

May 14, 2003

Mr. Fred Dacimo
Site Vice President
Indian Point Nuclear Generating Station, Unit 2
Entergy Nuclear Operations, Inc.
295 Broadway, Suite 1
Post Office Box 249
Buchanan, NY 10511-0249

SUBJECT: INDIAN POINT NUCLEAR GENERATING STATION, UNIT 2 OPERATOR
AND SENIOR REACTOR OPERATOR INITIAL EXAMINATION REPORT
NO. 50-247/03-301

Dear Mr. Dacimo:

This report transmits the results of the Reactor operator (RO) and Senior reactor operator (SRO) licensing examination conducted by the NRC during the period of March 10-21, 2003. This examination addressed areas important to public health and safety and was developed and administered using the guidelines of the "Examination Standards for Power Reactors" (NUREG-1021, Revision 8, Supplement 1).

Based on the results of the examination, two of three Senior Reactor Operator and two of the three Reactor Operator applicants passed all portions of the examination. One Senior Reactor Operator and one Reactor Operator applicant failed the simulator portion (Part "C" operating test) of the examination. Examination results indicated that generally the applicants were well prepared for the examination. On April 24, the NRC provided final examination results, including individual license numbers, during a telephone call between Mr. R. Conte and Mr. L. Cortopassi and others of your staff. No findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). These records include the final examination and are available in ADAMS RO and SRO Written - Accession Number ML031200103; RO and SRO Operating Section A - Accession Number ML031200138; RO and SRO Operating Section B - Accession Number ML031200207; and RO and SRO Operating Section C - Accession Number ML031200432, and Facility Post Examination Comments on the Written and Operating Exams - Accession No. ML031200391. ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Mr. Fred Dacimo

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Should you have any questions regarding this examination, please contact me at (610) 337-5183, or by E-mail at RJC@NRC.GOV.

Sincerely,

/RA/

Richard J. Conte, Chief
Operational Safety Branch
Division of Reactor Safety

Docket No. 50-247
License No. DPR-26

Enclosure: Initial Examination Report No. 50-247/03-301

cc w/encl:

G. J. Taylor, Chief Executive Officer, Entergy Nuclear
M. R. Kansler, President - Entergy Nuclear Northeast
J. Herron, Senior Vice President, Indian Point Energy Center
C. Schwarz, General Manager - Plant Operations
D. Pace, Vice President, Engineering
J. Knubel, Vice President, Operations Support
J. McCann, Manager, Nuclear Safety and Licensing
J. Kelly, Director, Nuclear Safety Assurance
C. Faison, Manager, Licensing
E. Libby, Manager - Operations Training
H. Salmon, Jr., Director of Oversight
J. Fulton, Assistant General Counsel, Entergy Nuclear Operations, Inc.
W. Flynn, President, New York State Energy, Research
and Development Authority
J. Spath, Program Director, New York State Energy Research
and Development Authority
P. Eddy, Electric Division, New York State Department of Public Service
C. Donaldson, Esquire, Assistant Attorney General, New York Department
of Law
T. Walsh, Secretary, NFSC, Entergy Nuclear Operations, Inc.
D. O'Neill, Mayor, Village of Buchanan
J. G. Testa, Mayor, City of Peekskill
R. Albanese, Executive Chair, Four County Nuclear Safety Committee
S. Lousteau, Treasury Department, Entergy Services, Inc.
Chairman, Standing Committee on Energy, NYS Assembly
Chairman, Standing Committee on Environmental Conservation, NYS Assembly
Chairman, Committee on Corporations, Authorities, and Commissions
M. Slobodien, Director, Emergency Planning
B. Brandenburg, Assistant General Counsel
P. Rubin, Operations Manager
Assemblywoman Sandra Galef, NYS Assembly

