Appendix	D		Scenario Outline	Form ES-D-1
Facility:	IP2		Scenario No.: 1 Op Test No.:	1
Examiner	s:		Candidates:	CRS
<u></u>				RO
	_ <u>,</u>			PO
Initial Cor	nditions: 1	00% power	MOL	
		21 Charging I		
		21 CCW Pum	•	
			be Leak < 5 GPD	
<u>Turnover:</u>		Reduce load vithin 60 min	to 900 MWe to remove 23 Condensate Pump fi utes	rom service
Critical Ta	asks: N	Aanual reacte	or trip	
		nitiate Bleed	and Feed Cooling	
Event	Malf.	Event		
No.	No.	Type*	Event Description	
1		R (RO)	Reduce power	
		N (BOP)		
		N (CRS)		
2	CNH	C (RO)	MFRV fails closed slowly in AUTO	
	PCS8D	C (CRS)		
3	XMT RCS20A	I (ALL)	Pressurizer level channel fails high	
4	ATS7B	C (ALL)	Feedwater Pump trip requiring rapid load decreas	se to 700 MWe
5	ATS7A	M (ALL)	Feedwater pump trip. Reactor trip required.	
6	BAT ESR.FAIL. RX.TRIP	C (RO)	Auto reactor trip failure. Manual trip required	
7	MOC AFW1	C (BOP)	21 MDAFW fails to start	
8	MOT AFW1	C (BOP)	21 MDAFW trips	
9	ATS5C	· ·	TDAFW trips	

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(N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Scenario Event Description IP2 NRC Simulator Scenario 1

The crew assumes the shift and initiates a power reduction IAW POP-3.1. The RO will commence RCS boration and the BOP will slowly reduce generator load.

23MFRV controller fails in automatic. The controller must be placed in manual IAW AOI 28.0 and/or AOI 21.1.1, and 23 SG level restored to the normal control band. The CRS will refer to Tech Specs.

Pressurizer level channel 460 (controlling channel) will fail high. The crew will respond IAW AOI-28.0 and AOI-28.7. The RO will operate charging pumps and pressurizer heaters manually while the BOP defeats the failed channel inputs and the CRS refers to Technical Specifications.

Subsequently, 22 MBFP will trip, requiring a plant runback to <745 MWe IAW AOI-21.1.1. 23 SG level must be controlled manually and normal boration will be performed for AFD control. If Rod Insertion Limits are exceeded, the RO will commence Emergency Boration.

When the plant is stabilized, 21 MBFP will trip, requiring a reactor trip. The reactor must be tripped manually IAW AOI-21.1.1, because automatic reactor trip is not functional.

Subsequent AFW failures result in the requirement to transition to FR-H.1, and restore Heat Sink using Bleed and Feed.

EOP flow path: E-0 - ES-0.1 - FR-H.1

Indian Point Unit 2 2003 NRC Initial License Examination Simulator Scenario Setup Scenario 1

21 Charging Pump OOS:

LOA EPS10 RACKED OUT Place pump control switch in TPO

21 CCW Pump OOS:

LOA EPS13 RACKED OUT Place pump control switch in TPO

AUTO reactor trip failure:

21 ABFP fail to auto start:

23 ABFP trips upon starting:

Run Batch File ESR.FAIL.RX.TRIP

MOC AFW1 OPTION 4 AUTO CLOSE FAILURE

MOT AFW2A OPTION 2 AFW 23 SHAFT SEIZURE

Materials needed for scenario:

- POP-3.1
- Graph Book
- Tags for tagged equipment
- Reactivity Summary Sheet

Allow crew to begin scenario brief approximately 30 minutes prior to entering simulator

Note: None

Scenario built from IC 3

Appendix D)		Operator Action						Form ES-D-2			
Op Test No.:	1		Scenario #		Event #			Page	5	of	29	
Event Descrip	otion:		Reduce Power									
Time Position Applicant's Actions or Behavior												

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CRS	Refers to POP 3.1
CRS	IF reducing Reactor Power for a maintenance support function, the SM SHALL DETERMINE the desired Reactor Power level OR Turbine load (MWe) to maintain while repairs are made/troubleshooting is performed Evaluator Note: 900 MWe indicated on turnover
CRS	REQUEST Test Group to determine if Pressurizer Level instrumentation must be re-calibrated. Evaluator cue: If asked, no re-calibration necessary
CRS	VERIFY LCV-1129, Excess Condensate Return to CST, is closed, <u>AND</u> isolated locally per SOP 20.2, Condensate System Operation.
i i i i i i i i i i i i i i i i i i i	
отс	BORATE per SOP 3.2, Reactor Coolant System Boron Concentration Control, as necessary to maintain control banks above insertion limits required by GRAPH RPC-6, Cycle 14 Core Operating Limits Report.
RO	IF necessary, PLACE rod control in MANUAL to maintain rods above the Insertion Limit.
RO	MAINTAIN delta flux within the target band.

Appendix [)		Operator Action					
Op Test No.:	_1	Scenario #		Event #	1	Page	<u>6</u> of	29
Event Descrip	otion:	Reduce Power						
Time Position Applicant's Actions or Behavior								

CREW	 IF PICS is NOT operable, PERFORM the following after load changes greater than 10% per AOI 29.12, Loss of PICS Computer: Quadrant Power Tilt Calculation using DSR-4B, Quadrant Power Tilt Calculation Sheet (Technical Specification 3.10.10). Log individual rod position indications using DSR-3, Rod Position Verification Log Sheet (Technical Specification 3.10.9).
BOP	MONITOR condenser sextants for sodium increase.
RO	VERIFY T _{AVE} <u>AND</u> Pressurizer Level are maintained on program per Graph RCS-2, Pressurizer Level V.S. T _{AVE} .
· RO	MAINTAIN steam generator levels between 40 and 50 percent Narrow Range.
RO	NULL manual setpoint on feedwater regulating valve control to facilitate rapid transfer from AUTOMATIC to MANUAL control.
CRS	NOTIFY nuclear and conventional NPOs that load reduction is in progress.

Appendix D)		Form ES-D-2							
Op Test No.:	_ 1	Scenario #	_1	Event #	_1		Page	7	of	29
Event Descrip	otion:	Reduce Power								
Time	Time Position			Applica	nt's Actio	ons or Beha	avior	<u></u>		

CRS	 DIRECT NPOs to perform the following during load reduction: MONITOR Main Turbine Oil Temperatures MONITOR Hydrogen Seal Oil Temperatures MONITOR MBFP Oil Temperatures BALANCE Heater Drain Tank Pump flows between the pump in Auto and Manual per SOP 19.1, Extraction Steam And Heater Drain Systems Operation. IF SJAEs are in service, MAINTAIN Steam pressure per SOP 20.1, Condenser Air Removal System Operation, <u>AND</u> periodically CHECK SJAEs for backfiring. IF FCV-1120, Flowpath A Controller Stop, is in MANUAL, ADJUST to maintain FCV-1113, Gland Steam Condenser Minimum Flow Control Valve, closed.
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<u>NOTE</u> <u>WHEN</u> erratic governor operation is observed, governor oil pressure may be raised above the controlling load limit to avoid adverse Main Turbine operation.

BOP	INITIATE load decrease using either of the following as directed by CRS: o Governor control o Load Limit control
отс	Refers to SOP-3.2 for boration
OTC	 DETERMINE RCS Boron concentration from reactor coolant sample analysis. o IF analysis following concentration adjustment is <u>NOT</u> yet available, ESTIMATE Boron concentration based on latest readings.

Appendix E)		Operator Action				Form ES-D-2				
Op Test No.:	1	Scenario #	1	Event #	1	Page	8	of	29		
Event Descrip	otion:	Reduce Power	, <u> </u>	-				-			
Time	Position		Applicant's Actions or Behavior								

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	<u>NOTE</u> be based upon rod position, RCS Boron concentration, Xenon ental Boron and rod worth, and operating experience.
OTC	 DETERMINE magnitude of increase in Boron concentration necessary to accomplish desired reactivity change using one or more of the following references: GRAPH CVCS-3A, Boration Nomograph for Hot RCS GRAPH CVCS-3B, Boration Nomograph for Cold RCS GRAPH CVCS-6, Boration - Dilution Tables GRAPH RV-2, Total Power Defect (PCM) as a Function of Power and Boron Concentration at MOL GRAPH RV-3, Differential Boron Worth (Hot Zero Power) at MOL GRAPH RV-4, Total Temperature Defect (PCM) as a Function of Temperature and Boron Concentration at MOL GRAPH RV-9, IP2 Cycle 15 Reactivity Equivalents POP 1.2, Reactor Startup WCR 1, Reactivity Summary
отс	IF the change in RCS Boron concentration is anticipated to be greater than or equal to 25 ppm, OPERATE Pressurizer heaters to open spray valve.
отс	ESTIMATE total volume of boron required for boration from boration graphs or references listed in step 4.5.2.
отс	PLACE the RCS Makeup Control switch to STOP.
отс	SET Boric Acid Integrator to amount determined in step 4.5.3.

Appendix	D	Operator Action							Form ES-D-2			
Op Test No.:	1	Scenario #		Event #	_1		Page	9	of	29		
Event Descri	ption:	Reduce Powe	r									
Time Position Applicant's Actions or Behavior												

PLACE the RCS Makeup Mode Selector switch to BORATE.
FCV-110A, Boric Acid Flow Control, may be left in AUTO or placed in MANUAL as directed by CRS.
VERIFY boric acid transfer pumps are in AUTO.
PLACE the RCS Makeup Control switch to START.

NOTE BATPs shifting to fast speed is verified by both counter operation and by the respective fast speed lights illuminating.

ОТС	VERIFY BATPs shift to fast speed.
отс	IF in manual, ADJUST FCV-110A, Boric Acid Flow Control, to obtain desired boric acid flow rate (may be greater than meter range).
	<u>IF</u> desired to maximize Boron flow, CLOSE the appropriate BATP recirculation valve.
отс	 HCV-104 CVCS/Boric Acid Tank 22 BA Inlet
	 HCV-105 CVCS/Boric Acid Tank 21 BA Inlet

Appendix D			Ор	erator Actio		Form ES-D-2		
Op Test No.:		_ Scenario #	_1	_ Event #	1	Page	<u>10</u> of	29
Event Descri	otion:	Reduce Powe	۶r					
Time	Positio	n	Applicant's Actions or Behavior					

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отс	MONITOR Nuclear Instrumentation, Rod position, and RCS temperature closely during any reactivity changes.
отс	 WHEN boration operation has been completed, PERFORM the following: PLACE the RCS Makeup Control switch to STOP. PLACE the Makeup Mode Selector switch to MANUAL. ADJUST FCV-110A, Boric Acid Flow Control, dial setting to the new RCS Boron concentration per applicable CVCS Graph: GRAPH CVCS-1A, Blended Makeup - (0-500) with 120 Gpm PW GRAPH CVCS-1B, Blended Makeup - (0-2000) with 120 Gpm PW GRAPH CVCS-1C, Blended Makeup with Various PW Flows
отс	VERIFY FCV-110A control switch in AUTO.
отс	PLACE the RCS Makeup Control switch to START.
отс	VERIFY approximately 30 gallons of blended makeup flows through blender.
ОТС	PLACE RCS Makeup Control switch to STOP.
ОТС	SELECT AUTO on RCS Makeup Mode Selector switch.

Appendix [)		Ope	erator Actic	n		Form ES-D		
Op Test No.:		Scenario #	_1	Event #	_1	Page	<u>11</u> of	29	
Event Descrip	otion:	Reduce Powe	r						
Time	Position		Applicant's Actions or Behavior						

отс	PLACE the RCS Makeup Control switch to START.
CRS	IF Reactor is shutdown, REQUEST a sample for RCS Boron concentration within 30 minutes of completing the boration.
отс	LOG amount of boric acid added to system for boration in CCR Log.

