ES-301 Control Room Systems and Facility Walk-Through Test Outline

Form-301-2

Faci Exar	lity: <u>Hope Creek</u> m Level: <u>SRO(L)</u>	Date of Examina Operating T	tion: <u>3/17/(</u> est No.: 1	<u>)</u> 3
B.1:	Control Room System	ns	<u> </u>	
	System	JPM Description	Type Code*	Safety Function
S.1	215004 G2.1.23 (4.0) Source Range Monitor	Ability to perform specific system and integrated plant procedures during different modes of plant operation.	D, L	IC
		SRM/IRM Rod Block Bypassing during refueling operations IAW HC.OP-SO.SE-0001 Section 5.4. Perform independent verification of installed jumpers.		
S.2	204000 G2.1.20 (4.2) RWCU	Ability to execute procedure steps.	N, R, E, L, A	AUX/ DHR
		Align RWCU for Alternate Heat Removal Alternate path for bypassing RHX for additional cooling.		
S.3	234000 G2.2.28 (3.5) Fuel Handling Systems	Knowledge of new and spent fuel movement procedures.	N, R, A	FHE
		Manual transfer of dummy bundle within Spent Storage Pool. Unexpected Slack Cable / Bent Mast IAW HC.OP-SO.KE-0001 Attachment 2 (perform or simulate) (JPM-KE-014 Modified for Alternate path due to unexpected Slack Cable.)		
S.4	234000 A3.02 (3.7) Fuel Handling	Interlock operation	N, R	FHE
	Systems	Test HC.OP-FT.KE-0001 Section 5.4.1 through 5.4.15 (perform or simulate)		
S.5	234000 A3.01 (3.6)	Crane/refuel bridge movement.	N, R	FHE
	Systems	Automatic dummy bundle transfer in the Spent Fuel Pool (perform actual movement). Draft based on Peach Bottom JPM.		
B.2:	Facility Walk-Throug	h (Same as RO In-Plant Walkthrough)		<u> </u>
NΔ	NA	NA	NA	NA

STATION:	HOPE CREEK							
SYSTEM:	215004 Source Range Monitor							
TASK:	SRM/IRM Rod E Section 5.4. Per	Block Bypassii form indepen	ng during refu dent verificatio	eling opera on of install	itions IAW ed jumper	HC.OP- s.	-SO.SE-0001	
TASK NUMBER:	215004 A1.04							
JPM NUMBER:	2003-NRC-LSRO-	-S1						
ALTERNATE PATH	ł: 🛄		L			233000	A2 02	
APPLICABILITY: EO		SRO X	IMPORTAN LSRO X	CE FACTO	R: <u>3</u> R: <u>8</u>	.1 .0	3.3 SRO	
EVALUATION SET	TING/METHOD:	REACTOR B	UILDING/ SIMI	JLATE				
REFERENCES:	HC.OP-SO.SE-00	01 Rev 10						
TOOLS AND EQUI	PMENT: Flashlig	ht						
VALIDATED JPM (COMPLETION TIME	i:	5 min.					
	NTIFIED FOR TIME	CRITICAL ST	EPS:	N/A	min.			
APPROVED:								
N/A BARGAININ REPRESENT	G UNIT	TRAINING S	UPERVISOR		OPER/	ATIONS I or Desig	MANAGER	
CAUTION:	No plant equipm following: 1. Permission f 2. Direct oversi permission k 3. Verification c	ent shall be o rom the OS or ght by a quali based on plant of the "as left"	perated during Unit CRS; fied individual conditions).	y the perfor (determine a qualified	mance of a ed by the ir individual.	ı JPM wit ıdividual	thout the granting	
L								
ACTUAL JPM CON	IPLETION TIME:		min.					
ACTUAL JPM CON ACTUAL TIME CR	MPLETION TIME:	ON TIME:	min. N/A	min.				
ACTUAL JPM COM ACTUAL TIME CR JPM PERFORMED	MPLETION TIME: ITICAL COMPLETIC BY:	ON TIME:	min. N/A	min. GRADE:	SAT		UNSAT	
ACTUAL JPM COM ACTUAL TIME CR JPM PERFORMED REASON, IF UNSA	MPLETION TIME: ITICAL COMPLETION BY: TISFACTORY:	ON TIME:	min. N/A	min. GRADE:	SAT		UNSAT	

•

NC.TQ-WB.ZZ-0310(Z)

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: _____

DATE: _____

SYSTEM: 215004 Source Range Monitor

 TASK:
 SRM/IRM Rod Block Bypassing during refueling operations IAW HC.OP-SO.SE-0001

 Section 5.4. Perform independent verification of installed jumpers

TASK 215004 A1.04

INITIAL CONDITIONS:

- You are the Refueling SRO.
- The plant is in Operational Condition 5.
- All fuel has been moved to the Fuel Pool.
- The CRS has directed I&C to bypass the SRM/IRM Rod Blocks IAW HC.OP-SO.SE-0001

Section 5.4.

• I&C has performed step 5.4.3 of HC.OP-SO.SE-0001.

INITIATING CUE: Perform the Independent Verification requirements of steps 5.4.4 and 5.4.6 following I&C jumper installation IAW HC.OP-SO.SE-0001.

Successful Completion Criteria:

- 1. All critical steps completed.
- 2. All sequential steps completed in order.
- 3. All time-critical steps completed within allotted time.
- 4. JPM completed within validated time. Completion time may exceed the validated time if satisfactory progress is being made (and NRC concurrence is obtained).

NAME: ______ DATE: _____

NC.TQ

.ZZ-0310(Z)

SYSTEM: 215004 Source Range Monitor

TASK: SRM/IRM Rod Block Bypassing during refueling operations IAW HC.OP-SO.SE-0001 Section 5.4. Perform independent verification of installed jumpers.

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
*	5.4.4	START TIME: PERFORM an independent verification of the installed jumper from Step 5.4.3. INITIAL Attachment 1.	 Operator proceeds to panel 10C635, opens the Cabinet door and locates terminal strip DD. Operator locates terminals DD-19 and DD-20. Examiner Cue: A jumper is installed on the terminals indicated. Examiner Note: Initialing Attachment 1 is not critical. Operator initials Attachment 1 as independent verifier. 		
	5.4.5	INSTALL a jumper in Panel 10C636 from AA35 to AA36. INITIAL Attachment 1.	Examiner Cue: "The I&C technician has completed step 5.4.5"		

NAME:

NC.TQ-....ZZ-0310(Z)

DATE:

SYSTEM: 215004 Source Range Monitor

TASK: SRM/IRM Rod Block Bypassing during refueling operations IAW HC.OP-SO.SE-0001 Section 5.4. Perform independent verification of installed jumpers.

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
*	5.4.6	PERFORM an independent verification of the installed jumper from Step 5.4.5. INITIAL Attachment 1.	Operator proceeds to panel 10C636, opens the Cabinet door and locates terminal strip AA. Operator locates terminals AA-35 and AA- 36. Examiner Cue: A jumper is installed on the terminals indicated. Operator initials Attachment 1 as independent verifier.		
		STOP TIME:	Operator notifies Control Room that independent verification is complete. Terminating Cue: This JPM is complete.		

INITIAL CONDITIONS:

- You are the Refueling SRO.
- The plant is in Operational Condition 5.
- All fuel has been moved to the Fuel Pool.
- The CRS has directed I&C to bypass the SRM/IRM Rod Blocks IAW HC.OP-SO.SE-0001 Section 5.4.
- I&C has performed step 5.4.3 of HC.OP-SO.SE-0001.

