATA	3.66E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	5.76E-06	2.00E-05	9.33E-06	NO DATA	1.28E-05	NO DATA	8.47E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	8.37E-05	1.97E-04	9.14E-05	NO DATA	6.26E-05	2.39E-05	2.45E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	1.12E-04	1.49E-04	5.19E-05	NO DATA	5.07E-05	1.97E-05	2.12E-06	129	23.30	4.18E-02	5.55E-02	1.93E-02	0.00E+00	1.89E-02	7.34E-03	7.90E-04
Н-3	NO DATA	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	1.06E-07	128	509.00	0.00E+00	8.63E-04	8.63E-04	8.63E-04	8.63E-04	8.63E-04	8.63E-04
						Dose Comm	litment (mr	em) =		4.18E-02	5.64E-02	2.02E-02	8.63E-04	1.98E-02	8.21E-03	1.65E-03

# McGuire Nuclear Station Dose from Shoreline Sediment Pathway for 2002 Data Maximum Exposed Teen

Shoreline Recreation =	67	hr (in one year)
Shore Width Factor =	0.3	(lake shore - location 129)
Shore Width Factor =	0.2	(river shoreline - location 130)
Sediment Surface Mass =	40	kg/m²

Teen Dose from Shoreline Sediment Pathway (mrem) = Shoreline Recreation (hr) x External Dose Factor (mrem/hr per pCi/m<sup>2</sup>) x Shore Width Factor x Sediment Surface Mass (kg/nf) x Sediment Concentration (pCi/kg)

External Dose Factor Standing on Contaminated Ground			Highest Ar <u>Mean Conc</u>	nual Net entration	<u>Dose</u>			
(mrer	n/hr per pCi/i	m²)	Indicator	Sediment	(mr	em)		
Radionuclide	T. Body	Skin	Location	(pCi/kg)	T. Body	Skin		
Mn-54	5.80E-09	6.80E-09	130	22.40	6.96E-05	8.16E-05		
Co-58	7.00E-09	8.20E-09	ALL	0.00	0.00E+00	0.00E+00		
Co-60	1.70E-08	2.00E-08	ALL	0.00	0.00E+00	0.00E+00		
Cs-134	1.20E-08	1.40E-08	ALL	0.00	0.00E+00	0.00E+00		
Cs-137	4.20E-09	4.90E-09	130	47.00	1.06E-04	1.23E-04		
:	Dose Commit	ment (mrem)	=		1.75E-04	2.05E-04		

# McGuire Nuclear Station Dose from Drinking Water Pathway for 2002 Data Maximum Exposed Adult

**Highest Annual** 

#### Adult Dose from Drinking Water Pathway (mrem) = Usage (I) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/l)

Usage (intake in one year) = 730 l

								Net N	lean							
				Ingestion	n Dose Fa	actor		Concent	tration				Dose (m	<u>rem)</u>		
Radionuclide	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Indicator Location	Water (pCi/l)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	4.57E-06	8.72E-07	ΝΟ DATA	1.36E-06	NO DATA	1.40E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-58	NO DATA	7.45E-07	1.67E-06	NO DATA	NO DATA	NO DATA	1.51E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Fe-59	4.34E-06	1.02E-05	3.91E-06	NO DATA	NO DATA	2.85E-06	3.40E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Co-60	NO DATA	2.14E-06	4.72E-06	NO DATA	NO DATA	NO DATA	4.02E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zn-65	4.84E-06	1.54E-05	6.96E-06	NO DATA	1.03E-05	NO DATA	9.70E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Nb-95	6.22E-09	3.46E-09	1.86E-09	NO DATA	3.42E-09	NO DATA	2.10E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Zr-95	3.04E-08	9.75E-09	6.60E-09	NO DATA	1.53E-08	NO DATA	3.09E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
I-131	4.16E-06	5.95E-06	3.41E-06	1.95E-03	1.02E-05	NO DATA	1.57E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-134	6.22E-05	1.48E-04	1.21E-04	NO DATA	4.79E-05	1.59E-05	2.59E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Cs-137	7.97E-05	1.09E-04	7.14E-05	NO DATA	3.70E-05	1.23E-05	2.11E-06	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BaLa-140	2.03E-05	2.55E-08	1.33E-06	NO DATA	8.67E-09	1.46E-08	4.18E-05	ALL	0.00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
Н-3	NO DATA	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	101	564.00	0.00E+00	4.32E-02	4.32E-02	4.32E-02	4.32E-02	4.32E-02	4.32E-02

Dose Commitment (mrem) =

0.00E+00 4.32E-02 4.32E-02 4.32E-02 4.32E-02 4.32E-02 4.32E-02

McGuire Nuclear Station Dose from Fish Pathway for 2002 Data Maximum Exposed Adult

Adult Dose from Fish Pathway (mrem) = Usage (kg) x Dose Factor (mrem/pCi ingested) x Concentration (pCi/kg) H-3 Concentration in Fish = Surface Water pCi/l x Bioaccumulation Factor 0.9 pCi/kg per pCi/l = 565 pCi/l x 0.9 = 509 pCi/kg Usage (intake in one year) = 21 kg

#### Highest Annual Net Mean

			Ingestio	n Dose Fa	actor			<u>Concen</u>	<u>tration</u>				Dose (m	rem)		
Radionuclide	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI	Location	(pCi/kg)	Bone	Liver	T. Body	Thyroid	Kidney	Lung	GI-LLI
Mn-54	NO DATA	4.57E-06	8.72E-07	NO DATA	1.36E-06	NO DATA	1.40E-05	ALL	0.00	0.00E+00						
Co-58	NO DATA	7.45E-07	1.67E-06	ΝΟ DATA	ΝΟ DATA	ΝΟ DATA	1.51E-05	ALL	0.00	0.00E+00						
Fe-59	4.34E-06	1.02E-05	3.91E-06	NO DATA	ΝΟ DATA	2.85E-06	3.40E-05	ALL	0.00	0.00E+00						
Co-60	NO DATA	2.14E-06	4.72E-06	NO DATA	NO DATA	NO DATA	4.02E-05	ALL	0.00	0.00E+00						
Zn-65	4.84E-06	1.54E-05	6.96E-06	NO DATA	1.03E-05	NO DATA	9.70E-06	ALL	0.00	0.00E+00						
Cs-134	6.22E-05	1.48E-04	1.21E-04	NO DATA	4.79E-05	1.59E-05	2.59E-06	ALL	0.00	0.00E+00						
Cs-137	7.97E-05	1.09E-04	7.14E-05	NO DATA	3.70E-05	1.23E-05	2.11E-06	129	23.30	3.90E-02	5.33E-02	3.49E-02	0.00E+00	1.81E-02	6.02E-03	1.03E-03
н-з	NO DATA	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	1.05E-07	128	509.00	0.00E+00	1.12E-03	1.12E-03	1.12E-03	1.12E-03	1.12E-03	1.12E-03

 Dose Commitment (mrem) =
 3.90E-02
 5.45E-02
 3.61E-02
 1.12E-03
 1.92E-02
 7.14E-03
 2.15E-03

# McGuire Nuclear Station Dose from Shoreline Sediment Pathway for 2002 Data Maximum Exposed Adult

Shoreline Recreation =	12	hr (in one year)
Shore Width Factor =	0.3	(lake shore - location 129)
Shore Width Factor =	0.2	(river shoreline - location 130)
Sediment Surface Mass =	40	kg/m <sup>2</sup>

Adult Dose from Shoreline Sediment Pathway (mrem) = Shoreline Recreation (hr) x External Dose Factor (mrem/hr per pCi/m<sup>2</sup>) x Shore Width Factor x Sediment Surface Mass (kg/m<sup>2</sup>) x Sediment Concentration (pCi/kg)

External Dose Factor Standing on Contaminated Ground			Highest Ar <u>Mean Con</u>	nnual Net centration	Dose			
Radionuclide	(mrem/hr p T. Body	er pCi/m²) Skin	Indicator Location	Sediment (pCi/kg)	(mrem) T. Body Skir			
Mn-54	5.80E-09	6.80E-09	130	22.40	1.25E-05	1.46E-05		
Co-58	7.00E-09	8.20E-09	ALL	0.00	0.00E+00	0.00E+00		
Co-60	1.70E-08	2.00E-08	ALL	0.00	0.00E+00	0.00E+00		
Cs-134	1.20E-08	1.40E-08	ALL	0.00	0.00E+00	0.00E+00		
Cs-137	4.20E-09	4.90E-09	130	47.00	1.90E-05	2.21E-05		
	Dose Comn	nitment (mrer	n) =		3.14E-05	3.67E-05		

# **5.0 QUALITY ASSURANCE**

# 5.1 SAMPLE COLLECTION

EnRad Laboratories, Fisheries, and Aquatic Ecology performed the environmental sample collections as specified by approved sample collection procedures.

