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Exelon SM
Nuclear

10 CFR 50.90

RS-03-075

April 18, 2003

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

Subject: LaSalle County Station, Units 1 and 2
Facility Operating License Nos. NPF-11 and NPF-18
NRC Docket Nos. 50-373 and 50-374

Request for Amendment to Technical Specifications
Table 3.3.6.1-1, "Primary Containment Isolation Instrumentation"

In accordance with 10 CFR 50.90, Exelon Generation Company (EGC), LLC, hereby requests the following amendment to Appendix A, Technical Specifications (TS), of Facility Operating License Nos. NPF-11 and NPF-18. Specifically, the proposed change will modify TS Table 3.3.6.1-1, "Primary Containment Isolation Instrumentation" to add the requirement to perform a Channel Check in accordance with Surveillance Requirement (SR) 3.3.6.1.1 to thirteen listed instrument functions. The proposed change is the result of the replacement of existing plant equipment with equipment that has the capability of permitting the performance of a Channel Check with the plant in MODES 1, 2 and 3. The proposed change is consistent with the wording specified in NUREG -1434, "Standard Technical Specifications General Electric Plants, BWR/6," Revision 2, dated June 2001.

The information supporting the proposed TS change is subdivided as follows.

Attachment 1 is the notarized affidavit.
Attachment 2 provides our evaluation supporting the proposed change.
Attachment 3 contains a copy of the marked up TS page.
Attachment 4 provides the retyped TS page.

The proposed TS change has been reviewed by the LaSalle County Station Plant Operations Review Committee (PORC) and approved by the Nuclear Safety Review Board (NSRB) in accordance with the Quality Assurance Program.

EGC is notifying the State of Illinois of this application for amendment by transmitting a copy of this letter and its attachments to the designated State Official.

We request approval of the proposed change to occur by May 1, 2004 with an implementation period of 60 days.

A001

Should you have any questions concerning this submittal, please contact Mr. T. W. Simpkin at (630) 657-2821.

Sincerely,

T. W. Simpkin

for Keith R. Jury
Director-Licensing
Mid-West Regional Operating Group

Attachments:

Attachment 1. Affidavit
Attachment 2. Evaluation of Proposed Change
Attachment 3. Markup of Proposed Technical Specification Page Changes
Attachment 4. Retyped Pages for Technical Specification Change

cc: Regional Administrator – NRC Region III
NRC Project Manager, NRR
NRC Senior Resident Inspector – LaSalle County Station
Office of Nuclear Facility Safety – Illinois Department of Nuclear Safety

ATTACHMENT 1
Affidavit

STATE OF ILLINOIS	)	
COUNTY OF DUPAGE	)	
IN THE MATTER OF:	)	
EXELON GENERATION COMPANY (EGC), LLC	)	**Docket Numbers**
LASALLE COUNTY STATION - UNIT 1 and UNIT 2	)	**50-373 and 50-374**

SUBJECT: **Request for Amendment to Technical Specifications Table 3.3.6.1-1, "Primary Containment Isolation Instrumentation"**

AFFIDAVIT

I affirm that the content of this transmittal is true and correct to the best of my knowledge, information, and belief.

T. W. Simpkin
Manager-Licensing
Mid-West Regional Operating Group

Subscribed and sworn to before me, a Notary Public in and for the State above named, this 18th day of April, 2003

Anese L. Grigsby
Notary Public

OFFICIAL SEAL
ANESE L. GRIGSBY
NOTARY PUBLIC, STATE OF ILLINOIS
MY COMMISSION EXPIRES 3-13-2005

ATTACHMENT 2
Evaluation of Proposed Change
Page 1 of 6

1.0 INTRODUCTION

2.0 DESCRIPTION OF PROPOSED AMENDMENT

3.0 BACKGROUND

4.0 REGULATORY REQUIREMENTS & GUIDANCE

5.0 TECHNICAL ANALYSIS

6.0 REGULATORY ANALYSIS

7.0 NO SIGNIFICANT HAZARDS CONSIDERATION (NSHC)

8.0 ENVIRONMENTAL CONSIDERATION

9.0 PRECEDENT

1.0 INTRODUCTION

In accordance with 10 CFR 50.90, Exelon Generation Company (EGC), LLC, hereby requests the following amendment to Appendix A, Technical Specifications (TS), of Facility Operating License Nos. NPF-11 and NPF-18. Specifically, the proposed change will modify TS Table 3.3.6.1-1, "Primary Containment Isolation Instrumentation" to add the requirement to perform a Channel Check in accordance with Surveillance Requirement (SR) 3.3.6.1.1 to thirteen listed instrument functions. The proposed change is the result of the replacement of existing plant equipment with equipment that has the capability of permitting the performance of a Channel Check with the plant in MODES 1, 2 and 3. The proposed change is consistent with the wording specified in NUREG -1434, "Standard Technical Specifications General Electric Plants, BWR/6," Revision 2, dated June 2001.