INITIATING CUE: Perform the Independent Verification requirements of steps 5.4.4 and 5.4.6 following I&C jumper installation IAW HC.OP-SO.SE-0001.

TVE ON-THE-SPOT CHA	anges must be at	TTACHED FOR FIELD USE		Page 1 c	f 1
PSEG Internal Use Onl	<u>y</u> PSEG	NUCLEAR L.L.C.			
	HOPE CREEK	GENERATING STATIO	N		
	HC.OP-SO	D.SE-0001(Q) - Rev. 10			
			PATION		
NU	CLEAR INSTRUMI	ENTATION SYSTEM OP	CIATION		
NU USE CATEGORY: I	CLEAR INSTRUMI	ENTATION SYSTEM OP			
NUO USE CATEGORY: I A.	CLEAR INSTRUMI	Biennial Review performed	Yes	No N/	A
NUC USE CATEGORY: I A. B. Change Package(s) and	CLEAR INSTRUMI	Biennial Review performed	Yes	No N/	 A
NUC USE CATEGORY: I A. B. Change Package(s) and CP No.	CLEAR INSTRUMI	Biennial Review performed iumber(s) incorporated into thi AD No.	Yes s revision. AD Rev. No.	No N/ or Non	A
NUC USE CATEGORY: I A. B. Change Package(s) and CP No. C. OTSC(s) incorporated	CLEAR INSTRUMI d Affected Document N CP Rev. No into this revision:	Biennial Review performed Number(s) incorporated into thi AD No.	Yes s revision. AD Rev. No.	No N/ or Non	A c

REVISION SUMMARY

- 1. Based on request made under **Orders 70021956** and **80038788** the direction in steps 5.1.4 and 5.3.5 was revised to include verification that proper bypass indication to the RBM or CMS is observed when bypassing an LPRM, and that this indication is cleared when the LPRM is returned to service.
- 2. Based on writers review, several steps where changed from alphanumerical lists to bulleted lists since the steps did not require sequential performance. These changes are editorial in nature and revision bars have been omitted. This affected steps 3.2.1, 5.1.2.B, 5.1.3.C, 5.1.5.B, and Note 5.3. Also, revised Step 6.1 by changing NC.NA-AP.ZZ-0003(Q), Document Management Program to NC.NA-AP.ZZ-0011(Q), Records Management Program. This change is also editorial in nature as described in NC.DM-AP.ZZ-0001(Q). Also, Notes 5.1 and 5.3, which pointed out that compliance with a list of T/Ss was a requirement were removed. The requirements stated in these notes is currently stated as a limitation 3.2.1 making it redundant information.

IMPLEMENTATION REQUIREMENTS

Effective date 02/12/02

None		
	(\cdot)	
APPROVED:		626502
	Manager - Hope Creek Operations	Date

HC.OP-SO.SE-0001(Q)

NUCLEAR INSTRUMENTATION SYSTEM OPERATION

TABLE OF CONTENTS

SECTION	TITLE		PAGE
1.0	PURPOS	SE	2
2.0	PREREC	QUISITES	2
3.0	PRECA	UTIONS & LIMITATIONS	3
4.0	EQUIPN	IENT REQUIRED	5
5.0	PROCE	DURE	6
	5.1	Placing the System in Service	6
	5.2	SRM/IRM Detector Drive Operation	10
	5.3	System Bypassing	12
	5.4	SRM/IRM Rod Block Bypassing during Refueling operations (all Fuel Removed from Vessel)	17
6.0	RECOR	DS	19
7.0	REFER	ENCES	19
ATTACHMI	ENTS		
Attachment	l	Installation and Removal of jumpers for SRM/IRM Rod Blocks	20

HC.OP-SO.SE-0001(Q)

NUCLEAR INSTRUMENTATION SYSTEM OPERATION

1.0 PURPOSE

This procedure outlines the steps necessary for the operation of the Nuclear Instrumentation System.

2.0 **PREREQUISITES**

2.1 Placing the System in Service

- 2.1.1 Reactor Protection System is in service to receive inputs from the Nuclear Instrumentation System.
- 2.1.2 Applicable Precautions and Limitations have been reviewed by each procedure user.

2.2 SRM/IRM Detector Drive Operation

- 2.2.1 Nuclear Instrumentation System is in service IAW Section 5.1 of this procedure.
- 2.2.2 Applicable Precautions and Limitations have been reviewed by each procedure user.

2.3 System Bypassing

- 2.3.1 Nuclear Instrumentation System is in service IAW Section 5.1 of this procedure.
- 2.3.2 Applicable Precautions and Limitations have been reviewed by each procedure user.

2.4 SRM/IRM Rod Block Bypassing during Refueling Operations

- 2.4.1 Nuclear Instrumentation System is in service IAW Section 5.1 of this procedure.
- 2.4.2 All Fuel is removed from Reactor Vessel.
- 2.4.3 Applicable Precautions and Limitations have been reviewed by each procedure user.

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE

HC.OP-SO.SE-0001(Q)

3.0 PRECAUTIONS AND LIMITATIONS

3.1 Precautions

- 3.1.1 Control Rods shall not be moved when the SRM OR IRM Detectors are in motion.
- 3.1.2 An APRM Channel is inop $\underline{IF} < 2 \text{ LPRM}$ inputs per level $\underline{OR} < 14 \text{ LPRM}$ inputs per APRM Channel.
- 3.1.3 Before removing a joystick from Bypass, ensure bypassed Channel is reading approximately the same as the other Channels <u>AND</u> no HIGH or HIGH HIGHS exist on the bypassed Channel.

3.2 Limitations

- 3.2.1 Ensure compliance with the following Technical Specifications:
 - T/S 2.2.1 Reactor Protection System Instrumentation Setpoints
 - T/S 3/4.2.2 APRM Setpoints
 - T/S 3/4.3.1 Reactor Protection System Instrumentation
 - T/S 3/4.3.6 Control Rod Block Instrumentation
 - T/S 3/4.3.7.6 Source Range Monitors
 - T/S 3/4.9.2 Refueling Operations Instrumentation

TABLE SE-001				
PARAMETER	SETPOINT	BYPASSED		
SRM Downscale	3 cps	IRM's \geq range 3 or Reactor Mode Switch in RUN		
SRM Upscale	1 X 10 ⁵ cps	IRM's \geq range 8 or Reactor Mode Switch in RUN.		
SRM Inoperative	a) Module unpluggedb) Low Voltagec) Mode Switch out of Operate	IRM's ≥ range 8 <u>or</u> Reactor Mode Switch in RUN.		
SRM Detector Wrong Position	\leq 100 cps and Detectors not fully inserted	IRM's \geq range 3 or Reactor Mode Switch in RUN.		
IRM Downscale	5/125 of full scale	IRM's in range 1 position or Reactor Mode Switch in RUN.		
IRM Upscale	108/125 of full scale	Reactor Mode Switch in RUN		
IRM Inoperative	 a) Module unplugged b) Low Voltage c) Mode Switch out of Operate 	Reactor Mode Switch in RUN.		
IRM Detector Wrong Position	Detector not fully inserted	Reactor Mode Switch in RUN.		
APRM Upscale Thermal Power (Flow Biased)	.66(W-dW)+ 42% *FRTP CMFLPD	Reactor Mode Switch not in RUN.		
APRM Downscale	4% of Rated Thermal Power	Reactor Mode Switch not in RUN.		
APRM Upscale	12% of Rated Thermal Power	Reactor Mode Switch in RUN.		
APRM Inoperative	 a) ** < 14 LPRM inputs b) Module unplugged c) Mode Switch not in Operate 	None		
Flow Unit Inoperative	a) Module unpluggedb) Mode Switch not in Operate	Reactor Mode Switch not in RUN.		
Flow Unit Upscale	111% of Rated Flow	Reactor Mode Switch not in RUN.		
Comparator Trip	10% between Flow Unit outputs	Reactor Mode Switch not in RUN.		

