

**COMMISSION BRIEFING SLIDES/EXHIBITS**

**BRIEFING ON RESULTS OF  
AGENCY ACTION REVIEW MEETING**

**MAY 15, 2003**



# **AARM COMMISSION BRIEFING**

**William Travers, EDO**  
**R. William Borchardt, NRR**  
**Thomas Boyce, NRR**  
**Cynthia Carpenter, NRR**  
**Margaret Federline, NMSS**  
**James Dyer, Region III**

# **Results of the Agency Action Review Meeting**

- **Industry Trends Program**
- **ROP Self-Assessment Program**
- **Material facilities process improvements**
- **Davis-Besse update**
- **Plant discussions**

# **Elements of the Agency Action Review Meeting**

- **Conducted IAW Management Directive 8.14**
- **Review of agency actions:**
  - ▶ Individual plants per action matrix
  - ▶ Industry trends (SECY-03-0057)
  - ▶ ROP self-assessment (SECY-03-0062)
- **Material facility concerns, as applicable**

# **Assessment Process under the ROP**

- **End-of-cycle meetings - all plants**
- **EOC summary meetings - specific plants based on ROP action matrix column**
- **Annual assessment letters - all plants**
- **Annual public meetings - all plants**
- **Agency Action Review Meeting - specific plants based on ROP action matrix column**

# **Industry Trends**

- **Background**
- **Program purposes & role**
- **FY 2002 results**
- **Program development**

# **Background**

- **Improving industry trends contributed to decision to revise ROP**
- **Performance goal measure of “no statistically significant adverse industry trends in safety performance”**
- **Currently use indicators from AEOD**

# Purposes & Role

- **Means to confirm that operating reactor safety is being maintained**
- **By communicating performance, enhance stakeholder confidence**
- **Complements agency processes**
  - **Plant-specific oversight by ROP**
  - **Generic communications process**
  - **Generic safety issues process**

# **FY 2002 Results**

- **No statistically significant adverse industry trends in safety performance**
  - **AEOD indicators**
  - **ASP Program**
- **Insufficient data (<4 years) on ROP indicators for long term trending**
- **ASP data increase since 1997 - currently investigating IAW ITP process**

# **Program Development**

- **Additional operating experience**
  - ▶ **Davis-Besse lessons learned**
  - ▶ **Equipment reliability studies**
  - ▶ **Foreign operating experience**
- **Additional, more risk-informed indicators**
  - ▶ **Indicators for ROP cornerstones**
  - ▶ **Industry Initiating Event Performance Indicator (IIEPI)**

# **Program Development (cont)**

- **Risk-informed thresholds**
  - **SRM to develop thresholds**
  - **Focus on IIEPI and MSPI development**
- **Improved data collection & reporting**
  - **Consolidation of LER databases**
  - **Common reporting to NRC & INPO**
- **New performance goal measure**
  - **Plants in multiple/repetitive column  $\leq 5$**
  - **Reporting effective FY 2004**

# **ROP Self-Assessment**

- **IMC 0307- Self-Assessment Program to ensure ROP meets agency's performance goals and ROP program goals**
- **Diverse data sources for self-assessment**

# Overall Results

- **Successfully supported the agency's four performance goals and ROP goals**
- **Effective in monitoring plant activities and focusing resources**
- **Continued to improve ROP as a result of feedback and lessons learned**
- **Most self-assessment metrics were met**
- **Stakeholder perception on ROP is mixed**

# **Performance Indicator Program**

- **Significant activities/results**
  - **MSPI pilot program**
  - **PIs provide useful information**
  - **All self-assessment metrics met but one**
- **Challenges and planned actions**
  - **Complete and analyze results of MSPI pilot program**
  - **Barrier integrity and initiating events improvements**

# **Inspection Program**

- **Significant activities/results**
  - ▶ **Baseline Inspection Program completed**
  - ▶ **Completed review of all inspection procedures**
  - ▶ **All self-assessment metrics met**
- **Challenges and planned actions**
  - ▶ **Implement Davis-Besse task group recommendations**
  - ▶ **Revise physical protection inspection procedures**

# **Significance Determination Process**

- **Significant activities/results**
  - ▶ **SDP improvement initiative progress**
  - ▶ **Continued benchmarking phase 2 notebooks and development of SDPs**
  - ▶ **SDP task group & OIG recommendations**
  - ▶ **Four self-assessment metrics not met**
- **Challenges and planned actions**
  - ▶ **SDP improvement initiatives and task action plan**

# **Assessment Program**

- **Significant activities/results**
  - **Clarification of action matrix issues**
  - **First action matrix deviation**
  - **All self-assessment metrics met**
- **Challenges and planned actions**
  - **Public meeting frequency**
  - **Substantive cross-cutting issues**
  - **Enhance IMC 0350 guidance**

# **Self-Assessment Program**

- **Significant revision to IMC 0307**
  - **Annual review of baseline inspection program**
- **4 of 19 overall program metrics not met**

# **Resources / Resident Demographics**

- **Resource expenditure trends**
- **CY 2002 resource challenges and coping strategies**
- **Potential long-term improvement strategies**
- **Steady or improving overall demographic trends**

# **Self-Assessment Conclusions**

- **Successfully supported the agency's four performance goals and ROP goals**
- **Effective in monitoring plant activities and focusing resources**
- **Continued to improve ROP as a result of feedback and lessons learned**

# **NMSS Assessment Program Discussion - Background**

- **SECY 98-078, review process**
  - ▶ More formal performance review
  - ▶ Screening meetings with regions
  - ▶ Standardized evaluation templates
  - ▶ Lack of specific criteria

# **NMSS Assessment Program Discussion - Current Process**

- **SECY 02-0216 - selection process**
  - ▶ Criteria for selecting licensees
  - ▶ Annual report to the Commission
- **SECY 03-0600 - annual report**
  - ▶ Evolving process, existing data
  - ▶ Future reports: risk insights, NMP

