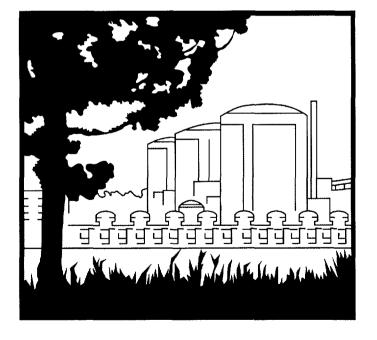
NRC FORM 658 (9-1999)			U.S. NUCLEAR REGULATORY COMMISSION								
(9.1999)											
TRANSMITTAL OF MEETING HANDOUT MATERIALS FOR IMMEDIATE PLACEMENT IN THE PUBLIC DOMAIN											
person who iss materials, will l circumstances	sued the meeting notice). The co	omplet I Desk	v the person who announced the meeting (i.e., the ted form, and the attached copy of meeting handout on the same day of the meeting; under no g day after the meeting.								
DATE OF MEETING	1										
04/30/2003	The attached document(s), which was/were handed out in this meeting, is/are to be placed in the public domain as soon as possible. The minutes of the meeting will be issued in the near future. Following are administrative details regarding this meeting:										
	Docket Number(s)	50-	269, 50-270, 50-287								
	Plant/Facility Name	<u> </u>	CONEE NUCLEAR STATION, UNITS 1, 2, AND 3								
	TAC Number(s) (if available)	MB6651, MB6652, AND MB6653									
	Reference Meeting Notice	AP	RIL 3, 2003								
	Purpose of Meeting (copy from meeting notice)	то	DISCUSS NON-SEISMIC PIPING IN THE								
		OCONEE AUXILIARY BUILDING									
NAME OF PERSON WH	IO ISSUED MEETING NOTICE		TITLE								
L. N. OLSHAN			PROJECT MANAGER								
OFFICE											
NRR											
DIVISION DLPM											
BRANCH											
PD II-1		_									
	s form and attachments:										
Docket File/Centr PUBLIC	al File		DFOI								
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Auxiliary Building Flooding Licensing Amendment Request – RAI Responses

Oconee Nuclear Station April 30, 2003

# Duke<br/>Energy<br/>MAB Flooding LAR – RAI<br/>Response

#### **Duke Attendees**

Larry Nicholson, Regulatory Compliance Manager Reene' Gambrell, Regulatory Compliance Engineer Boyd Shingleton, Regulatory Compliance Engineer Duncan Brewer, Severe Accident Analysis Group Manager Steve Nader, SAAG Engineer Ed Burchfield, Engineering Supervisor Jeff Robertson, Senior Engineer George Mc Aninch, Engineering Supervisor John Richards, Sr. Civil Engineer

# Duke<br/>Energy...AB Flooding LAR – RAI<br/>Response

#### Introduction

- Duke met with NRC on 8/22/02 to describe licensing strategy to resolve non-conforming condition related to the impact of non-seismic piping failure in the Auxiliary Building on safetyrelated equipment
- Duke submitted LAR on 11/1/02 requesting NRC to allow certain portions of the non-seismic piping in the Auxiliary Building to remain non-seismic based on low risk significance



## Introduction (continued)

- Duke received Request for Additional Information (RAI) on the PRA justification for the change on 1/16/03; provided response on 2/11/03
- Duke received another RAI related to HSPW and LPSW piping design and the ABS calculation on 2/13/03 & 3/6/03; provided response on 4/3/03

# Duke<br/>Energy<br/>MAB Flooding LAR – RAI<br/>Response

#### **PRA Discussion – Steve Nader**

**Civil Discussion – John Richards** 

# PRA Evaluation-Overview

#### Overview

- Piping has been identified that could have an effect on safety related equipment if it failed in a seismic event
- Solution: Design study concluded that upgrading piping to seismic is the most practical solution
  - Estimated cost ~ \$1 million
- PRA Task: Evaluate the decreased risk if the pipe is upgraded to seismic

# PRA Evaluation-Overview

Base Seismic PRA model

- ➢Part of IPEEE submittal
- ≻Submitted to NRC 12-28-95
- SER received 3-15-00
- ➢ Model periodically updated (2003)
- Self initiated internal audit Spring 2003
  - Reviewed model, inputs, etc.
  - Conclusion: Model is complete, thorough
  - No items identified that affect AB flood issues

# PRA Evaluation-Overview

Base Seismic PRA model (continued)