3.2.2 The signals listed in Table SE-001 will initiate a Control Rod block.

* IF FRTP < 1

CMFLPD

** < 2 LPRM inputs per level is an Administrative limit imposed by Tech. Spec. Table 3.3.1-1 note (e). There are no automatic actions associated with this limit. Operator action is required to implement the appropriate response for an inoperable APRM.

HC.OP-SO.SE-0001(Q)

TABLE SE-002				
PARAMETER	SETPOINT	BYPASSED		
SRM Upscale	2 X 10 ⁵ cps	Shorting Links installed		
IRM Upscale	120/125 of full scale	Reactor Mode Switch in RUN		
IRM Inoperative	a) Module unpluggedb) Low Voltagec) Mode Switch not in Operate	Reactor Mode Switch in RUN		
APRM Upscale	118% of Rated Thermal Power	None		
APRM Upscale	15% of Rated Thermal Power	Reactor Mode Switch in RUN.		
APRM Upscale Thermal Power	.66(W-dW)+ 51% * <u>FRTP</u> CMFLPD 113.5% maximum with high flow clamped	None		
APRM Inoperative	 a) **< 14 LPRM inputs b) Module unplugged c) Mode Switch not in Operate 	None		

3.2.3 The signals listed in Table SE-002 will initiate a Reactor Scram.

CMFLPD

** < 2 LPRM inputs per level is an Administrative limit imposed by Tech. Spec. Table 3.3.1-1 note (c). There are no automatic actions associated with this limit. Operator action is required to implement the appropriate response for an inoperable APRM.</p>

3.2.4 The signals listed in Table SE-003 will enable the Oscillation Power Range Monitor (OPRM) :

TABLE SE-003				
PARAMETER	SETPOINT	BYPASSED		
APRM POWER	> 30%	NONE		
CORE FLOW	< 60%	NONE		

4.0 EQUIPMENT REQUIRED

None

ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE

HC.OP-SO.SE-0001(Q)

<u>NOTE</u> 5.0

All operations are performed from 10C651C unless otherwise noted.

5.0 **PROCEDURE**

5.1 Placing the System in Service

- 5.1.1 **ENSURE** all prerequisites have been satisfied IAW Section 2.1.
- 5.1.2 **PLACE** Source Range Monitors in service as follows:
 - A. PLACE SRM A,B,C & D MODE SWITCHES in OPERATE. (10C635 & 10C636)
 - B. **OBSERVE** the following annunciators are clear:
 - SRM UPSCALE OR INOPERATIVE
 - SRM PERIOD
 - SRM DOWNSCALE
 - C. **OBSERVE all SOURCE** RANGE NEUTRON MONITORING MONITOR STATUS lights are OFF.
 - D. OBSERVE STARTUP RANGE MONITOR RECORDERS A AND C AND B AND D indicate > 3 cps. (10C650C)

5.1.3 **PLACE** Intermediate Range Monitors in service as follows:

- A. **PLACE IRM A,B,C,D,E,F,G&H MODE SWITCHES in** OPERATE. (10C635 & 10C636)
- B. **PLACE** CHANNEL A,B,C,D,E,F,G&H IRM RANGE SELECT RANGE SWITCHES to 1.
- C. **OBSERVE** the following annunciators are clear:
 - IRM UPSCALE
 - IRM A/B/E/F UPSCALE INOP/ TRIP
 - IRM C/D/G/H UPSCALE INOP/ TRIP

Continued Next Page

Hope Creek

Page 6 of 20

Rev. 10

HC.OP-SO.SE-0001(Q)

5.1.3 (Continued)

D. **OBSERVE** IRM RPS TRIP CHANNELS A,B MONITOR STATUS lights are OFF (except DNSC).

E. **PRESS** RECORDER INPUT R603 A,B,C&D IRM A, B, C, D, E,F, G&H PBs.

CAUTION 5.1.4

LPRM readings should be consistent with plant conditions before returning to service.

5.1.4	PLA	CE Local Power Range Monitors in service as follows:
	А.	VERIFY LPRM reading is consistent with plant conditions.
	Β.	BYPASS APRM(s) in the affected cabinet.
	C.	At Panel 10C608, PLACE LOCAL POWER RANGE MONITOR Control Switch to OP <u>THEN</u> OBSERVE the LPRM BYPASS light is OFF.
	D.	OBSERVE APRM AND LPRM UPSCALE annunciator is clear.
	E.	At Panel 10C651, SELECT a control rod so that the LPRM is displayed on the Four Rod Display <u>THEN</u> OBSERVE the following:
		1. The associated BYPASS light on the Four Rod Display is OFF.
		2. The LPRM reading is consistent with plant conditions.
	F.	At the CMS, SELECT OD-8 <u>THEN</u> OBSERVE that the LPRM is reading consistent with plant conditions.
Continued Next Page	G.	VERIFY APRM GAFS are within ± 2% of rated CTP, <u>AND</u> , <u>IF</u> necessary, ADJUST GAFS IAW HC.RE-ST.SE-0002(Q).

LIL ATTNE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE

HC.OP-SO.SE-0001(Q)

- $ -$	
11. UNBTRASS AFRICA(S) in the affected cubined.	
I. NOTIFY Reactor Engineer of LPRM placed in service.	
PLACE Average Power Range Monitors in service as follows:	
A. PLACE APRM A,B,C,D,E.F MODE SWITCHES in OPERATE. (10C608)	
B. OBSERVE the following annunciators are clear:	
APRM SYS A UPSCALE TRIP/INOP	
APRM SYS B UPSCALE TRIP/INOP	
APRM UPSCALE	
C. OBSERVE APRM RPS TRIP CHANNEL A.B MONITOR STATUS lights are OFF (except DNSC).	
D. OBSERVE APRM DOWNSCALE annunciator is energized.	· <u> </u>
PLACE Oscillation Power Range Monitor in service as follows:	
 A. For the following Oscillation Power Range Monitors, PLACE the following OPRM MANUAL BYPASS/AUTO/OPRM MANUAL ENABLE Switch (S4) in the AUTO position (10C608) 	1):
 OPRMA1 OPRMA2 OPRMB1 OPRMB2 OPRMC1 OPRMC2 OPRMD1 	
	 NOTIFY Reactor Engineer of LPRM placed in service. PLACE Average Power Range Monitors in service as follows: A. PLACE APRM A,B,C,D,E,F MODE SWITCHES in OPERATE. (10C608) B. OBSERVE the following annunciators are clear: APRM SYS A UPSCALE TRIP/INOP APRM SYS B UPSCALE TRIP/INOP APRM UPSCALE C. OBSERVE APRM RPS TRIP CHANNEL A.B MONITOR STATUS lights are OFF (except DNSC). D. OBSERVE APRM DOWNSCALE annunciator is energized. PLACE Oscillation Power Range Monitor in service as follows: A. For the following Oscillation Power Range Monitors, PLACE the following OPRM MANUAL BYPASS/AUTO/OPRM MANUAL ENABLE Switch (S4) in the AUTO position (10C608