2.0 DESCRIPTION OF PROPOSED AMENDMENT

The proposed change will modify TS Table 3.3.6.1-1 to add the requirement to perform a Channel Check in accordance with SR 3.3.6.1.1 to the following thirteen instrument functions.

- 1.e. Main Steam Line Tunnel Differential Temperature – High
- 3.e. RCIC Equipment Room Temperature – High
- 3.f. RCIC Equipment Room Differential Temperature – High
- 3.g. RCIC Steam Line Tunnel Temperature – High
- 3.h. RCIC Steam Line Tunnel Differential Temperature – High
- 4.c. RWCU Heat Exchanger Area Temperature – High
- 4.d. RWCU Heat Exchanger Area Ventilation Differential Temperature – High
- 4.e. RWCU Pump and Valve Area Temperature – High
- 4.f. RWCU Pump and Valve Area Differential Temperature – High
- 4.g. RWCU Holdup Pipe Area Temperature – High
- 4.h. RWCU Holdup Pipe Area Ventilation Differential Temperature – High
- 4.i. RWCU Filter/ Demineralizer Valve Room Area Temperature – High
- 4.j. RWCU Filter/ Demineralizer Valve Room Area Ventilation Differential Temperature – High

3.0 BACKGROUND

The current TS for LaSalle County Station (LSCS) does not require the performance of a Channel Check in MODES 1, 2 and 3 for the above identified thirteen instrument functions listed on TS Table 3.3.6.1-1. The current LSCS TS requirements were based on the capability of existing plant equipment. This existing equipment requires the manipulation of toggle switches to observe the different leak detection channels. Due to the one out of one isolation logic associated with these channels, manipulation of the

toggle switches presented a risk of inadvertent isolation from channel spiking as the individual channel switches are selected. During a refueling outage in January/February of 2003, Unit 2 replaced its leak detection system thermocouple monitors. The new thermocouple monitors have continuous reading, simultaneous digital displays that permit the performance of a Channel Check without risk of inadvertent system isolations with the Unit in MODE 1, 2 and 3. After the return to service of Unit 2 in March of 2003, LSCS verified that due to the replacement of this equipment that a Channel Check could be successfully performed on the above listed thirteen instrument functions and as an interim administrative control is currently performing shiftly Channel Checks of the instrument channels. LSCS is currently scheduled to perform a similar equipment replacement on Unit 1 in its upcoming refueling outage in February 2004. The proposed change to add the performance of a Channel Check to the above listed thirteen instrument functions will provide further assurance that the instrumentation is performing as designed.

4.0 REGULATORY REQUIREMENTS & GUIDANCE

10 CFR 50.36(c)(2)(ii)(c), "Criterion 3.", requires that a structure, system or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier be included in the TS.

5.0 TECHNICAL ANALYSIS

5.1 Design Bases

The leak detection system at LSCS uses ambient or differential temperature increases to detect small primary coolant boundary leaks in the Main Steam Line Tunnel and in various rooms of the Reactor Core Isolation Cooling (RCIC) System and the Reactor Water Cleanup (RWCU) System. The existing thermocouple monitors did not have the capability to allow a Channel Check to be performed in MODES 1, 2 and 3 without undue risk of initiating an inadvertent system isolation. Thus, the LSCS TS took exception to the guidance contained in NUREG –1434 and did not specify that a Channel Check be performed.

The new thermocouple monitors have continuously reading digital displays that permit the performance of a Channel Check without risk of inadvertent system isolations with the Unit in MODE 1, 2 and 3. The new thermocouple digital diplays were installed on Unit 2 during the January/February 2003 refuel outage and are scheduled to be installed in Unit 1 during the upcoming January 2004 refuel outage. After the return to service of Unit 2 in March of 2003, LSCS verified that the thermocouple digital displays do permit a Channel Check to be successfully performed on the above listed thirteen instrument functions. Therefore, LSCS is requesting that TS Table 3.3.6.1-1 be modified

to specify that a SR 3.3.6.1.1 Channel Check be performed in MODES 1, 2 and 3, consistent with the guidance contained in NUREG-1434, Rev. 2.