At Lead Evaluator's discretion, proceed to Event 2

Appendix [)		Operator Action					Form ES-D-2		
Op Test No.:		Scenario #		_ Event #	2		Page	12	of	29
Event Descri	ption:	MFRV Fails C	losed S	Slowly						
Time	Position		Applicant's Actions or Behavior							

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Booth Ins CNH PCS	tructor: Whe	n directed, insert the following command: TART 65 RAMP TIME 300 SEVERITY 25
	CRS	Refer to AOI 28.0, Instrument Failures, or 21.1.1, Loss of Feedwater
		Note: The following 2 steps are the actions of AOI-28.0
	отс	VERIFY The Following Controls: Turbine load - STABLE Rod Control - STABLE PRZR pressure control - NORMAL PRZR level control - NORMAL MBFP Speed - NORMAL S/G levels - NORMAL
	ОТС	Place 23 MFRV in MANUAL and restore SG level
		Note: The following steps are the actions of AOI-21.1.1. The crew may use either procedure to stabilize level prior to diagnosing the failure
	CRS	IF Main Feed Regulator Valve(MFRV) has failed, GO TO Section 5.7
		· ·
	отс	TRANSFER failed MFRV to MANUAL and RESTORE normal level
	1	

Appendix E	D	·····	Operator Action Form ES-D-						S-D-2	
Op Test No.: Event Descrip		Scenario # MFRV Fails 0	_1 Closed S	Event #	2		Page	<u>13</u>	of	29
Time	Position		Applicant's Actions or Behavior							

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отс	POSITION MBFP recirculation valve switch at the direction of the CRS o FCV-1115 21 MBFP Recirc VIv o FCV-1116 22 MBFP Recirc VIv
CRS	IF SG Levels can be stabilized, GO TO step 5.13 to stabilize plant
CREW	ADJUST TURBINE LOAD AS DIRECTED BY THE SM: VERIFY that both PORV block valves (MOV-535 and MOV- 536) are closed. VERIFY that the MBFP recirculation valve switches are in AUTO • FCV-1115 21 MBFP Recirc VIv • FCV-1116 22 MBFP Recirc VIv ESTABLISH plant conditions as directed by the SM: • SHUT DOWN the Reactor per POP 3.1, Plant Shutdown from Full Power Operation to Zero Power Condition

Appendix [)		Ope	erator Actio		Form E	S-D-2	
Op Test No.:	1	Scenario #	1	Event #	3	Page	<u>14</u> of	29
Event Descri	otion:	Pressurizer L	evel Ch	annel Fails H	igh	·		
Time	Position			Applica	nt's Actions	or Behavior		

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	en directed, insert the following command: RITY 100 RAMP TIME 180						
CRS	Refer to AOI-29.0, Instrument Failures						
отс	VERIFY The Following Controls: • Turbine load - STABLE • Rod Control - STABLE • PRZR pressure control - NORMAL • PRZR level control - NORMAL • MBFP Speed - NORMAL • S/G levels – NORMAL						
отс	Place Charging Pump Speed control in manual						
CRS	CHECK PRZR instrumentation – NORMAL • PRZR levels • GO to AOI 28.7 (Fails High), PRZR LEVEL CHANNEL FAILURE						
отс	 IF failed Channel is controlling Pressurizer level TURN OFF any unnecessary Pressurizer back-up heaters PLACE charging pump speed control in MANUAL CONTROL Pressurizer level in normal band. (Refer to Graph, RCS-2, PRESSURIZER LEVEL vs. TAVE, in Graph Book). 						

Appendix D)		Operator Action					S-D-2
Op Test No.:		Scenario #	_1	Event #	3	Page	<u>15</u> of	29
Event Description: Pressurizer Level Channel Fails High								
Time	Position		Applicant's Actions or Behavior					

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	BOP	In Foxboro rack B6, PLACE Pressurizer Level Defeat switch (L 460A) to DEFEAT #, for affected channel
	отс	RETURN charging pump speed control to AUTOMATIC
	отс	RETURN Pressurizer bulk heater control to AUTOMATIC <u>OR</u> MANUAL as directed by SOP 1.4, Pressurizer Pressure Control
	POP	PLACE Hi Level Trip Bistable Trip switch for affected channel in TRIP
·	BOP	o 460, Rack A-12, WHITE
		VEDIEV as a viscourse of Technical Constitution in Table 2.5.2
	CRS	VERIFY requirements of Technical Specification in Table 3.5-2 are met.

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When Channel is removed from service, Proceed to Event 4

Appendix [)		Оре	erator Actio	on		For	mΕ	S-D-2
·····							······································		<u></u>
Op Test No.:	1	Scenario #		Event #	4	Page	16	of	29
Event Descri	ption:	Feedwater Pu	mp Trip	Requiring F	apid Load	Decrease to 700	MWe		
Time	Position			Applica	nt's Action	s or Behavior			

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		· · ·
	CRS	Refer to AOI-21.1.1, Loss of Feedwater
	CRS	IF one MBFP has tripped, GO TO Section 5.3
	CREW	IF Turbine Load greater than 745 MWe, VERIFY automatic turbine runback has reduce Turbine Load to approximately 74 MWe • IF NOT, REDUCE Turbine Load to approximately 745 MWe using Load Limits (preferred) or Governor
he recircul	lation valve OTC	NOTE should remain closed UNTIL load has been stabilized VERIFY the Recirculation Valve for tripped MBFP is closed
he recircul		should remain closed <u>UNTIL</u> load has been stabilized
he recircul		should remain closed <u>UNTIL</u> load has been stabilized
he recircul	OTC	should remain closed <u>UNTIL</u> load has been stabilized VERIFY the Recirculation Valve for tripped MBFP is closed <u>IF</u> necessary, ADJUST turbine load to MATCH Steam flows to Feedwater flows and MAINTAIN Steam Generator levels on
	отс	should remain closed <u>UNTIL</u> load has been stabilized VERIFY the Recirculation Valve for tripped MBFP is closed <u>IF</u> necessary, ADJUST turbine load to MATCH Steam flows to Feedwater flows and MAINTAIN Steam Generator levels on
	отс	should remain closed <u>UNTIL</u> load has been stabilized VERIFY the Recirculation Valve for tripped MBFP is closed <u>IF</u> necessary, ADJUST turbine load to MATCH Steam flow Feedwater flows and MAINTAIN Steam Generator levels or program

Appendix [)		Оре	erator Actio	on		Form	ES-D-2
					<u></u>	·		·····
Op Test No.:	1	Scenario #	_1	_ Event #		Page	<u>17</u> c	of <u>29</u>
Event Descrip	otion:	Feedwater Pu	ımp Trip	Requiring R	apid Load I	Decrease to 700 N	ИWe	
Time	Positio	n		Applica	nt's Actions	or Behavior		

	VERIFY control rods are in AUTO
отс	VERIFY control rods are in AUTO
отс	IF rods are <u>NOT</u> responding, manually DRIVE rods in to control temperature
BOP	VERIFY that the standby condensate pump is running
BOP	VERIFY that 21 and 23 AFW Pumps are running
BOP	VERIFY that the SGBD Isolation Valves are closed
	VERIFY that the turbine runback has terminated, as follows
отс	 Load Limit oil pressure stable TURBINE RUNBACK ACTUATED alarm on Panel SEF(Window 3-8) extinguished
отс	<u>IF</u> the runback has <u>NOT</u> terminated, PLACE the Loss of MBFP Turbine Runback Control Switch in DEFEAT (Panel FAF).
When plant is stable,	or at Lead Evaluator's discretion, proceed to Event 5

Appendix [)		Oper	ator Actio	on		Form	n ES	-D-2
Op Test No.:	_1	Scenario #	_1	Event #	5, 6, 7, 8, 9	Page	18	of _	29
Event Descrip	otion:				rip Required. Auto rips; MDAFW Fails				s.
Time	Position			Applica	nt's Actions or Beh	avior			·····

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Booth Instructor: When directed, insert the following command: MAL ATS7A MOT AFW1A OPTION 2 SHAFT SEIZURE CRS May refer to AOI-21.1.1 CRS Direct entry to E-0, Reactor Trip or Safety Injection Verify reactor trip:

отс	 Verify reactor trip: Trip breakers open Flux decreasing Rod bottom lights lit Rod position indicators all less than 7.5 inches
отс	 Verify Turbine Trip All stop valves closed
BOP	 Verify power to 480 Volt busses: At least one energized: 2A and 3A 5A 6A All Energized

Appendix D)		Oper	ator Actio	on		Form	ES-D-2
Op Test No.:	_1	Scenario #	_1	Event #	5, 6, 7, 8, 9	Page	<u>19</u> of	f <u>29</u>
Event Descrip	otion:	Feedwater P Manual Trip I	ump Trip. Required.	Reactor T TDAFW T	rip Required. Aut rips; MDAFW Fail	o Reactor T s to Start; N	rip Failur MDAFW T	e. rips.
Time	Position	- 1		Applica	ant's Actions or Be	havior		

OTC/BOP	Check if SI is actuated: o SI annunciator lit OR o Any SI pumps running
 CREW	Check if SI is required (NO)
CRS	GO to ES-0.1 Reactor Trip Response
отс	Check RCS temperature stable at or trending to 547°F
OTC	Check Generator Output breaker open
BOP	Verify 480 volt busses energized by offsite power
отс	 Check pressurizer level control: Level greater than 18% Charging and Letdown in service Any CCW pump running Level trending to 37%
отс	 Check pressurizer pressure control: o Pressure greater than 1840 psig o Trending to 2235 psig

Appendix E)		Opera	ator Action	on		Form	ES-D-2
Op Test No.:	_1	Scenario #		Event #	5, 6, 7, 8, 9	Page	<u>20</u> of	29
Event Descrip	otion:	Feedwater Pr Manual Trip F	ump Trip. Required.	Reactor T TDAFW T	rip Required. Auto rips; MDAFW Fails	Reactor T to Start; N	rip Failure /IDAFW T	e. rips.
Time	Position	<u> </u>		Applica	ant's Actions or Beh	avior		

Booth Instructor:	When directed, insert the following command:
MAL ATS5C	

Determine Red Path exists on Heat Sink CSF status tree. Direct transition to FR-H.1, Loss of Heat Sink
Check if secondary heat sink is required:
 RCS pressure greater than any non-faulted SG pressure
 RCS temperature greater than 350°F
-

CAUTION

- If average of the 3 lowest wide range SG levels is less than 41% (54% Adverse containment) due to loss of secondary heat sink, RCPs should be tripped and steps 9 15 should be immediately initiated for bleed and feed
- City Water for AFW pumps will be necessary if CST level decreases to less than 2 ft
- Radiation levels and harsh environment conditions should be evaluated prior to performing local actions

CREW	Determine Bleed and Feed criteria is met					
отс	Trip RCPs					
	Actuate SI					
OTC/BOP	o Train A o Train B					

Appendix [2	Operator Action					Form ES-D-2			
Op Test No.:	_1	Scenario #	_ <u>1</u> _E	Event #	5, 6, 7, 8, 9	Page	21	of	29	
Event Descri	ption:				rip Required. Auto rips; MDAFW Fails				DS.	
Time	Position			Applica	ant's Actions or Beha	avior				

OTC/BOP	 Verify RCS Feed path Check All SI pumps running 22 SI pump discharge isolation MOV-851A and MOV- 851B open Check proper emergency SI valve alignment for operating pumps: SI pump cold leg injection valves open RHR HX CCW outlet valves open RHR HX Motor operated valves open
	 Check feed path established