Continued Next Page

5.1.6 (Continued)

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 10030408

B.	For the following Oscillation Power Range Monitors.
	PLACE the OPERATE/TEST Key Switch in the OPERATE
	position. (10608):

	1.	OPRMA1	
	2.	OPRMA2	<u> </u>
	3.	OPRMB1	
	4.	OPRMB2	
	5.	OPRMC1	
	6.	OPRMC2	
	7.	OPRMD1	
	8.	OPRMD2	
C.	OBS	ERVE the following annunciator window is clear :	
	Wind	dow Box C3-F3 - OPRM TRIP BYP/INOP/TRBL	
D.	OBS	ERVE the local LED indicators for the following Oscillation	
	Powe	er Range Monitors are off (except READY). (10C608)	
	1.	OPRMA1	
	2.	OPRMA2	
	3.	OPRMB1	
	4.	OPRMB2	
	5.	OPRMC1	
	6.	OPRMC2	
	7.	OPRMD1	
	8.	OPRMD2	

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030108

HC.OP-SO.SE-0001(Q)

5.2 SRM/IRM Detector Drive Operation

5.2.1 **ENSURE** all prerequisites have been satisfied IAW Section 2.2.

<u>NOTE</u> 5.2.2

- A. When all IRM Range Switches are on range 3 <u>OR</u> above, SRM detectors may be fully withdrawn without causing a rod withdrawal block.
- B. When Reactor Mode Switch is in RUN, IRM detectors may be fully withdrawn without causing a rod withdrawal block.

CAUTION 5.2.2

Control Rods shall not be moved when the SRM OR IRM Detectors are in motion.

- 5.2.2 <u>WHEN</u> desired, **DRIVE OUT** SRM (IRM) Detectors as follows:
 - A. **PRESS** DETECTOR DRIVE POWER ON/OFF PB POWER ON comes on.

NOTE 5.2.2.B

Only one SRM Detector should be selected during drive out operation when SRM's are required to be operable.

- B. **PRESS** required SRM (IRM) DETECTOR SELECT PBs. Backlight comes on.
- C. <u>IF</u> driving out SRM detectors, MAINTAIN STARTUP RANGE MONITOR RECORDERS A(C), B(D) (10C650C) between 100 cps AND 10⁵ cps.
- D. **PRESS** AND HOLD DETECTOR DRIVE OUT PB. DRIVE OUT comes on.

Continued next page

5.2.2 (continued)

- <u>WHEN STARTUP RANGE MONITOR RECORDERS</u> (10C650C) indicates desired level
 <u>OR SRM (IRM) DETECTOR SELECT OUT is on,</u> **RELEASE** DETECTOR DRIVE OUT PB. DRIVE OUT goes off.
- F. **PRESS** selected SRM (IRM) DETECTOR SELECT PBs Backlight goes off.
- G. <u>WHEN</u> all SRM (IRM) DETECTOR SELECT OUT are on, **PRESS** DETECTOR DRIVE POWER ON/OFF PB. OFF comes on.
- 5.2.3 <u>WHEN</u> desired, DRIVE IN SRM (IRM) Detectors as follows:
 - A. **PRESS** DETECTOR DRIVE POWER ON/OFF PB. POWER ON comes on.
 - B. **PRESS** required SRM (IRM) DETECTOR SELECT PBs. Backlight comes on.

NOTE 5.2.3.C

"DRIVE IN" is a seal-in signal. All selected detectors will drive in until their respective "DETECTOR SELECT IN" is on.

- C. **PRESS** DETECTOR DRIVE IN PB. DRIVE IN AND DRIVING IN indicators come on.
- <u>WHEN</u> all selected SRM (IRM) DETECTOR SELECT IN indicators are on,
 VERIFY DRIVING IN is off.
 PRESS DETECTOR DRIVE IN PB. (DRIVE IN goes off).
- E. **PRESS** selected SRM (IRM) DETECTOR SELECT PBs. Backlight goes off.
- F. **PRESS** DETECTOR DRIVE POWER ON/OFF PB. OFF comes ON.

5.3 System Bypassing

- 5.3.1 **ENSURE** all prerequisites have been satisfied IAW Section 2.3.
- 5.3.2 To bypass a Source Range Monitor. **PERFORM** the following:

<u>NOTE</u> 5.3.2.A

If the Reactor Mode Switch is in the RUN position, the Source Range Neutron Monitoring Status Bypass light will NOT illuminate.

A. **PLACE** SOURCE RANGE NEUTRON MONITORING MONITOR BYPASS joystick to the SRM Channel position to be bypassed. **OBSERVE** associated SOURCE RANGE NEUTRON MONITORING MONITOR STATUS BYPASS is on.

CAUTION 5.3.2.B

- The bypassed Channel should be reading approximately the same as the other Channels.
- No upscale alarm <u>OR</u> upscale trip exists on the bypassed Channel before removing a joystick from Bypass.
- Placing a joystick to BYPASS defeats the MONITOR STATUS lights (except BYPASS) for the respective channel.
- Status of upscale alarms and trips must be checked at local panel.
 - B. <u>WHEN</u> SRM bypass is no longer required, PLACE SOURCE RANGE NEUTRON MONITORING MONITOR BYPASS joystick to center position. OBSERVE associated SOURCE RANGE NEUTRON MONITORING MONITOR STATUS BYPASS is off.

5.3.3 To bypass an Intermediate Range Monitor, **PERFORM** the following:

NOTE 5.3.3.A

If the Reactor Mode Switch is in the RUN position, the Intermediate Range Neutron Monitoring Status Bypass light will NOT illuminate.

PLACE IRM RPS TRIP CHANNEL A(B) MONITOR BYPASS joystick to the IRM Channel position to be bypassed.
 OBSERVE associated IRM RPS TRIP CHANNEL A(B) MONITOR STATUS BYPASS is on.

CAUTION 5.3.3.B

- The bypassed Channel should be reading approximately the same as the other Channels.
- No upscale alarm or upscale trip exists on the bypassed Channel before removing a joystick from Bypass.
- Placing a joystick to BYPASS defeats the MONITOR STATUS lights (except BYPASS) for the respective channel.
 - Status of upscale alarms and trips must be checked at local panel.
 - B. <u>WHEN</u> IRM bypass is no longer required,
 PLACE IRM RPS TRIP CHANNEL A(B) MONITOR BYPASS joystick to center position.
 OBSERVE associated IRM RPS TRIP CHANNEL
 A(B) MONITOR STATUS BYPASS is off.

- 5.3.4 To bypass an Average Power Range Monitor, **PERFORM** the following:
 - PLACE APRM RPS TRIP CHANNEL A(B) MONITOR BYPASS joystick to the APRM Channel position to be bypassed <u>THEN</u> OBSERVE associated APRM RPS TRIP CHANNEL A(B) MONITOR STATUS BYPASS is on.