### 5.2 <u>SAMPLE ANALYSIS</u>

EnRad Laboratories performed the environmental sample analyses as specified by approved analysis procedures. EnRad Laboratories is located in Huntersville, North Carolina, at Duke Power Company's Environmental Center.



# 5.3 DOSIMETRY ANALYSIS

Duke Power Company's Environmental Center

The Radiation Dosimetry and Records group performed environmental dosimetry measurements as specified by approved dosimetry analysis procedures.

# 5.4 LABORATORY EQUIPMENT QUALITY ASSURANCE

## 5.4.1 DAILY QUALITY CONTROL

EnRad Laboratories has an internal quality assurance program which monitors each type of instrumentation for reliability and accuracy. Daily quality control checks ensure that instruments are in proper working order and these checks are used to monitor instrument performance.

## 5.4.2 CALIBRATION VERIFICATION

National Institute of Standards and Technology (NIST) standards that represent counting geometries are analyzed as unknowns at various frequencies ranging from weekly to annually to verify that efficiency calibrations are valid. The frequency is dependent upon instrument use and performance. Investigations are performed and documented should calibration verification data fall out of limits.

### 5.4.3 BATCH PROCESSING

Method quality control samples are analyzed with sample analyses that are processed in batches. These include gross beta in drinking water and all tritium analyses.

### 5.5 DUKE POWER INTERCOMPARISON PROGRAM

EnRad Laboratories participated in the Duke Power Nuclear Generation Department Intercomparison Program during 2002. Interlaboratory cross-check standards, including, Marinelli beakers, air filters, air cartridges, gross beta on smears, and tritium in water samples were analyzed at various times of the year by the four counting laboratories in Duke Power Company for this program. A summary of these Intercomparison Reports for 2002 is documented in Table 5.0-A.

## 5.6 DUKE POWER AUDITS

The McGuire Radiation Protection Section was audited by the Quality Assurance Group in January of 2002. There were no findings as a result of this 2002 audit.

EnRad Laboratories was audited by the Quality Assurance Group in June of 2002. Laboratory practices and procedures were reviewed. No significant problems were identified as a result of this 2002 audit.

## 5.7 U.S. NUCLEAR REGULATORY COMMISSION INSPECTIONS

The McGuire Nuclear Station Radiological Environmental Monitoring Program was not audited by the NRC in 2002. EnRad Laboratories was not audited by the NRC in 2002.

## 5.8 STATE OF NORTH CAROLINA INTERCOMPARISON PROGRAM

EnRad Laboratories routinely participates with the State of North Carolina Department of Environmental Health and Natural Resources (DEHNR) in an intercomparison program. EnRad Laboratories sends air, water, milk, vegetation, sediment, and fish samples which have been collected to the State of North Carolina Radiation Protection Section for intercomparison analysis.

## 5.9 TLD INTERCOMPARISON PROGRAM

### 5.9.1 NUCLEAR TECHNOLOGY SERVICES INTERCOMPARISON PROGRAM

Section 5 - Page 2

Radiation Dosimetry and Records participates in a quarterly TLD intercomparison program administered by Nuclear Technology Services, Inc.

of Roswell, GA. Nuclear Technology Services irradiates environmental dosimeters quarterly and sends them to the Radiation Dosimetry and Records group for analysis of the unknown estimated delivered exposure. A summary of the Nuclear Technology Services Intercomparison Report is documented in Table 5.0-B.

### 5.9.2 STATE OF NORTH CAROLINA INTERCOMPARISON PROGRAM

Radiation Dosimetry and Records routinely participates in a TLD intercomparison program. The State of North Carolina Radiation Protection Section irradiates environmental dosimeters and sends them to the Radiation Dosimetry and Records group for analysis of the unknown estimated delivered exposure. A summary of the State of North Carolina Environmental Dosimetry Intercomparison Report for 2002 is documented in Table 5.0-B.

### 5.9.3 INTERNAL CROSSCHECK (DUKE POWER)

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Radiation Dosimetry and Records participates in a quarterly TLD intracomparison program administered internally by the Dosimetry Lab. The Dosimetry Lab Staff irradiates environmental dosimeters quarterly and submits them for analysis of the unknown estimated delivered exposure. A summary of the Internal Cross Check (Duke Power) Result is documented in Table 5.0-B.

# TABLE 5.0-A DUKE POWER COMPANY INTERLABORATORY COMPARISON PROGRAM 2002 CROSS-CHECK RESULTS FOR ENRAD LABORATORIES

Cross-Check samples are normally analyzed a minimum of three times. A status of "3 Pass" indicates that all three analyses yielded results within the designated acceptance range. A status of "1 Pass" indicates that one analysis of the cross-check was performed.

Footnote explanations are included following this data table.

Gamma in Water 3.5 liters

-						
Reference	Sample I.D.	Nuclide	Acceptance	Reference	Mean Reported	Cross Check
Date		- -	Range	Value	Value	Status
			pCi/l	pCi/l	pCi/l	
3/15/2002	Q021GWSL	Cr-51	0.95 - 1.69 E5	1.27 E5	1.25 E5	3 Pass
		Mn-54	0.82 - 1.45 E5	1.09 E5	1.13 E5	3 Pass
		Fe-59	4.17 - 7.40 E4	5.56 E4	5.78 E4	3 Pass
		Co-60	5.75 - 10.20 E4	7.67 E4	7.68 E4	3 Pass
		Zn-65	0.80 - 1.43 E5	1.07 E5	1.12 E5	3 Pass
		Cs-134	4.46 - 7.91 E4	5.95 E4	5.36 E4	3 Pass
		Cs-137	0.97 - 1.72 E5	1.29 E5	1.24 E5	3 Pass
		Ce-139	0.00 - 0.00 E0	0.00 E0	1.82 E3	3 Pass <sup>(1)</sup>
		Ce-141	1.17 - 2.07 E5	1.56 E5	1.55 E5	3 Pass
5/14/2002	Q022GWR	Cr-51	6.77 - 12.01 E3	9.03 E3	9.32 E3	3 Pass
		Mn-54	1.38 - 2.45 E3	1.84 E3	1.98 E3	3 Pass
		Co-58	1.82 - 3.24 E3	2.43 E3	2.48 E3	3 Pass
		Fe-59	1.75 - 3.10 E3	2.33 E3	2.44 E3	3 Pass
		Co-60	1.71 - 3.04 E3	2.28 E3	2.38 E3	3 Pass
		Zn-65	2.66 - 4.71 E3	3.54 E3	3.73 E3	3 Pass
		Cs-134	1.68 - 2.98 E3	2.24 E3	2.10 E3	3 Pass
		Cs-137	1.24 - 2.20 E3	1.65 E3	1.60 E3	3 Pass
		Ce-141	2.32 - 4.11 E3	3.09 E3	3.13 E3	3 Pass
8/16/2002	Q023GWS	Cr-51	1.58 - 2.80 E5	2.10 E5	2.10 E5	3 Pass
		Mn-54	5.73 - 10.16 E4	7.64 E4	8.16 E4	3 Pass
		Co-58	4.47 - 7.93 E4	5.96 E4	6.20 E4	3 Pass
		Fe-59	4.76 - 8.44 E4	6.34 E4	6.85 E4	3 Pass
		Co-60	5.29 - 9.38 E4	7.06 E4	7.20 E4	3 Pass
		Zn-65	7.16 - 12.70 E4	9.55 E4	1.03 E5	3 Pass
1		Cs-134	4.79 - 8.49 E4	6.39 E4	5.78 E4	3 Pass
		Cs-137	4.48 - 7.95 E4	5.98 E4	5.76 E4	3 Pass
		Ce-141	1.00 - 1.78 E5	1.34 E5	1.38 E5	3 Pass
	·,					