- CAUTION
 If offsite power is lost after SI reset, then manual action may be required to restart safeguards equipment
 Placing key switches to DEFEAT will prevent auto SI actuation

	 o Check all CCW pumps running
OTC/BOP	 Place controls for main and bypass feedwater regulating valves to close Verify automatic safeguards actuation key switches on panel SB-2 in DEFEAT position Train A SIA-1 Train B SIA-2 One at a time, depress safety injection reset buttons Train A Train B Verify Train A and B reset

	Appendix D	<u></u>	Operator Action					Form ES-D-2			
.	Op Test No.:	15	Scenario #		Event #	5, 6, 7, 8, 9	Page	<u>22</u> of	29		
· · ·	Event Description:	F N	Feedwater Pu Manual Trip R	imp Trip. Required.	Reactor Tr TDAFW Tr	ip Required. Auto ips; MDAFW Fails	Reactor T to Start; M	rip Failure IDAFW Tr	ips.		
	Time F	Position			Applica	nt's Actions or Beh	avior				

	BOI	 Reset Containment isolation phase A and Phase B Place IVSW switches to open on SN panel Place CNTMT RAD MON WCPS valves control switch to open on SN panel Verify personnel and equipment hatch solenoid control switches to incident on SN panel Place control switches for all remaining Phase A isolation valves to close on SN panel One at a time, depress Phase A reset pushbuttons CI Phase A Train A CI Phase A Train B Verify Train A and B reset
	BOI	Check Phase B actuated (NO)
	BO	 Establish Instrument Air to Containment Open PCV-128
· .	OT	 Establish RCS Bleed Path Verify power to PRZR PORV Block Valves available Verify PRZR POPRV Block Valves both open Open both PORVs Monitor PORV Nitrogen alarms
	OT	Verify adequate bleed path

Appendix [)	Operator Action					Form ES-D-		
Op Test No.:	_1	Scenario #		Event #	5, 6, 7, 8, 9	Page	<u>23</u> of	29	
Event Descrip	otion:				rip Required. Auto rips; MDAFW Fails				
Time	Position			Applica	ant's Actions or Be	havior			

 OTC/BOP	Perform steps 1-9 of E-0, Reactor Trip or Safety Injection

Terminate scenario when RCS Bleed and Feed is established and verified

Appendix D	·		Ор	erator Action	}			Form	ES-D-2
Op Test No.:		Scenario #	All	Event #	Attachment 1	Page	24	of	29
Event Descrij	otion:	Attachment 1,	Automa	atic Action V	erification				
Time	Position	1		Applica	nt's Actions or Beh	avior			

This attachment must AC Power.	<u>Note</u> be terminated upon CRS transition to ECA-0.0, Loss of All
	Verify proper Charging system operation:
BOP	 a. Start at least one charging pump in manual at maximum speed b. Align charging pump suction to the RWST Open charging pump suction valve from RWST LCV-112B Close charging pump suction valve from VCT LCV-112C Place RCS Makeup Control switch to STOP
· ·	Note

- Notify CRS of any automatic actions that failed to occur during performance of this attachment
- Equipment found misaligned due to operator action should NOT be repositioned.

BOP	Check generator output breakers – OPEN

	Appendix D		Operator Action					Form E		
	Op Test No.:		Scenario #	All	Event #	Attachment 1	_ Page	-25	of	29
_*	Event Description:		Attachment 1, Automatic Action Verification							
	Time	Position	1		Applica	ant's Actions or Beh	avior		. / . /	

вор	Check status of 480 volt busses: a. All 480V busses – ENERGIZED BY OFF- SITE POWER b. Dispatch NPO to reset: o All lighting o MCC 24A o MCC 27A
	o MCC 29A c. Stop all Condensate Pumps
· ·	
BOP	 Verify FW Isolation: Main Boiler Feed Pumps – TRIPPED Main Boiler Feed Pump Discharge Valves – CLOSED FW Regulating valves – CLOSED FW Stop Valves – CLOSED SG Blowdown Isolation Valves - CLOSED
BOP	Check if Main Steam Lines should be isolated: a. Check for either: • High Steam Line flow with EITHER Tave less than 541 deg F OR Steam line pressure less than 525 psig. OR • Containment pressure – EVER GREATER THAN 24 psig b. Verify MSIVs - CLOSED

	Appendix D		· · · · · · · · · · · · · · · · · · ·	Ор	erator Action)			Form	ES-D-2
-	Op Test No.:	1	Scenario #	All	Event #	Attachment 1	Page	26	of	29
	Event Descrip	ition:	Attachment 1	, Automa	atic Action V	erification				
	Time	Position			Applica	nt's Actions or Beha	avior	····	- *	••••••••••••••••••••••••••••••••••••••

		Verify proper Service Water System operation:
	BOP	 a. Three Service Water Pumps – Running on Essential Header b. Service Water valves from Diesel Generator - OPEN
		Verify SI system pumps running:
	BOP	 a. Three SI pumps – RUNNING b. 22 SI pump discharge isolation MOV-851A AND MOV-851B – OPEN c. Two RHR pumps - RUNNING
	BOP	Verify proper emergency SI System valve alignment: a. SI pump cold leg injection valves – OPEN o 856A o 856E o 856D b. RHR HX CCW outlet valves – OPEN o 822A o 822B c. RHR HX MOVs – OPEN o 746 o 747
		Verify Containment Fan Coolers – IN SERVICE:
	BOP	 a. Five fan coolers - RUNNING b. Charcoal Filter valves - OPEN c. Fan normal discharge valves – CLOSED d. TCV-1104 and TCV-1105 – BOTH OPEN
	BOP	Verify AFW flow to all SGs

Appendix D		· · · · · · · · · · · · · · · · · · ·	Op	erator Actior)			Form	ES-D-2
Op Test No.:	1	Scenario #	All	Event #	Attachment 1	Page	27	of	29
Event Descri	otion:	Attachment 1,	Automa	atic Action V	erification				
Time	Position			Applica	nt's Actions or Beha	avior			÷.,

BOP	 a. Containment Purge Valves – CLOSED FCV-1170 FCV-1171 FCV-1172 FCV-1173 b. Containment Pressure Relief Valves - CLOSED PCV-1190 PCV-1191 PCV-1192
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Appendix D		· · · · · · · · · · · · · · · · · · ·	Ор	erator Action				Form	ES-D-2
Op Test No.:		Scenario #	All	Event #	Attachment 1	Page	28	of	29
Event Descri	otion:	Attachment 1,	Automa	atic Action Ve	erification				
Time	Position	· · · · · · · · · · · · · · · · · · ·		Applica	nt's Actions or Beha	avior		<u></u>	<u>á</u>

	nment Isolation Phase A
b. c. d. BOP	Phase A – ACTUATED • Train A master relay CA1 (above rack E) • Train B master relay CA2 (above rack F) Phase A valves – CLOSED IVSW valves – OPEN • 1410 • 1413 • SOV-3518 • SOV-3519 WCP valves – OPEN: • PCV 1238 • PCV 1238 • PCV 1239 • PCV 1240 • PCV 1240 • PCV 1241 Place personnel and equipment hatch solenoid control switches to INCIDENT on SM panel Dispatch NPO to periodically check • IVSW Tank • Level – GREATER THAN 92% • Pressure – GREATER THAN 57 PSIG • WCP header pressures – GREATER THAN 52 PSIG

Evaluator note: The following step is intended for High Containment pressure condition. It will not be performed if conditions aren't met

Appendix D			Оре	erator Action	<u>۱</u>			Form	ES-D-2
Op Test No.:	: <u>1</u>	Scenario #	All	Event #	Attachment 1	Page	29	of	29
Event Descri	iption:	Attachment 1,	Automa	tic Action V	erification				
Time	Position		Applicant's Actions or Behavior						

BOP	 a. Containment Pressure – EVER GREATER THAN 24 PSIG b. Verify spray pumps – RUNNING c. Verify spray pump discharge valves – OPEN MOV-866A MOV-866B MOV-866D d. Verify Containment Isolation Phase B valves CLOSED STOP all RCPs f. Verify IVSW Isolation Valves – OPEN 7864 7865 7866 7867
BOP	Verify CCR Air Conditioner Train A and B – RUNNING IN INCIDENT MODE 2
BOP	Notify CRS that Attachment 1 is complete

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Appendix	D	·····	Scenario Outline	Form ES-D-1
Facility: Examiner	IP2		Scenario No.: 2 Op Tes Candidates:	st No.: 1 CRS
Examinen	J	· · · · · · · · · · · · · · · · · · ·		RO
				R0
Initial Con	ditions:	100% power N	IOL	
		21 Charging P	ump OOS	
		21 CCW Pum	OOS	
		Small SG Tub	e Leak < 5 GPD	
<u>Turnover:</u>			ture disc is leaking. Reduce Power to emove Main Turbine and Generator fro	
Critical Ta	<u>isks:</u>	Manual Turbin	e Trip	
		Initiate Emerge	ency Boration	
Event No.	Malf. No.	Event Type*	Event Description	on
1		R (RO) N (BOP) N (CRS)	Reduce load	
2	XMT MSS054A	I (CRS)	First Stage Shell Pressure PT-412B fails I	ow
3	CCW1 CCW2 CCW3	C (BOP/CRS)	CCW Pump Trip. Standby does not autor	natically start
4	CVC6 CCW8 RCP7A	C (ALL)	RCP TBHX leak. RCP vibration	
5	RCP21 PPL3-4	M (ALL)	RCP sheared shaft; ATWS	un 1999 - 1997 -
6	TCA1-6 PPL43-48	C (RO)	Turbine Trip failure	
7	CVC9	C (ALL)	Boration failure	

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* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Scenario Event Description IP2 Simulator Scenario 2

The crew will assume the shift and initiate a load decrease in accordance with POP-3.1.

First Stage Shell Pressure transmitter PT-412B will fail low. The crew will place steam dumps in Pressure Control Mode and trip SI steam flow bistables IAW AOI-28.0 and AOI-28.14. The CRS will refer to Technical Specifications.

A running CCW pump will trip. The standby pump will have to be manually started. The crew will respond IAW AOI-4.1.1. Subsequently, a TBHX leak will develop, and action to isolate the TBHX leak will be taken IAW AOI-4.1.2.

During the TBHX tube leak, RCP vibration will rise, eventually resulting in failure of the RCP shaft. The crew will respond using AOI-1.3. A reactor trip will be required, but will not automatically occur.

The RO will attempt to manually trip the reactor, but the reactor will not trip. The turbine must be manually tripped, and emergency boration will fail, requiring an alternate method for emergency boration.

Safety Injection will actuate due to lowering RCS pressure. The crew will perform the necessary actions for SI actuation along with FR-S.1 action.