CAUTION 5.3.4.B

- The bypassed Channel should be reading approximately the same as the other Channels.
- No upscale alarm or upscale trip exists on the bypassed Channel before removing a joystick from Bypass.
- Placing a joystick to BYPASS defeats the MONITOR STATUS lights (except BYPASS) for the respective channel.
- Status of upscale alarms and trips must be checked at local panel.
 - B. <u>WHEN</u> APRM bypass is no longer required,
 PLACE APRM RPS TRIP CHANNEL A(B) MONITOR BYPASS joystick to center position
 <u>THEN</u> OBSERVE associated APRM RPS TRIP CHANNEL A(B) MONITOR STATUS BYPASS is off.

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE

HC.OP-SO.SE-0001(Q)

NO1	ΓE	5.	3.	.5

- A. An APRM Channel is inop <u>IF</u> < 2 LPRM inputs per level <u>OR</u> < 14 LPRM inputs per APRM Channel.
- B. The Rod Block Monitor receives input from LPRMs. A RBM channel is considered administratively inoperable (for affected control rods)
 <u>IF</u> < 50% of the LPRM inputs on a given LPRM level are available unless a situation specific evaluation has been performed. [70005801]
 - 5.3.5 To bypass a Local Power Range Monitor, PERFORM the following:
 - A. <u>IF LPRM is in an APRM,</u> <u>THEN, VERIFY operability IAW T/S Table 3.3.1-1 Note (c)</u> <u>OR, DECLARE associated APRM INOP.</u>
 - B. **VERIFY** RBM Channel operability for the affected control rods IAW T/S 3.3.6.
 - C. **BYPASS** APRM(s) in the affected cabinet.
 - D. At Panel 10C608,
 PLACE LOCAL POWER RANGE MONITOR Control Switch to BY THEN, OBSERVE the LPRM BYPASS light is ON.
 - E. At Panel 10C651,
 SELECT a control rod so that the LPRM is displayed on the Four Rod Display
 <u>THEN</u> OBSERVE that the associated BYPASS light on the Four Rod Display is ON.
 - F. At the CMS, SELECT OD-8 THEN OBSERVE that the LPRM is indicating '3' in a 'RED' field.
 - G. VERIFY APRM GAFS are within ± 2% of rated CTP AND, IF necessary, ADJUST GAFS IAW HC.RE-ST.SE-0002(Q).

Continued Next Page

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030108

HC.OP-SO.SE-0001(Q)

5.3.5 (Continued)

- H. UNBYPASS APRM(s) in the affected cabinet.
- I. NOTIFY Reactor Engineer of LPRM bypassed.
- J. <u>WHEN LPRM bypass is no longer required</u>, **PLACE LPRM in service** IAW step 5.1.4.

<u>NOTE</u> 5.3.6

Bypassing a Recirc Flow Unit with the joystick only removes it from the Comparator Circuit. To fully bypass unit requires I&C to bypass the unit at Panel 10C608 Pwr Range Neutron Mon Cab.

- 5.3.6 To bypass a Recirc Flow Unit from the Comparator Circuit, **PERFORM** the following:
 - PLACE RECIRC FLOW MONITORING RPS TRIP CHANNEL A(B) MONITOR BYPASS joystick to required flow channel position.
 OBSERVE associated RECIRC FLOW MONITORING RPS TRIP CHANNEL A(B) MONITOR STATUS FLOW BYPASS is on.
 - B. <u>WHEN</u> Recirc Flow Unit bypass is no longer required,
 PLACE RECIRC FLOW MONITORING RPS TRIP CHANNEL
 A(B) MONITOR BYPASS joystick to center position.
 OBSERVE associated RECIRC FLOW MONITORING RPS
 TRIP CHANNEL A(B) MONITOR STATUS FLOW BYPASS is off.

- 5.3.7 To bypass (trip disable) an OPRM, **PERFORM** the following:
 - A. PLACE OPRM MANUAL BYPASS/AUTO/OPRM MANUAL ENABLED Switch (S4) to the OPRM MANUAL BYPASS position. (10C608)
 - B. **OBSERVE** Annunciator Window Box C3-F3 TRIP BYP/INOP/TRBL is illuminated.
 - C. <u>WHEN</u> OPRM MANUAL BYPASS is not required, PLACE MANUAL BYPASS/AUTO/OPRM MANUAL ENABLED Switch (S4) to the AUTO position. (10C608)
 - D. **OBSERVE** Annunciator Window Box C3-F3 TRIP BYP/INOP/TRBL is extinguished.

5.4 SRM/IRM Rod Block Bypassing during Refueling Operations.

<u>NOTE 5.4</u>

- A. All steps in this Section are to be completed in sequence.
- B. Installation and removal of jumpers should be performed by a qualified I&C Technician.
 - 5.4.1 **ENSURE** all prerequisites are satisfied IAW Section 2.4.
 - 5.4.2 LOG bypassing of SRM/IRM Control Rod Blocks in the Tech Spec Action Statement Log.
 - 5.4.3 INSTALL a jumper in Panel 10C635 from DD19 to DD20. INITIAL Attachment 1.
 - 5.4.4 **PERFORM** an independent verification of the installed jumper from Step 5.4.3. **INITIAL** Attachment 1.

HC.OP-SO.SE-0001(Q)

5.4.5	INSTALL a jumper in Panel 10C636 from AA35 to AA36.
	INITIAL Attachment 1.

- 5.4.6 **PERFORM** an independent verification of the installed jumper from Step 5.4.5. **INITIAL** Attachment 1.
- 5.4.7 SRM/IRM Rod Blocks are now bypassed. Control Rod withdrawal should be permitted.
- 5.4.8 <u>WHEN</u> the SRM/IRM refueling Rod Block bypass is no longer desired. **REMOVE** the jumper in Panel 10C635 from DD19 to DD20. **INITIAL** Attachment 1.
- 5.4.9 PERFORM an independent verification of the removed jumper from Step 5.4.8.
 INITIAL Attachment 1.
- 5.4.10 **REMOVE** the jumper in Panel 10C636 from AA35 to AA36. **INITIAL** Attachment 1.
- 5.4.11 PERFORM an independent verification of the removed jumper from step 5.4.10.
 INITIAL Attachment 1.
- 5.4.12 VERIFY SRM/IRM rod blocks are functioning AND CLEAR the action statement log.

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030108

HC.OP-SO.SE-0001(Q)

6.0 RECORDS

- 6.1 **RETAIN** the following IAW NC.NA-AP.ZZ-0011(Q); Records Management Program:
 - Procedure cover page
 - Attachment 1 Installation and Removal of Jumpers for SRM/IRM Rod Blocks

7.0 **REFERENCES**

- 7.1 Panel Drawings: J-0651-1; Sht. 6, Sht. 7
- 7.2 <u>GE Documents</u>: GEK-90300, Vol. IV, Pt. 1, Sept. 1983 GEK-90300, Vol. IV, Pt. 3, Sept. 1983
- 7.3 **Procedures** HC.RE-ST.SE-0002(Q)
- 7.4 Commitment Documents

CD-849A CR 960911068

7.5 Other Documents

DCP 4EC-3523 BP 970729283 70005801

ATTACHMENT 1 (Page 1 of 1) Installation and Removal of Jumpers for SRM/IRM Rod Blocks

STEP	NOMENCLATURE	PERFORMER	VERIFIER
5.4.3 &	JUMPER INSTALLED PANEL 10C635		
5.4.4	DD19 TO DD20	I&C	
5.4.5 &	JUMPER INSTALLED PANEL 10C636		
5.4.6	AA35 TO AA36	l&C	
5.4.8 &	JUMPER REMOVED PANEL 10C635		
5.4.9	DD19 TO DD20	i&C	
5.4.10 &	JUMPER REMOVED PANEL 10C636		
5.4.11	AA35 TO AA36	I&C	

Hope Creek

.

