



**FPL Energy**  
**Seabrook Station**

FPL Energy Seabrook Station  
P.O. Box 300  
Seabrook, NH 03874  
(603) 773-7000

April 21, 2003

Docket No. 50-443

NYN-03032

Re: NYN-02095

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555-0001

Seabrook Station  
Revision to License Amendment Request 02-03,  
“Changes to Technical Specifications Associated with Nuclear Instrumentation”

FPL Energy Seabrook, LLC (FPLE Seabrook) has enclosed herein a revision to License Amendment Request (LAR) 02-03, NYN-02095, dated October 11, 2002. This revision withdraws the LAR 02-03 proposed change to Technical Specification (TS) Table 4.3-1 to extend the TRIP ACTUATING DEVICE OPERATIONAL TEST surveillance interval for Functional Units 16a, Turbine Trip on Low Fluid Oil Pressure; and Functional Unit 16b, Turbine Trip on Turbine Stop Valve Closure, from 31 days to 92 days. The surveillance interval for Functional Units 16a and 16b will remain as currently licensed, i.e., 31 days. TS Table 4.3-1 and its associated Table Notations have been revised accordingly to reflect the 31 day interval. The revised markup and retype pages are enclosed and supercede those submitted in LAR 02-03.

This revision to LAR 02-03 does not make any additional changes to LAR 02-03 or its conclusion. That is, the proposed changes do not involve a significant hazards consideration pursuant to the requirements of 10 CFR 50.92 and the proposed changes continue to meet the criteria of 10CFR 51.22(c)(9) for a categorical exclusion from the requirements for an Environmental Impact Statement. Therefore, this revision to LAR 02-03 will neither adversely affect nor endanger the health and safety of the general public.

Should you have any questions regarding this letter, please contact Mr. James M. Peschel, Regulatory Programs Manager, at (603) 773-7194.

Very truly yours,  
FPL Energy Seabrook, LLC.

  
\_\_\_\_\_  
Gene F. St. Pierre  
Station Director

A001

cc: H. J. Miller, NRC Regional Administrator  
V. Nerses, NRC Project Manager, Project Directorate I-2  
G. Dentel, NRC Senior Resident Inspector

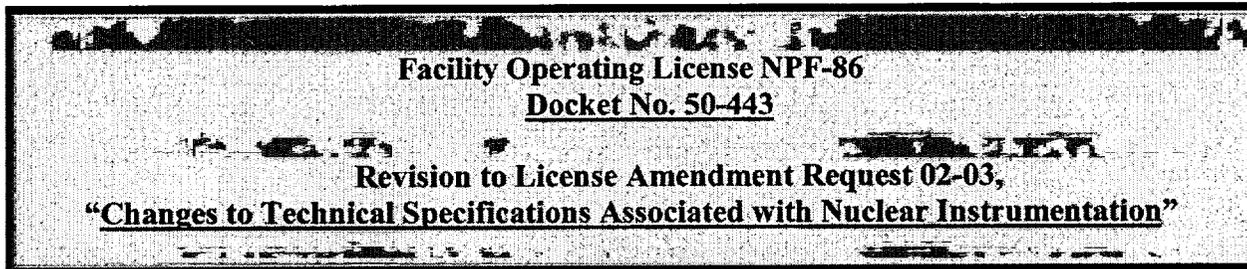
Mr. Donald Bliss, Director  
New Hampshire Office of Emergency Management  
State Office Park South  
107 Pleasant Street  
Concord, NH 03301



**FPL Energy**

**Seabrook Station**

**SEABROOK STATION UNIT 1**



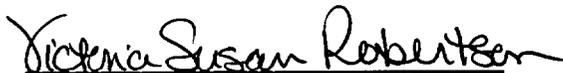
FPL Energy Seabrook, LLC submits this revision to License Amendment Request 02-03 pursuant to 10CFR50.90. The following information is enclosed in support of this revised License Amendment Request:

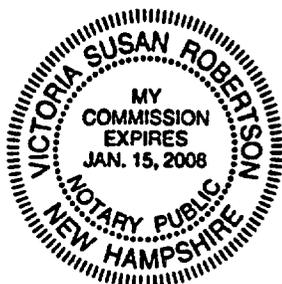
- Markup and Retype of the Proposed Change

I, Gene F. St. Pierre, Station Director of FPL Energy Seabrook, LLC hereby affirm that the information and statements contained within this License Amendment Request are based on facts and circumstances which are true and accurate to the best of my knowledge and belief.