EOP flow path: E-0 - FR-S.1 - E-0 - ES-1.1

Indian Point Unit 2 2003 NRC Initial License Examination Simulator Scenario Setup Scenario 2

21 Charging Pump OOS:

LOA EPS10 1,0,D Place pump control switch in TPO

21 CCW Pump OOS:

LOA EPS13 1,0,D Place pump control switch in TPO

23 CCW Pump auto start fail:

Fail Reactor trip breakers as is:

Fail Rod Drive MG Set breaker as is:

MOC CCW3 Option 4

BKR PPL003 Option 5 BKR PPL004 Option 5

BKR CRF1 Option 5 BKR CRF2 Option 5 BKR EPS31 Option 5 BKR EPS32 Option 5

MOV-CVC9 Option 5

Fail MOV-333 Closed:

Fail Auto turbine trip:

Run Batch files

FAIL.AUTO.TURB.TRIPS.BAT MAN.TURB.TRIP.ENABLE.BAT

Materials needed for scenario:

- POP-3.1
- Graph Book
- Tags for tagged equipment
- Reactivity Summary Sheet

Allow crew to begin scenario brief approximately 30 minutes prior to entering simulator

Note: None

Scenario built from IC 2

Indian Point Unit 2 2003 NRC Initial License Examination Simulator Scenario Turnover Information Scenario 2

- The plant is at 100% power, steady state conditions exist.
- Middle of Life, C_b is 862 ppm.
- EFPD = 340
- Control Bank D = 220 steps
- Tavg = 559°F
- RCS Pressure = 2235 psig
- Pressurizer Level = 45%
- A small Steam Generator Tube Leak exists on 23 SG, less than 5 gallons per day.
- Risk Assessment = GREEN
- Daily Risk Factor = 0.94

The following equipment is out of service:

- 21 Charging Pump. Return expected in approximately 6 hours.
- 21 Component Cooling Water Pump. Return to service in approximately 8 hours.

Crew instructions:

- The Main Turbine 21 Rupture Disc is leaking approximately 12 SCFM.
- In accordance with POP-3.1, shut down at a rate of 200 MWe per hour and remove the Main Turbine and Generator from service.

Appendix D)		Operator Action				Form ES-D-2			
Op Test No.:	1	Scenario #	2	Event #		Page	_5	_ of	27	
Event Descrip	otion:	Reduce Load								
Time	Position			Applica	nt's Action	s or Behavior				

Refers to POP 3.1
IF reducing Reactor Power for a maintenance support function the SM SHALL DETERMINE the desired Reactor Power level <u>OR</u> Turbine load (MWe) to maintain while repairs are made/troubleshooting is performed
REQUEST Test Group to determine if Pressurizer Level instrumentation must be re-calibrated.
Note: Calibration not required
VERIFY LCV-1129, Excess Condensate Return to CST, is closed, <u>AND</u> isolated locally per SOP 20.2, Condensate System Operation.
BORATE per SOP 3.2, Reactor Coolant System Boron Concentration Control, as necessary to maintain control banks above insertion limits required by GRAPH RPC-6, Cycle 14 Core Operating Limits Report.
IF necessary, PLACE rod control in MANUAL to maintain rods above the Insertion Limit.
MAINTAIN delta flux within the target band.

Appendix	D		Operator Action					Form ES-D-2			
Op Test No.	: _1	Scenario #	2	Event #	_1	Page	6	of	27		
Event Descr	iption:	Reduce Load									
Time	Position	<u> </u>		Applica	int's Actions	or Behavior					

CREW	 IF PICS is <u>NOT</u> operable, PERFORM the following after load changes greater than 10% per AOI 29.12, Loss of PICS Computer: Quadrant Power Tilt Calculation using DSR-4B, Quadrant Power Tilt Calculation Sheet (Technical Specification 3.10.10). Log individual rod position indications using DSR-3, Rod Position Verification Log Sheet (Technical Specification 3.10.9).
 BOP	MONITOR condenser sextants for sodium increase.
отс	VERIFY T _{AVE} <u>AND</u> Pressurizer Level are maintained on program per Graph RCS-2, Pressurizer Level V.S. T _{AVE} .
отс	MAINTAIN steam generator levels between 40 and 50 percent Narrow Range.
отс	NULL manual setpoint on feedwater regulating valve control to facilitate rapid transfer from AUTOMATIC to MANUAL control.
CRS	NOTIFY nuclear and conventional NPOs that load reduction is in progress.

Appendix	D		Ор	erator Actio	on		Form ES-D-2			
Op Test No.	: <u>1</u>	Scenario #	2	Event #		Page	7	of	27	
Event Descr	iption:	Reduce Load								
Time Position				Annlica	int's Actio	ns or Behavior				

CRS	 DIRECT NPOs to perform the following during load reduction: MONITOR Main Turbine Oil Temperatures MONITOR Hydrogen Seal Oil Temperatures MONITOR MBFP Oil Temperatures BALANCE Heater Drain Tank Pump flows between the pump in Auto and Manual per SOP 19.1, Extraction Steam And Heater Drain Systems Operation.
	 IF SJAEs are in service, MAINTAIN Steam pressure per SOP 20.1, Condenser Air Removal System Operation, <u>AND</u> periodically CHECK SJAEs for backfiring. IF FCV-1120, Flowpath A Controller Stop, is in MANUAL, ADJUST to maintain FCV-1113, Gland Steam Condenser Minimum Flow Control Valve, closed.

<u>NOTE</u> <u>WHEN</u> erratic governor operation is observed, governor oil pressure may be raised above the controlling load limit to avoid adverse Main Turbine operation.

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отс	INITIATE load decrease using either of the following as directed by CRS: • Governor control • Load Limit control
отс	Refers to SOP-3.2 for boration
отс	 DETERMINE RCS Boron concentration from reactor coolant sample analysis. <u>IF</u> analysis following concentration adjustment is <u>NOT</u> yet available, ESTIMATE Boron concentration based on latest readings.

Appendix [)		Ope	erator Actio	on		Form ES-D-		
Op Test No.:	_1	Scenario #	2	Event #		Page	8	_ of	27
Event Descrip	otion:	Reduce Load							
Time	Position			Annlica	int's Actions	s or Behavior	<u></u>		

	<u>NOTE</u> be based upon rod position, RCS Boron concentration, Xenon ental Boron and rod worth, and operating experience.
OTC	 DETERMINE magnitude of increase in Boron concentration necessary to accomplish desired reactivity change using one or more of the following references: GRAPH CVCS-3A, Boration Nomograph for Hot RCS GRAPH CVCS-3B, Boration Nomograph for Cold RCS GRAPH CVCS-6, Boration - Dilution Tables GRAPH RV-2, Total Power Defect (PCM) as a Function of Power and Boron Concentration at MOL GRAPH RV-3, Differential Boron Worth (Hot Zero Power) at MOL GRAPH RV-4, Total Temperature Defect (PCM) as a Function of Temperature and Boron Concentration at MOL GRAPH RV-9, IP2 Cycle 15 Reactivity Equivalents POP 1.2, Reactor Startup WCR 1, Reactivity Summary
ОТС	IF the change in RCS Boron concentration is anticipated to be greater than or equal to 25 ppm, OPERATE Pressurizer heaters to open spray valve.
ОТС	ESTIMATE total volume of boron required for boration from boration graphs or references listed in step 4.5.2.
OTC	PLACE the RCS Makeup Control switch to STOP.
отс	SET Boric Acid Integrator to amount determined in step 4.5.3.

Appendix D			Operator Action				Form ES-D-2				
Op Test No.:	_1	Scenario #	2	_ Event #	1	Page	9	of	27		
Event Descri	ption:	Reduce Load									
Time	Position		Applicant's Actions or Behavior					-			

 отс	PLACE the RCS Makeup Mode Selector switch to BORATE.
 отс	FCV-110A, Boric Acid Flow Control, may be left in AUTO or placed in MANUAL as directed by CRS.
отс	VERIFY boric acid transfer pumps are in AUTO.
 отс	PLACE the RCS Makeup Control switch to START.

NOTE BATPs shifting to fast speed is verified by both counter operation and by the respective fast speed lights illuminating.

 отс	VERIFY BATPs shift to	o fast speed.
отс	IF in manual, ADJUST obtain desired boric ac range).	FCV-110A, Boric Acid Flow Control, to cid flow rate (may be greater than meter
	IF desired to maximize BATP recirculation val	e Boron flow, CLOSE the appropriate ve.
отс	o HCV-104 Inlet	CVCS/Boric Acid Tank 22 BA
	o HCV-105 Inlet	CVCS/Boric Acid Tank 21 BA

Appendix I		Operator Action					Form ES-D-2		
Op Test No.:	_1	Scenario #	2	_ Event #	_1	Page	10	of	27
Event Descri	ption:	Reduce Load	•						
Time	Position		Applicant's Actions or Behavior					<u></u>	

отс	MONITOR Nuclear Instrumentation, Rod position, and RCS temperature closely during any reactivity changes.
OTC	 WHEN boration operation has been completed, PERFORM the following: PLACE the RCS Makeup Control switch to STOP. PLACE the Makeup Mode Selector switch to MANUAL. ADJUST FCV-110A, Boric Acid Flow Control, dial setting to the new RCS Boron concentration per applicable CVCS Graph: GRAPH CVCS-1A, Blended Makeup - (0-500) with 120 Gpm PW GRAPH CVCS-1B, Blended Makeup - (0-2000) with 120 Gpm PW GRAPH CVCS-1C, Blended Makeup with Various PW Flows
отс	VERIFY FCV-110A control switch in AUTO.
отс	PLACE the RCS Makeup Control switch to START.
отс	VERIFY approximately 30 gallons of blended makeup flows through blender.
отс	PLACE RCS Makeup Control switch to STOP.
отс	SELECT AUTO on RCS Makeup Mode Selector switch.

Appendix D			Ope	erator Actio	· · · · · · · · · · · · · · · · · · ·	Form ES-			
Op Test No.:	1	Scenario #	2	Event #	_1	Page	11	of	27
Event Descrip	otion:	Reduce Loa	d						
Time	Positio	on	Applicant's Actions or Behavior						37 27

2

отс	PLACE the RCS Makeup Control switch to START.
CRS	IF Reactor is shutdown, REQUEST a sample for RCS Boron concentration within 30 minutes of completing the boration.
отс	LOG amount of boric acid added to system for boration in CCR Log Book.
At Lead Evaluator's	discretion, proceed to Event 2

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Appendix D		Operator Action					Form ES-D-2		
Op Test No.:	_1	Scenario #	2	Event #	_2	Page	<u>12</u> of	27	
Event Descrip	tion:	First Stage Sł	nell Pre	ssure PT-412	B Fails Low				
Time	Position	<u> </u>		Applica	nt's Actions or	Behavior			

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CRS	Refers to AOI-28.0, Instrument Failures
отс	VERIFY The Following Controls: • Turbine load - STABLE • Rod Control - STABLE • PRZR pressure control - NORMAL • PRZR level control - NORMAL • MBFP Speed - NORMAL • S/G levels - NORMAL
	Note: The instrumentation steps can be performed in a order
отс	 CHECK PRZR instrumentation - NORMAL: PRZR pressures PRZR levels
отс	 CHECK S/G instrumentation – NORMAL S/G levels S/G pressures S/G feedwater flow S/G steam flows

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Appendix [)		Operator Action				Form	ES-D-2
Op Test No.:		Scenario #	2	_ Event #	2	Page	<u>13</u> (of <u>27</u>
Event Descrip	otion:	First Stage Sh	nell Pres	sure PT-412	B Fails Low			
Time	Position		Applicant's Actions or Behavior					

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	CHECK RCS instrumentation:
отс	 O CHECK RCS loop temperatures: ○ Loop Tavg – NORMAL ○ Actual loop △T – NORMAL
	 CHECK Power Range Channels – NORMAL CHECK RCS coolant loop flow channels – NORMAL
	CHECK Turbine first stage pressure – NORMAL
CRS	 Go to AOI 28.14, 1ST STAGE PRESSURE CHANNEL FAILURE
отс	SET steam dump pressure controller for 1005 psig (83%) steam pressure
отс	MOVE steam dump control selector switch to Pressure Mode
CREW	MINIMIZE transients which will cause Tavg to increase above 559°F
CRS/BOP	DETERMINE <u>IF</u> tripping Steam Flow SI bistable trip switches will cause an SI
· · ·	
	CRS OTC OTC CREW