C. Terry, Niagara Mohawk Power Corporation
County Clerk, Westchester County Legislature
A. Spano, Westchester County Executive
R. Bondi, Putnam County Executive
C. Vanderhoef, Rockland County Executive
E. A. Diana, Orange County Executive
T. Judson, Central NY Citizens Awareness Network
M. Elie, Citizens Awareness Network
D. Lochbaum, Nuclear Safety Engineer, Union of Concerned Scientists
Public Citizen's Critical Mass Energy Project
M. Mariotte, Nuclear Information & Resources Service
F. Zalzman, Pace Law School, Energy Project
L. Puglisi, Supervisor, Town of Cortlandt
Congresswoman Sue W. Kelly
Congresswoman Nita Lowey
Senator Hilary Rodham Clinton
Senator Charles Schumer
J. Riccio, Greenpeace
A. Matthiessen, Executive Director, Riverkeepers, Inc.
M. Kapolwitz, Chairman of County Environment & Health Committee
A. Reynolds, Environmental Advocates
M. Jacobs, Director, Longview School
D. Katz, Executive Director, Citizens Awareness Network
P. Gunter, Nuclear Information & Resource Service
P. Leventhal, The Nuclear Control Institute
K. Copeland, Pace Environmental Litigation Clinic
R. Witherspoon, The Journal News

Mr. Fred Dacimo

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Distribution w/encl: **(VIA E-MAIL)**

Region I Docket Room (with concurrences)

H. Miller, RA

J. Wiggins, DRA

H. Nieh, RI EDO Coordinator

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P. Eselgroth, DRP

R. Laufer, NRR

P. Milano, PM, NRR

G. Vissing, PM, NRR (Backup)

W. Cook, DRP

R. Martin, DRP

T. Byron, INPO (ByronTR@Inpo.org)

W. Lanning, DRS

R. Crlenjak, DRS

R. Conte, DRS

J. Caruso, Chief Examiner, DRS

C. Buracker, DRS (Master Exam File)

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U. S. NUCLEAR REGULATORY COMMISSION

REGION I

Docket No: 50-247

License No: DPR-26

Report No: 50-247/03-301

Licensee: Entergy

Facility: Indian Point 2

Dates: March 21, 2003 (Written Examination Administration)
March 10-14, 2003 (Operating Test Administration)
March 21-April 4, 2003 (Examination Grading)

Examiners: J. Caruso, Senior Operations Engineer (Chief Examiner)
S. Barr, Operations Engineer
J. Laughlin, Operations Engineer

Approved by: Richard J. Conte, Chief
Operational Safety Branch
Division of Reactor Safety

Enclosure

SUMMARY OF FINDINGS

IR 05000247/03-301; March 10-21, 2003; Indian Point 2; Initial Operator Licensing Examination. Four of six applicants passed the examination (2 ROs, 1 SRO instant, and 1 SRO upgrade).

The written examinations were administered by the facility and the operating tests were administered by three NRC region-based examiners.

A. Inspector Identified Findings

No findings of significance were identified.

B. Licensee Identified Findings

No findings of significance were identified.

Report Details

1. REACTOR SAFETY

Mitigating Systems - Reactor Operator (RO) and Senior Reactor Operator (SRO) Initial License Examination

a. Scope of Review

The licensee's examination team developed the written and operating initial examinations and together with NRC personnel verified or ensured, as applicable, the following:

- The examination was prepared and developed in accordance with the guidelines of Revision 8, Supplement 1 of NUREG-1021, "Operator Licensing Examination Standards for Power Reactors." A review was conducted both in the Region I office and at the Indian Point 2 plant and training facility. Final resolution of comments and incorporation of test revisions were conducted during and following the onsite preparation week.
- Simulation facility operation was proper.
- A test item analysis was completed on the written examination for feedback into the systems approach to training program.
- Examination security requirements were met. However, the licensee reported to the NRC per telephone conversation, before the written examination was approved by the NRC for administration, that it was their intention to replace the first 25 questions on the written exam. The licensee proposed this action because they had terminated the employment of an individual who had previously reviewed and had knowledge of the first 20-22 questions on the proposed written examination. The facility recommended replacement of the first 25 questions on the written exam as a conservative measure to ensure exam security would not be compromised. The NRC concurred with the facilities' recommendation at the time to replace these questions prior to exam administration.

The NRC examiners administered the operating portion of the examination to all applicants from March 10-14, 2003. Indian Point 2 training staff administered the written examination on March 21, 2003.

b. Findings

Grading and Results

Four applicants (2 SROs and 2 ROs) passed all portions of the initial licensing examination. One SRO and one RO applicant failed the simulator portion (Part "C" of the operating test) of the examination.