  
Gene F. St. Pierre  
Station Director

Sworn and Subscribed  
before me this  
21<sup>st</sup> day of April, 2003

  
Notary Public



### **MARKUP AND RETYPE PAGES**

The attached markup and retype pages reflect the currently issued revision of the Technical Specifications listed below. Pending Technical Specifications or Technical Specification changes issued subsequent to this submittal are not reflected in the enclosed markup and retype pages. The enclosed retype should be checked for continuity with Technical Specifications prior to issuance.

The following Technical Specifications and Bases are included in the attached markups:

<b>Technical Specification</b>	<b>Title</b>	<b>Page(s)</b>
Table 4.3-1	Reactor Trip System Instrumentation Surveillance Requirements	3/4 3-10
Table 4.3-1 Table Notations	Reactor Trip System Instrumentation Surveillance Requirements	3/4 3-12

SEABROOK - UNIT 1

3/4 3-10

Amendment No. 36

TABLE 4.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERATIONAL TEST</u>	<u>ACTUATION LOGIC TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
13. Steam Generator Water Level-- Low-Low	S	R	Q	N.A.	N.A.	1, 2
14. Undervoltage - Reactor Coolant Pumps	N.A.	R	N.A.	Q	N.A.	1
15. Underfrequency - Reactor Coolant Pumps	N.A.	R	N.A.	Q	N.A.	1
16. Turbine Trip						
a. Low Fluid Oil Pressure	N.A.	R	N.A.	S/U( <sup>8</sup> Y, 10)	N.A.	1
b. Turbine Stop Valve	N.A.	R	N.A.	S/U( <sup>8</sup> Y, 10)	N.A.	1
17. Safety Injection Input from ESF	N.A.	N.A.	N.A.	R	N.A.	1, 2
18. Reactor Trip System Interlocks						
a. Intermediate Range Neutron Flux, P-6	N.A.	R(4)	R	N.A.	N.A.	2**
b. Low Power Reactor Trips Block, P-7	N.A.	R(4)	R	N.A.	N.A.	1
c. Power Range Neutron Flux, P-8	N.A.	R(4)	R	N.A.	N.A.	1
d. Power Range Neutron Flux, P-9	N.A.	R(4)	R	N.A.	N.A.	1

1  
1  
1

ROTATE TO  
 → HERE

TABLE 4.3-1 (Continued)

TABLE NOTATIONS

\*Only if the Reactor Trip System breakers happen to be closed and the Control Rod Drive System is capable of rod withdrawal.

\*\*Below P-6 (Intermediate Range Neutron Flux Interlock) Setpoint.

\*\*\*Below P-10 (Low Setpoint Power Range Neutron Flux Interlock) Setpoint.