STATION: HOPE CREEK							
SYSTEM	204000 RWCU System						
	2010001111						
TASK:	Align RWCU 0009	for Alternate D	ecay Heat	Removal IAW	Sectio	on F of HC.()P-AB.RPV-
TASK NUMBER:	295021AA10)4					
JPM NUMBER:	2003-NRC-L	SRO-S2					
ALTERNATE PATH	ł: X				-	205024	
			MOODT		к: 	295021	AA 1.04
APPLICABILITY: EO	RO	SRO X	LSRO		/K:	RO	SRO
EVALUATION SET	TING/METHO	D: REACT	OR BUILD	ING – SIMUL	ATE		
REFERENCES:	HC.OP-AB.RF	vV-0009(Q) Rev	0; HC.OF	P-SO.BG-0001	(Q) Re	ev 33	
TOOLS AND EQUI	PMENT: S	urvey map of R	loom 4506				
VALIDATED JPM C	COMPLETION	TIME:	20	min.			
TIME PERIOD IDENTIFIED FOR TIME CRITICAL STEPS: N/A min.							
APPROVED:							
N/A	N/A						
BARGAINING REPRESENTA		TRAINING S	UPERVIS	OR	OPE	RATIONS or Desig	MANAGER

CAUTION:	No plant equipment shall be operated during the performance of a JPM without the following:
	1. Permission from the OS or Unit CRS;
	Direct oversight by a qualified individual (determined by the individual granting permission based on plant conditions).
	3. Verification of the "as left" condition by a qualified individual.

ACTUAL JPM COMPLETION TIME:	min.
ACTUAL TIME CRITICAL COMPLETION TIME:	N/A min.
JPM PERFORMED BY:	GRADE: SAT UNSAT
REASON, IF UNSATISFACTORY:	
EVALUATOR'S SIGNATURE:	DATE:

NAME:	
DATE:	

SYSTEM: 204000 RWCU System

TASK: Align RWCU for Alternate Decay Heat Removal IAW Section F of HC.OP-AB.RPV-0009

TASK NUMBER: 295021AA1.04

INITIAL CONDITIONS:

- The plant is in Operational Condition 4
- Reactor Recirc Pumps are not available
- SDC Common Suction header has isolated
- RWCU is in service in a normal line-up
- Reactor coolant temperature is rising slowly
- RWCU is required for Alternate Decay Heat Removal IAW Condition F of HC.OP-AB.RPV-0009

INITIATING CUE:

Perform the required actions for Condition F of HC.OP-AB.RPV-0009.

Successful Completion Criteria:

- 1. All critical steps completed.
- 2. All sequential steps completed in order.
- 3. All time-critical steps completed within allotted time.
- 4. JPM completed within validated time. Completion time may exceed the validated time if satisfactory progress is being made (and NRC concurrence is obtained).

NAME: ______ DATE: _____

NC.TQ

.ZZ-0310(Z)

SYSTEM: 204000 RWCU System

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
		Operator obtains procedure HC.OP- AB.RPV-0009.	Operator obtains the correct procedure. Examiner Note: Once operator locates correct procedure, provide copy of HC.OP- AB.RPV-0009 procedure to operator for markup as necessary.		
		Operator reviews AB-RPV-0009 List of Conditions.	Operator determines Section F of HC.OP- AB.RPV-0009 is applicable.		
		Operator determines beginning step of the procedure	Operator determines correct beginning step to be F.1		
	F.	RWCU is required for Alternate Decay Heat Removal	Operator reads steps F.1, F.2, and F.3.		
	F.1	Ensure RWCU is in service	Operator verifies RWCU is in service from Cue sheet.		
			Examiner Note:		
			Initialing steps is not critical.		

NC.TQ _Z-0310(Z)

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: DATE:

SYSTEM: 204000 RWCU System

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
*	F.2	FULLY OPEN ED-V035.	Operator determines valve location from page 10 of HC.OP-AB.RPV-0009. Operator enters Reactor Building and locates ED-V035 in room 4504E (145 ft elev. Corridor outside RWCU Regen & Non-Regen HX Room along wall) *Operator simulates opening ED-V035 fully by rotating the valve handle clockwise until the pointer shows 100 percent open. Examiner Cue: The valve operator indicated has been rotated in the direction indicated and has come to the position indicated.		
	F.3	If necessary, <u>THEN</u> Bypass the Regenerative heat exchanger to maximize decay heat removal.	Operator asks if Bypassing the Regenerative Heat Exchanger is necessary. Examiner Cue: Ten minutes has passed. RPV temperatures are still increasing.		

NAME: DATE:

SYSTEM: 204000 RWCU System

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
			Operator determines Bypassing the Regenerative Heat Exchanger is necessary.		
		Operator obtains procedure HC.OP- SO.BG-0001.	Operator obtains the correct procedure. Examiner Note : Once operator locates correct procedure, provide copy of HC.OP- SO.BG-0001 procedure to operator for markup as necessary.		
		Operator reviews precautions and limitations.	Operator reviews precautions and limitations. Examiner Cue: If excessive time is taken reviewing precautions and limitations, inform operator that all are satisfied.		
	5.9	Bypassing the Regenerative Heat Exchangers	Operator determines correct beginning step to be 5.9.1		

NAME: _____ DATE:

SYSTEM: 204000 RWCU System

TASK: 295021AA1.04 Align RWCU for Alternate Decay Heat Removal IAW Section F of HC.OP-AB.RPV-0009

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
	5.9.1	ENSURE all prerequisites of Section 2.9 are satisfied.	Operator asks for permission to open the Regen HX Bypass line. Cue: OS/CRS permission has been obtained to open the Regen Hx bypass line. If Operator asks if Full or Partial Bypass is required, Cue: Full bypass		
	5.9.2	IF bypass of the Regenerative Heat Exchangers is required, <u>THEN</u> PERFORM the following:			

NC.TQ .ZZ-0310(Z)

.

OPERATOR TRAINING PROGRAM

NAME: _____ DATE: _____

JOB PERFORMANCE MEASURE

SYSTEM: 204000 RWCU System

TASK: 295021AA1.04 Align RWCU for Alternate Decay Heat Removal IAW Section F of HC.OP-AB.RPV-0009

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
*	5.9.2.A	UNLOCK AND OPEN V231, Bypass Valve.	 Examiners note: The following values are located inside the Regen & Non-Regen HX room, which is a high radiation, contaminated area. Provide the operator with room map. Some values are visible through the viewing window. Operator simulates removal of locking device and rotates V231 handwheel in the Counter-clockwise direction until the handwheel stops in the full open position. Examiner Cue: The value stated is in the position stated. 		
*	5.9.2.B	UNLOCK AND OPEN V233, Bypass Valve.	Operator simulates removal of locking device and rotates V233 handwheel in the Counter-clockwise direction until the handwheel stops in the full open position. Examiner Cue: The valve stated is in the position stated.		