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
11/19/2002	Q024GWR	Cr-51	2.08 - 3.70 E4	2.78 E4	3.09 E4	3 Pass
	l l	Mn-54	5.96 - 10.56 E3	7.94 E3	9.12 E3	3 Pass
		Co-57	0.00 - 0.00 E0	0.00 E0	6.65 E1	3 Pass <sup>(1)</sup>
		Co-58	6.54 - 11.60 E3	8.72 E3	9.79 E3	3 Pass
		Fe-59	3.72 - 6.60 E3	4.96 E3	5.95 E3	3 Pass
		Co-60	6.68 - 11.85 E3	8.91 E3	1.02 E4	3 Pass
		Zn-65	0.75 - 1.33 E4	1.00 E4	1.22 E4	3 Pass
		Cs-134	4.04 - 7.16 E3	5.38 E3	5.46 E3	3 Pass
		Cs-137	0.89 - 1.58 E4	1.19 E4	1.27 E4	3 Pass
		Ce-141	6.32 - 11.21 E3	8.43 E3	9.56 E3	3 Pass

### Gamma in Water 3.5 liters continued

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### Gamma in Water 1.0 liter

Reference	Sample I.D.	Nuclide	Acceptance	Reference	Mean Reported	Cross Check
Date			Range	Value	Value	Status
			pCi/l	pCi/l	pCi/l	
3/15/2002	Q021GWSL	Cr-51	0.95 - 1.69 E5	1.27 E5	1.28 E5	3 Pass
		Mn-54	0.82 - 1.45 E5	1.09 E5	1.16 E5	3 Pass
		Fe-59	4.17 - 7.40 E4	5.56 E4	5.85 E4	3 Pass
		Co-60	5.75 - 10.20 E4	7.67 E4	7.70 E4	3 Pass
		Zn-65	0.80 - 1.43 E5	1.07 E5	1.13 E5	3 Pass
		Cs-134	4.46 - 7.91 E4	5.95 E4	5.51 E4	3 Pass
1		Cs-137	0.97 - 1.72 E5	1.29 E5	1.27 E5	3 Pass
		Ce-139	0.00 - 0.00 E0	0.00 E0	1.84 E3	3 Pass <sup>(1)</sup>
		Ce-141	1.17 - 2.07 E5	1.56 E5	1.60 E5	3 Pass
5/14/2002	Q022GWR	Сг-51	6.77 - 12.01 E3	9.03 E3	9.07 E3	3 Pass
		Mn-54	1.38 - 2.45 E3	1.84 E3	2.00 E3	3 Pass
		Co-58	1.82 - 3.24 E3	2.43 E3	2.43 E3	3 Pass
		Fe-59	1.75 - 3.10 E3	2.33 E3	2.57 E3	3 Pass
		Co-60	1.71 - 3.04 E3	2.28 E3	2.44 E3	3 Pass
		Zn-65	2.66 - 4.71 E3	3.54 E3	3.86 E3	3 Pass
		Cs-134	1.68 - 2.98 E3	2.24 E3	1.98 E3	3 Pass
		Cs-137	1.24 - 2.20 E3	1.65 E3	1.57 E3	3 Pass
		Ce-141	2.32 - 4.11 E3	3.09 E3	3.13 E3	3 Pass
8/16/2002	Q023GWS	Cr-51	1.58 - 2.80 E5	2.10 E5	2.11 E5	3 Pass
	-	Mn-54	5.73 - 10.16 E4	7.64 E4	8.09 E4	3 Pass
		Co-58	4.47 - 7.93 E4	5.96 E4	6.17 E4	3 Pass
		Fe-59	4.76 - 8.44 E4	6.34 E4	6.67 E4	3 Pass
		Co-60	5.29 - 9.38 E4	7.06 E4	7.02 E4	3 Pass
		Zn-65	7.16 - 12.70 E4	9.55 E4	1.01 E5	3 Pass
		Cs-134	4.79 - 8.49 E4	6.39 E4	5.82 E4	3 Pass
		Cs-137	4.48 - 7.95 E4	5.98 E4	5.76 E4	3 Pass
	<u>                                     </u>	Ce-141	1.00 - 1.78 E5	1.34 E5	1.37 E5	3 Pass
		Cs-134 Cs-137 Ce-141	4.79 - 8.49 E4 4.48 - 7.95 E4 1.00 - 1.78 E5	6.39 E4 5.98 E4 1.34 E5	5.82 E4 5.76 E4 1.37 E5	

## Gamma in Water 1.0 liter continued

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Reference	Sample I.D.	Nuclide	Acceptance	Reference	Mean Reported	Cross Check
Date			Range	Value	Value	Status
			pCi/l	pCi/l	pCi/l	
11/19/2002	Q024GWR	Cr-51	2.08 - 3.70 E4	2.78 E4	3.15 E4	3 Pass
		Mn-54	5.96 - 10.56 E3	7.94 E3	9.12 E3	3 Pass
		Co-57	0.00 - 0.00 E0	0.00 E0	3.87 E1	3 Pass <sup>(1)</sup>
		Co-58	6.54 - 11.60 E3	8.72 E3	9.71 E3	3 Pass
		Fe-59	3.72 - 6.60 E3	4.96 E3	6.07 E3	3 Pass
		Co-60	6.68 - 11.85 E3	8.91 E3	1.01 E4	3 Pass
		Zn-65	0.75 - 1.33 E4	1.00 E4	1.22 E4	3 Pass
		Cs-134	4.04 - 7.16 E3	5.38 E3	5.24 E3	3 Pass
		Cs-137	0.89 - 1.58 E4	1.19 E4	1.29 E4	3 Pass
		Ce-141	6.32 - 11.21 E3	8.43 E3	9.53 E3	3 Pass
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### Gamma in Water 0.5 liter

Nuclide	Acceptance	Reference	Mean Reported	Cross Check
	Range	Value	Value	Status
	pCi/l	pCi/l	pCi/l	
Cr-51	0.95 - 1.69 E5	1.27 E5	1.25 E5	3 Pass
Mn-54	0.82 - 1.45 E5	1.09 E5	1.14 E5	3 Pass
Fe-59	4.17 - 7.40 E4	5.56 E4	5.91 E4	3 Pass
Co-60	5.75 - 10.20 E4	7.67 E4	7.65 E4	3 Pass
Zn-65	0.80 - 1.43 E5	1.07 E5	1.14 E5	3 Pass
Cs-134	4.46 - 7.91 E4	5.95 E4	5.06 E4	3 Pass
Cs-137	0.97 - 1.72 E5	1.29 E5	1.23 E5	3 Pass
Ce-139	0.00 - 0.00 E0	0.00 E0	1.76 E3	3 Pass <sup>(1)</sup>
Ce-141	1.17 - 2.07 E5	1.56 E5	1.54 E5	3 Pass
		·		
Cr-51	6.77 - 12.01 E3	9.03 E3	9.29 E3	3 Pass
Mn-54	1.38 - 2.45 E3	1.84 E3	1.96 E3	3 Pass
Co-58	1.82 - 3.24 E3	2.43 E3	2.47 E3	3 Pass
Fe-59	1.75 - 3.10 E3	2.33 E3	2.51 E3	3 Pass
Co-60	1.71 - 3.04 E3	2.28 E3	2.40 E3	3 Pass
Zn-65	2.66 - 4.71 E3	3.54 E3	3.76 E3	3 Pass
Cs-134	1.68 - 2.98 E3	2.24 E3	2.03 E3	3 Pass
Cs-137	1.24 - 2.20 E3	1.65 E3	1.62 E3	3 Pass
Ce-141	2.32 - 4.11 E3	3.09 E3	3.11 E3	3 Pass
			······	
Cr-51	1.58 - 2.80 E5	2.10 E5	2.10 E5	3 Pass
Mn-54	5.73 - 10.16 E4	7.64 E4	8.06 E4	3 Pass
Co-58	4.47 - 7.93 E4	5.96 E4	6.02 E4	3 Pass
Fe-59	4.76 - 8.44 E4	6.34 E4	6.77 E4	3 Pass
Co-60	5.29 - 9.38 E4	7.06 E4	7.09 E4	3 Pass
Zn-65	7.16 - 12.70 E4	9.55 E4	1.03 E5	3 Pass
Cs-134	4.79 - 8.49 E4	6.39 E4	5.48 E4	3 Pass
Cs-137	4.48 - 7.95 E4	5.98 E4	5.65 E4	3 Pass
Ce-141	1.00 - 1.78 E5	1.34 E5	1.34 E5	3 Pass
-	Cs-134 Cs-137 Ce-141	Cs-1344.79 - 8.49E4Cs-1374.48 - 7.95E4Ce-1411.00 - 1.78E5	Cs-1344.79 - 8.49E46.39E4Cs-1374.48 - 7.95E45.98E4Ce-1411.00 - 1.78E51.34E5	Cs-1344.79 - 8.49E46.39E45.48E4Cs-1374.48 - 7.95E45.98E45.65E4Ce-1411.00 - 1.78E51.34E51.34E5

