

May 15, 2003

Mr. J. A. Price
Site Vice President - Millstone
c/o Mr. David W. Dodson
Dominion Nuclear Connecticut, Inc.
Rope Ferry Road
Waterford, CT 06385

SUBJECT: MILLSTONE POWER STATION, UNIT NO. 1 - ISSUANCE OF AMENDMENT
RE: CHANGES TO SELECTED RADIOLOGICAL-RELATED TECHNICAL
SPECIFICATIONS (TAC NO. MB5077)

Dear Mr. Price:

The Commission has issued the enclosed Amendment No. 112 to Facility Operating License No. DPR-21 for the Millstone Power Station, Unit No. 1, in response to your application dated May 13, 2002. It should be noted that this application also requested an amendment to Millstone Power Station, Unit Nos. 2 (MP2) and 3 (MP3). The MP2 and MP3 amendments were issued by separate letter.

The amendment revises the Permanently Defueled Technical Specifications (PDTSS) changing selected radiological-related PDTSSs. These changes are due to the revision to Part 20 of Title 10 of the *Code of Federal Regulations*.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA by Brian Benney for/

Drew G. Holland, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-245

Enclosures: 1. Amendment No. 112 to DPR-21
2. Safety Evaluation

cc w/encls: See next page

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DATE	4/22/03	5/2/03

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DOMINION NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-245

MILLSTONE POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112
License No. DPR-21

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Dominion Nuclear Connecticut, Inc. (the licensee) dated May 13, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will be decommissioned in conformity with the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. DPR-21 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 112, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Stephen Dembek, Chief, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications

Date of Issuance: May 15, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 112

FACILITY OPERATING LICENSE NO. DPR-21

DOCKET NO. 50-245

Replace the following pages of the Appendix A Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

5.0-13

5.0-13

5.0-14

5.0-14

5.0-15

5.0-15

5.0-18

5.0-18

5.0-19

5.0-19

5.0-20

5.0-21

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. DPR-21

DOMINION NUCLEAR CONNECTICUT, INC.

MILLSTONE POWER STATION, UNIT NO. 1

DOCKET NO. 50-245

1.0 INTRODUCTION

By letter dated May 13, 2002, Dominion Nuclear Connecticut, Inc. (licensee or DNC), submitted a license amendment request to amend the Millstone Power Station, Unit No. 1 Permanently Defueled Technical Specifications (PDTs). The proposed amendment would make changes to the following PDTs:

- Occupational Radiation Exposure Report (PDTs 5.7.1)
- High Radiation Area (PDTs 5.8.1)
- Radioactive Effluent Controls Program (PDTs 5.6.4)

2.0 REGULATORY EVALUATION

Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36, "Technical Specifications," specifies the categories of items to be included in the plant technical specifications (TSs). These include safety limits, limiting safety system settings, limiting control settings, limiting conditions for operation, surveillance requirements (SRs), design features, and administrative controls. In addition, 10 CFR Part 20 and Appendix I of 10 CFR Part 50 provide requirements for radiological control.

Over the years, the NRC staff (staff) has published a number of guidance documents regarding TS requirements related to radiological control. These guidance documents provide such information as methods acceptable to the staff for implementing specific parts of the Commission's regulations. These include the following:

- Regulatory Guide (RG) 1.16, "Report of Operating Information — Appendix A Technical Specifications."
- RG 8.38, "Control of Access to High and Very High Radiation Areas in Nuclear Power Plants."

- Generic Letter (GL) 89-01, "Implementation of Programmatic Controls for Radiological Effluent Technical Specifications (RETS) in the Administrative Controls Section of the Technical Specifications and the Relocation of Procedural Details of RETS to the Offsite Dose Calculation Manual or to the Process Control Program."
- NUREG-1302, "Offsite Dose Calculation Manual Guidance: Standard Radiological Effluent Controls for Boiling Water Reactors (GL 89-01, Supplement No. 1)."
- GL 95-01, "Relocation of Selected Technical Specification Requirements Related to Instrumentation."

In addition to these guidance documents, the staff issued guidance to improve a plant's TSs. This guidance is located in:

- NUREG-1433, Revision 2, "Standard Technical Specifications, General Electric Plants, BWR/4."