NC.TQ. .ZZ-0310(Z)

NAME: ______ DATE: _____

SYSTEM: 204000 RWCU System

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
*	5.9.2.C	CLOSE V230, Hx Outlet Valve.	Operator simulates rotating the V230 handwheel in the Clockwise direction until the handwheel stops in the full closed position. Examiner Cue: The valve stated is in the position stated.		
	5.9.2.D	ADJUST system flowrate (for 2 pump operation) to approx. 300- 320 gpm. (flow rate may change due to the difference in system resistance due to heat exchanger being removed from service. Flow adjustments should be made using HV-F042.)	Operator should request the Control Room to adjust flow IAW 5.9.2.D Examiner Cue: The Control Room crew will complete this step.		
		STOP TIME	Examiner Cue: This JPM is complete.		

AREA DESCRIPTION RX 145' EL REGEN HEAT EXCHANGER ROOM WP 1 4506 MANIPULATE VALVES 1730 DATE THE 03/26/99 PURPOSE OF SURVEY AIR SAMPLE REPORT & NIA 12 SRMP / SRMP+ RMIY ROZA 7# 7A SURVEY INST(S) . 7 BIG 183t NST ID # REMARKS NIA HWCL AX POWER: REOB fi uCI /cc NIA AIR SAMPLE RESULTS: Jalanson SUPERVISOR REVIEW SURVEYED BY SIGNATURE JACLAUSON U \cap HRA SIRIHR NGURPA LAKED 112 337 160 RA CA 100 Ð ler RX WATER CLEAN-UP RECENERATIVE 1913 300 120 JOCH HEAT EX. B67230 IN OH 200 T B61231 IN O/H 100 140 \odot 180 BGV233 RX WATER CLEAN-UP NON RECENERATIVE NEAT EX (2 HOUNTED HORIZ.) L


INITIAL CONDITIONS:

- The plant is in Operational Condition 4
- Reactor Recirc Pumps are not available
- SDC Common Suction header has isolated
- RWCU is in service in a normal line-up
- Reactor coolant temperature is rising slowly
- RWCU is required for Alternate Decay Heat Removal IAW Condition F of HC.OP-AB.RPV-0009

INITIATING CUE:

Perform the required actions for Condition F of HC.OP-AB. RPV-0009.

TI

PSEG Internal Use Only

PSEG NUCLEAR L.L.C.

HOPE CREEK GENERATING STATION

HC.OP-SO.BG-0001(Q) - Rev. 33

REACTOR WATER CLEANUP SYSTEM OPERATION

USE CATEGOR	Y: 11	
A.	Biennial Review performed	Yes No \checkmark N/A
B. Change Package	e(s) and Affected Document Number(s) incorporate	ed into this revision.
• CP No.	CP Rev. No AD No	AD Rev. No or None
C. OTSC(s) incorp	porated into this revision:	
• OTSC No(s	s)	or None 🖌
	· · ·	

REVISION SUMMARY

1. Based on request made under Order 80037826 (T/S Amendment 140) prerequisite 2.8.3, and Notes 5.2.3.1 and 5.2.4.F were revised to change reference from T/S 3/4.4.4 to a reference to UFSAR section 5.2.3.2.2.2 (where the T/S direction was moved to). Editorial Change IAW NC.DM-AP.ZZ-0001(Q) allowances.

IMPLEMENTATION REQUIREMENTS

Effective date 11/15/02

Implementation of T/S Amendment 140

APPROVED:

ug

1/2/02 Date

Manager - Hope Greek Operations

Page 1 of 1

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

REACTOR WATER CLEANUP SYSTEM OPERATION

TABLE OF CONTENTS

SECTION	<u>TITLE</u>		PAGE
1.0	PURPO	SE	2
2.0	PRERE	QUISITES	2
3.0	PRECA	UTIONS AND LIMITATIONS	. 6
4.0	EQUIP	MENT REQUIRED	. 10
5.0	PROCE	DURE	. 11
	5.1	System Fill and Vent	. 11
	5.2	Placing RWCU In-Service	. 15
	5.3	Warm-up of RWCU Recirc Pumps	. 20
	5.4	Blowdown Operation	. 28
	5.5	Hot Standby Operation Without Reactor Recirc Flow - Maximizing Bottom Head Drain Flow	. 31
	5.6	Blowdown During Refueling Operation	. 33
	5.7	Reducing Flow through or Bypassing the Filter/Demins	. 35
	5.8	Removing RWCU Pump/System from Service	37
· .	5.9	Bypassing the Regenerative Heat Exchanger	40
:	5.10	Placing the Regenerative Heat Exchangers In-Service	44
	5.11	Flushing A RWCU Pump prior to Maintenance	45
6.0	REFEI	RENCES	48
ATTACHM	ENTS		
Attachment Attachment Attachment	1 - Maxin 2 - Maxin 3 - Comp	num RWCU Return To Feedwater Flow - 2 Pump Operation num RWCU Return To Feedwater Flow - 1 Pump Operation uter Point A196 Substitute Value Instructions	50 51 52

Hope Creek

Rev. 33

REACTOR WATER CLEANUP SYSTEM OPERATION

1.0 PURPOSE

This procedure outlines the steps necessary for System Fill and Vent, Warm-up, Startup, Shutdown, Blowdown, Hot Standby, and Refueling Operation of the Reactor Water Cleanup (RWCU) System.

2.0 PREREQUISITES

2.1 System Fill and Vent

- 2.1.1 RWCU System Valves are aligned IAW applicable SAP/WCM operational mode.
- 2.1.2 RWCU Major Component Electrical lineup is complete IAW applicable SAP/WCM operational mode.
- 2.1.3 BB-HV-F023A, Reactor Recirc Pumps Suction Valve, and/or BB-HV-F023B, Reactor Recirc Pumps Suction Valve, are open.
- 2.1.4 Radiation Protection should be contacted prior to performing venting and/or draining in this procedure. The individual(s) performing the venting and/or draining should obtain instructions <u>AND</u> approval from the Radiation Protection Shift Technician OR Radiation Protection Supervisor.

2.2 Placing RWCU System in Service

- 2.2.1 RWCU System is filled AND vented IAW Section 5.1 of this procedure.
- 2.2.2 An idle RWCU Recirc pump shall be warmed up IAW Section 5.3, <u>IF</u> the temperature difference between the pump casing <u>AND</u> Reactor Vessel water is > 150°F <u>AND</u> Reactor water temperature is > 212°F. [CD-886X]
- 2.2.3 RWCU System Valves are aligned IAW applicable SAP/WCM operational mode.
- 2.2.4 RWCU Major Component Electrical lineup is complete IAW applicable SAP/WCM operational mode.

Hope Creek

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 22 <u>PSEG Internal Use Only</u>

·_____.

HC.OP-SO.BG-0001(Q)

hal Use Or	
2.2.5	Chemistry Technician is available <u>AND</u> at least one RWCU Filter/Demin is ready for service (IF desired to place Filter/Demin in-service immediately upon placing the system in-service).
2.2.6	Reactor Auxiliaries Cooling Water (RACS) is in-service supplying water to the RWCU Heat Exchangers and Pumps.
2.2.7	Instrument Air System is in-service supplying air to air operated valves.
2.2.8	BB-HV-F023A, Reactor Recirc Pump Suction Valve, and/or BB-HV-F023B, Reactor Recirc Pump Suction Valve, are open.
2.2.9	AE-HV-F011A, Feedwater Inlet Valve, and/or AE-HV-F011B, Feedwater Inlet Valve, are open.
2.2.10	AE-HV-F074A, Feedwater Outboard Isolation Check Valve, and/or AE-HV-F074B, Feedwater Outboard Isolation Check Valve, actuator open.
2.2.11	<u>WHEN</u> both RWCU Pumps have been idle, <u>THEN</u> I&C should be notified to vent the delta flow transmitters FT-N041A(D), FT-N036A(D), <u>AND</u> FT-N012A(D).
Warm-	up of RWCU Recirc Pumps
2.3.1	RWCU System Valves are aligned IAW applicable SAP/WCM operational mode (except for individual RWCU Pump(s).
2.3.2	RWCU Major Component Electrical lineup is complete IAW applicable SAP/WCM operational mode.
2.3.3	Condensate Storage and Transfer System is in-service.
2.3.4	RWCU Recirc Pump (s) oil level at "oil level" mark <u>AND</u> oiler cup full.
2.3,5	Instrument Air System in-service.
2.3.6	BB-HV-F023A, Reactor Recirc Pump Suction Valve, and/or BB- HV-F023B, Reactor Recirc Pump Suction Valve,

Hope Creek

are open.