- Based on internal events PRA
- Internal events PRA evaluated internal flooding (from pipe breaks)
- Internal events PRA concluded that the important consequence of flooding in the AB is failure of the HPI pumps (piping failures from any cause)
- Internal events PRA flooding analysis also concluded that TB floods were much more important than AB floods; therefore AB floods screened out

# PRA Evaluation-Overview

## ■Base Seismic PRA model (continued)

- Internal events PRA conclusions were carried forward into the seismic PRA
- >No AB floods due to pipe breaks are modeled
- Piping breaks are modeled as one input to the system reliability
- TB floods are modeled (Condenser expansion joints)
- TB floods fail equipment in TB basement (e.g., EFW, cooling water to HPI)

# PRA Evaluation-Overview

#### Base Case Risk Calculation

- Start with the base seismic model
- > Add AB piping failures to the model
- Determine consequences
- Input appropriate fragilities for the piping
- Run the seismic model with new failure mode(s)
- Results: AB piping failures contribute <1% to total CDF

# PRA Evaluation-Overview

 Modified Plant Risk Calculation
Input new fragility value for piping based on ONS seismic design requirements
Re-run the seismic PRA model
Results: CDF decreased by 3E-07

# PRA Evaluation-Important Inputs

Assumption: AB Piping Failures Fail HPI Pumps

≻Internal events PRA :

- Plant layout, physical arrangement of AB
- AB drain system
- Walkdown results
- ➢ Most rooms of AB screened out
  - Minimal flood potential (rate, volume)
  - Adequate drainage
- >HPI, LPI, RBS located in basement
  - Vulnerable to flood

# PRA Evaluation-Important Inputs

Assumption: AB Piping Failures Fail HPI Pumps (continued)

➢ RBS, LPI used to mitigate large LOCA

- Probability of large LOCA and seismic failure of this piping is very small- screened out
- LPI, RBS used in other transients- however always with HPI
- Conclusion: HPI pumps are important for evaluation of AB piping failure. HPI bounds failure of other AB equipment.

# PRA Evaluation-Important Inputs

Assumption: AB Piping Failures Fail HPI Pumps (continued)

- Are other important safety functions potentially lost?
- Use same screening methods as IPEEE model
  - Secondary side heat removal- no
  - Ability to trip reactor- no
  - Loss of power no
  - Standby Shutdown Facility no
  - Component cooling water possibly lost

# PRA Evaluation-Important Inputs

Assumption: AB Piping Failures Fail CC Pumps

- Large floods (high flow rate, unisolated)
- > 500,000 gallons to fill to next level
  - Operating experience shows the largest AB flood is 164,000 gallons at WNP-2 (June 1998)
- IPEEE concluded CC Pump motor control centers would not be flooded
- LAR evaluation conservatively assumed <u>CC is lost</u> as a consequence of the flood

# PRA Evaluation-Important Inputs

# ■HPI, CC Important For Seal LOCAs

#### >ONS mitigates seal LOCAs with

- HPI seal injection
- Component Cooling of RCP thermal barrier
- Standby Shutdown Facility primary make-up

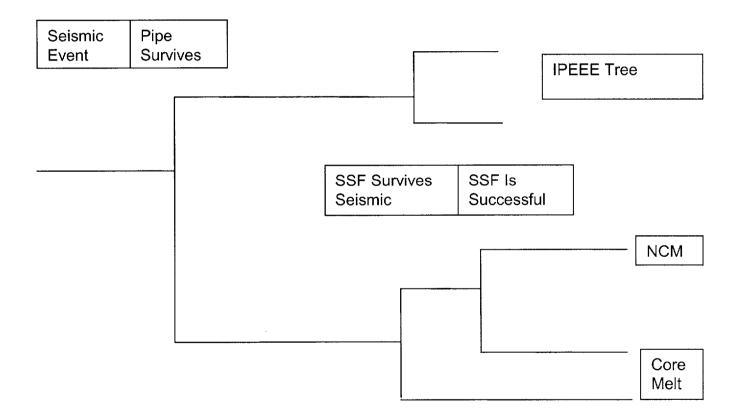
# PRA Evaluation-Important Inputs

New Core Melt Sequences Added to the Seismic PRA Model

Seismic Pipe Break

- Assume large, unisolated- HPI, CC fails
- SSF fails
  - Could fail due to seismic event
  - Could fail randomly