## Gamma in Water 0.5 liter continued

Reference	Sample I.D.	Nuclide	Acceptance	Reference	Mean Reported	Cross Check
Date			Range	Value	Value	Status
			pCi/l	pCi/l	pCi/l	
11/19/2002	Q024GWR	Cr-51	2.08 - 3.70 E4	2.78 E4	3.10 E4	3 Pass
		Mn-54	5.96 - 10.56 E3	7.94 E3	8.98 E3	3 Pass
		Co-57	0.00 - 0.00 E0	0.00 E0	6.61 E1	3 Pass <sup>(1)</sup>
		Co-58	6.54 - 11.60 E3	8.72 E3	9.64 E3	3 Pass
		Fe-59	3.72 - 6.60 E3	4.96 E3	5.92 E3	3 Pass
		Co-60	6.68 - 11.85 E3	8.91 E3	9.97 E3	3 Pass
		Zn-65	0.75 - 1.33 E4	1.00 E4	1.19 E4	3 Pass
		Cs-134	4.04 - 7.16 E3	5.38 E3	5.08 E3	3 Pass
		Cs-137	0.89 - 1.58 E4	1.19 E4	1.24 E4	3 Pass
		Ce-141	6.32 - 11.21 E3	8.43 E3	9.15 E3	3 Pass

### Gamma in Filter

Reference	Sample I.D.	Nuclide	Acceptance	Reference	Mean Reported	Cross Check
Date			Range	Value	Value	Status
			pCi/total	pCi/total	pCi/total	
6/13/2002	E3197-37	Cr-51	1.05 - 2.42 E2	1.59 E2	1.79 E2	3 Pass
	[	Mn-54	4.80 - 8.51 E1	6.40 E1	6.72 El	3 Pass
	[	Co-58	5.03 - 8.91 E1	6.70 E1	6.78 E1	3 Pass
	[	Fe-59	3.89 - 7.51 El	5.40 E1	6.46 E1	1/3 High <sup>(2)</sup>
		Co-60	6.30 - 11.17 E1	8.40 E1	8.70 E1	3 Pass
		Zn-65	0.91 - 1.61 E2	1.21 E2	1.30 E2	3 Pass
		Cs-134	6.08 - 10.77 E1	8.10 E1	7.99 E1	3 Pass
		Cs-137	4.58 - 8.11 E1	6.10 E1	5.73 E1	3 Pass
		Ce-141	4.58 - 8.11 E1	6.10 E1	6.84 E1	3 Pass
12/5/2002	E3459-37	Cr-51	1.80 - 3.28 E2	2.43 E2	2.55 E2	3 Pass
		Mn-54	0.75 - 1.33 E2	1.00 E2	1.01 E2	3 Pass
		Co-58	7.35 - 13.03 E1	9.80 E1	9.61 E1	3 Pass
	[	Fe-59	3.83 - 6.78 E1	5.10 E1	5.71 El	3 Pass
	[	Co-60	0.87 - 1.54 E2	1.16 E2	1.19 E2	3 Pass
	[	Zn-65	0.95 - 1.68 E2	1.26 E2	1.28 E2	3 Pass
	[	Cs-134	5.25 - 9.31 E1	7.00 E1	6.50 E1	3 Pass
	[	Cs-137	1.16 - 2.06 E2	1.55 E2	1.45 E2	3 Pass
	[ [	Ce-141	5.85 - 10.37 E1	7.80 E1	8.05 E1	3 Pass

## Iodine in Water

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
5/28/2002	Q022LIW1	I-131	1.38 - 2.59 E0	1.89 E0	2.31 E0	3 Pass
5/28/2002	Q022LIW2	I-131	2.03 - 3.60 E1	2.71 E1	3.87 E1	3/3 High <sup>(3)</sup>
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5/28/2002	Q022LIW3	I-131	3.33 - 5.91 E2	4.44 E2	4.72 E2	3 Pass

### Iodine in Milk

Reference Date	Sample I.D.	Nuclide	Acceptance Range . pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
11/5/2002	Q024LIM1	I-131	2.80 - 4.97 E2	3.73 E2	3.52 E2	3 Pass
11/5/2002		T 121		0.00 50	0.00 50	
11/5/2002	QU24LIM2	1-131	0.00 - 0.00 E0	0.00 E0	0.00 E0	3 Pass

### Iodine Cartridge

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi	Reference Value pCi	Mean Reported Value pCi	Cross Check Status
3/8/2002	A15502-04	I-131	1.47 - 2.61 E5	1.96 E5	2.31 E5	3 Pass
6/13/2002	E3198-37	I-131	6.98 - 12.37 E1	9.30 E1	10.7 E1	3 Pass
8/9/2002	A16102-04	I-131	1.89 - 3.35 E5	2.52 E5	2.95 E5	3 Pass
12/5/2002	E3460-37	I-131	7.28 - 12.90 E1	9.70 E1	1.12 E2	3 Pass

### Beta in Water

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi/l	Reference Value pCi/l	Mean Reported Value pCi/l	Cross Check Status
6/13/2002	E3199-37	Beta	2.10 - 3.72 E2	2.80 E2	2.59 E2	3 Pass
6/13/2002	E3258-37	Beta	2.10 - 3.72 E2	2.80 E2	2.63 E2	3 Pass

### Beta Air Particulate

Reference Date	Sample I.D.	Nuclide	Acceptance Range pCi	Reference Value pCi	Mean Reported Value pCi	Cross Check Status
2/15/2002	A15504-04	Gross Beta	5.64 - 10.00 E3	7.52 E3	7.97 E3	3 Pass

Reference	Sample I.D.	Nuclide	Acceptance	Reference	Mean Reported	Cross Check
Date			Range	Value	Value	Status
			pCi/l	pCi/l	pCi/l	
3/15/2002	Q021TWSL1	H-3	0.93 - 1.66 E4	1.25 E4	1.26 E4	3 Pass
2/15/2002		11.2		0.00 E0		2 Dece
3/15/2002	QU211WSL2	н-э	0.00 - 0.00 E0	0.00 E0	0.00 E0	5 Pass
5/14/2002	Q022TWR1	H-3	4.25 - 8.20 E2	5.90 E2	5.60 E2	3 Pass
5/14/2002	Q022TWR2	H-3	0.00 - 0.00 E0	0.00 E0	0.00 E0	3 Pass
5/14/2002	Q022TWR3	H-3	2.06 - 3.66 E3	2.75 E3	2.84 E3	3 Pass
8/15/2002	Q023TWSL1	Н-3	0.00 - 0.00 E0	0.00 E0	0.00 E0	3 Pass
8/15/2002	0023TWSL2	H-3	2.94 - 5.21 E4	3.92 E4	3.37 E4	3 Pass
11/22/2002	Q024TWR1	H-3	0.00 - 0.00 E0	0.00 E0	0.00 E0	3 Pass
11/22/2002	Q024TWR2	H-3	1.65 - 2.93 E3	2.20 E3	1.89 E3	3 Pass

 Gamma in Water, Sample ID Q021GWSL, Reference Date 3/15/2002: 3.5 L Marinelli, 1.0 L Marinelli, 0.5 L Marinelli,

Ce-139 was observed in cross-checks and was attributed to a contaminant arriving with the source. The nuclide was determined to be present, but there was no reference activity applicable to the results.