- (1) If not performed in previous <sup>92</sup>31 days.
- (2) Comparison of calorimetric to excore power indication above 15% of RATED THERMAL POWER. Adjust excore channel gains consistent with calorimetric power if absolute difference is greater than 2%. The provisions of Specification 4.0.4 are not applicable to entry into MODE 2 or 1.
- (3) Single point comparison of incore to excore AXIAL FLUX DIFFERENCE above 50% of RATED THERMAL POWER. Recalibrate if the absolute difference is greater than or equal to 3%. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1. For the purposes of this surveillance requirement, monthly shall mean at least once per 31 EFPD.
- (4) Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (5) Initial plateau curves shall be measured for each detector. Subsequent plateau curves shall be obtained, evaluated and compared to the initial curves. For the Intermediate Range and Power Range Neutron Flux channels the provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.
- (6) Incore - Excore Calibration, above 75% of RATED THERMAL POWER. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1. For the purposes of this surveillance requirement, quarterly shall mean at least once per 92 EFPD.
- (7) Each train shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (8) ~~(Not used)~~ IF NOT PERFORMED IN PREVIOUS 31 DAYS.
- (9) Surveillance in MODES 3\*, 4\*, and 5\* shall also include verification that permissives P-6 and P-10 are in their required state for existing plant conditions by observation of the permissive annunciator window.
- (10) Setpoint verification is not applicable.
- (11) The TRIP ACTUATING DEVICE OPERATIONAL TEST shall independently verify the OPERABILITY of the undervoltage and shunt trip attachments of the Reactor Trip Breakers.

TABLE 4.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERATIONAL TEST</u>	<u>ACTUATION LOGIC TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
13. Steam Generator Water Level-- Low-Low	S	R	Q	N.A.	N.A.	1, 2
14. Undervoltage - Reactor Coolant Pumps	N.A.	R	N.A.	Q	N.A.	1
15. Underfrequency - Reactor Coolant Pumps	N.A.	R	N.A.	Q	N.A.	1
16. Turbine Trip						
a. Low Fluid Oil Pressure	N.A.	R	N.A.	S/U(8, 10)	N.A.	1
b. Turbine Stop Valve	N.A.	R	N.A.	S/U(8, 10)	N.A.	1
17. Safety Injection Input from ESF	N.A.	N.A.	N.A.	R	N.A.	1, 2
18. Reactor Trip System Interlocks						
a. Intermediate Range Neutron Flux, P-6	N.A.	R(4)	R	N.A.	N.A.	2**
b. Low Power Reactor Trips Block, P-7	N.A.	R(4)	R	N.A.	N.A.	1
c. Power Range Neutron Flux, P-8	N.A.	R(4)	R	N.A.	N.A.	1
d. Power Range Neutron Flux, P-9	N.A.	R(4)	R	N.A.	N.A.	1

TABLE 4.3-1 (Continued)

TABLE NOTATIONS

\*Only if the Reactor Trip System breakers happen to be closed and the Control Rod Drive System is capable of rod withdrawal.

\*\*Below P-6 (Intermediate Range Neutron Flux Interlock) Setpoint.

\*\*\*Below P-10 (Low Setpoint Power Range Neutron Flux Interlock) Setpoint.

- (1) If not performed in previous 92 days.
- (2) Comparison of calorimetric to excore power indication above 15% of RATED THERMAL POWER. Adjust excore channel gains consistent with calorimetric power if absolute difference is greater than 2%. The provisions of Specification 4.0.4 are not applicable to entry into MODE 2 or 1.
- (3) Single point comparison of incore to excore AXIAL FLUX DIFFERENCE above 50% of RATED THERMAL POWER. Recalibrate if the absolute difference is greater than or equal to 3%. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1. For the purposes of this surveillance requirement, monthly shall mean at least once per 31 EFPD.
- (4) Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (5) Initial plateau curves shall be measured for each detector. Subsequent plateau curves shall be obtained, evaluated and compared to the initial curves. For the Intermediate Range and Power Range Neutron Flux channels the provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.
- (6) Incore - Excore Calibration, above 75% of RATED THERMAL POWER. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1. For the purposes of this surveillance requirement, quarterly shall mean at least once per 92 EFPD.
- (7) Each train shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (8) If not performed in previous 31 days.
- (9) Surveillance in MODES 3\*, 4\*, and 5\* shall also include verification that permissives P-6 and P-10 are in their required state for existing plant conditions by observation of the permissive annunciator window.
- (10) Setpoint verification is not applicable.
- (11) The TRIP ACTUATING DEVICE OPERATIONAL TEST shall independently verify the OPERABILITY of the undervoltage and shunt trip attachments of the Reactor Trip Breakers.