# PRA Evaluation-Event Tree



# PRA Evaluation-Uncertainties

Uncertainties

Base Case Results

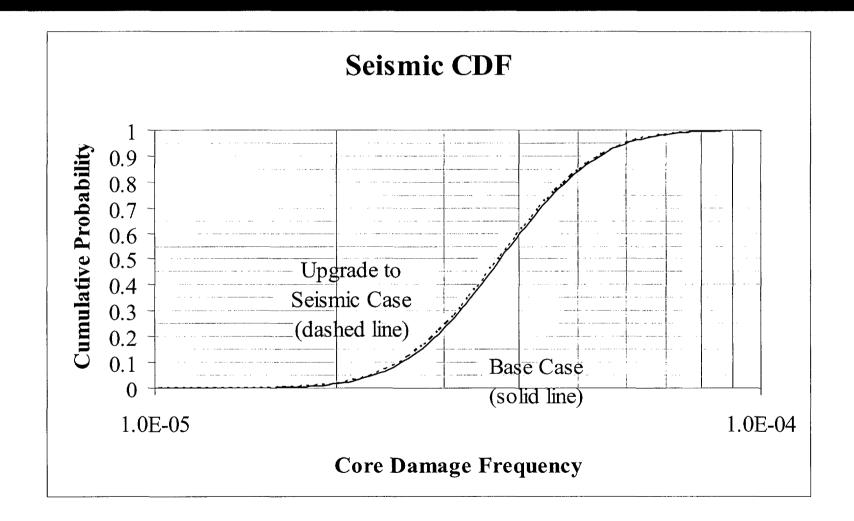
- Mean = 3.89E-05
- Std Dev = 1.16E-05
- Best Estimate = 3.25E-05

➢ Upgrade to Seismic Case

- Mean = 3.86E-05
- Std Dev = 1.17E-05
- Best Estimate = 3.23E-05

➢ Delta: 3.89E-05 – 3.86E-05 = <u>3E-07</u>

# PRA Evaluation-Uncertainties



# Computation of Piping Fragilities

Compute piping fragilities to support two cases

- Currently installed piping
- Upgraded piping consistent with Oconee design requirements

# **Duke** Energy Currently Installed Piping

Primarily rod hung piping

- Does not include seismic design
- ■Large diameter piping (≥ 3") is welded carbon or stainless steel
- ■Small diameter piping (≤ 2½") is threaded or welded carbon or stainless steel

# **Duke** Energy Currently Installed Piping

# Fragility Calculations

- Performed walkdowns of installed piping with contract technical experts (Bob Campbell, ABS)
- Confirmed installed piping consistent with piping in earthquake experience data
  - Materials, design and construction similar
  - Vulnerabilities leading to failures evaluated
- Selected controlling supports for analysis

# **Duke** Energy Currently Installed Piping

### Fragility Calculations (continued)

- Computed fragility using UHS for Oconee
- Controlling fragilities based on experience of Fire Protection Piping in the Loma Prieta earthquake

> $A_m = 0.85g \ \beta_R = 0.3 \ \beta_U = 0.46$ >HCLPF = 0.24g (ONS SSE is 0.10g)

# **Upgraded** Piping

Computed fragility assuming existing piping is upgraded to comply with Oconee design requirements

- Considered piping, supports, anchors
- Computed fragility using UHS for Oconee

# **Upgraded Piping**

# Fragility Calculations

- Investigated numerous components to determine governing element (pipe, supports, anchors, welds, etc.)
- Controlling fragilities conservatively based on welds in the supports

$$>A_{\rm m} = 1.95g \quad \beta_{\rm R} = 0.33 \quad \beta_{\rm U} = 0.59$$

≻HCLPF = 0.43g

Upgraded piping would be approximately twice as rugged as the installed piping

# Follow-Up Questions

- Received follow-up questions on 2/13/03 and 3/6/03
- Responded on 4/3/03
- Key outstanding issues from conference call on 4/10/03
  - Haven't shown experience is applicable to ONS piping
  - Experience vertical spectrum doesn't envelope ONS UHS spectrum at high frequencies
  - Uncertainty for the analysis case seems unreasonably high
  - Reports of corrosion problems

# Applicability of Experience Data

- Experience data primarily from about 20 sites in the 1989 Loma Prieta earthquake (NUREG/CR-5580, Appendix C)
- Includes experience from 1,000 water sprinkler systems
- Construction dates range from about 1930 to the present
- Construction generally complies with pre-1989 versions of NFPA-13
- Some piping included lateral bracing and some did not

# Applicability of Experience Data

#### Only 13 Direct Piping Related Failures Including

- Inadvertent actuation of deluge valves
- Minor pipe leaks
- Support failures not resulting in leaks
- Connection failures
- Sprinkler head interaction failures
- Other Failures Not Considered Direct Piping Related
  - Several failures due to soil liquefaction
  - Several pipe connection and sprinkler head failures in extensively damaged buildings