Gamma in Water, Sample ID Q024GWR, Reference Date 11/19/2002: 3.5 L Marinelli, 1.0 L Marinelli, 0.5 L Marinelli

Co-57 was observed in cross-checks and was attributed to a contaminant arriving with the source. The nuclide was determined to be present, but there was no reference activity applicable to the results.

(2) Gamma in Filter, Sample ID E3197-37, Reference Date 6/13/2002

Three results for Fe-59 [1099.2 keV] were reported, with one being above acceptance limit. Calibration verifications for detector 4 (SN: 35-P31076A) were evaluated and yielded acceptable results. General Office PIP G-02-00278 was written to record investigative actions.

(3) Iodine in Water, Sample ID Q022LIW2, Reference Date 5/28/2002

Three results were reported for this cross-check, all of which were above the cross-check acceptance limit. Investigation yielded the most likely cause for the high results was failure to shake resin dish prior to analysis. General Office PIP G-02-00254 was written to record investigative actions.

# TABLE 5.0-B **2002 ENVIRONMENTAL DOSIMETER CROSS-CHECK RESULTS**

				NUCIE	ar Techn	ology Se	rvices				
1st Quart	ter 2002					2nd Quar	ter 2002				
TLD	Delivered	Reported	Bias	Pass/Fail		TLD	Delivered	Reported	Bias	Pass/Fail	
Number	(mrem)	(mrem)	(% diff)	Criteria	Pass/Fail	Number	(mrem)	(mrem)	(% diff)	Criteria	Pass/Fail
100141	88	88.1	0.11	<+/-15%	Pass	100340	99	93.9	-5.15	<+/-15%	Pass
100157	88	92.8	5.45	<+/-15%	Pass	101179	99	97.5	-1.52	<+/-15%	Pass
100215	88	91.1	3.52	<+/-15%	Pass	100349	99	96.3	-2.73	<+/-15%	Pass
100145	88	91.2	3.64	<+/-15%	Pass	100397	99	98.9	-0.10	<+/-15%	Pass
100463	88	90.7	3.07	<+/-15%	Pass	100022	99	98.5	-0.51	<+/-15%	Pass
Average Bias (B)		3.16				Avera	ge Bias (B)	-2.00			
Standard Deviation (S)		1.93				Standard De	eviation (S)	2.03			
Mea	Measure Performance  B +S		5.09	<15%	Pass	Mea	sure Perform	ance  B +S	4.03	<15%	Pass
3rd Quar	ter 2002					4th Quart	er 2002				
TLD	Delivered	Reported	Bias	Pass/Fail		TLD	Delivered	Reported	Bias	Pass/Fail	
Number	(mrem)	(mrem)	(% diff)	Criteria	Pass/Fail	Number	(mrem)	(mrem)	(% diff)	Criteria	Pass/Fail
100601	66	64.0	-3.03	<+/-15%	Pass	101275	70	63.5	-9.29	<+/-15%	Pass
100841	66	65.1	-1.36	<+/-15%	Pass	100478	70	64.2	-8.29	<+/-15%	Pass
100868	66	65.3	-1.06	<+/-15%	Pass	100503	70	63.1	-9.86	<+/-15%	Pass
100880	66	68.2	3.33	<+/-15%	Pass	101011	70	64.6	-7.71	<+/-15%	Pass
100989	66	65.4	-0.91	<+/-15%	Pass	100538	70	67.0	-4.29	<+/-15%	Pass
	Avera	ge Bias (B)	-0.61				Avera	ge Bias (B)	-7.89		
	Standard De	eviation (S)	2.36				Standard De	eviation (S)	2.18		
Mea	sure Perforn	nance  B +S	2.97	<15%	Pass	Mea	sure Perform	ance  B +S	10.06	<15%	Pass

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### State of North Carolina, Division of Radiation Protection

Spring 20	002					Fall 2002					
TLD	Delivered	Reported	Bias	Pass/Fail		TLD	Delivered	Reported	Bias	Pass/Fail	
Number	(mrem)	(mrem)	(% diff)	Criteria	Pass/Fail	Number	(mrem)	(mrem)	(% diff)	Criteria	Pass/Fail
100008	90	80.8	-10.22	<+/-15%	Pass	100777	151.6	127.3	-16.03	<+/-15%	Fail
100030	90	83.1	-7.67	<+/-15%	Pass	100704	151.6	127.9	-15.63	<+/-15%	Fail
100050	90	81.3	-9.67	<+/-15%	Pass	100740	151.6	130.9	-13.65	<+/-15%	Pass
100064	90	84.6	-6.00	<+/-15%	Pass	100680	151.6	132.4	-12.66	<+/-15%	Pass
100117	90	82.6	-8.22	<+/-15%	Pass	101020	151.6	123.7	-18.40	<+/-15%	Fail
100166	90	81.1	-9.89	<+/-15%	Pass	100747	151.6	131.3	-13.39	<+/-15%	Pass
100208	90	79.9	-11.22	<+/-15%	Pass	100759	151.6	130.6	-13.85	<+/-15%	Pass
101122	90	82.5	-8.33	<+/-15%	Pass	100103	151.6	128.6	-15.17	<+/-15%	Fail
Average Bias (B)		-8.90				Avera	ge Bias (B)	-14.85			
Standard Deviation (S)		1.67				Standard De	eviation (S)	1.85			
Mea	sure Perforn	nance  B +S	10.57	<15%	Pass	Meas	sure Perform	nance  B +S	16.70	<15%	Fail <sup>(1)</sup>

lst Quart	er 2002					2nd Quar	ter 2002				
TLD	Delivered	Reported	Bias	Pass/Fail		TLD	Delivered	Reported	Bias	Pass/Fail	
Number	(mrem)	(mrem)	(% diff)	Criteria	Pass/Fail	Number	(mrem)	(mrem)	(% diff)	Criteria	Pass/Fa
101102	40	37.0	-7.50	<+/-15%	Pass	100103	33	31.5	-4.55	<+/-15%	Pass
100109	40	37.1	-7.25	<+/-15%	Pass	100740	33	31.6	-4.24	<+/-15%	Pass
101122	40	38.4	-4.00	<+/-15%	Pass	100747	33	31.9	-3.33	<+/-15%	Pass
100268	40	37.1	-7.25	<+/-15%	Pass	100770	33	31.3	-5.15	<+/-15%	Pass
100807	40	39.8	-0.50	<+/-15%	Pass	100772	33	32.4	-1.82	<+/-15%	Pass
100810	40	37.7	-5.75	<+/-15%	Pass	101017	33	31.7	-3.94	<+/-15%	Pass
100169	40	37.3	-6.75	<+/-15%	Pass	101020	33	30.8	-6.67	<+/-15%	Pass
100366	40	38.1	-4.75	<+/-15%	Pass	101025	33	32.3	-2.12	<+/-15%	Pass
100815	40	37.2	-7.00	<+/-15%	Pass	101035	33	31.3	-5.15	<+/-15%	Pass
100823	40	38.8	-3.00	<+/-15%	Pass	101036	33	32.3	-2.12	<+/-15%	Pass
	Avera	ge Bias (B)	-5.38				Avera	ge Bias (B)	-3.91		
	Standard De	eviation (S)	2.31				Standard De	viation (S)	1.57		
Mea	sure Perform	nance  B +S	7.68	<15%	Pass	Meas	sure Perform	nance  B +S	5.48	<15%	Pass
Brd Quar	ter 2002					4th Quart	er 2002				
TLD	Delivered	Reported	Bias	Pass/Fail		TLD	Delivered	Reported	Bias	Pass/Fail	
Number	(mrem)	(mrem)	(% diff)	Criteria	Pass/Fail	Number	(mrem)	(mrem)	(% diff)	Criteria	Pass/Fa
100792	106	100.8	-4.91	<+/-15%	Pass	100389	66	64.5	-2.27	<+/-15%	Pass
101274	106	100.0	-5.66	<+/-15%	Pass	101114	66	62.3	-5.61	<+/-15%	Pass
100314	106	100.0	-5.66	<+/-15%	Pass	100114	66	66.6	0.91	<+/-15%	Pass
100460	106	100.9	-4.81	<+/-15%	Pass	100649	66	63.0	-4.55	<+/-15%	Pass
100101	106	101.9	-3.87	<+/-15%	Pass	100469	66	65.2	-1.21	<+/-15%	Pass
100783	106	101.8	-3.96	<+/-15%	Pass	100763	66	62.0	-6.06	<+/-15%	Pass
100760	106	100.8	-4.91	<+/-15%	Pass	100304	66	62.8	-4.85	<+/-15%	Pass
101349	106	98.1	-7.45	<+/-15%	Pass	100068	66	67.2	1.82	<+/-15%	Pass
100794	106	101.1	-4.62	<+/-15%	Pass	100203	66	65.0	-1.52	<+/-15%	Pass
100012	106	100.1	-5.57	<+/-15%	Pass	100418	66	64.7	-1.97	<+/-15%	Pass
	Avera	ge Bias (B)	-5.14				Avera	ge Bias (B)	-2.53		
	Standard De	eviation (S)	1.03				Standard De	viation (S)	2.69		
Mea	sure Perforn	nance  B +S	6.17	<15%	Pass	Meas	sure Perform	ance  B +S	5.22	<15%	Pass