Appendix I	0	······································	Operator Action					Form ES-D-2		
Op Test No.:		Scenario #	2	Event #	_2	Page	<u>14</u> of	27		
Event Descri	ption:	First Stage Sh	iell Pres	ssure PT-412	B Fails Low					
Time	Position		Applicant's Actions or Behavior							

BOP	 IF tripping Steam Flow SI bistables will NOT cause an SI, THEN TRIP bistable for the failed channel Loop 1B High SF SI (White A-11) Loop 2B High SF SI (White A-10) Loop 3B High SF SI (White A-11) Loop 4B High SF SI (White A-10)
CREW	 <u>IF</u> it has been determined that the failed component will not effect rod control and/or steam dumps, they may be returned to AUTOMATIC per CR/SM direction RETURN rod control to AUTOMATIC RESET loss of load interlock
When Hi Steam Flow discretion, proceed t	O ENSURE steam dump controller is returned to temperature mode bistables have been tripped or at Lead Evaluator's o Event 3

Appendix D			Operator Action				Form ES-D-2			
Op Test No.:	_1	Scenario #	2	Event #	3	Page	<u>15</u> of	27		
Event Descript	lion:	CCW Pump T	rip.							
Time	Position			Applica	nt's Actions of	or Behavior				

Booth Instructor: When directed, insert the following commands: MOT CCW2 Option 2 CCW pump 22 shaft seizure HTX CVC6 Option 1 VALUE 50 RAMP 15:00 TBHX LEAK SLF RCP7A VALUE 10 RAMP 10:00 21 RCP HIGH VIBRATION WITH 5 MINUTE DELAY

Note: Allow entry to AOI-4.1.2, Leakage into CCW system, prior to initiating the vibration alarm. Either select an appropriate timer or wait until after procedure is in use.

CRS	Refers to AOI-4.1.1, Loss of Component Cooling
BOP	 VERIFY CCW Pump discharge pressure is greater than 80 psig with one pump in service or greater than 107 psig with two pumps in service IF NOT, START additional CCW Pumps as necessary to provide the required CCW flow IF CCW Pump discharge pressure can NOT be maintained greater than 107 psig with two pumps in service, INITIATE a plant shutdown in accordance with Technical Specification 3.3.E.2
CRS	VERIFY the requirements of Technical Specification 3.3.E.1 or 3.3.E.2 are met.

Event 4 is initiated on timer from this event. Proceed to Event 4 when alarms are received for TBHX leakage.

Appendix D	Operator Action	Form ES-D-2				
Op Test No.: 1	Scenario #2 Event #4	Page 16 of 27				
Event Description:	RCP TBHX Leak. RCP Vibration					
Time Positi	on Applicant's Actions or E	Applicant's Actions or Behavior				

	BOP	 VERIFY automatic actions in Section 3 have occurred o If CC Surge tank level is increasing, close RCV-017 DIRECT NPO to CLOSE 831, Surge Tank Makeup Valve
25 Queen To	CRS	DIRECT NPO to CLOSE 831, Surge Tank Makeup Valve
25 Surray Ta		
	ank Relief BOP	NOTE Valve is set at 52 psig MONITOR CCW Surge Tank Pressure / Level AND WHUT Level to detect lifting of 835, Surge Tank Relief Valve
the leaking the approval	compone of the SW	<u>NOTE</u> nt is known, Operator may go directly to appropriate Attachment /S, to isolate the in-leakage
	CREW	If surge tank level is increasing, isolate sources of leakage one at a time using the following attachments: • Attachment 1, RCP Thermal Barrier

discretion

Appendix E)	Operator Action			on	· · · · · · · · · · · · · · · · · · ·	For	m E	S- D-2
Op Test No.:	_1	Scenario #	_2	Event #	5, 6, 7	Page	<u>17</u>	of	_27
Event Descrip	otion:	RCP Sheared	l Shaft; /	ATWS; Turbi	ine Trip Failure;	Boration Fail	ure		
Time	Position			Applica	nt's Actions or E	Behavior			

Booth Instructor: When directed, insert the following command: MOT RCP5 OPTION 1 21 RCP Shaft break REMOVE CCW Pump 23 AUTO START FAILURE (MOC CCW3)

	CREW	Determine reactor trip required. Reactor did not trip.
	CRS	Direct reactor trip and entry to E-0, Reactor Trip or Safety Injection
	отс	 Verify reactor trip Attempt to manually trip reactor
	CRS	Direct entry to FR-S.1, Response to Nuclear Power Generation/ATWS
	CREW	 Verify reactor trip Attempt to manually trip the reactor Manually insert control rods Dispatch NPO to locally trip reactor trip breakers or MG set breakers
Critical Task	отс	 Verify Turbine trip Manually trip the turbine
	BOP	Check AFW pumps running

Appendix [)	Operator Action					Form ES-D-2			
Op Test No.:	_1	Scenario #	2	Event #	5, 6, 7	Page	<u>18</u> of	27		
Event Descrij	ption:	RCP Sheared	l Shaft;	ATWS; Turbi	ine Trip Failure;	Boration Faile	ure			
Time	Position			Applica	int's Actions or I	Behavior				

	BOP	Initiate emergency boration o Start Charging pumps o Open MOV-333 (Will not Open)
Critical Task	BOP	Align one of the following flowpaths RWST flow path Open LCV-112B Close LCV-112C Place Makeup control switch to STOP Establish maximum charging flow OR Normal boration flow path FCV-110 in MANUAL Both boric acid pumps in high speed Adjust FCV-110 for maximum flow

Booth Instructor: When boration is initiated, remove Reactor trip malfunctions to allow rods to drop

отс	Check PRZR pressure less than 2335 psig

CAUTION

Radiation levels and harsh environment conditions should be evaluated prior to performing local actions

	Verify containment ventilation isolation
BOP	 Containment Purge valves closed Containment pressure relief valves closed

Appendix D)	Operator Action Fo				Operator Action				
Op Test No.:		Scenario #	_2	Event #	5, 6, 7	Page	19	of	_27	
Event Descrip	tion:	RCP Sheared	Shaft; A	ATWS; Turbi	ine Trip Failure;	Boration Failu	ıre			
Time	Position		Applicant's Actions or Behavior							

If an SI sig with this p		CAUTION occurs, steps 1-9 of E-0 should be performed while continuing
	BOP	Perform Steps 1-9 of E-0, Reactor Trip or Safety Injection
		While determining whether the RCPs have CCW cooling, the BOP may determine that CCW flow is insufficient to maintain long term thermal barrier cooling. He may decide to trip RCPs, but is not required to, because minimal cooling will be available at this time.
	отс	 Check if the following trips have occurred Reactor trip Turbine trip Dispatch NPO to open MG set output breakers

CAUTION City Water for AFW pumps will be necessary if CST level decreases to less than 2 feet

отс	 Check SG levels NR level in at least one SG greater than 10% Control feed flow to maintain 10-50%
отс	 Verify dilution paths isolated o FCV-111A closed o FI-111 no flow indicated

Appendix [)		Operator Action				Form ES-D-2			
Op Test No.:	_1	Scenario #	2	Event #	5, 6, 7	Page	20	of	27	
Event Descrip	otion:	RCP Sheared	Shaft;	ATWS; Turbi	ne Trip Failure;	Boration Failu	ire			
Time	Position		Applicant's Actions or Behavior							

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	отс	 Check for reactivity insertion from uncontrolled cooldown Check RCS temperature decreasing in an uncontrolled manner Check any SG pressure decreasing in an uncontrolled manner Stop controlled cooldown
	BOP	Check core exit TCs less than 1200°F
(отс	Verify reactor subcritical
(CRS	Return to E-0, step 1
-		Evaluator Note: E-0, Steps 1-9 should be in progress or complete. Attachment 1 actions from step 5 of E-0 are included at back of this guide. (page 22)
C	отс	 Check RCS temperature stable at or trending to 547°F (NO) Stop dumping steam
С	отс	Check PRZR PORVs and spray valves closed
С	тс	Check if RCPs should be stopped (NO)

Appendix [Operator Action					Form ES-D-2			
Op Test No.:	_1	Scenario #	2	_ Event #	5, 6, 7	Page	<u>21</u> of	27		
Event Descrip	otion:	RCP Sheared	l Shaft; A	ATWS; Turbi	ne Trip Failure;	Boration Failu	ure			
Time	Position		Applicant's Actions or Behavior							

 отс	 Check if any SG is faulted Any SG depressurizing in an uncontrolled manner Any SG depressurizing
CREW	Check if SG tubes are intact (YES)
 CREW	Check if RCS is intact (YES)
 CREW	Check if SI should be terminated (YES)
 CRS	Direct transition to ES-1.1, SI Termination

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Appendix D		Operator Action							Form ES-D-2		
Op Test No.:		Scenario #	All	Event #	Attachment 1	Page	22	of	27		
Event Descrip	otion:	Attachment 1,	Automa	atic Action V	erification						
Time	Position			Applica	nt's Actions or Beha	avior			<u>.</u>		

	·····	Note
This attac AC Power		be terminated upon CRS transition to ECA-0.0, Loss of All
		Verify proper Charging system operation:
	BOP	 a. Start at least one charging pump in manual at maximum speed b. Align charging pump suction to the RWST Open charging pump suction valve from RWST LCV-112B Close charging pump suction valve from VCT LCV-112C Place RCS Makeup Control switch to STOP
		Note

- Notify CRS of any automatic actions that failed to occur during performance of this attachment
- Equipment found misaligned due to operator action should NOT be repositioned.

BOP	Check generator output breakers – OPEN

Appendix D	· · · · · ·	Operator Action						Form ES-D-2		
Op Test No.:	_1	Scenario #	All	Event #	Attachment 1	Page	23	_ of	27	
Event Descrip	otion:	Attachment 1,	Automat	tic Action V	erification					
Time	Position	<u> </u>		Applica	nt's Actions or Beha	ivior				

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BOP	Check status of 480 volt busses: a. All 480V busses – ENERGIZED BY OFF- SITE POWER b. Dispatch NPO to reset: o All lighting o MCC 24A o MCC 27A o MCC 29A c. Stop all Condensate Pumps
BOP	Verify FW Isolation: • Main Boiler Feed Pumps – TRIPPED • Main Boiler Feed Pump Discharge Valves – CLOSED • FW Regulating valves – CLOSED • FW Stop Valves – CLOSED • SG Blowdown Isolation Valves - CLOSED
BOP	Check if Main Steam Lines should be isolated: a. Check for either: • High Steam Line flow with EITHER Tave less than 541 deg F OR Steam line pressure less than 525 psig. OR • Containment pressure – EVER GREATER THAN 24 psig b. Verify MSIVs - CLOSED

Appendix D		······································	Operator Action					Form	ES-D-2
Op Test No.:	1	Scenario #	All	Event #	Attachment 1	Page	24	of	27
Event Descrij	ption:	Attachment 1,	Automa	atic Action Ve	erification				
Time	Positio	n	Applicant's Actions or Behavior						

	Verify proper Service Water System operation:
ВОР	a. Three Service Water Pumps – Running on Essential Header
	b. Service Water valves from Diesel Generator - OPEN
	Verify SI system pumps running:
BOP	a. Three SI pumps – RUNNING b. 22 SI pump discharge isolation MOV-851A
	AND MOV-851B – OPEN
	c. Two RHR pumps - RUNNING
	Verify proper emergency SI System valve alignment:
	a. SI pump cold leg injection valves – OPEN o 856A o 856E o 856C
BOP	 856D b. RHR HX CCW outlet valves – OPEN
	o 822A o 822B
	c. RHR HX MOVs – OPEN o 746
	0 740
	Verify Containment Fan Coolers – IN SERVICE:
BOP	a. Five fan coolers - RUNNINGb. Charcoal Filter valves - OPEN
	 c. Fan normal discharge valves – CLOSED d. TCV-1104 and TCV-1105 – BOTH OPEN