# Applicability of Experience Data

# Primary Causes of Damage Soil liquefaction Large deformation and damage to structures poorly designed for earthquakes Groove type victaulic couplings Loss of support from C-clamp supports None of these conditions exist in the Service Water piping at Oconee

#### Distribution of Experience vs. Peak Ground Acceleration

	Peak Ground Acceleration # Systems / # Failures								
Facility	0.1g		0.2g		0.3g		0.4g		
Almeda Navel Air Station					90	2			
Oakland Army Base					22	1			
Oakland Naval Station					9	91			
Port of Oakland					50	32			
Treasure Island/Hunters Pt			34	2					
San Francisco Airport					80	1			
California Hospitals	61	0	26	1			4	0	
Lockheed			300	0			20	1	
Moffett Navel Air Station			280	0					
Hewlett Packard			20	2		<u> </u>	23	2	
Totals	61	0	660	5	251	5	47	3	
Failure Rate	0.00%		0.76%		1.99%		6.38%		

Notes: 1) Failures due to soil liquefaction or C-Clamp supports (therefore, not included in totals)

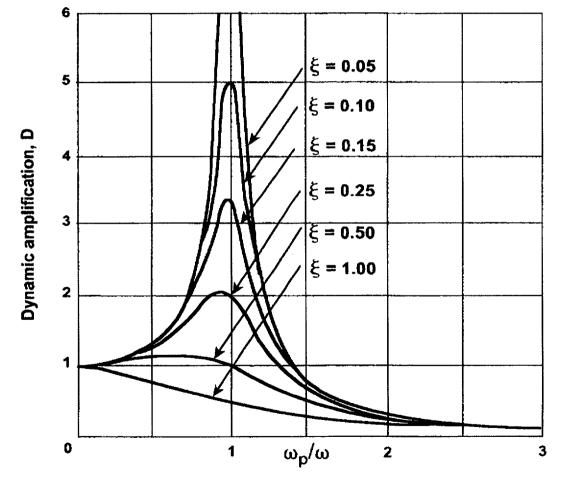
2) Two of these failures are related to structure failures (therefore, not included in totals)

# Applicability of Experience Data

- Huge volume of data permits development of generic fragility
- Support design considerations at Oconee (vertical and lateral) evaluated by separate analyses of bounding cases
- From that analysis, Oconee supports have higher fragility than experience data results, therefore, supports were not controlling
- Experience data is an acceptable estimate of Oconee Service Water Piping fragility

# DukeResponse to HighEnergyFrequency Accelerations

High frequency input does not excite low frequency modes



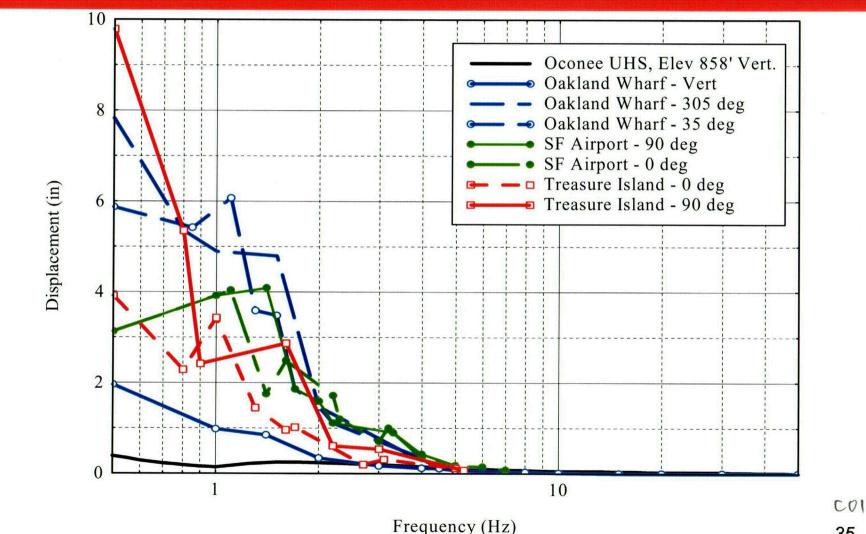
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# DukeResponse to HighEnergyFrequency Accelerations

- Analysis of multi-degree of freedom systems shows that
  - response (amplification, stress, etc) is typically dominated by lower frequency modes where
    - the majority of the system mass is vibrating
    - participation factors are largest
- Higher frequency modes have low participation and do not significantly contribute to displacements and stresses