### Table 5.0-B Footnote Explanations

There was an unusual amount of time between anneal, irradiation and readout for the Fall 2002 Environmental Crosscheck with the State of NC Division of Radiation Protection. Normal times between irradiation and readout are typically less than 7 days. In the Fall 2002 crosscheck, due to the ice storm and holiday season, the time between irradiation and readout was 18 days. Fading correction was not applied for the report results; fading correction is typically NOT USED.

When fading correction was applied, the cross-checks were within acceptance criteria. Fading correction will be applied to future cross-checks.

Additional information is documented in PIP G-03-00021.

# 6.0 REFERENCES

- 6.1 McGuire Selected License Commitments
- 6.2 McGuire Technical Specifications

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- 6.3 McGuire Updated Final Safety Analysis Report
- 6.4 Duke Power Company McGuire Offsite Dose Calculation Manual
- 6.5 McGuire Annual Radiological Environmental Operating Report 1979 2001
- 6.6 McGuire Annual Radioactive Effluent Release Report 2002
- 6.7 Probability and Statistics in Engineering and Management Science, Hines and Montgomery, 1969, pages 287-293.
- 6.8 Practical Statistics for the Physical Sciences, Havilcek and Crain, 1988, pages 83-93.
- 6.9 Nuclear Regulatory Commission Regulatory Guide 1.109, Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purposes of Evaluating Compliance with 10CFR50, Appendix I.
- 6.10 EnRad Laboratories Operating Procedures
- 6.11 RETDAS, Radiological Effluent Tracking and Dose Assessment Software, Vertechs Version 3.5.0, Duke Power Revision # 3.0
- 6.12 NRC Integrated Inspection Report (no audit performed in 2002)
- 6.13 Duke Power Company EnRad Laboratory Charcoal Cartridge Study, performed 2001

# **APPENDIX** A

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# ENVIRONMENTAL SAMPLING & ANALYSIS PROCEDURES

# **APPENDIX A**

# ENVIRONMENTAL SAMPLING AND ANALYSIS PROCEDURES

Adherence to established procedures for sampling and analysis of all environmental media at McGuire Nuclear Station was required to ensure compliance with Station Selected Licensee Commitments. Analytical procedures were employed to ensure that Selected Licensee Commitments detection capabilities were achieved.

Environmental sampling and analyses were performed by EnRad Laboratories, Dosimetry and Records, and Fisheries and Aquatic Ecology.

This appendix describes the environmental sampling frequencies and analysis procedures by media type.

### I. <u>CHANGE OF SAMPLING PROCEDURES</u>

No changes were made to the sampling procedures during 2002.

### II. DESCRIPTION OF ANALYSIS PROCEDURES

Gamma spectroscopy analyses are performed using high purity germanium gamma detectors and Canberra analytical software. Designated sample volumes are transferred to appropriate counting geometries and analyzed by gamma spectroscopy. Perishable samples such as fish and broadleaf vegetation are ground to achieve a homogeneous mixture. Soils and sediments are dried, sifted to remove foreign objects (rocks, clams, glass, etc.) then transferred to appropriate counting geometry. Ten percent of samples receiving gamma analysis are analyzed as duplicate analyses.

Low-level iodine analyses are performed by passing a designated sample aliquot through an ion exchange resin to remove and concentrate any iodine in the aqueous sample (milk). The resin is then dried and transferred to appropriate counting geometry and analyzed by gamma spectroscopy.

Tritium analyses are performed quarterly by using low-level environmental liquid scintillation analysis technique on a Packard 2550 liquid scintillation system. Tritium samples are batch processed with a tritium spike to verify instrument performance and sample preparation technique are acceptable.

Gross beta analysis is performed by concentrating a designated aliquot of sample precipitate and analyzing by gas-flow proportional counters. Samples are batch processed with a blank to ensure sample contamination has not occurred.

### III. CHANGE OF ANALYSIS PROCEDURES

No analysis procedures were changed during 2002.

### IV. SAMPLING AND ANALYSIS PROCEDURES

### A.1 AIRBORNE PARTICULATE AND RADIOIODINE

Airborne particulate and radioiodine samples at each of seven locations were composited continuously by means of continuous air samplers. Air particulates were collected on a particulate filter and radioiodines were collected in a charcoal cartridge situated behind the filter in the sampler. The samplers are designed to operate at a constant flow rate (in order to compensate for any filter loading) and are set to sample approximately 2 cubic feet per minute. Filters and cartridges were collected weekly. A weekly gross beta analysis was performed on each filter and a weekly gamma analysis was performed on each charcoal cartridge. Filters were segregated by location and a quarterly gamma analysis was performed on the filter composite. The filter and charcoal cartridge were analyzed independently. The continuous composite samples were collected from the locations listed below.

Location 120	=	Site Boundary (0.5 mi. NNE)
Location 121	=	Site Boundary (0.5 mi. NE)
Location 125	=	Site Boundary (0.4 mi. SW)
Location 133	=	Cornelius, NC (6.2 mi. NE)
Location 134	=	East Lincoln Junior High School (8.8 mi. WNW)
Location 192	=	Peninsula development (2.8 mi. NNE)
Location 195	=	Fishing Access Road (0.2 mi. N)

### A.2 DRINKING WATER

Biweekly composite samples were collected. A gross beta and gamma analysis was performed on monthly composites. Tritium analysis was performed on the quarterly composites. The composites were collected biweekly from the locations listed below.

Location 101	=	North Mecklenburg Water Supply (3.3 mi E)
Location 119	=	Mt. Holly Municipal Water Supply (7.4 mi. SSW)
Location 132	=	Charlotte Municipal Water Supply (11.2 mi. SSE)
Location 136	Ξ	Mooresville Municipal Water Supply (12.7 mi. NNE)
Location 194	=	East Lincoln Water Supply (6.7 mi. NNW)

### A.3 SURFACE WATER

Biweekly composite samples were collected. A gamma analysis was performed on the monthly composites. Tritium analysis was performed on the quarterly composites sample. The composites were collected biweekly from the locations listed below.

Location 128	=	Discharge Canal Bridge (0.4 mi. NE)
Location 131	Ξ	Cowans Ford Dam (0.6 mi. WNW)
Location 135	=	Plant Marshall Intake Canal (11.9 mi. N)

### A.4 <u>MILK</u>

Biweekly grab samples were collected at each dairy. A gamma and low-level Iodine-131 analysis was performed on each sample. The biweekly grab samples were collected from the locations listed below.

Location 138	Ξ	Henry Cook Dairy - COWS (3.1 mi. ESE)
Location 139	Ξ	William Cook Dairy - COWS (2.5 mi. E)
Location 141	=	Lynch Dairy - COWS (14.8 mi. WNW)

### A.5 BROADLEAF VEGETATION

Monthly samples were collected as available and a gamma analysis was performed on each sample. The samples were collected from the locations listed below.