# **Davis-Besse Informational Update**

- **Preliminary determination was that reactor head wastage had high safety significance (red)**
- **IMC 0350 oversight panel restart checklist used to manage issues necessary to address before restart consideration**
- **Decision to authorize restart will not be made until NRC senior managers are satisfied that the facility can be operated safely**

# **Davis-Besse Informational Update**

- **Planned oversight will remain enhanced should restart be authorized**
- **Public access and stakeholder involvement remain at heightened levels**
- **NRC budget implications of Davis-Besse oversight are significant**



# **PLANT DISCUSSION BRIEFING**

**William Travers, EDO**  
**William Kane, DEDR**  
**R. William Borchardt, NRR**  
**Hubert Miller, Region I**  
**Luis Reyes, Region II**  
**James Dyer, Region III**  
**Ellis Merschoff, Region IV**

# **Plant Discussion - Indian Point 2**

- **Background**
- **Inspection activities**
- **Current status**
- **Public interface**
- **Next steps**

# **Plant Discussion - Oconee**

- **Background**
- **Inspection activities**
- **Current status**
- **Public interface**
- **Next steps**

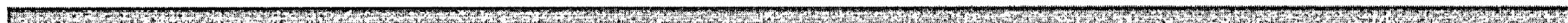
# **Plant Discussion - Point Beach**

- **Background**
- **Inspection activities**
- **Current status**
- **Public interface**
- **Next steps**

# **Plant Discussion - Cooper**

- **Background**
- **Inspection activities**
- **Current status**
- **Public interface**
- **Next steps**

# **BACKUP SLIDES**



## OVERALL RESULTS 2002 PERFORMANCE INDICATORS

9 Performance Indicators (White or Greater) Crossed Thresholds (By Unit)

<u>Cornerstone</u>	<u>White</u>	<u>Yellow</u>	<u>Red</u>
Initiating Events	4		
Mitigating Systems	4		
Barrier Integrity			
Emergency Preparedness	1		
Public Radiation Safety			
Occupational Radiation Safety			
Physical Protection			
Total	9	0	0

NOTE: 9 PIs greater-than-green in ROP-3 (CY2002) is fewer than the 9 PIs over 3 quarters in ROP-2 (shortened 9-month cycle) and 20 PIs in ROP-1. The ROP started in April 2000 with 24 PIs greater-than-green.

## OVERALL RESULTS 2002 INSPECTION FINDINGS

36 Inspection Findings (White or Greater) Processed by SERP (By Unit)

<u>Cornerstone</u>	<u>White</u>	<u>Yellow</u>	<u>Red</u>
Initiating Events			
Mitigating Systems	14	1	2
Barrier Integrity	1		
Emergency Preparedness	12		
Public Radiation Safety	5		
Occupational Radiation Safety			
Physical Protection		1	
Total	32	2	2

NOTE: 28 findings (by site) in ROP-3 is approximately the same rate as 19 findings (by site) over 3 quarters in ROP-2 (shortened 9-month cycle) and 22 findings in ROP-1

## OVERALL RESULTS 2002 PLANT PERFORMANCE SUMMARY

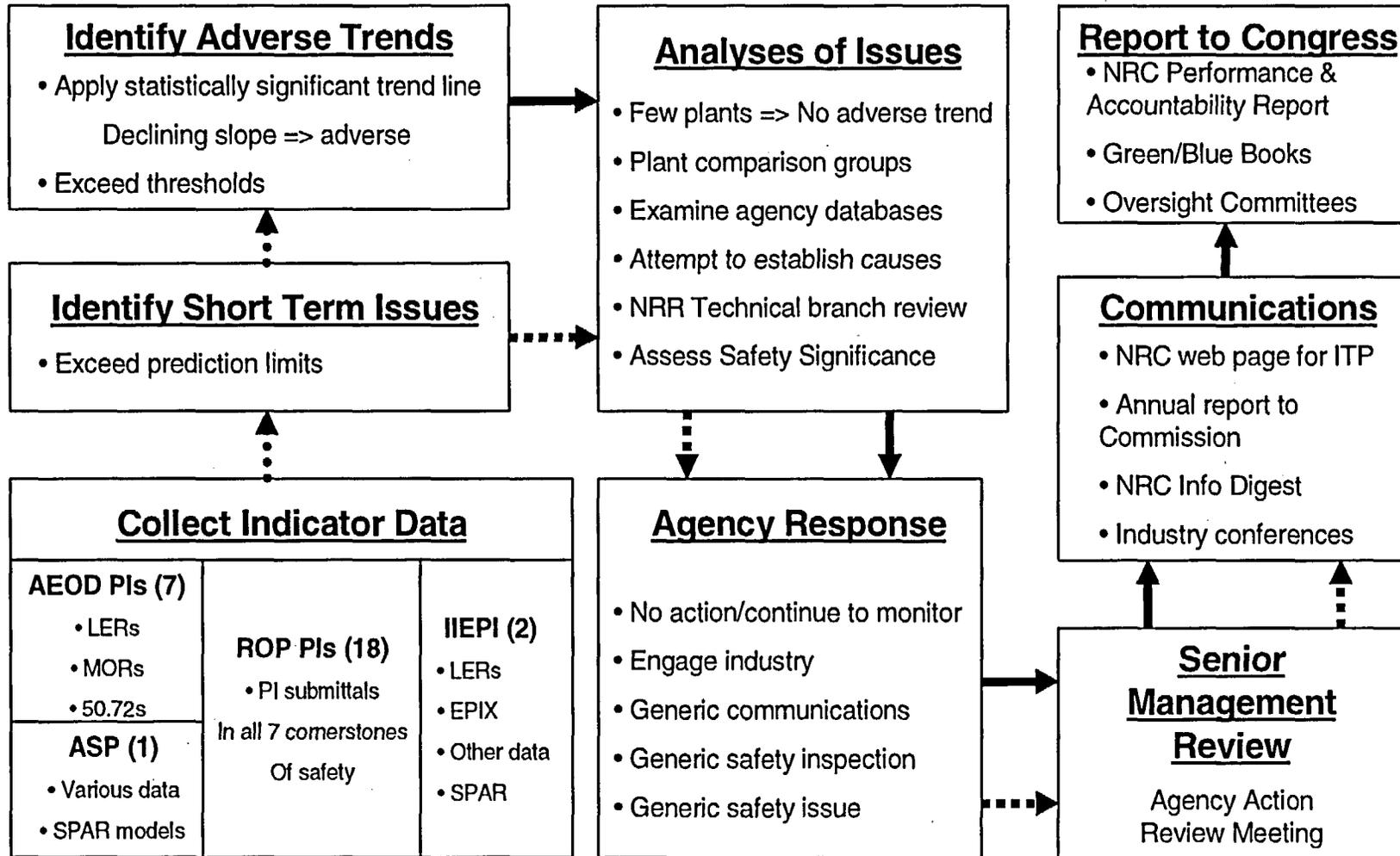
Most Significant Column in 2002 for Each of 102 Units