3.0 TECHNICAL EVALUATION

3.1 PDTS Section 5.7.1, "Occupational Radiation Exposure Report"

The licensee has proposed the following changes:

1. Clarification of the requirement for the annual tabulation of personnel receiving exposures greater than 100 mrem (deep dose equivalent) by indicating it applies only to those for whom monitoring was required to be performed.
2. Revision of the annual tabulation of personnel from those "receiving exposures greater than 100 mrem/yr and their associated man-rem exposure" to apply to those "receiving an annual deep dose equivalent >100 mrem and the associated collective deep dose equivalent (reported in person-rem)."
3. Addition of a sentence clarifying that the annual tabulation supplements the requirements of 10 CFR 20.2206.
4. Change the term "pocket dosimeter" to "pocket ionization chamber."
5. State that dose assignments can be estimated using electronic dosimeters.
6. Change the term "whole body dose" to "deep dose equivalent" to reflect current 10 CFR Part 20 terminology.
7. Change the required submittal date from "March 1" to "April 30."

The proposed changes are in accordance with the guidance and methodology contained in RG 1.16, GL 89-01 and NUREG-1433, and therefore, satisfy the requirements of 10 CFR Part 20. Therefore, the staff considers the proposed changes are acceptable.

3.2 PDTS Sections 5.8.1, 5.8.2 and 5.8.3, "High Radiation Area"

The licensee has proposed to incorporate changes for consistency with the current 10 CFR Part 20 by adopting the wording of the "High Radiation Area" specification from the current revision (Revision 2) of NUREG-1433, so that each program will reflect identical wording of requirements.

The proposed changes are in accordance with the guidance and methodology contained in GL 89-01 and NUREG-1433, and therefore, satisfy the requirements of 10 CFR Part 20. Therefore, the staff considers the proposed changes are acceptable.

3.3 PDTS Section 5.6.4, "Radioactive Effluent Controls Program"

- a. The licensee has proposed to revise PDTS Section 5.6.4.b to clarify that the program includes "limitations on the concentrations of radioactive material released in liquid effluents to unrestricted areas, conforming to ten times the concentration values in Appendix B, Table 2, Column 2 to 10 CFR Part 20.1001-20.2402."
- b. The licensee has proposed to revise PDTS Section 5.6.4.g to clarify that the program includes "limitations on the dose rate resulting from radioactive material released in gaseous effluents from the site to areas at or beyond the site boundary."
- c. The licensee has proposed to revise PDTS Section 5.6.4.j to clarify that the program includes "limitations on the annual dose or dose commitment to any member of the public, beyond the site boundary, due to releases of radioactivity and to radiation from uranium fuel cycle sources."
- d. The licensee has proposed to add a new paragraph to PDTS Section 5.6.4.j to reflect that the 25 percent extension of the surveillance interval provisions of SR 3.0.2 are applicable to the Radiological Environmental Controls Program (RECP) surveillance frequency. The provisions of SR 3.0.3, which indicate that compliance with the requirements to declare a limiting condition for operation not met, may be delayed for up to 24 hours from the time of discovery (when time limits of the Action are less than 24 hours) to permit completion of the surveillance will also be included.

The provisions of SR 3.0.2 are applied to the RECP surveillance frequencies of PDTS 5.6.4.j to allow for scheduling flexibility. SR 3.0.2 permits a 25 percent extension of the interval specified in the frequency (31 days).

As applied to the 31-day frequency of PDTS 5.6.4.j, SR 3.0.3 would allow up to 31 days to complete the surveillance (dose calculation) if it is discovered that the surveillance was not performed within 38 days and 18 hours (the specified interval plus the 25 percent extension). Allowing 31 days to complete the cumulative dose and projected dose calculation for the current quarter/year is acceptable because it will have no effect on the outcome of the calculations and has no impact on the risk associated with plant operation. In addition, operating experience has shown that the calculated dose is usually well within limits. Thus, it is considered unlikely that a potential greater time interval between dose calculations will result in inadvertent effluent releases exceeding

the specified limits. PDTS 5.6.4.j requires the RECP, which is contained in the Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMODOCM), to include remedial measures established in the event dose limits are exceeded. Therefore, delaying implementation of these remedial measures for 31 days from the time of discovery of a missed dose calculation is acceptable. In the event the calculations are not completed within the time limits allowed by SR 3.0.2 and SR 3.0.3, these specifications direct the licensee to take the RECP remedial measures.

- e. The licensee has proposed to revise PDTS Section 5.6.4.e to clarify the wording to require projected dose contributions as directed by the methodology of the REMODOCM.

The proposed changes are in accordance with the guidance and methodology contained in GL 89-01 and NUREG-1433, and therefore, satisfy the requirements of 10 CFR Part 20. Therefore, the staff considers the proposed changes are acceptable.

3.4 Summary

The staff has reviewed the licensee's submittal and based on the review discussed above, the staff finds the proposed changes of the selected PDTSs to be acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Date: May 15, 2003

Millstone Power Station Unit 1

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