Location 120	Ξ	Site Boundary (0.5 mi. NNE)
Location 125	Ξ	Site Boundary (0.4 mi. SW)
Location 134	=	East Lincoln Junior High School (8.8 mi. WNW)
Location 193	=	Site Boundary (0.2 mi. N)

### A.6 FOOD PRODUCTS

Samples were collected monthly when available during the harvest season and a gamma analysis was performed on each. The samples were collected at the location listed below.

Location 188 = Garden (2.8 mi N)

### A.7 <u>FISH</u>

Semiannual samples were collected and a gamma analysis was performed on the edible portions of each sample. Boney fish (i.e. Sunfish) were prepared whole

minus the head and tail portions. The samples were collected from the locations listed below.

Location 129	=	Discharge Canal Entrance to Lake Norman (0.5 mi.ENE)
Location 137	=	Pinnacle Access Area (12.0 mi. N)

#### A.8 <u>SHORELINE SEDIMENT</u>

Semiannual samples were collected and a gamma analysis was performed on each following the drying and removal of rocks and clams. The samples were collected from the locations listed below.

Location 129	=	Discharge Canal Entrance to Lake Norman (0.5 mi. ENE)
Location 130	=	Highway 73 Bridge Downstream (0.5 mi. SW)
Location 137	=	Pinnacle Access Area (12.0 mi. N)

### A.9 DIRECT GAMMA RADIATION (TLD)

Thermoluminescent dosimeters (TLD) were collected quarterly at forty-one locations. A gamma exposure rate was determined for each TLD. The TLDs were placed as indicated below.

- \* An inner ring of 14 TLDs at the site boundary, one in each available meteorological sector. The site boundary locations in the N and NNW sectors are over water; however, two special interest TLD's were placed in these sectors inside the site boundary in March, 1991.
- \* An outer ring of 16 TLDs, one in each meteorological sector in the 6 to 8 kilometer range.
- \* The remaining TLDs were placed in special interest areas such as population centers, residential areas, schools, and control locations.

TLD locations are listed in Table 2.1-B.

### A.10 ANNUAL LAND USE CENSUS

An annual Land Use Census was conducted to identify within a distance of 8 kilometers (5.0 miles) from the station, the nearest location from the site boundary in each of the sixteen meteorological sectors, the following:

- \* The Nearest Residence
- \* The Nearest Meat Animal

- \* The Nearest Garden greater than 50 square meters or 500 square feet
- \* The Nearest Milk-giving Animal (cow, goat, etc.)

This census was initiated on June 12, 2002 and completed on June 14, 2002. Results are shown in Table 3.10.

In the environmental program, the air deposition parameters (D/Q) are used to determine air, broadleaf vegetation and milk sampling locations. McGuire's sectors with the three highest values did not change in 2002.

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# **APPENDIX B**

# RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

# SUMMARY OF RESULTS

#### Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

Report Period: 01-JAN-2002 to 31-DEC-2002

Medium or Pathway Sampled	Type and Numt of	Type and Total Lower Number Limit of of Detection		All Indicator Locations	All Indicator LocationsLocation with Highest Annual Mean Name, Distance, DirectionMean (Fraction) RangeLocation CodeMean (Fraction) Range		Control Location	No.of Non- Routine Report Meas.
Unit of Measurement	Analyses Performed		(LLD)	Mean (Fraction) Range			Mean (Fraction) Range	
Air Particulate (pCi/m3)							134 (8.8 mi WNW)	
	BETA	364	1.00E-02	1.54E-2 (312/312)	121	1.57E-2 (52/52)	1.72E-2 (52/52)	0
			-	3.68E-3 - 2.38E-2	(0.5 mi NE)	4.56E-3 - 2.19E-2	7.48E-3 - 2.59E-2	
	CS-134	28	5.00E-02	0.00 (0/24)		0.00 (0/4)	0.00 (0/4)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-137	28	6.00E-02	0.00 (0/24)		0.00 (0/4)	0.00 (0/4)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	I-131	28	7.00E-02	0.00 (0/24)		0.00 (0/4)	0.00 (0/4)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

### Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

Report Period: 01-JAN-2002 to 31-DEC-2002

Medium or Pathway Sampled	Type a Tota Numl of	and al oer	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non- Routine Report Meas.
Unit of Measurement	Analy Perform	ses med	(LLD)	Mean (Fraction) Range	Location Code	Mean (Fraction) Range	Mean (Fraction) Range	
<del></del>	I		II		<u></u>		L	
Air Radioiodine							134	
(pCi/m3)							(8.8 mi WNW)	
	CS-134	364	5.00E-02	0.00 (0/312)		0.00 (0/52)	0.00 (0/52)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-137	364	6.00E-02	0.00 (0/312)		0.00 (0/52)	0.00 (0/52)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	I-131	364	7.00E-02	0.00 (0/312)		0.00 (0/52)	0.00 (0/52)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
				···· · · · · · · · · · · · · · · · · ·				

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction) Zero range indicates no detectable activity measurements

Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

Report Period: 01-JAN-2002 to 31-DEC-2002

Medium or Pathway Sampled	Type and Total Number of		Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non- Routine Report Meas.
Unit of Measurement	Analyses Performed		(LLD)	Mean (Fraction) Range	Location Code	Mean (Fraction) Range	Mean (Fraction) Range	
Drinking Water (pCi/liter)				. <u></u>			136 (12.7 mi NNE)	
	BAI 4-140	65	15	0.00(0/52)		0.00 (0/13)	0.00 (0/13)	0
	DALLA 140	05	15	0.00 - 0.00	- ··· ·	0.00 - 0.00	0.00 - 0.00	
	BETA	65	4	2.32 (52/52)	194	2.47 (13/13)	2.08 (13/13)	0
				1.15 - 3.23	(6.7 mi NNW)	1.92 - 2.91	1.07 - 2.65	
	CO-58	65	15	0.00 (0/52)	<u> </u>	0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CO-60	65	15	0.00 (0/52)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-134	65	15	0.00 (0/52)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00	<b>_</b>	0.00 - 0.00	0.00 - 0.00	
	CS-137	65	18	0.00 (0/52)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	FE-59	65	30	0.00 (0/52)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	H-3	20	2000	450 (12/16)	101	564 (4/4)	0.00 (0/4)	0
				279 - 643	(3.3 mi E)	509 - 643	0.00 - 0.00	
	I-131	65	15	0.00 (0/52)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	MN-54	65	15	0.00 (0/52)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	NB-95	65	15	0.00 (0/52)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	ZN-65	65	30	0.00 (0/52)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	ZR-95	65	15	0.00 (0/52)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction) Zero range indicates no detectable activity measurements

Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

Report Period: 01-JAN-2002 to 31-DEC-2002

								No of Nor
Medium or	Type and Total		Lower	All Indiantor	Location	with Highest	Control	NO. OF NON-
Pathway	Numbe	er	Limit of		Annı	ual Mean	Location	Routine
Sampled	of		Detection	Locations	Name, Dist	ance, Direction	Location	Report
								Meas.
Unit of	Analyse	es		Mean (Fraction)	Location	Mean (Fraction)	Mean (Fraction)	
leasurement	Perform	ed	(LLD)	Range	Code	Range	Range	
							<del></del>	
Surface Water							135	
(pCi/liter)							(11.0  m; N)	
(permer)							(11.9 mill)	
	BALA-140	39	15	0.00 (0/26)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CO-58	39	15	0.00 (0/26)		0.00 (0/13)	0.00 (0/13)	0
			· · ·	0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CO-60	39	15	0.00 (0/26)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-134	39	15	0.00 (0/26)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00	••••	0.00 - 0.00	0.00 - 0.00	
	CS-137	39	18	0.00 (0/26)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	FE-59	39	30	0.00 (0/26)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	H-3	12	2000	492 (8/8)	128	565 (4/4)	0.00 (0/4)	0
				408 - 736	(0.4 mi NE)	420 - 736	0.00 - 0.00	
	<u>I-131</u>	39	15	0.00 (0/26)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	MN-54	39	15	0.00 (0/26)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	NB-95	39	15	0.00 (0/26)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	ZN-65	39	30	0.00 (0/26)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	ZR-95	39	15	0.00 (0/26)		0.00 (0/13)	0.00 (0/13)	0
				0.00 - 0.00	-	0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction) Zero range indicates no detectable activity measurements

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### Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

#### Report Period: 01-JAN-2002 to 31-DEC-2002

Medium or Pathway Sampled	Type and Total Number of		Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non- Routine Report Meas.
Unit of Measurement	Analyses Performed		(LLD)	Mean (Fraction) Range	Location Code	Mean (Fraction) Range	Mean (Fraction) Range	
Milk (pCi/liter)							141 (14.8 mi WNW)	
	BALA-140	78	15	0.00 (0/52)		0.00 (0/26)	0.00 (0/26)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-134	78	15	0.00 (0/52)		0.00 (0/26)	0.00 (0/26)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-137	78	18	0.00 (0/52)		0.00 (0/26)	0.00 (0/26)	0
	·			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	I-131	78	15	0.00 (0/52)		0.00 (0/26)	0.00 (0/26)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	LLI-131	78	1	0.00 (0/52)		0.00 (0/26)	0.00 (0/26)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

### Facility: McGuire Nuclear Station

50-369,370 Docket No.