<u>Action Matrix Column</u>	<u>Number of Units</u>
Licensee Response	63
Regulatory Response	28
Degraded Cornerstone	6
Multiple/Repetitive Degraded Cornerstone	5

NOTE: This distribution is similar to previous ROP cycles, with the exception of 5 units moving to the MRDC column (previously only 1 per ROP cycle).

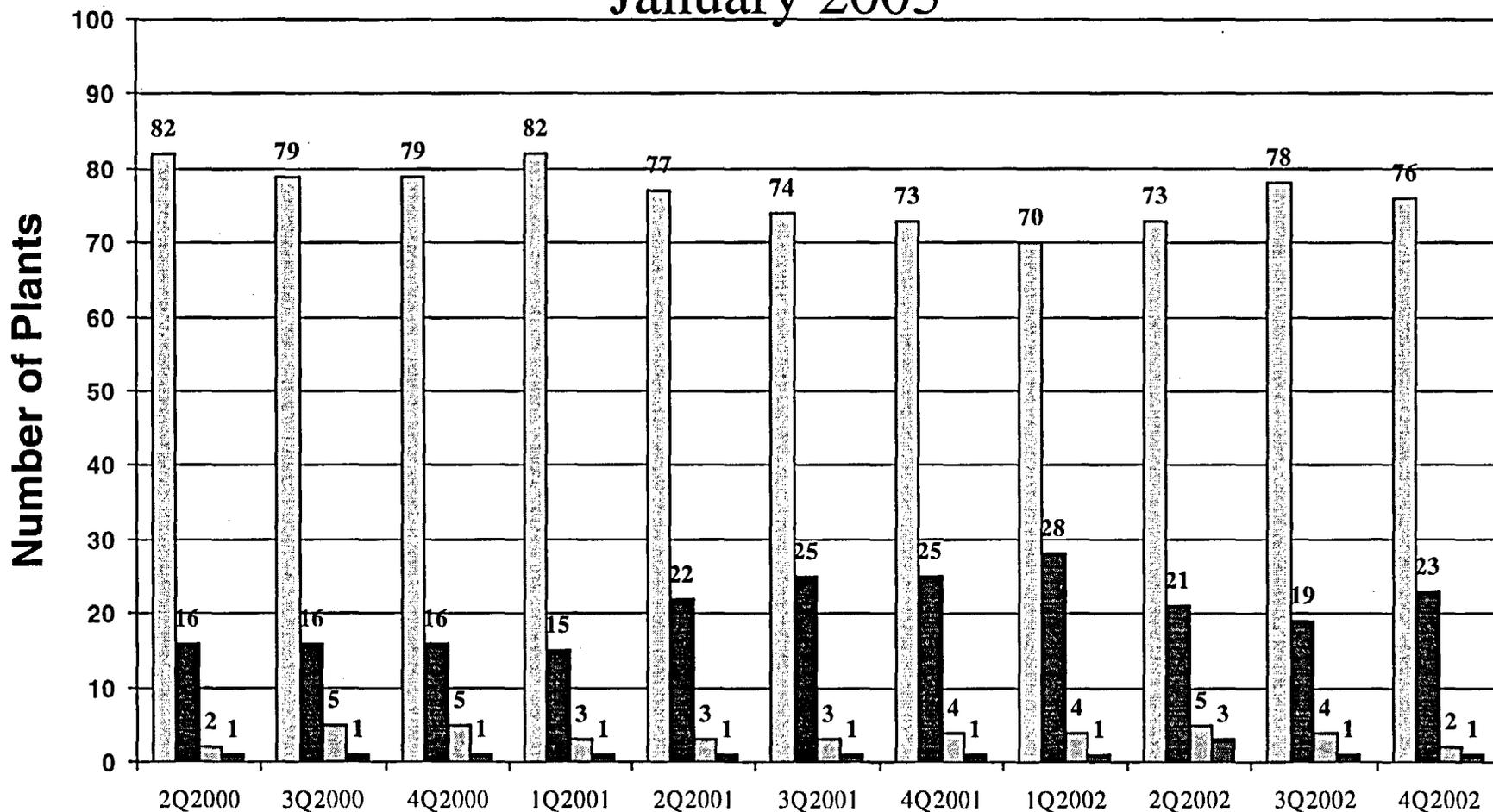
NOTE: Davis-Besse is Outside the Action Matrix and Under IMC 0350 Process.