Appendix D		Operator Action					Form ES-D-2			
Op Test No.:	_1	Scenario #	All Event #	Attachment 1	Page	25	_ of	27		
Event Description: Attachment 1, Automatic Action Verification										
Time	Position		Annlica	nt's Actions or Beha	avior		ile in solution of			

BOP	Verify AFW flow to all SGs
	Verify Containment Ventilation Isolation:
BOP	 a. Containment Purge Valves – CLOSED FCV-1170 FCV-1171 FCV-1172 FCV-1173 b. Containment Pressure Relief Valves - CLOSED PCV-1190 PCV-1191 PCV-1192

Appendix D		Operator Action					Form ES-D-2				
Op Test No.:		Scenario #	All	Event #	Attachment 1	Page	26	of	27		
Event Descri	ption:	Attachment 1,	Automa	tic Action V	erification						
Time	Position	Position Applicant's Actions or Behavior									

BOP Verify Containment Isolation Phase A a. Phase A – ACTUATED o Train A master relay CA1 (above rack E) o Train B master relay CA2 (above rack F) b. Phase A valves – CLOSED c. IVSW valves – OPEN o 1410 o 1413 o SOV-3518 o SOV-3519 d. WCP valves – OPEN: o PCV 1238 o PCV 1239 o PCV 1240 o PCV 1241 e. Place personnel and equipment hatch solenoid control switches to INCIDENT on SM panel f. Dispatch NPO to periodically check o IVSW Tank o Level – GREATER THAN 92%	 	· · · · · · · · · · · · · · · · · · ·
o Train A master relay CA1 (above rack E) o Train B master relay CA2 (above rack F) b Phase A valves – CLOSED c IVSW valves – OPEN o 1410 o 1413 o SOV-3518 o SOV-3519 d WCP valves – OPEN: o PCV 1238 o PCV 1239 o PCV 1240 o PCV 1241 e Place personnel and equipment hatch solenoid control switches to INCIDENT on SM panel f. Dispatch NPO to periodically check o Level – GREATER THAN 92%		Verify Containment Isolation Phase A
o WCP header pressures – GREATER THAN 52 PSIG	BOP	 Train A master relay CA1 (above rack E) Train B master relay CA2 (above rack F) Phase A valves – CLOSED IVSW valves – OPEN 1410 1413 SOV-3518 SOV-3519 WCP valves – OPEN: PCV 1238 PCV 1239 PCV 1240 PCV 1241 Place personnel and equipment hatch solenoid control switches to INCIDENT on SM panel Dispatch NPO to periodically check IVSW Tank Level – GREATER THAN 92% Pressure – GREATER THAN 57 PSIG WCP header pressures – GREATER

Evaluator note: The following step is intended for High Containment pressure condition. It will not be performed if conditions aren't met

Appendix D		Operator Action					Form ES-D-2			
Op Test No.	: _1	Scenario #	All	Event #	Attachment 1	Page	27	of <u>27</u>		
Event Descr	iption:	Attachment 1	, Automa	tic Action Ve	erification					
Time	Position			Applica	nt's Actions or Beh	avior				

BOP	Check if Containment Spray should be actuated: a. Containment Pressure – EVER GREATER THAN 24 PSIG b. Verify spray pumps – RUNNING c. Verify spray pump discharge valves – OPEN o MOV-866A o MOV-866B o MOV-866C o MOV-866D d. Verify Containment Isolation Phase B valves – CLOSED e. STOP all RCPs f. Verify IVSW Isolation Valves – OPEN o 7864 o 7865 o 7867
BOP	Verify CCR Air Conditioner Train A and B – RUNNING IN INCIDENT MODE 2
BOP	Notify CRS that Attachment 1 is complete

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Appendix D

Scenario Outline

Form ES-D-1

r				
Facility:	IP2		Scenario No.: 3 Op Test No.: 1	
Examiners	5:		Candidates:	CRS
				RO
		·····		PO
		······		-
Initial Con	aitions: 6	% power BO	L	
	P	lant startup i	n progress	
Turnover:	R	laise power a	nd synchronize the Main Generator	
Critical Ta	<u>sks:</u> S	top SI pumps	3	
	ls	solate rupture	d SG	
Event	Malf.	Event		
No.	No.	Type*	Event Description	
1		R (RO)	Raise reactor power. Synchronize Main Generator	
		N (BOP)		
		N (CRS)		
2	XMT RCS036A	I (ALL)	Tcold instrument fails high	
3	RCS14C	C (ALL)	Steam Generator Tube Leak	
4	RCS14C	M (ALL)	SGTR	
5	XMT	C (RO)	Atmospheric Dump valve on ruptured SG fails open	
	SGN43			
6	SWI PPL030B	C (BOP)	CIA fails to reset. Manual action to bypass and reset CIA	

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Scenario Event Description IP2 Simulator Scenario 3

The crew will assume the shift to raise power and synchronize the Main Generator to the grid IAW POP-1.3 and SOP-26.4.

When the generator is on-line, a Tcold instrument will fail high. IAW AOI-28.0 and 28.1, the RO will place the running Charging Pump in manual, the BOP will trip bistables, and the CRS will refer to Technical Specifications.

When the plant is stable, a steam generator tube leak will develop, requiring action IAW AOI-1.2. Leak rate is quantified, secondary systems are isolated, and the crew will begin a plant shutdown IAW POP-3.1 based on excessive SG tube leakage. The CRS will again refer to Technical Specifications.

While the crew is shutting the plant down, the tube leak will increase in severity. The crew will determine that pressurizer level cannot be maintained, and a reactor trip will be required.

When the reactor trips, one Atmospheric Dump Valve will fail open, requiring manual action to close it to minimize the radioactive release to atmosphere. Additionally, CIA will fail to reset. The crew must bypass and manually reset CIA to provide instrument air to containment, avoiding RCS depressurization using PORVs

EOP flow path: E-0 - E-3 - ES-3.1

Indian Point Unit 2 2003 NRC Initial License Examination Simulator Scenario Setup Scenario 3

No equipment Out of Service

CIA Reset Failure:

SWI PPL030B 1

Materials needed for scenario:

- POP-1.3
- SOP-26.4
- Graph Book
- Reactivity Summary Sheet

Allow crew to begin scenario brief approximately 30 minutes prior to entering simulator

Note: None

Scenario built from IC 20

Indian Point Unit 2 2003 NRC Initial License Examination Simulator Scenario Turnover Information Scenario 3

- The plant is at 6% power, ready to synchronize the Main Generator.
- Beginning of Life, C_b is 1275 ppm.
- EFPD = 100
- Control Bank D = 157 steps
- Tavg = 547°F
- RCS Pressure = 2235 psig
- Pressurizer Level = 37%
- Risk Assessment = GREEN
- Daily Risk Factor = 0.83

The following equipment is out of service:

None

Crew instructions:

- In accordance with POP-1.3 and SOP-26.4, synchronize the Main Generator and prepare to raise load to 100%.
- POP-1.3 is complete through step 4.32
- SOP-26.4 step 4.6.7 in progress
- D.O has confirmed that the switchyard is properly aligned and all grounds removed
- D.O directs you to synchronize using breaker 7 and close breaker 9 when ready in accordance with procedure
- Generator is to be synchronized in MANUAL

Appendix E)	Operator Action						Form ES-D-2			
Op Test No.:	_1	Scenario #	3	_ Event #		Page	5	of	29		
Event Description: Raise reactor power. Synchronize Main Generator.											
Time	Position		Applicant's Actions or Behavior								

	CRS	Refer to POP-1.3
E	30P	SYNCHRONIZE the Generator to the bus and CONTINUE with SOP 26.4, Turbine Generator Startup, Synchronizing, Voltage Control and Shutdown, "Initial Loading of the Generator and Closing of the Second Generator Output Breaker", section
	CRS	REQUEST the DO to indicate which generator breaker (7 or 9) is to be used for synchronizing
E	BOP	PLACE Bkr 7 or 9 Sync Pot Control Selector switch in the appropriate position
E	BOP	VERIFY Synchroscope Switch is in MAN
At synchronization greater than sys		<u>CAUTION</u> Main Transformer Secondary voltage should be 8 to 20 KV age

 BOP	UNLESS directed otherwise by the DO, slowly ADJUST the AC regulator UNTIL 346 to 358 KV (nominally 350 KV) on the high side of the generator main power transformers has been established
 BOP	ADJUST Turbine speed <u>UNTIL</u> the Synchroscope is observed to be rotating slowly in the fast direction

Appendix E)		Оре	erator Actio	n		Foi	rm E	S-D-2
							• • • •		
Op Test No.:	1	Scenario #	3	Event #	1	Page	6	of	29
Event Description: Raise reactor power. Synchronize Main Generator.									
Time	Position		Applicant's Actions or Behavior						

		NOTE of the Generator may be required because the Auto- has been determined to be inoperable						
	BOP	VERIFY Synchroscope Switch is in MAN						
	BOP WHEN the Synchroscope is between one minute BOP BEFORE 12, and 12 O'clock (top center), CLOSE the selected generator breaker CRS NOTIFY the SO that Unit 2 is synchronized to the bus							
	CRS	NOTIFY the SO that Unit 2 is synchronized to the bus						
· · · · · · · · · · · · · · · · · · ·	CRS	RECORD the time in the CRS log book						
		<u>CAUTION</u> ain Generator above 40 MWe with 6.9 KV Buses 1 through 4 in Auxiliary Transformer and the Unit Auxiliary Transformer						
	CREW	VERIFY Generator output voltage is within the capability limits in Graph EL-1, Capability Curve Voltage Regulator in Service and Out of Service						
o Ref	 <u>NOTE</u> Refer to Graph EL-3, Generator Load Changing Curves. 							
o The	e maximum G	tenerator H_2 heat up rate is 10°C/hr (18°F/hr.)						