Enclosure

The licensee submitted two written post-examination comments one on the operating exam and one on the written exam and a third verbal comment was made regarding the Operating Exam. Facility Post Examination Comments on the Written and Operating Exams can be found in ADAMS - Accession No. ML031200391. NRC resolution of the licensee's comments is included as Attachment 2. NRC staff accepted both comments.

Examination Preparation and Quality

The quality of the draft examinations was within acceptable range.

Examination Administration and Performance

NRC examiners did not note generic performance errors by the applicants during examination administration.

4. OTHER ACTIVITIES

4OA6 Meetings, including Exit

On April 24, 2003, the NRC provided conclusions and examination results to Mr. L. Cortopassi, IPEC Training Manager, via telephone. License numbers for four of the six applicants that passed all portions of the initial licensing examination were also provided during this time.

The NRC expressed appreciation for the cooperation and assistance that was provided during the preparation and administration of the examination by the licensee's training staff.

Enclosure

Attachment 1

KEY POINTS OF CONTACT

LICENSEE

| | |
|---------------|---|
| L. Cortopassi | Manager, Training, Indian Point Station |
| F. Wilson | Superintendent, Operations Training |
| W. Altic | Licensed Operator Training Unit 2 |

NRC

| | |
|-------------|----------------------------|
| J. Caruso | Senior Operations Engineer |
| S. Barr | Operations Engineer |
| J. Laughlin | Operations Engineer |

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

None.

ATTACHMENT 2

NRC RESOLUTION OF LICENSEE COMMENTS

Question: SRO 097

Comment: The question asks the Technical Specification (TS) requirements for a loss of both intermediate range instruments during a start-up. The question is recommended for deletion since none of the 4 choices correctly answers the question.

NRC Resolution: Recommendation accepted. The question is deleted due to no correct answer. Referring to TS table 3.5-2, item #3 indicates with no intermediate channels operable if two or more power range channels are greater than 10% power then the intermediate channels are not required. Intermediate range instruments are required until greater than 10% power by the Unit 2 TS and the question stem stated that power is 6%. Therefore, Unit 2 TS requires Hot Shutdown condition but there is no stated time limit. For example, answer "D" was incorrect because it stated a time limit of 6 hours to be in hot shutdown.

Administrative JPM, A.4 SRO, "Perform Event Classification"

Comment: This JPM was designed to provide initial conditions such that the applicants would have determined that the RCS Barrier had been lost (LOCA in progress), Fuel Barrier had a potential loss (degraded core cooling FR-C.2 has been entered and exited), and the Containment Barrier had failed (Containment pressure has increased to 2.8 psig and has stabilized) resulting in a General Emergency classification. However, when considering the Containment Barrier integrity based on 25-40 minutes into the accident sequence, with equipment functioning as specified in the JPM conditions, containment pressure will be single digits. Two correct answers are recommended for this JPM either a Site Area Emergency (i.e., Loss or potential loss of 2 barriers) or General Area Emergency (i.e., Loss or potential loss of 3 barriers).

NRC Resolution: Recommendation accepted. Two correct answers were accepted. In order for the applicants to assess failure or potential failure of the Containment Barrier, they needed information that was not provided in the initial conditions regarding core spray system operation and containment pressures reached during the accident. Depending on the applicants' analysis of the initial conditions and assumptions, the applicants may have determined that either the Containment Barrier had failed (i.e., a loss of all 3 barriers) or that the Containment Barrier had never been challenged (i.e., a loss of only 2 barriers).

Administrative JPM, A.1b SRO, "Review Control Room Log Entries"

Comment: This SRO JPM was designed to examine an SRO applicant's ability to perform a supervisory review of completed control room logs. The initiating cue stated, "review log entries taken on the 1900-0700 shift for approval." The facility provided a verbal post exam comment that 2 of the 3 out of specification readings that were intended to be identified were actually listed under the 0700-1900 shift.

NRC Resolution: This exam item was evaluated based on the examiner's verbal direction given during exam administration. During administration of this exam item, the examiner verbally briefed the SRO applicants that they were to carry out their normal supervisory responsibilities in reviewing the entire log.