# Industry Trends Program

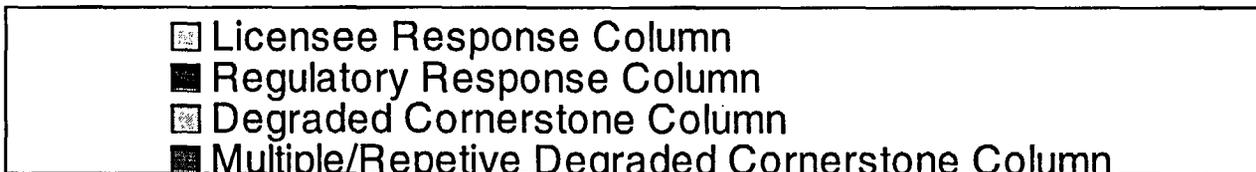


# Action Matrix Summary

January 2003

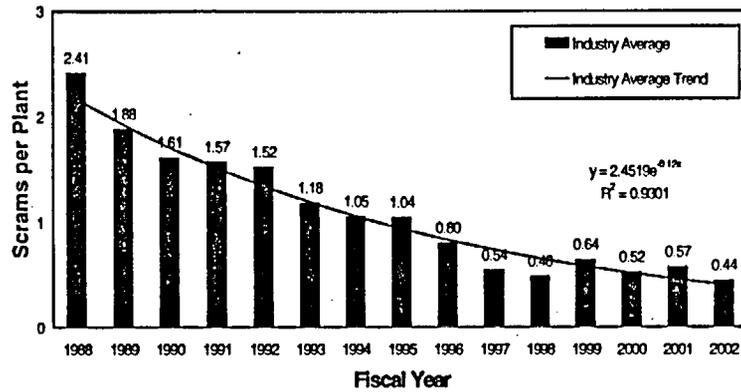


Includes DC Cook Units 1 & 2 starting in 2Q2001; Davis-Besse under IMC 0350 starting in 2Q2002

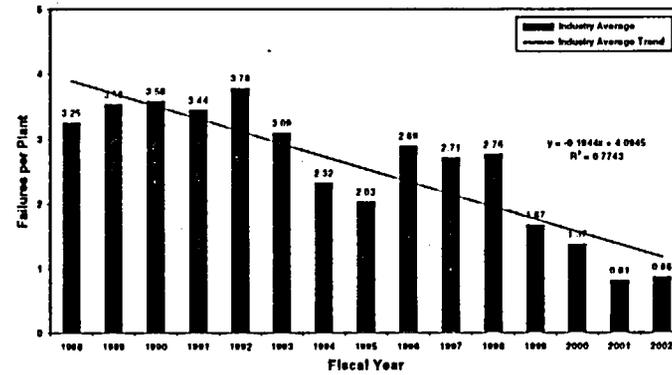


# AEOD PIs

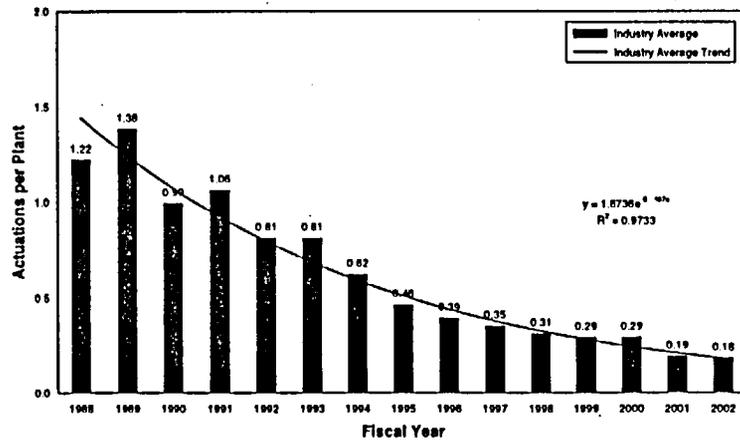
Automatic Scrams While Critical



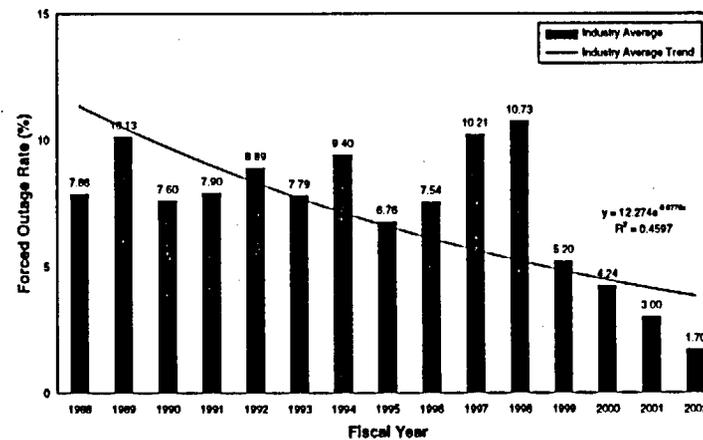
Safety System Failures



Safety System Actuations

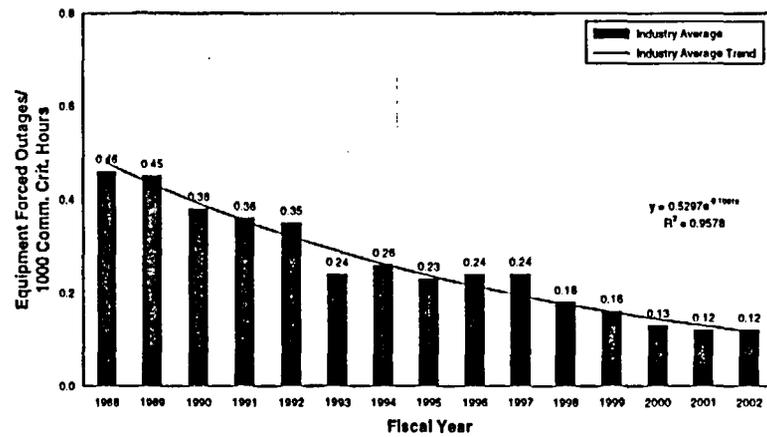


Forced Outage Rate (%)