Appendix D)	Operator Action					Form ES-D-2			
	<u></u>	- <u></u>								
Op Test No.:	1	Scenario #	3	Event #	1	Page	7	of	29	
Event Descrip	otion:	Raise reactor	power.	Synchronize	e Main Gene	erator.				
Time	Position			Applica	nt's Actions	or Behavior				

	BOP	COMMENCE turbine generator load increase as plant conditions allow
	BOP	As Generator load increases, VERIFY all phase ammeters approximately equal
	BOP	As directed by the DO, ADJUST Generator AC regulator to obtain Reactive loading (VARS)
Booth Instr MWe	uctor: If aske	ed, D.O. requests 50 MVARs OUT until Generator load is 200
slov	vly, minimizin	NOTE btain 30 to 40 MWe Turbine Generator load should be done of the effects of swell on SG level and to avoid exceeding the P-10 Permissive setpoint.
		e to 40 MWe should be done at a rate at which the operator is proximately 10 to 15 minutes).
o Cro	ssover steam	n temperature increases SHALL <u>NOT</u> exceed 75°F/hr.
	BOP	ADJUST Governor for 30 to 40 MWe Turbine Generator load
	BOP	ALIGN the 25000 Speedomax Bearing Temperature Monitor per Operator Aid 97-02, as directed by the CRS
		Note: May N/A this step

Appendix D		Operator Action					Form ES-D-2		
Op Test No.:	_1	_ Scenario #	_3	Event #	1	Pag	e <u>8</u>	of	29
Event Descrip	otion:	Raise reactor	powe r .	Synchronize	e Main Ge	nerator.			
Time	Position	1	Applicant's Actions or Behavior						

	BOP	<u>WHEN</u> closure of the other generator output breaker is to be performed, PLACE the Synchroscope Switch to MAN, and PLACE Bkr 7 or 9 Sync Pot Control Selector Switch in the appropriate position
sinc	e there shou	<u>CAUTION</u> pe should <u>NOT</u> rotate when the second breaker is selected, Id be no phase difference across the breaker. ope is rotating, do <u>NOT</u> close second breaker
	BOP	CLOSE the second breaker with the DO's permission
	отс	 WHEN Reactor power level exceeds 10 percent as indicated by the LOW POWER PERMISSIVE BLOCK NOT ENGAGED alarm and illumination of the POWER ABOVE P-10 light BLOCK the intermediate range trip and rod stop OBSERVE the INTERMED RANGE TRIP BLOCKED light is illuminated BLOCK the low power range trip OBSERVE the LOW PWR RANGE TRIP BLOCKED light is illuminated VERIFY the LOW POWER PERMISSIVE BLOCK NOT ENGAGED Alarm clears
	BOP	INITIATE Steam Generator Blowdown per SOP 7.1, Steam Generator Blowdown System Operation
If the Main loads	Generator is	<u>CAUTION</u> carrying greater than 40 MWe, do <u>NOT</u> transfer the auxiliary

Appendix D)	Operator Action					Form ES-D-2			
Op Test No.:	1	Scenario #	3	Event #		Page	9	of	29	
Event Descrip	otion:	Raise reactor	power.	Synchronize	Main Gene	rator.				
Time	Position			Applica	nt's Actions	or Behavior		<u>.</u>		

вор	PRIOR to exceeding 40 MWe, TRANSFER Bus Sections 1 through 4 from the Station Auxiliary Transformer to the Unit Auxiliary Transformer per SOP 27.1.4, 6900 Volt System
When busses are trai	nsferred or at Lead Evaluator's discretion, proceed to Event 2

Appendix E)	Operator Action Fc					Fo	Form ES-D-2		
1										
Op Test No.:	1	Scenario #	3	Event #	2	Pa	ge <u>10</u>	of	29	
Event Descrip	otion:	Tcold Instrum	ent Fai	ls High				_		
Time	Positio	n		Applica	nt's Actio	ns or Behavior				

 CREW	Refers to AOI-28.0, Instrument Failures
CREW	VERIFY The Following Controls: • Turbine load - STABLE • Rod Control - STABLE • PRZR pressure control - NORMAL • PRZR level control - NORMAL • MBFP Speed - NORMAL • S/G levels - NORMAL
CREW	CHECK PRZR instrumentation - NORMAL: o PRZR pressures o PRZR levels
CREW	 CHECK S/G instrumentation – NORMAL S/G levels S/G pressures S/G feedwater flow S/G steam flows
CREW	 CHECK RCS instrumentation: CHECK RCS loop temperatures: Loop Tavg – NORMAL Actual loop ΔT – NORMAL CHECK Power Range Channels – NORMAL CHECK RCS coolant loop flow channels – NORMAL

Appendix D)		Оре	erator Actic	n			For	mΕ	S-D-2
On Test No.		Scenario #		Event#	2		Page	11	of	29
Op Test No.:		Tcold Instrum	ont Fail	-			i age	<u> </u>	0.	
Event Descrip Time	Position				nt's Action	ns or Beha	vior			

CRS	GO to AOI 28.1, NARROW RANGE HOT/COLD LEG TEMPERATURE CHANNEL FAILS HIGH/LOW
 отс	PLACE Rod Control bank selector switch in MAN
 CRS	OBSERVE actual insertion limits. (REFER to GRAPH RPC-6, Cycle 15 Core Operating Limits Report)
 отс	PLACE charging pump speed control in MANUAL
отс	CONTROL Pressurizer level in Normal band. (Refer to GRAPH RCS-2, Pressurizer Level Program vs. Tave, in the Graphs Book)
 BOP	In Foxboro Rack D10, PLACE T AVE DEFEAT switch (T/412A OR T/412B) to DEFEAT LOOP #, for affected loop
	In Foxboro Rack B8, PLACE DELTA-T DEFEAT switch
 BOP	(T/411A <u>OR</u> T/411B) to DEFEAT LOOP #, for affected loop
 отс	PLACE Rod Control Bank selector switch in AUTO, <u>UNLESS</u> directed otherwise by the CRS
отс	RETURN charging pump speed control to AUTOMATIC

Appendix E)	······································		Form ES-D-2					
Op Test No.:	1	Scenario #	3	Event #	2		Page	<u>12</u> of	29
Event Descrij	ption:	Tcold Instrum	ent Fai	ls High					
Time	Position			Applica	nt's Actio	ons or Beh	avior		

CRS	REFER to Technical Specification Tables 3.5-2, 3.5-3, 3.5-4 <u>AND</u> Bistable Trip status lights on Panel SO to determine if tripping Bistable trip switches will cause a Reactor Trip
вор	<u>IF</u> tripping the Bistable Trip Switches will <u>NOT</u> cause a Reactor Trip, <u>THEN</u> TRIP the appropriate Loop Bistable trip switches per Table 1, List of Loop Temperature Bistable Trip Switches
When bistables are	tripped or at Lead Evaluator's discretion, proceed to Event 3

Appendix [)		Operator Action				Form ES-D-2			
Op Test No.:	1	Scenario #	3	Event #	3	Page	<u>13</u>	of	29	
Event Descri	ption:	- Steam Gener	ator Tul	- be Leak						
Time	Position	1		Applica	int's Actions	or Behavior				

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the SHA <u>IF</u> A min	next action le ALL be re-en attachment 8 ute intervals	NOTE hile in this procedure, steam generator leak rate increases abo evel as specified in Step 2 of FOLDOUT PAGE, this procedure tered at Step 3 is used for Leak Rate Estimation, it shall be performed at 15 until the leakrate is stable for 1 hour (≤10% increase during a en the time interval may be relaxed to 2 hours
	CRS	IF a significant increase is observed on R-45, PERFORM the following NOTIFY Health Physics PERFORM Leak Rate Estimate per Attachment 8
		If requested for leak rate attachment calculation, air In-Leakage ity is 0.06 μ Ci/cc IF steam is available on the secondary side, DIRECT
	CRS	 Chemistry to perform Leak Rate Calculation MAINTAIN steady state conditions while Chemistry performs Leak Rate Calculation

Appendix [)	Operator Action					Form ES-D-2			
Op Test No.:	_ 1	Scenario #	3	Event #	3		Page	<u>14</u>	of	29
Event Descrip	otion:	Steam Gener	ator Tut	be Leak						
Time	Position	ion Applicant's Actions or Behavior								

<u>IF</u> i lea ○ <u>W</u> H	it is not practi kage should <u>IEN</u> available	NOTE pecified in Step 3 apply to leakage in any one steam generator. cal to assign the leakage to an individual steam generator, all be assumed to be from one steam generator a, the Nitrogen 16 Monitor SHALL be used for the initial leak rate. (Note: N-16 not available below 30% power)					
	CREW	CHECK Primary To Secondary Leak Rate Leak Rate - GREATER THAN 5 GPD 					
	CREW	Leak Rate - GREATER THAN OR EQUAL TO 30 GPD					
more in las	it hour, monit	<u>NOTE</u> rate is greater than 75 gpd <u>AND</u> has increased by 30 gpd or ored in at least 30 minutes intervals, Reactor Power should be % within 1 hour and placed in hot shutdown within 2 additional					
	CREW	Leak Rate - GREATER THAN 75 GPD AND INCREASED BY 30 GPD IN LAST HOUR MONITORED IN AT LEAST 30 MINUTES INTERVALS					
<u>IF</u> performi	ng a Rapid P	<u>NOTE</u> lant Shutdown, Chemistry HOLD for sampling is <u>NOT</u> required					
Booth Instr that leak ra	Booth Instructor Note: Approximately 15 minutes after call for Chemistry sample, report that leak rate estimation per Chemistry sample results indicate a leak rate of 80 GPD.						

Appendix E)		Operator Action Form E					
				· · ·				
Op Test No.:	1	Scenario #	_3	Event #	3	Page	<u>15</u> of	_29
Event Descrip	otion:	Steam Gener	ator Tul	be Leak				
Time	Position			Applica	nt's Actions	or Behavior		

	CREW	PERFORM a rapid plant shutdown using POP 3.1, Plant Shutdown from Full Power Operation to Zero Power Condition concurrently with the rest of this procedure such that reactor power is less than 50% within 1 hour <u>AND</u> in hot shutdown within 2 additional hours.
When dec	ision is mad	le to shut down the unit, proceed to Event 4

Appendix D				Operator Action Fo						S- D-2
Op Test No.:		1	Scenario #	3	_ Event #	4, 5, 6	Page	16	of	29
Event Descrip	otion:		SGTR, Atmos	pheric I	Dump Valve	on ruptured SG	fails open; Cl	A rese	t fail	ure
Time	Po	sition	Applicant's Actions or Behavior							

Booth Instructor: When directed, insert the following command: MAL RCS14C ACT,5,0,0,0,D XMT SGN43 1,1400,600,0,C,JBKRTA.EQ.0

 CRS	Direct entry to E-0, Reactor Trip or Safety Injection
 отс	Verify reactor trip
отс	Verify turbine trip
BOP	Verify power to 480 V busses
отс	Check if SI is actuated
BOP	Perform attachment 1 while continuing with this procedure (Attachment 1 actions begin on page 24 of this scenario guide)
ОТС	Verify AFW pumps running
отс	Verify total AFW flow greater than 400 GPM

Appendix D		Operator Action Form E						S- D-2	
Op Test No.:		Scenario #	3	_ Event #	4, 5, 6	Page	<u>17</u>	of	29
Event Descrip	otion:	SGTR, Atmos	pheric [Dump Valve	on ruptured SG	fails open; Cl	A rese	et fail	ure
Time	Position		Applicant's Actions or Behavior						

manner t	ssure should b o less than 320 o supply water	<u>CAUTION</u> e monitored. If RCS pressure decreases in an uncontrolled 0 psig any RHR pump placed in PULLOUT must be manually to the RCS
	отс	 Verify SI system flow RCS pressure less than 1660 psig SI pump flow indicated RCS pressure less than 320 psig (NO) Place one RHR pump in PULLOUT
	отс	 Check RCP seal cooling 3 CCW pumps running CCW flow to RCP thermal barriers normal Service Water system aligned for 3 header operation Locally verify SWN-4 and SWN-5 closed Start one Service Water pump on Non-Essential header on bus supplied by off-site power
	e containment e RCS tempera	CAUTION conditions exist, use wide range cold leg temperatures to ature
	отс	 Check RCS temperature stable at or trending to 547°F (NO) Manually close Atmospheric Dump valve for 23 SG (Failed open due to failed pressure transmitter)
	отс	Check PRZR PORVs and spray valves closed

Appendix D		Operator Action F								S-D-2
Op Test No.:		1	Scenario #	3	Event #	4, 5, 6	Page	18	of	29
Event Description: SGTR, Atmospheric Dump Valve on ruptured SG fails open; CIA reset failure										
Time	F	ositior	Applicant's Actions or Behavior							

отс	Check if RCPs should be stopped (NO)
отс	 Check if any SG is faulted Any SG depressurizing in an uncontrolled manner Any SG depressurizing
CREW	Check if SG tubes are intact (NO)
CRS	Direct transition to E-3, Steam Generator Tube Rupture
	OTC

CAUTION

FRPs should NOT be implemented prior to completion of E-0, Reactor Trip or Safety Injection, Attachment 1, Automatic Action Verification