### Spectral Displacement Comparison



# Uncertainty for the Analysis Case

- β<sub>U</sub> in the analysis case doesn't significantly affect the final results
- Performed sensitivity analysis assuming knowledge about the analyzed piping was near perfect ( $\beta_R = 0.10$   $\beta_U = 0.10$ )
- This conservatively increases the seismic capacity (HCLPF would be 1.40g, >300% increase)
- Negligible impact on CDF
- Therefore, even if the  $\beta_U$  was significantly reduced, the results would be the same

# **Corrosion Concerns**

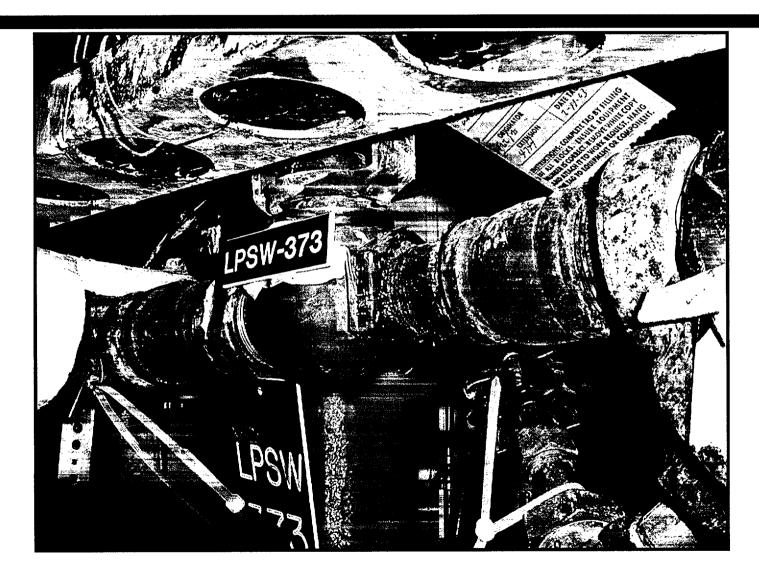
Corrosion concerns from conference call on 4/10/03

- Picture of valve LPSW-373 which shows corrosion
- > Question regarding leaks in system
- Question regarding piping replacement

# SW Corrosion

- Internal Pitting
  - Lake water (clean by most standards)
  - Crud Corrosion Products collect over many years of operation
- LPSW surface corrosion
  - Anti sweat insulation
  - Anti corrosion paint
  - Moisture intrusion over many years







# Leakage Past Threads

Drops per minute

Liquid Management goal is 0 leakage

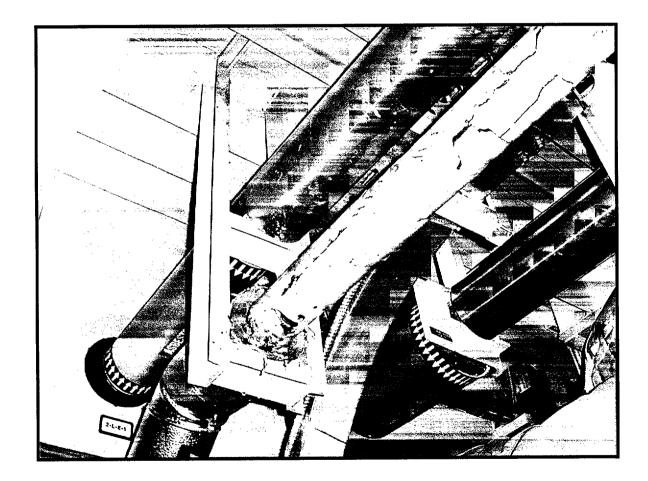
≻Work order to replace nipple

Leak management to evaluate

- Corrosion
- Structural



#### Pin Leak on Pipe



# Pin Leak on Pipe

Not a structural issue
Work order written to replace pipe
PIP written to evaluate pipe
> Required by NSD-413
> Screened using pre-established criteria

-5

# Pipe Replacement

Leaking components are replaced
SW Inspection Program performs UT
Small bore piping refurbishment
>Life extension
>To minimize resistance to flow
>Not related to structural integrity

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# Leakage Summary

Zero Leakage Threshold
SW Inspection Program
Isolated pin hole leaks
Evaluated via PIP
Screened using pre-existing criteria
Not a structural issue

# Duke<br/>Energy<br/>MAB Flooding LAR – RAI<br/>Response

Conclusion - Larry Nicholson