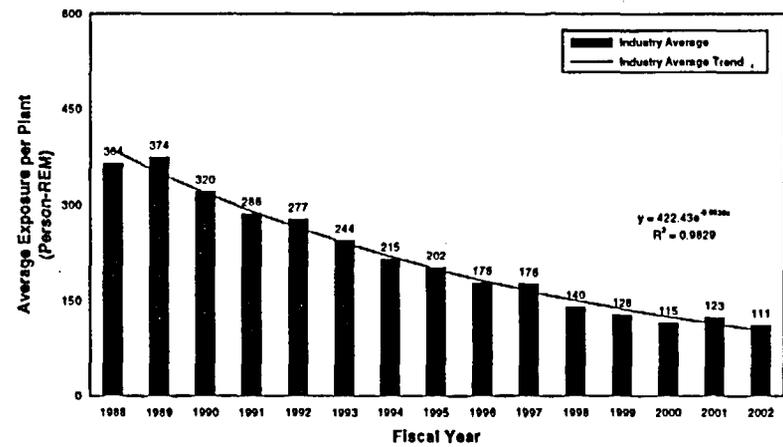


# AEOD PIs (cont.)

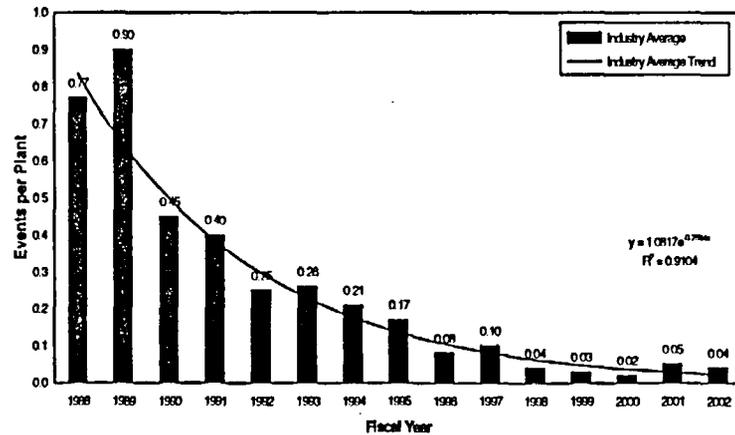
Equipment Forced Outages/  
1000 Commercial Critical Hours