5.2 Risk Information

This submittal is not based on risk informed decision making.

6.0 REGULATORY ANALYSIS

TS Table 3.3.6.1-1 lists primary containment isolation instrumentation that are required to function, in combination with other accident mitigation features, to limit fission product release during and following postulated Design Basis Accidents (DBAs) to within limits. Therefore, TS Table 3.3.6.1-1 must be included in LSCS TS in accordance with 10 CFR 50.36(c)(2)(ii)(c).

7.0 NO SIGNIFICANT HAZARDS CONSIDERATION

EGC has evaluated the proposed change to the TS for LaSalle County Station, Unit 1 and Unit 2, and has determined that the proposed change does not involve a significant hazards consideration and is providing the following information to support a finding of no significant hazards consideration.

Does the change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No

The proposed change to Technical Specifications (TS) Table 3.3.6.1-1, "Primary Containment Isolation Instrumentation" will incorporate into the LaSalle County Station (LSCS) TS, wording specified in NUREG -1434, "Standard Technical Specifications General Electric Plants, BWR/6," Revision 2, dated June 2001. The proposed change will modify TS Table 3.3.6.1-1 to add the requirement to perform a Channel Check in accordance with Surveillance Requirement (SR) 3.3.6.1.1 to thirteen listed instrument functions. The performance of TS surveillance testing is not a precursor to any accident previously evaluated. A Channel Check is a monitoring activity that does not represent an accident initiator. Thus, the proposed change does not have any affect on the probability of an accident previously evaluated.

The function of instrumentation listed on TS Table 3.3.6.1-1, in combination with other accident mitigation features, is to limit fission product release during and following postulated design Basis Accidents (DBAs) to within limits. The surveillance testing specified in TS Table 3.3.6.1-1 will provide assurance that

the instrumentation will perform as designed. Thus, the radiological consequences of any accident previously evaluated are not increased.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Does the change create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No

The proposed change does not affect the control parameters governing unit operation or the response of plant equipment to transient conditions. The failure modes of the new instrumentation do not give rise to a new or different kind of accident. The proposed change does not introduce any new modes of system operation or failure mechanisms.

Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

Does the change involve a significant reduction in a margin of safety?

Response: No

The leak detection system at LaSalle County Station uses ambient or differential temperature increases to detect small primary coolant boundary leaks in the Main Steam Line Tunnel and in various rooms of the Reactor Core Isolation Cooling (RCIC) System and the Reactor Water Cleanup (RWCU) System. The existing thermocouple monitors did not have the capability to allow a Channel Check to be performed without undue risk of initiating an inadvertent system isolation in MODES 1, 2 and 3. Thus, the LSCS TS took exception to the guidance contained in NUREG –1434 and did not specify on TS Table 3.3.6.1-1 that a SR 3.3.6.1.1 Channel Check be performed on the above listed thirteen instrument functions.

The new thermocouple monitors have continuously reading digital displays that permit the performance of a Channel Check with the Unit in MODE 1, 2 and 3 without risk of inadvertent system isolations. The new thermocouple digital displays have been installed on Unit 2 during the January/February 2003 refuel outage and are scheduled to be installed in Unit 1 during the up coming January 2004 refuel outage. LSCS after the return to service of Unit 2 in March of 2003, verified that the thermocouple digital displays do permit a Channel Check to be successfully performed on the above listed thirteen instrument functions. Therefore, LSCS is requesting that TS Table 3.3.6.1-1 is modified to specify that

a SR 3.3.6.1.1 Channel Check be performed in MODES 1, 2 and 3, consistent with the guidance contained in NUREG-1434, Rev. 2.

Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

Based upon the above, EGC concludes that the proposed amendment presents no significant hazards consideration under the standards set forth in 10 CFR50.92(c), and, accordingly, a finding of "no significant hazards consideration" is justified.

8.0 ENVIRONMENTAL CONSIDERATION

A review has determined that the proposed amendment would change a requirement with respect to installation or use of a facility component located within the restricted area, as defined in 10 CFR 20, or would change an inspection or surveillance requirement. However, the proposed amendment does not involve (i) a significant hazards consideration, (ii) a significant change in the types or significant increase in the amounts of any effluent that may be released offsite, or (iii) a significant increase in individual or cumulative occupational radiation exposure. Accordingly, the proposed amendment meets the eligibility criterion for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the proposed amendment.

9.0 PRECEDENT

The proposed amendment incorporates into the LaSalle County Station a change to TS Table 3.3.6.1-1 that is consistent with the wording and intent of NUREG-1434, Rev. 2.

