

# LIS ORIGINAL

SINS No.: 6835  
IN 86-17

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
OFFICE OF INSPECTION AND ENFORCEMENT  
WASHINGTON, DC 20555

March 24, 1986

IE INFORMATION NOTICE NO. 86-17: UPDATE OF FAILURE OF AUTOMATIC SPRINKLER  
SYSTEM VALVES TO OPERATE

Addressees:

All nuclear power reactor facilities holding an operating license (OL) or a construction permit (CP).

Purpose:

This is a notice of the continuing problems of the operational failure of 6-inch deluge and pre-action fire protection water control valves, identified as Model C, manufactured by Automatic Sprinkler Corporation of America (ASCOA) of Cleveland, Ohio. This notice supplements information contained in Information Notice (IN) 84-16, issued on March 2, 1984.

It is expected that recipients will review the information for applicability to their facilities and consider actions, if appropriate, to preclude similar problems from occurring at their facilities. However, suggestions contained in this information notice do not constitute NRC requirements; therefore, no specific action or written response is required.

Description of Circumstances:

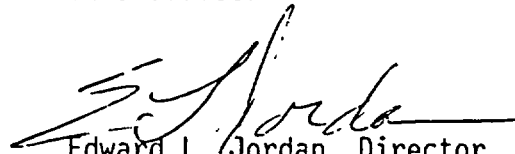
The NRC received a copy of a notification from ASCOA in January of 1986 stating that previous instructions issued to customers in late 1984 or early 1985 concerning maintenance to the 6-inch Model C deluge and pre-action valve was inadequate to prevent the failure-to-actuate problems that had been experienced.

In September of 1983, Mississippi Power and Light reported the failure of one of these valves to actuate during a fire in the emergency diesel/generator and the failures of two other identical valves to actuate during tests. These events and subsequent licensee findings are detailed in IN 84-16. That information notice stated that ASCOA was evaluating the failures.

In June of 1984, ASCOA concluded in a letter to NRC that the Grand Gulf events were plant unique. In November of 1984, ASCOA informed the NRC that three separate reports of the 6-inch Model C valve failing to actuate under certain circumstances had caused them to re-evaluate their previous conclusion. They provided NRC with an advanced copy of instructions to customers on the nature of the problem, identification of affected systems, and a recommended course of action to be taken to correct the problem. ASCOA stated this notification would be sent to all their nuclear customers.

The most recent ASCOA instructions (circa January 1986) supersede the previous instructions and state that the previously recommended maintenance procedure is inadequate. These instructions provide what is termed a "temporary solution." A tentative date for availability of a permanent solution, additional information for identification of affected valves, and a course of action described by ASCOA as "required" for the elimination of the problem are also given. However, it is not clear that this notification has been sent to all nuclear customers. Licensees who have not received this communication from ASCOA are encouraged to contact their ASCOA representative for this important information.

No specific action or written response is required by this notice. If you have any questions about this matter, please contact the Regional Administrator of the appropriate regional office or this office.



Edward L. Jordan, Director  
Division of Emergency Preparedness  
and Engineering Response  
Office of Inspection and Enforcement

Technical Contact: Mary S. Wegner, IE  
301-492-4511

Attachment: List of Recently Issued IE Information Notices

LIST OF RECENTLY ISSUED  
IE INFORMATION NOTICES

Information Notice No.	Subject	Date of Issue	Issued to
86-16	Failures To Identify Containment Leakage Due To Inadequate Local Testing of BWR Vacuum Relief System Valves	3/11/86	All power reactor facilities holding an OL or CP
86-15	Loss Of Offsite Power Caused By Problems In Fiber Optics Systems	3/10/86	All power reactor facilities holding an OL or CP
86-14	PWR Auxiliary Feedwater Pump Turbine Control Problems	3/10/86	All power reactor facilities holding an OL or CP
86-13	Standby Liquid Control System Squib Valves Failure To Fire	2/21/86	All BWR facilities holding an OL or CP
86-12	Target Rock Two-Stage SRV Setpoint Drift	2/25/86	All power reactor facilities holding an OL or CP
86-11	Inadequate Service Water Protection Against Core Melt Frequency	2/25/86	All power reactor facilities holding an OL or CP
84-69 Sup. 1	Operation Of Emergency Diesel Generators	2/24/86	All power reactor facilities holding an OL or CP
86-10	Safety Parameter Display System Malfunctions	2/13/86	All power reactor facilities holding an OL or CP
86-09	Failure Of Check And Stop Check Valves Subjected To Low Flow Conditions	2/3/86	All power reactor facilities holding an OL or CP
86-08	Licensee Event Report (LER) Format Modification	2/3/86	All power reactor facilities holding an OL or CP

OL = Operating License  
CP = Construction Permit