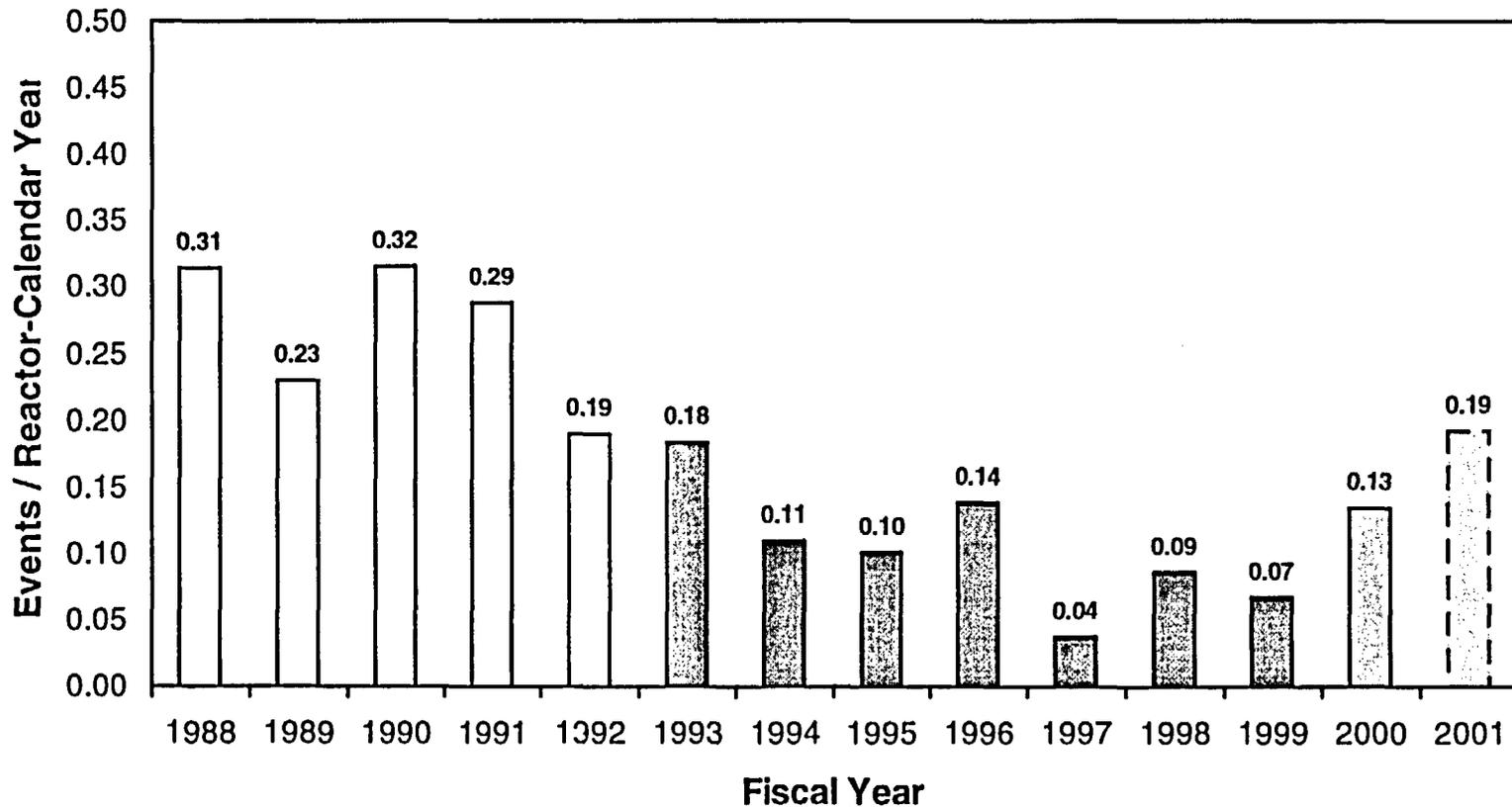
Collective Radiation Exposure



Significant Events



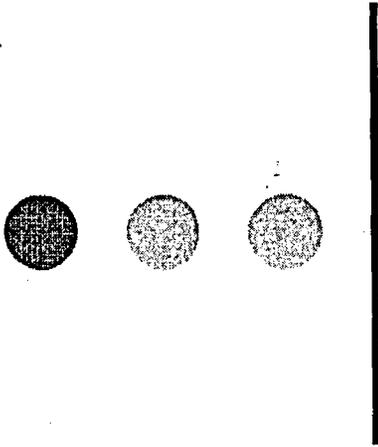
# All ASP Events/Conditions



## RESOURCES EXPENDED

(Total Staff Effort Expended at Operating Power Reactors)

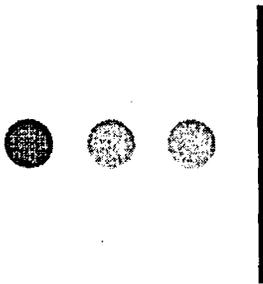
	52 weeks initial implementation 4/2/00 - 4/1/01	52 weeks FY 2001 9/24/00 - 9/22/01	52 weeks FY 2002 9/23/01 - 9/21/02
<b>Baseline/Core</b>			
Direct Inspection Effort	128,447	130,330	119,884
Inspection Prep/Doc	115,935	109,227	91,385
Plant Status	<u>43,751</u>	<u>46,191</u>	<u>44,228</u>
Subtotal	288,133	285,748	255,497
 <b>Plant Specific Inspections</b>			
Direct Inspection Effort	11,295	8,436	9,354
Inspection Prep/Doc	<u>6,683</u>	<u>6,161</u>	<u>7,715</u>
Subtotal	17,978	14,597	17,069
 <b>GSI/SI</b>			
	2,416	918	1,718
 <b>Performance Assessment</b>			
	21,017	19,845	17,293
 <b>Other Activities</b>			
Inspection Related Travel	47,190	49,471	43,627
Routine Communication			
Regional Support			
Enforcement Support			
Significance Determination Process			
Review of Technical Documents			
 <b>Total Staff Effort</b>			
(regular + nonreg hrs)	376,734 hrs	370,579 hrs	335,204 hrs
 <b>Total Staff Effort/Operating Site</b>			
	5,623 hrs/site	5531 hrs/site	5003 hrs/site



# Reactor Oversight Process: Insights and Challenges

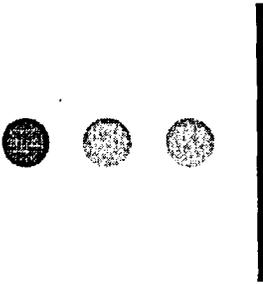
John McGaha  
President, Entergy Operations, Inc.  
May 15, 2003





# Insights and Observations

- A significant improvement and evolving
- Effective and adds value: focus on safety significant issues
- Allows the inspector and the licensee to manage risk and the regulatory process
- Can performance be too green??
- Encourage risk-informing regulations
- Room for improvement



# Challenges

- Risk-significance evaluation process
- Resolve NRC SPAR vs. licensee PSA
- Oversight to new standards
- Improve indicators
- Better risk-inform the Security, Rad Protection and Emergency Preparedness SDP



# Conclusion

- A significant improvement
- Has resulted in safety performance improvement
- Results are more visible and timely
- Has focused our resources on the safety significant areas
- The process can be further improved

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**Statement of James Riccio  
Nuclear Policy Analyst  
Greenpeace  
Before the  
U.S. Nuclear Regulatory Commission  
On  
Results of the Agency Action Review Meeting**

May 15, 2003

Good morning Chairman Diaz, Commissioners Dicus, McGaffigan and Merrifield, Greenpeace welcomes this opportunity to present our views to the Commission on the results of the Agency Action Review Meeting and on the NRC's Reactor Oversight Process.

Admittedly, I have never been a big fan of the Revised Reactor Oversight Process (ROP). I was originally a participant in the pilot evaluation panel. However, when it became evident that differing opinions would not be tolerated or reported back to the Commission I withdrew my participation. It was evident to me then that in the transition to the new reactor oversight process the agency would lose much of the transparency that it had taken years to achieve.

The old "watch list" process became so transparent that it became evident that the NRC senior managers were short-circuiting the regulatory processes by failing to take the regulatory actions warranted by the performance indicators and watch list process. If I could figure out which reactors needed regulatory attention, why couldn't the NRC? I was using your data! Fortunately those senior managers are no longer with the agency and are now pulling paychecks from the industry they worked so hard to protect while they were supposedly protecting the public health and safety.

The NRC has always had the information necessary to make the correct assessments of problem nuclear plants. NRC senior managers either lacked the will or the integrity to act upon the data they had in hand. As the U.S. General Accounting Office pointed out, "NRC has not taken aggressive enforcement action to force the licensees to fix their long-standing safety problems on a timely basis. As a result, the plant's condition has worsened, making safety margins smaller."<sup>1</sup> Sadly, this statement is as true today as it was the day it was written.

Unfortunately, little has changed in the three years since the implementation of the new oversight process. The NRC continues to lurch from one crisis to the next and in the process has undermined the public confidence in the Commission, NRC senior management and ultimately the nuclear industry.

