

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

IE Bulletin No. 79-10
Date: May 11, 1979
Page 1 of 2

REQUALIFICATION TRAINING PROGRAM STATISTICS

Description of Circumstances:

Because of the apparent failure of the operators at Three Mile Island to recognize certain plant conditions and take appropriate action to effectively cool the core and contain fission products after release from the core, the NRC Commissioners are evaluating licensee's requalification training programs. As one element in this evaluation, the NRC is interested in obtaining statistics about the failure rate on the annual requalification examinations. The information requested below along with other information will then be used to evaluate the effectiveness of the operator requalification training program.

Action to be Taken by Licensees:

For all power reactor facilities with an operating license.

1. Provide both the total number and percentage of operators who have failed the annual requalification examination.
2. Provide the percentage of those operators who take the annual requalification examination and are required to attend lectures on categories of material for which they received a grade of less than 80 percent. Also provide the total number of supplemental lectures attended (e.g., 3 operators had to attend 2 lectures, 1 operator had to attend 3 lectures, etc.).
3. Provide both the total number and percentage of operators under the requalification program that participated in accelerated training because they either scored less than 70 percent overall on the annual written examination or had an unsatisfactory performance on the oral examination.
4. Provide the same information required by 1 thru 3 on Senior Operators.

7905110048

5. Provide the above information from four annual examinations (written and oral). If the requalification program has not been implemented for that period of time or records are not available, provide the maximum information that is available.
6. Record information in format shown in Table 1.
7. Within 14 calendar days of the date of issue of this Bulletin, report in writing to the Director of the appropriate Regional Office the results of your review. A copy of your report should be sent to the United States Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation, Division of Project Management, Operator Licensing Branch, Washington, D.C. 20555.

Approved by GAO, B180225 (R0072), clearance expires 7/31/80. Approval was given under a blanket clearance specifically for identified generic problems.

Enclosure: As Stated

TABLE 1 (Continued)

	Operators				Senior Operators			
	Exam 1	Exam 2	Exam 3	Exam 4	Exam 1	Exam 2	Exam 3	Exam 4
C. Total number attending accelerated training because unsatisfactory oral examination								
D. Percent attending accelerated training because unsatisfactory oral examination								

Date of Exam 1

2

3

4

LISTING OF IE BULLETINS
ISSUED IN LAST TWELVE MONTHS

Bulletin No.	Subject	Date Issued	Issued To
78-06	Defective Cutler-Hammer, Type M Relays With DC Coils	5/31/78	All Power Reactor Facilities with an OL or CP
78-07	Protection afforded by Air-Line Respirators and Supplied-Air Hoods	6/12/78	All Power Reactor Facilities with an OL, all class E and F Research Reactors with an OL, all Fuel Cycle Facilities with an OL, and all Priority I Material Licensees
78-08	Radiation Levels from Fuel Element Transfer Tubes	6/12/78	All Power, Test and Research Reactor Facilities with an OL having Fuel Element Transfer Tubes
78-09	BWR Drywell Leakage Paths Associated with Inadequate Drywell Closures	6/14/78	All BWR Power Reactor Facilities with an OL (for action) or CP (for information)
78-10	Bergen-Paterson Hydraulic Shock Suppressor Accumulator Spring Coils	6/27/78	All BWR Power Reactor Facilities with an OL or CP
78-11	Examination of Mark I Containment Torus Welds	7/24/78	BWR Power Reactor Facilities with an OL for action: Peach Bottom 2 and 3, Quad Cities 1 and 2, Hatch 1, Monticello and Vermont Yankee. All other BWR Power Reactor Facilities with an OL for information

LISTING OF IE BULLETINS
ISSUED IN LAST TWELVE MONTHS (CONTINUED)

Bulletin No.	Subject	Date Issued	Issued To
78-12	Atypical Weld Material in Reactor Pressure Vessel Welds	9/29/78	All Power Reactor Facilities with an OL or CP
78-12A	Atypical Weld Material in Reactor Pressure Vessel Welds	11/24/78	All Power Reactor Facilities with an OL or CP
78-12B	Atypical Weld Material in Reactor Pressure Vessel Welds	3/19/79	All Power Reactor Facilities with an OL or CP
78-13	Failures In Source Heads of Kay-Ray, Inc., Gauges Models 7050, 7050B, 7051, 7051B, 7060, 7060B, 7061 and 7061B	10/27/78	All General and Specific Licensees with the subject Kay-Ray, Inc. Gauges
78-14	Deterioration of Buna-N Components In ASCO Solenoids	12/19/78	All GE BWR Facilities with an OL (for action), and all other Power Reactor Facilities with an OL or CP (for information)
79-01	Environmental Qualification of Class IE Equipment	2/8/79	All Power Reactor Facilities with an OL, except the 11 Systematic Evaluation Program Plants (for action), and all other Power Reactor Facilities with an OL or CP (For Information)
79-02	Pipe Support Base Plate Design Using Concrete Expansion Anchor Bolts	3/8/79	All Power Reactor Facilities with an OL or CP

LISTING OF IE BULLETINS
ISSUED IN LAST TWELVE MONTHS (CONTINUED)

Bulletin No.	Subject	Date Issued	Issued to
79-03	Longitudinal Weld Defects in ASME SA-312 Type 304 Stainless Steel Pipe Spools Manufactured by Youngstown Welding and Engineering Company	3/12/79	All Power Reactor Facilities with an OL or CP
79-04	Incorrect Weights for Swing Check Valves Manufactured by Velan Engineering Corporation	3/30/79	All Power Reactor Facilities with an OL or CP
79-05	Nuclear Incident at Three Mile Island	4/1/79	All Babcock and Wilcox Power Reactor Facilities with an OL, Except Three Mile Island 1 and 2 (For Action), and All Other Power Reactor Facilities With an OL or CP (For Information)
79-05A	Nuclear Incident at Three Mile Island - Supplement	4/5/79	Same as 79-05
79-06	Review of Operational Errors and System Misalignments Identified During the Three Mile Incident	4/11/79	All Pressurized Water Power Reactor Facilities with an OL Except B&W Facilities (For Action), All Other Power Reactor Facilities with an OL or CP (For Information)

LISTING OF IE BULLETINS
ISSUED IN LAST TWELVE MONTHS (CONTINUED)

Bulletin No.	Subject	Date Issued	Issued to
79-06A	Same Title as 79-06	4/14/79	All Westinghouse Designed Pressurized Power Reactor Facilities with an OL (For Action), and All Other Power Reactor Facilities with an OL or CP (For Information)
79-06A (Revision 1)	Same Title as 79-06	4/18/79	All Westinghouse Designed Pressurized Power Reactor Facilities with an OL (For Action), and All Other Power Reactor Facilities with an OL or CP (For Information)
79-06B	Same Title as 79-06	4/14/79	All Combustion Engineering Designed Pressurized Power Reactor Facilities with an OL (For Action), and All Other Power Reactor Facilities with an OL or CP (For Information)
79-07	Seismic Stress Analysis of Safety-Related Piping	4/14/79	All Power Reactor Facilities with an OL or CP

LISTING OF IE BULLETINS
ISSUED IN LAST TWELVE MONTHS (CONTINUED)

Bulletin No.	Subject	Date Issued	Issued to
79-08	Events Relevant to Boiling Water Power Reactors Identified During Three Mile Island Incident	4/14/79	All BWR Power Reactor Facilities with an OL (For Action), All Other Power Reactor Facilities with an OL or CP (For Information)
79-09	Failures of GE Type AK-2 Type Circuit Breaker in Safety Related Systems	4/17/79	All Power Reactor Facilities with an OL or CP