Location: Mecklenburg County, North Carolina

### Report Period: 01-JAN-2002 to 31-DEC-2002

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non- Routine Report Meas.
Unit of	Analyses		Mean (Fraction)	Location	Mean (Fraction)	Mean (Fraction)	
Measurement	Performed		Range	Code	Range	Range	
Broadleaf Vegetation (pCi/kg-wet)						134 (8.8 mi WNW)	
	CS-134 28	60	0.00 (0/21)		0.00 (0/7)	0.00 (0/7)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-137 28	80	0.00 (0/21)		0.00 (0/7)	0.00 (0/7)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	I-131 28	60	0.00 (0/21)		0.00 (0/7)	0.00 (0/7)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

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#### Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

### Report Period: 01-JAN-2002 to 31-DEC-2002

Medium or Pathway Sampled	Type and T Numbe of	lotal T	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non- Routine Report Meas.
Unit of Measurement	Analyse Perform	ed	(LLD)	Mean (Fraction) Range	Location Code	Location Mean (Fraction) Code Range		
Food Products (pCi/kg-wet)	L						No Control Location	•
	<u>CS-134</u>	11	60	0.00 (0/11)		0.00 (0/11)	0.00 (0/0)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-137	11	80	0.00 (0/11)		0.00 (0/11)	0.00 (0/0)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	I-131	11	60	0.00 (0/11)		0.00 (0/11)	0.00 (0/0)	0
				0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

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### Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

### Report Period: 01-JAN-2002 to 31-DEC-2002

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location Annu Name, Dist	Location with Highest Annual Mean Name, Distance, Direction		No. of Non- Routine Report Meas.
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean (Fraction) Range	Mean (Fraction) Range	
Fish (pCi/kg-wet)			<b></b>	L <u></u>		137 (12.0 mi N)	
	<u>CO-58</u> 12	. 130	0.00 (0/6)		0.00 (0/6)	0.00 (0/6)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CO-60 12	. 130	0.00 (0/6)		0.00 (0/6)	0.00 (0/6)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-134 12	. 130	0.00 (0/6)		0.00 (0/6)	0.00 (0/6)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-137 12	. 150	23.3 (2/6)	129	23.3 (2/6)	0.00 (0/6)	0
			22.2 - 24.4	(0.5 mi ENE)	22.2 - 24.4	0.00 - 0.00	
	FE-59 12	, 260	0.00 (0/6)		0.00 (0/6)	0.00 (0/6)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	MN-54 12	. 130	0.00 (0/6)		0.00 (0/6)	0.00 (0/6)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	ZN-65 12	260	0.00 (0/6)		0.00 (0/6)	0.00 (0/6)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
				<u> </u>			

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction) Zero range indicates no detectable activity measurements

#### Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

#### Report Period: 01-JAN-2002 to 31-DEC-2002

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location Annı Name, Dist	with Highest Jal Mean Jance, Direction	Control Location	No. of Non- Routine Report Meas.
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean (Fraction) Range	Mean (Fraction) Range	
Shoreline Sediment (pCi/kg-dry)						137 (12.0 mi N)	
([8;,	MN-54 6	0	22.4 (1/4)	130	22.4 (1/2)	0.00 (0/2)	
			22.4 - 22.4	(0.5 mi SW)	22.4 - 22.4	0.00 - 0.00	
	CO-58 6	0	0.00 (0/4)		0.00 (0/2)	0.00 (0/2)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CO-60 6	0	0.00 (0/4)		0.00 (0/2)	0.00 (0/2)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-134 6	150	0.00 (0/4)		0.00 (0/2)	0.00 (0/2)	0
			0.00 - 0.00		0.00 - 0.00	0.00 - 0.00	
	CS-137 6	180	90.0 (4/4)	130	159 (2/2)	112 (1/2)	0
			19.8 - 248	(0.5 mi SW)	71.5 - 248	112 - 112	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

If LLD is equal to 0.00, then the LLD is not required by Selected Licensee Commitments

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### Facility: McGuire Nuclear Station

Docket No. 50-369,370

Location: Mecklenburg County, North Carolina

### Report Period: 01-JAN-2002 to 31-DEC-2002

Medium or Pathway Sampled	Type and Total Number of	Lower Limit of Detection	All Indicator Locations	Location with Highest Annual Mean Name, Distance, Direction		Control Location	No. of Non- Routine Report Meas.
Unit of Measurement	Analyses Performed	(LLD)	Mean (Fraction) Range	Location Code	Mean (Fraction) Range	Mean (Fraction) Range	
Direct Radiation TLD (mR/standard quarter)						175 (15.5 mi WNW)	
	164	0.00E+00	16.9 (160/160)	173	24.3 (4/4)	23.7 (4/4)	0
			9.80 - 29.5	(8.4 mi NNW)	22.1 - 26.0	22.2 - 24.8	

Mean and range based upon detectable measurements only

Fraction of detectable measurements at specified locations is indicated in parentheses, (Fraction)

Zero range indicates no detectable activity measurements

# **APPENDIX C**

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# SAMPLING DEVIATIONS & UNAVAILABLE ANALYSES

# **APPENDIX C**

# MCGUIRE NUCLEAR STATION SAMPLING DEVIATIONS & UNAVAILABLE ANALYSES

DEVIATION & UNAVAILABLE REASON CODES					
BF	Blown Fuse	PO	Power Outage		
FZ	Sample Frozen	PS	Pump out of service / Undergoing Repair		
IW	Inclement Weather	SL	Sample Loss/Lost due to Lab Accident		
LC	Line Clog to Sampler	SM	Motor / Rotor Seized		
OT	Other	TF	Torn Filter		
PI	Power Interrupt	VN	Vandalism		
PM	Preventive Maintenance				

# C.1 SAMPLING DEVIATIONS

# Air Particulate and Air Radioiodines

Location	Scheduled Collection Dates	Actual Collection Dates	Reason Code	Corrective Action
121	6/19 - 6/26/2002	6/19 - 6/26/2002		Power outage to sampling equipment occurred during the composite period.
195	6/19 - 6/26/2002	6/19 - 6/26/2002		Power was cut due to a tree falling on the main service line to the McGuire site.
120	6/19 - 6/26/2002	6/19 – 6/26/2002	РО	Power was interrupted for approximately three hours.

## **Surface Water**

Location	Scheduled Collection Dates	Actual Collection Dates	Reason Code	Corrective Action
		cup 7/10/2020	DO	Power outage to sampling equipment occurred during the composite period. Power was cut due to a tree falling on the main service line to the McGuire site. Power was interrupted for approximately
128	6/12 - 7/10/2002	6/12 - 7/10/2002	PO	three hours.

# C.2 UNAVAILABLE ANALYSES

There were no unavailable analyses for the 2002 REMP.

# **APPENDIX D**

# **ANALYTICAL DEVIATIONS**

No analytical deviations were incurred for the 2002 Radiological Environmental Monitoring Program

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# **APPENDIX E**

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# RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM RESULTS

This appendix includes all of the sample analysis reports generated from each sample medium for 2002. Appendix E is located separately from this report and is permanently archived at Duke Power Company's Environmental Center radiological environmental master file, located at the McGuire Nuclear Station Site in Huntersville, North Carolina.