ATTACHMENT 3

MARKUP OF PROPOSED TECHNICAL SPECIFICATION PAGE CHANGES

Revised TS Page

Table 3.3.6.1-1 (page 1 of 4)
Primary Containment Isolation Instrumentation

Change adds 3.3.6.1.1 to the following functions

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Main Steam Line Isolation					
a. Reactor Vessel Water Level – Low Low Low, Level 1	1,2,3	2	D	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5 SR 3.3.6.1.6	≥ -137.0 inches
b. Main Steam Line Pressure – Low	1	2	E	SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.5 SR 3.3.6.1.6	≥ 826.5 psig
c. Main Steam Line Flow – High	1,2,3	2 per MSL	D	SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.5 SR 3.3.6.1.6	≤ 128.0 psid
d. Condenser Vacuum – Low	1,2(a), 3(a)	2	D	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ 3.8 inches Hg vacuum
e. Main Steam Line Tunnel Differential Temperature – High	1,2,3	2	D	~~SR 3.3.6.1.1~~ ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 66.4°F
f. Manual Initiation	1,2,3	2	G	SR 3.3.6.1.5	NA
2. Primary Containment Isolation					
a. Reactor Vessel Water Level – Low Low, Level 2	1,2,3	2	H	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ -58.0 inches
b. Drywell Pressure – High	1,2,3	2	H	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 1.93 psig
c. Reactor Building Ventilation Exhaust Plenum Radiation – High	1,2,3	2	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 42.0 mR/hr
d. Fuel Pool Ventilation Exhaust Radiation – High	1,2,3	2	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 42.0 mR/hr

(continued)

(a) With any turbine stop valve not closed.

Table 3.3.6.1-1 (page 2 of 4)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
2. Primary Containment Isolation (continued)					
e. Reactor Vessel Water Level – Low Low Low, Level 1	1,2,3	2	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ -137.0 inches
f. Reactor Vessel Water Level – Low, Level 3	1,2,3	2	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ 11.0 inches
g. Manual Initiation	1,2,3	1	G	SR 3.3.6.1.5	NA
3. Reactor Core Isolation Cooling (RCIC) System Isolation					
a. RCIC Steam Line Flow – High	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.5	≤ 176.0 inches water
b. RCIC Steam Line Flow – Timer	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ 2.6 seconds and ≤ 5.5 seconds
c. RCIC Steam Supply Pressure – Low	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ 58.2 psig
d. RCIC Turbine Exhaust Diaphragm Pressure – High	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 300 inches water
e. RCIC Equipment Room Temperature – High	1,2,3	1	F	SR 3.3.6.1.1 ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 291.0°F
f. RCIC Equipment Room Differential Temperature – High	1,2,3	1	F	SR 3.3.6.1.1 ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 189.0°F
g. RCIC Steam Line Tunnel Temperature – High	1,2,3	1	F	SR 3.3.6.1.1 ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 277.0°F
h. RCIC Steam Line Tunnel Differential Temperature – High	1,2,3	1	F	SR 3.3.6.1.1 ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 155.0°F
i. Drywell Pressure – High	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 1.77 psig

(continued)

Table 3.3.6.1-1 (page 3 of 4)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
3. RCIC System Isolation (continued)					
j. Manual Initiation	1,2,3	1[(b)]	G	SR 3.3.6.1.5	NA
4. Reactor Water Cleanup (RWCU) System Isolation					
a. Differential Flow – High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 82.8 gpm
b. Differential Flow – Timer	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 48.9 seconds
c. RWCU Heat Exchanger Area Temperature – High	1,2,3	1 per area	F	~~SR 3.3.6.1.1~~ ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 157.0°F
d. RWCU Heat Exchanger Area Ventilation Differential Temperature – High	1,2,3	1 per area	F	~~SR 3.3.6.1.1~~ ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 38.5°F
e. RWCU Pump and Valve Area Temperature – High	1,2,3	1 per area	F	~~SR 3.3.6.1.1~~ ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 209.0°F
f. RWCU Pump and Valve Area Differential Temperature – High	1,2,3	1 per area	F	~~SR 3.3.6.1.1~~ ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 91.0°F
g. RWCU Holdup Pipe Area Temperature – High	1,2,3	1	F	~~SR 3.3.6.1.1~~ ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 209.0°F
h. RWCU Holdup Pipe Area Ventilation Differential Temperature – High	1,2,3	1	F	~~SR 3.3.6.1.1~~ ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 91.0°F
i. RWCU Filter/ Demineralizer Valve Room Area Temperature – High	1,2,3	1	F	~~SR 3.3.6.1.1~~ ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 209.0°F
j. RWCU Filter/ Demineralizer Valve Room Area Ventilation Differential Temperature – High	1,2,3	1	F	~~SR 3.3.6.1.1~~ ~~SR 3.3.6.1.2~~ SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 91.0°F

(continued)

(b) Only inputs into one of two trip systems.