NRC senior management has continued to place the economics of the nuclear industry ahead of public health and safety. According to the NRC's Inspector General report on Davis Besse, "(d)uring its review of the potentially hazardous condition at Davis-Besse, the NRC staff considered the financial impact to the licensee of an unscheduled plant shutdown."<sup>2</sup> Additionally the NRC's Inspector General found that:

(w)ith respect to Davis-Besse, one NRR senior official noted to OIG that the staff considered the large cost FENOC would incur if ordered to shut down, particularly if no cracking was found upon inspection.... the NRR Director had spoken with the FENOC President and was aware of the licensee's financial concerns pertaining to an unscheduled shutdown. According to the memorandum, the FENOC President told the NRR Director that the impact of a shutdown prior to February 2002 would be significant, and that Davis-Besse would be better positioned to shut down in February because of the availability of replacement fuel. The FENOC President confirmed to OIG that this discussion took place.<sup>3</sup>

The NRC's Inspector General has also reported that, "NRC appears to have informally established an unreasonably high burden of requiring absolute proof of a safety problem, versus lack of reasonable assurance of maintaining public health and safety, before it will act to shut down a power plant."<sup>4</sup>

Apparently, NRC's senior managers have deluded themselves into believing that their strategic performance goal of "reducing unnecessary regulatory burden" somehow trumps or takes precedence over the NRC's statutory responsibility to protect the public health and safety. Since the implementation of the new oversight process, NRC senior management has continued to scuttle efforts of its own staff to regulate the industry and has allowed reactors to operate to point of breakdown.

A pattern has seemingly developed that has gone unnoticed by this Commission. NRC staff attempts to enforce the regulations and potentially shut down a reactor. NRC senior management intervenes to prevent the "unnecessary regulatory burden" of actually complying with the regulations. The NRC allows the reactor to continue to operate until it is forced to shut down by incident or accident.

The debacle at Davis Besse is not an anomaly; it is merely NRC business as usual. It is the same pattern of regulatory neglect by the NRC that led to the steam generator tube rupture at Indian Point in 2000.<sup>5</sup>

It was my contention then and now, that the new oversight process does not regulate the industry, it regulates the agency. It circumscribes what action the NRC may take based upon a candy-color-coated ranking of performance indicators that are so meaningless as to be irrelevant. It handcuffs NRC regional inspectors unless reactor operation is so atrocious that it trips the line from green to white. However that's next to impossible because the industry and the agency set the thresholds so high that a reactor would never trip the indicator.<sup>6</sup>

The new oversight process has failed to curb the same abuses of authority by NRC senior management that led to the shut down of every reactor in the state of Connecticut and a re-examination of the reactor oversight process in the first place.

### **Performance Indicators**

Through the research and writing of three Nuclear Lemons reports encompassing a decade worth of reactor data, I became aware that if the industry and agency could not improve performance they would manipulate the performance indicators to achieve a downward trend.<sup>7</sup> Under the revised reactor oversight process this massaging of performance indicators has continued. Under the new assessment regime, NRC has manipulated the only indicator that it and NEI couldn't get to trend downward under the previous program, safety system failures. The NRC has allowed the industry to split hairs over the difference between functionality and operability by adding a caveat to the performance indicator. Rather than track safety system failures, the new program will track safety system functional failures. The NRC should not attempt to excuse these safety system failures by applying some ex-post facto justification based upon risk insights that may not be accurate.

In April 2000, the Commission asked the ACRS to review the new Reactor Oversight Process. Specifically, you asked the ACRS to review the use of performance indicators in the Reactor Oversight Process to ensure that they provide meaningful insight into aspects of plant operation that are important to safety.

The ACRS found that performance indicator thresholds for the white/yellow and the yellow/red thresholds for initiating events and mitigating systems are not meaningful. The ACRS has pointed out to the NRC staff that:

it would take more than 20 reactor trips per year to effect the initiating event risk category in a sufficient amount to cause a licensee to enter the red band. Clearly, 20 trips in a year is far worse than industry performance has been for at least four decades to my memory.

It would take over 2000 loss of heat sink events over a 3-year period or more than two per day to enter the red category for the loss of heat sink events. Clearly, these are not particularly meaningful. The same pattern occurs in the mitigating system category.<sup>8</sup>

The ACRS has repeatedly pointed out the failings of the performance indicators used in the NRC's color-coded system. I cannot understand how that NRC staff can think that these performance indicators are even worth the time and effort needed to collect the data. The fact that the staff is ignoring the ACRS is even more troubling. Why have advisory committees if the NRC is going to ignore their advice?

### **The Significance Determination Process – Justice Delayed is Justice Denied.**

The NRC is well aware of the timeliness concerns with the significance determination process (SDP).<sup>9</sup> It has become evident during the Davis Besse debacle that

the NRC's significance determination process is so slow and arbitrary that it can not provide meaningful input into the reactor oversight process. However, our concerns with the SDP go well beyond timeliness.

The SDP is so fatally flawed that the NRC should scrap it. It is so thoroughly incomprehensible that the NRC had to create workbooks for the staff in an effort to make the process repeatable. That too has not worked, so now the NRC wants to produce workbooks with pre-determined outcomes. As I pointed out to this Commission years ago, if the SDP is not repeatable it is certainly not science and is more akin to a black art. The NRC takes an accident or incident at a reactor, runs it through the SPD and magically the accident or incident is less significant than previously believed.

One small problem: the SDP is based upon probabilistic risk assessments (PRAs) that do not reflect reality. The PRAs do not account for deviations from the reactor's design and licensing basis and the NRC treatment of old design issues only exacerbates this problem. As my colleague from the Union of Concerned Scientists (UCS) has repeatedly pointed out to the Commission, your risk assessments don't even reflect the fact that the reactor vessel can fail.<sup>10</sup>

If the reactor vessel can't fail then the Pressurized Thermal Shock rule would not have been needed and would constitute an "unnecessary regulatory burden." If the vessel can fail, why isn't that reflected in the risk assessment? When I asked the NRC staff whether the NRC planned to revisit their risk assessments to reflect the reality of vessel failure, all I got in response was a long and drawn out bureaucratic no!

