



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 26, 1981

ALL BWR LICENSEES
(EXCEPT HUMBOLDT BAY AND LACROSSE)

RE: IMPLEMENTATION OF NUREG-0313, REV. 1, "TECHNICAL REPORT ON MATERIAL SELECTION AND PROCESSING GUIDELINES FOR BWR COOLANT PRESSURE BOUNDARY PIPING (GENERIC TASK A-42)
(Generic Letter 81-04)

Enclosed for your action is the final NUREG-0313, Rev. 1, "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping," dated July 1980, which incorporates the public comments on the "For Comment" edition of the same subject report published in October 1979. This final report constitutes the staff's resolution of the NRC's Generic Technical Activity A-42, "Pipe Cracks in Boiling Water Reactors," which was an "Unresolved Safety Issue" pursuant to Section 210 of the Energy Reorganization Act of 1974.

This report, NUREG-0313, Rev. 1, identifies NRC requirements resulting from the recent publication, NUREG-0531, "Investigation and Evaluation of Stress-Corrosion Cracking in Piping of Light Water Reactor Plants," by the NRC Pipe Crack Study Group (PCSG). The Study Group was specifically chartered to reexamine our previous conclusions and recommendations on this subject in view of cracks recently discovered in large diameter pipes, to evaluate the significance of safe end cracking at Duane Arnold relative to similar material and design aspects at other facilities, and to examine detection capabilities.

NUREG-0313, Rev. 1 sets forth the NRC staff's revised acceptable methods to reduce the intergranular stress corrosion cracking susceptibility of BWR ASME Code Class 1, 2, and 3 pressure boundary piping and safe ends. For plants that cannot fully comply with the material selection, testing and processing guidelines of this report, varying degrees of augmented inservice inspection and leak detection requirements are presented.

We request that you review all of the ASME Code Class 1 and 2 pressure boundary piping, safe ends, and fitting material, including weld metal, at your facility to determine if it meets the material selection, testing, and processing guidelines set forth in the enclosed report. You should identify to us any materials that do not meet these guidelines and propose appropriate changes to your Technical Specifications to incorporate the augmented inservice inspection requirements specified in Section IV of the enclosed report. In the case of "service sensitive" lines you should also provide your plans and schedule for the replacement, to the extent practicable, of nonconforming materials with those that conform to the staff's guidelines.

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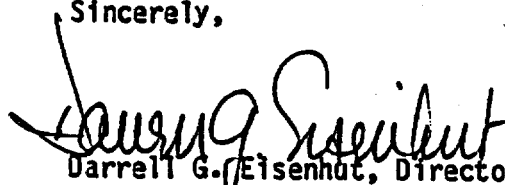
In addition, we request that if it is needed, you propose appropriate changes to your Technical Specifications to bring them into conformance with the enclosed report and the affected portion of the model Technical Specifications denoted with an asterisk in the margin of the text.

You should respond to this letter by July 1, 1981. In your response, you should propose a schedule to accomplish the above requested actions or a description, schedule and justification for alternative actions to accomplish the objectives of this program.

This request for information was approved by GAO under a blanket clearance number R0072 which expires November 30, 1983. Comments on burden and duplication may be directed to the U.S. General Accounting Office, Regulatory Reports Review, Room 5106, 441 G Street, N.W., Washington, D.C. 20548.

The response to this request should reference this letter by title and number.

Sincerely,


Darrell G. Eisenhut, Director
Division of Licensing

Enclosures:

1. NUREG-0313, Rev. 1
2. Model Technical Specifications

cc: Service Lists, w/o Enclosure 1

BOILING WATER REACTOR LICENSEES

2/26/81

Docket No. 50-293
Pilgrim Unit 1

Docket No. 50-325
Brunswick Unit 1

Docket No. 50-324
Brunswick Unit 2

Docket No. 50-10
Dresden 1

Docket No. 50-237
Dresden 2

Docket No. 50-249
Dresden 3

Docket No. 50-254
Quad-Cities Unit 1

Docket No. 50-265
Quad-Cities Unit 2

Docket No. 50-155
Big Rock Point

Docket No. 50-409
Lacrosse (AC)

Docket No. 50-321
Edwin I. Hatch Unit 1

Docket No. 50-366
Edwin I. Hatch Unit 2

Docket No. 50-331
Duane Arnold

Docket No. 50-219
Oyster Creek

Docket No. 50-220
Nine Mile Point Unit 1

Docket No. 50-298
Cooper Station

Docket No. 50-245
Millstone Unit 1

Docket No. 50-263
Monticello

Docket No. 50-133
Humboldt Bay

Docket No. 50-277
Peach Bottom Unit 2

Docket No. 50-278
Peach Bottom Unit 3

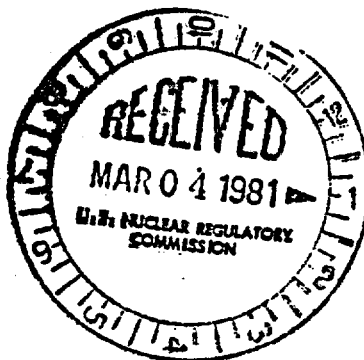
Docket No. 50-333
FitzPatrick

Docket No. 50-259
Browns Ferry Unit 1

Docket No. 50-260
Browns Ferry Unit 2

Docket No. 50-296
Browns Ferry Unit 3

Docket No. 50-271
Vermont Yankee



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