ATTACHMENT 4

RETYPED PAGES
FOR
TECHNICAL SPECIFICATION CHANGE

Retyped TS Page

Table 3.3.6.1-1 (page 1 of 4)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Main Steam Line Isolation					
a. Reactor Vessel Water Level–Low Low Low, Level 1	1,2,3	2	D	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5 SR 3.3.6.1.6	≥ -137.0 inches
b. Main Steam Line Pressure–Low	1	2	E	SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.5 SR 3.3.6.1.6	≥ 826.5 psig
c. Main Steam Line Flow–High	1,2,3	2 per MSL	D	SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.5 SR 3.3.6.1.6	≤ 128.0 psid
d. Condenser Vacuum–Low	1,2[(a)], 3[(a)]	2	D	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ 3.8 inches Hg vacuum
e. Main Steam Line Tunnel Differential Temperature–High	1,2,3	2	D	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 66.4°F
f. Manual Initiation	1,2,3	2	G	SR 3.3.6.1.5	NA
2. Primary Containment Isolation					
a. Reactor Vessel Water Level–Low Low, Level 2	1,2,3	2	H	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ -58.0 inches
b. Drywell Pressure–High	1,2,3	2	H	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 1.93 psig
c. Reactor Building Ventilation Exhaust Plenum Radiation–High	1,2,3	2	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 42.0 mR/hr
d. Fuel Pool Ventilation Exhaust Radiation–High	1,2,3	2	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 42.0 mR/hr

(continued)

(a) With any turbine stop valve not closed.

Table 3.3.6.1-1 (page 2 of 4)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
2. Primary Containment Isolation (continued)					
e. Reactor Vessel Water Level–Low Low Low, Level 1	1,2,3	2	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ -137.0 inches
f. Reactor Vessel Water Level–Low, Level 3	1,2,3	2	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ 11.0 inches
g. Manual Initiation	1,2,3	1	G	SR 3.3.6.1.5	NA
3. Reactor Core Isolation Cooling (RCIC) System Isolation					
a. RCIC Steam Line Flow–High	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.5	≤ 176.0 inches water
b. RCIC Steam Line Flow–Timer	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ 2.6 seconds and ≤ 5.5 seconds
c. RCIC Steam Supply Pressure–Low	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≥ 58.2 psig
d. RCIC Turbine Exhaust Diaphragm Pressure–High	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 300 inches water
e. RCIC Equipment Room Temperature–High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 291.0°F
f. RCIC Equipment Room Differential Temperature–High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 189.0°F
g. RCIC Steam Line Tunnel Temperature–High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 277.0°F
h. RCIC Steam Line Tunnel Differential Temperature–High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 155.0°F
i. Drywell Pressure–High	1,2,3	2	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 1.77 psig

(continued)

Table 3.3.6.1-1 (page 3 of 4)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
3. RCIC System Isolation (continued)					
j. Manual Initiation	1,2,3	1[b]	G	SR 3.3.6.1.5	NA
4. Reactor Water Cleanup (RWCU) System Isolation					
a. Differential Flow–High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 82.8 gpm
b. Differential Flow–Timer	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 48.9 seconds
c. RWCU Heat Exchanger Area Temperature–High	1,2,3	1 per area	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 157.0°F
d. RWCU Heat Exchanger Area Ventilation Differential Temperature–High	1,2,3	1 per area	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 38.5°F
e. RWCU Pump and Valve Area Temperature–High	1,2,3	1 per area	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 209.0°F
f. RWCU Pump and Valve Area Differential Temperature–High	1,2,3	1 per area	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 91.0°F
g. RWCU Holdup Pipe Area Temperature–High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 209.0°F
h. RWCU Holdup Pipe Area Ventilation Differential Temperature–High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 91.0°F
i. RWCU Filter/ Demineralizer Valve Room Area Temperature–High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 209.0°F
j. RWCU Filter/ Demineralizer Valve Room Area Ventilation Differential Temperature–High	1,2,3	1	F	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.5	≤ 91.0°F

(continued)

(b) Only inputs into one of two trip systems.