отс	Check if RCPs should be stopped

CAUTION

Radiation levels and harsh environment conditions should be evaluated prior to performing local actions

	отс	Identify Ruptured SG o 23 SG ruptured
the	turbine drive	<u>CAUTION</u> ven AFW pump is the only source of feed flow, steam supply to en AFW pump should be maintained from one SG must be maintained available for RCS cooldown

Appendix D			Operator Action				Form ES-D-				
					· · · · · · · · · · · · · · · · · ·						
Op Test No.:	1	Scenario #	3	Event #	4, 5, 6	Page	<u>19</u>	of	29		
Event Description: SGTR, Atmospheric Dump Valve on ruptured SG fails open; CIA reset failure											
Time	Position		Applicant's Actions or Behavior								

Critical Task (Isolation actions)	отс	 Isolate flow from ruptured SG Atmospheric Dump valve in AUTO set at 74% Atmospheric Dump valve closed Note: In manual due to transmitter failure
	отс	Check 22 and 23 SGs intact (NO) o Dispatch NPO to close MS-42 steam to TDAFW pump
	BOP	Verify blowdown isolation valves from 23 SG closed
	CRS	Dispatch NPO Close steam traps upstream of ruptured SG MSIV Verify ruptured SG MSIV bypass closed Close ruptured SG MSIV

CAUTION

If any ruptured SG is faulted, feed flow to that SG should remain isolated during subsequent recovery actions unless needed for RCS cooldown

 	Check ruptured SG NR level greater than 10%
отс	 Stop feed flow to 23 SG

CAUTION

Isolation of the ruptured SG steamlines from the intact SG steamlines including trip of turbine driven AFW pump or closing the steam supply valve to turbine driven AFW pump from the ruptured SG should be completed before continuing with step 5

Appendix D		Operator Action					Form ES-D-2				
Op Test No.:		Scenario #		Event #	4, 5, 6	Page	20	of	29		
Event Description: SGTR, Atmospheric Dump Valve on ruptured SG fails open; CIA reset failure											
Time	Position		Applicant's Actions or Behavior								

	отс	Verify ruptured SG pressure greater than 440 psig
Sta indi ⊙ To⊺	tus Tree, indi cation until a	<u>CAUTION</u> F running, the following steps may cause a false F.0.4, Integrity cation for the ruptured loop. Disregard this ruptured loop Tcold fter performing step 27 nline isolation, steam dump to condenser should NOT exceed ⁻ SG
	отс	 Initiate RCS cooldown Determine required core exit temperature Dump steam to condenser from intact SGs at maximum rate not to exceed 0.5E6 lbm/hr per SG Condenser available Steam Dump control to manual with zero output Place steam dump in pressure control Stop cooldown when desired temperature is achieved
	отс	 Check intact SG NR levels greater than 10% Control feed to maintain 10-50% NR level
	отс	 Check PRZR PORVs and Block Valves o Power available to block valves o PORVs closed o At least one block valve open o Open one block valve

Appendix D	···· · ·	Operator Action							Form ES-D-2			
Op Test No.:	_1	Scenario #	3	_ Event #	4, 5, 6	Page	21	of	29			
Event Description: SGTR, Atmospheric Dump Valve on ruptured SG fails open; CIA reset failure												
Time	Position		Applicant's Actions or Behavior									

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	 <u>CAUTION</u> If offsite power is lost after SI reset, then manually action may be requir to restart safeguards equipment Placing key switches to DEFEAT will prevent auto SI actuation 										
	BOP	Reset SI									
	BOP	 Reset CIA Must manually reset train 'A' relay. 									
	BOP	Establish Instrument Air to containment									
manner to		<u>CAUTION</u> e monitored. If RCS pressure decreases in an uncontrolled) psig any RHR pump placed in PULLOUT must be manually to the RCS									
	BOP	Check if RHR pumps should be stopped o Stop RHR pumps and place in auto									
		<u>CAUTION</u> s to less than 15 feet, charging pumps which are started or tored for loss of suction which may result in pump damage									
	OTC OTC OF Align suction which may result in pump damage OTC OF Align suction to RWST OF Establish maximum flow										

Appendix D			Operator Action					Form ES-D-2		
Op Test No.:		1	Scenario #	3	Event #	4, 5, 6	Page	22	of	29
Event Description: SGTR, Atmospheric Dump Valve on ruptured SG fails open; CIA reset failure									ire	
Time	F	Position	Applicant's Actions or Behavior							

ОТС	 Check if RCS cooldown should be stopped O Core exit TCs less than required
отс	Check ruptured SG pressure stable or increasing
 OTC	Check RCS subcooling based on CETs greater than required
отс	 Depressurize RCS to minimize break flow and refill pressurize Normal spray available Depressurize until pressurizer level is 71%, OR RCS pressure less than SG pressure and PRZR level greater than 14% OR RCS subcooling less than required
 отс	Stop depressurization

SI must be terminated when SI termination criteria are satisfied to prevent overfilling the ruptured SG

Appendix D			Ор	erator Actior)		Fo	rm E	S- D-2
Op Test No.:	_1	Scenario #	3	Event #	4, 5, 6	Page	23	of	29
Event Descrip	otion:	SGTR, Atmos	pheric I	Dump Valve	on ruptured SG	fails open; Cl	A rese	et fail	ure
Time	Position			Applica	nt's Actions or E	Behavior			

	отс	 Check if SI flow can be terminated RCS subcooling greater than required on table Secondary heat sink, either 400 gpm AFW flow or 10% NR in at least one SG RCS pressure stable or increasing PRZR level greater than 14%
Critical Task	отс	Stop SI pumps and place in AUTO
		vhen SI pumps are secured

Appendix D		· · · · · · · · · · · · · · · · · · ·	Оре	erator Actior	۱		F	orm E	S- D-2
Op Test No.:	_1	Scenario #	All	Event #	Attachment 1	Page	24	_ of	29
Event Descrip	otion:	Attachment 1,	Automa	atic Action V	erification				
Time	Position			Applica	nt's Actions or Beha	avior			

			Note
	is attac Power		be terminated upon CRS transition to ECA-0.0, Loss of All
		BOP	 Verify proper Charging system operation: a. Start at least one charging pump in manual a maximum speed b. Align charging pump suction to the RWST Open charging pump suction valve from RWST
		L	Note
0		CRS of any a tachment	utomatic actions that failed to occur during performance o
0		ment found m tioned.	isaligned due to operator action should NOT be
		BOP	Check generator output breakers – OPEN

Appendix D		· · · · · · · · · · · · ·	Op	erator Action)		F	orm E	S- D-2
Op Test No.:	1	Scenario #	All	Event #	Attachment 1	_ Page	25	of	29
Event Descrip	otion:	Attachment 1,	Automa	atic Action V	erification				
Time	Positior	۱		Applica	nt's Actions or Beha	avior			

BOP	Check status of 480 volt busses: a. All 480V busses – ENERGIZED BY OFF- SITE POWER b. Dispatch NPO to reset: o All lighting o MCC 24A o MCC 27A o MCC 29A c. Stop all Condensate Pumps
BOP	Verify FW Isolation: • Main Boiler Feed Pumps – TRIPPED • Main Boiler Feed Pump Discharge Valves – CLOSED • FW Regulating valves – CLOSED • FW Stop Valves – CLOSED • SG Blowdown Isolation Valves - CLOSED
BOP	Check if Main Steam Lines should be isolated: a. Check for either: • High Steam Line flow with EITHER Tave less than 541 deg F OR Steam line pressure less than 525 psig. OR • Containment pressure – EVER GREATER THAN 24 psig b. Verify MSIVs - CLOSED

Appendix D			Оре	erator Action			F	orm E	S- D-2
Op Test No.:	1	Scenario #	All	Event #	Attachment 1	Page	26	of	29
Event Descrip	otion:	Attachment 1,	Automa	atic Action V	erification				
Time	Position			Applica	nt's Actions or Beha	avior			

	Verify proper Service Water System operation:
	a. Three Service Water Pumps – Running on
BOP	Essential Header
	b. Service Water valves from Diesel Generator -
	OPEN
	Verify SI system pumps running:
	a. Three SI pumps – RUNNING
BOP	b. 22 SI pump discharge isolation MOV-851A
	AND MOV-851B – OPEN
	c. Two RHR pumps - RUNNING
	Verify proper emergency SI System valve alignment:
	venty proper emergency of System valve alignment.
	a. SI pump cold leg injection valves – OPEN
	o 856A
	o 856E
	o 856C
BOP	 856D b. RHR HX CCW outlet valves – OPEN
	\circ 822A
	o 822B
	c. RHR HX MOVs – OPEN
	o 746
	o 747
	Verify Containment Fan Coolers – IN SERVICE:
	a. Five fan coolers - RUNNING
BOP	b. Charcoal Filter valves - OPEN
	c. Fan normal discharge valves – CLOSED
	d. TCV-1104 and TCV-1105 – BOTH OPEN
BOP	Verify AFW flow to all SGs

Appendix D	· · · · ·		Оре	erator Action)		F	orm E	S- D-2
Op Test No.:	_1	Scenario #	All	Event #	Attachment 1	Page	27	_ of	_29
Event Descrip	otion:	Attachment 1,	Automa	tic Action V	erification				
Time	Position			Applica	nt's Actions or Beha	avior	• • • • • • • • • • • • • • • • • • • •		

	Verify Containment Ventilation Isolation:
BOP	 a. Containment Purge Valves – CLOSEL FCV-1170 FCV-1171 FCV-1172 FCV-1173 Containment Pressure Relief Valves - CLOSED PCV-1190 PCV-1191 PCV-1192
	Note

Appendix D			Ор	erator Actior)		F	Form E	S- D-2
Op Test No.:	_1	Scenario #	All	Event #	Attachment 1	Page	28	of	29
Event Descrip	otion:	Attachment 1,	Automa	atic Action V	erification				
Time	Position			Applica	nt's Actions or Beha	vior			

	Verify Containment Isolation Phase A
	a. Phase A – ACTUATED o Train A master relay CA1 (above rack E) o Train B master relay CA2 (above rack F)
	b. Phase A valves – CLOSED
	c. IVSW valves – OPEN
	o 1410
	o 1413
	 SOV-3518 SOV-3519
	d. WCP valves – OPEN:
	\circ PCV 1238
BOP	• PCV 1239
	o PCV 1240
	o PCV 1241
	e. Place personnel and equipment hatch solenoid control switches to INCIDENT on St
	panel f. Dispatch NPO to periodically check
	 IVSW Tank
	• Level – GREATER THAN 92%
	 Pressure – GREATER THAN 57 PSIG
	 WCP header pressures – GREATER
	THAN 52 PSIG

condition. It will not be performed if conditions aren't met

Appendix D			Operator Action			Fo	Form ES-D-2		
Op Test No.:		Scenario #	All	Event #	Attachment 1	Page	29	of	29
Event Descrip	otion:	Attachment 1,	Automa	itic Action V	erification				
Time	Position		Applicant's Actions or Behavior						

BOP	Check if Containment Spray should be actuated: a. Containment Pressure – EVER GREATER THAN 24 PSIG b. Verify spray pumps – RUNNING c. Verify spray pump discharge valves – OPEN o MOV-866A o MOV-866B o MOV-866D d. Verify Containment Isolation Phase B valves – CLOSED e. STOP all RCPs f. Verify IVSW Isolation Valves – OPEN o 7864 o 7865 o 7866 o 7867
BOP	Verify CCR Air Conditioner Train A and B – RUNNING IN INCIDENT MODE 2
BOP	Notify CRS that Attachment 1 is complete