### **The NRC Has Failed to Learn the Lessons of Past Regulatory Failures**

Unless the NRC is honest about its own shortcomings in regards to the reactor oversight process it will be impossible to improve the process and declining reactor performance will continue to result in accidents, incidents and other surprises for the NRC and the industry.

The section below never appeared in the Davis Besse Lessons Learned Task Force report. In a discussion last week with the NRC, I was told that the section below was purportedly outside the scope of the DBLLTF and so was not incorporated in the final document. However, this omitted portion of the report is directly on point for this morning's discussion:

#### **3.4.2 The NRC Failed to Provide Adequate Reactor Oversight Process Guidance.**

The LLTF found that the staff was having difficulty characterizing the significance of the Davis-Besse (Sic) event. This difficulty appeared to stem from technical limitations of risk assessments and SDPs in that pressure boundary integrity does not appear to be treated explicitly in PRAs. As a result, the type and extent of wastage of the RCS pressure boundary encountered at Davis-Besse appeared to be more within the scope of traditional deterministic analyses than in a risk-informed framework. In fact, as of the time of the LLTF review, the SDP for this

event had been in progress for 5 months, with no resolution. Members of the NRC staff expressed the opinion that, in the transition to the ROP, the agency has placed an over-reliance on risk information as opposed to deterministic methods.<sup>11</sup>

Greenpeace is left to wonder how many other regulatory issues and insights were deemed to be "beyond the scope" of the Davis Besse Lessons Learned Task Force. We have received file boxes full of FOIA documents, unfortunately many of those documents and emails were sanitized prior to being publicly released.

## CONCLUSION

When the NRC first instituted the revised reactor oversight process, the staff was surveyed. The results, as reported in Inside NRC, should have given the Commission cause for concern:

- 70% of those surveyed believed that the new process would not catch declining performance "before a significant reduction in safety margins."
- 70% of NRC's resident inspectors believed that the new process "may not identify and halt degrading performance."
- 79% of NRC staff either had no opinion or believed that the new performance indicators did not provide an adequate indication of declining performance.
- 75% of the NRC staff thought that the nuclear industry and NEI had too much influence and input into the new process.<sup>12</sup>

Guess what? The NRC staff was right!

When the NRC solicited comments on the 2002 reactor oversight process, the agency stated that the revised reactor oversight process inherently encompassed the NRC's performance goals. However, if we hold the revised reactor oversight process up to the NRC performance goals on can only conclude that the process has been an abysmal failure:

- The oversight process failed to maintain safety and failed to ensure that reactors are operated safely.
- The oversight process failed to enhance public confidence by failing to increase the predictability, consistency, and objectivity of the NRC and by failing to provide timely and understandable information.
- The NRC has failed to improve the effectiveness, efficiency, and realism of the oversight process by ignoring the lessons learned from past regulatory failures.

The NRC may have reduced the regulatory burden on the nuclear industry but the agency has failed to maintain safety and has further undermined the public's confidence in the NRC as an independent and unbiased regulator of the nuclear industry.

I thank the Commission for their time and consideration of our comments.

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<sup>1</sup> U.S. General Accounting Office, Nuclear Regulation: Preventing Problem Plants Requires More Effective NRC Action, GAO/RCED-97-145, May 1997, pp. 2 & 3.

<sup>2</sup> U.S. Nuclear Regulatory Commission, Office of the Inspector General, NRC's Regulation Of Davis-Besse Regarding Damage To The Reactor Vessel Head Case No. 02-03S, December 30, 2002, p.23.

<sup>3</sup> U.S. Nuclear Regulatory Commission, Office of the Inspector General, NRC's Regulation Of Davis-Besse Regarding Damage To The Reactor Vessel Head Case No.02-03S, December 30, 2002, p.17.

<sup>4</sup> U.S. Nuclear Regulatory Commission, Office of the Inspector General, NRC's Regulation Of Davis-Besse Regarding Damage To The Reactor Vessel Head Case No. 02-03S, December 30, 2002, p.23.

<sup>5</sup> U.S. Nuclear Regulatory Commission, Office of the Inspector General, NRC's Response to the February 15, 2000 Steam Generator Rube Rupture at Indian Point 2 Power Plant, Case No. 00-03S, August 29, 2000.

<sup>6</sup> Riccio, James, Comments of James Riccio Greenpeace Nuclear Policy Analyst to The U.S. Nuclear Regulatory Commission on the Third Year of Implementation of the Reactor Oversight Process, January 7, 2003.

<sup>7</sup> Riccio, James, Nuclear Lemons: An Assessment of America's Worst Nuclear Power Plants, Public Citizen's Critical Mass Energy Project, 1996, p. 24.

<sup>8</sup> U.S. Nuclear Regulatory Commission, Meeting with the Advisory Committee on Reactor Safeguards, December 5, 2001, p. 3.

<sup>9</sup> U.S. Nuclear Regulatory Commission, Office of the Inspector General, Review of NRC's Significance Determination Process, OIG-02-A-15, August 21, 2002.

<sup>10</sup> Lochbaum, David, Union of Concerned Scientists, Dickens of A Story: Ghosts of Past Present and Future at Davis Besse, To: John A Grobe, Chairman, Davis Besse 0350 panel and Edwin Hackett, Assistant Team Leader, Davis Besse Reactor Vessel Head Degradation Lesson Learned Task Force, July 3, 2002, p. 3.

<sup>11</sup> U.S. Nuclear Regulatory Commission, Freedom Of Information Act Response, FOIA/PA 2003-0018, March 20, 2003, Appendix C, Document C/6.

<sup>12</sup> Jenny Weil, "Some Regional Staffers Question Adequacy of New Oversight Process." Inside NRC, January 17, 2000, p. 1.