. 2.3

Page 3 of 53

Rev. 33

.____

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

men	lai Use U	<u>uiy</u>	
	2.3.7	Reactor Auxiliaries Cooling Water (RACS) is in-service to supply cooling water to RWCU Recirc Pumps.	
	2.3.8	NUMAC's 1SKXR-11497 and 1SKXR-11499 RWCU differential flow displays have been checked <u>AND</u> <u>IF</u> DP "WC/(MA) column Channel "A" does not agree with Channel "D" (should read 3.0 to 4.0 MA with no RWCU flow) <u>THEN</u> NOTIFY L&C to fill and vent Delta flow transmitters.	
2.4	Blowdo	wn Operation	
	2.4.1	RWCU System is in-service IAW Section 5.2 of this procedure.	~
	2.4.2	At least one of the following systems is in-service:	
	•	 Radwaste Equipment Drain (Waste Collection Tanks) Main Condenser 	
2.5	<u>Hot Sta</u> Bottom	undby Operation Without Reactor Recirc Flow - Maximizing Head Drain Flow	
	2.5.1	RWCU System is in-service IAW Section 5.2 of this procedure.	
	2.5.2	Reactor is in Hot Standby condition, with both Recirc Pumps out of service.	<u></u> .
2.6	Blowde	own During Refueling Operation	
	2.6.1	RWCU System is in-service IAW Section 5.2 of this procedure.	
	2.6.2	Reactor is shutdown for refueling.	
	2.6.3	At least one of the following systems is in-service:	
		 Radwaste Equipment Drain (Waste Collector Tanks) Main Condenser 	
2.7	Reduc	ing Flow through or Bypassing the Filter/Demins	
	RWCU	USystem is in-service IAW Section 5.2 of this procedure.	

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

0			
2.8	Removii	ng RWCU Pump/System from Service	
	2.8.1	RWCU System is in-service IAW Section 5.2 of this procedure.	
	2.8.2	Chemistry Technician is available to remove Filter Demins from service.	
	2.8.3	Chemistry Technician is cognizant of <u>OR</u> has been notified of the following:	
		• Sampling requirements of UFSAR section 5.2.3.2.2.2 (IF applicable)	
	-	• Required pH analysis once per 72 hours, per UFSAR section 5.2.3.2.2.2, when RWCU is taken out-of-service OR flow to pH analyzer is interrupted.	;
2.9	Bypassi	ing the Regenerative Heat Exchangers	
	OS/CRS	S permission has been obtained to open the Regen Hx bypass line.	
2.10	Placing	the Regenerative Heat Exchangers in Service	
	None		
2.11	Flushir	ng a RWCU Pump prior to Maintenance.	
	2.11.1	Radwaste has been notified that pump flushing is to start.	
	2.11.2	RWCU Pump to be flushed has been secured.	
	2.11.3	Rad Protection standing by to survey RWCU Pump.	

Hope Creek

Page 5 of 53

Rev. 33

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

3.0 PRECAUTIONS AND LIMITATIONS

- 3.1 Precautions
 - 3.1.1 Improper operation of RWCU Pumps can cause cavitation, improper pressurization of idle pumps, and internal thermal shock, which can lead to premature seal failures. [CD-445D]
 - 3.1.2 Following a trip, it is recommended to start RWCU Pump immediately, if possible, to avoid thermal stresses to piping and associated components. An idle RWCU Recirc Pump shall be warmed up if the temperature difference between pump casing and Reactor Vessel water is > 150°F and Reactor water temperature is > 212°F. [CD-886X]
 - 3.1.3 To avoid thermal stress to the Feedwater nozzles, the maximum RWCU flow should be maintained when a low flow condition exists. [CD-786D]
 - 3.1.4 HV-F102, RECIRC LOOP SUCT HDR, should <u>NOT</u> be closed to the point where the RWCU Recirc Pump(s) start to cavitate on low suction pressure. This corresponds to approximately 135 gpm on computer point B2058 REAC BOTTOM HEAD DRAIN (Ambient conditions <u>AND</u> normal RPV level).
 - 3.1.5 The rate of RWCU Pump warm-up shall <u>NOT</u> exceed 25°F per minute to prevent pump damage.
 - 3.1.6 Loss of a RWCU Pump, when in two pump operation, can cause pump runout. Flow must be reduced immediately by closing HV-F044, FILTER DEMIN BYPASS, (if open) <u>OR</u> throttling HV-F042, REGEN HX RTN ISLN, <u>UNTIL</u> Chemistry can remove a Filter/Demin from service.

3.1.7 To prevent a containment isolation of the BG-HV-F001 <u>AND</u> BG-HV-F004 valves while venting RWCU delta transmitters (2.2.11). The following guidance should be taken:

A. All the transmitters in one channel should be vented first before venting the transmitters in the other channel.

AND

B. The associated breaker should be open for the channel being venting IAW Tech Spec 3.6.3 prior to venting.

HC.OP-SO.BG-0001(Q)

- 3.1.8 Chemistry should isolate CAVS System prior to removing RWCU from service
 <u>AND</u> when securing the RWCU System for leak isolation.
- 3.1.9 The RWCU System contains potentially radioactive contaminated fluid.

3.1.10 A gross failure of a differential pressure transmitter 1SKXR-11497 <u>OR</u> 1SKXR-11499 (10C609, 10C611), does <u>NOT</u> cause an automatic RWCU isolation for the associated inbd/otbd isol vlv (transmitter input > 21 ma <u>OR</u> < 1 ma as sensed by the monitor). In this condition, a RWCU System differential flow between influent <u>AND</u> effluent outside Containment \geq 56 gpm for 45 seconds (time delay) isolation function will <u>NOT</u> occur if a high flow condition exists. Tech Spec Action Statement 3.3.2, Isolation Actuation Instrumentation, shall be entered. (The same logic is programmed into the monitors for the case of a failed thermocouple unit, i.e.., <u>NO</u> isolation occurs, <u>AND</u> is intended to minimize isolations due to sensor failures).

3.1.11

Increasing blowdown flow beyond 215 gpm (CRIDS A2947) will cause the delta flow transmitters FT-N012A(D) differential pressure range to be exceeded (output clamped at a maximum value of 215 gpm, up arrows indicated at the NUMAC). While <u>NO</u> additional increase in blowdown flow will be indicated in gpm, ma indication will increase until a gross failure occurs (reference Precaution 3.1.10), <u>AND</u> feedwater return flow will continue to indicate a decrease.

3.1.12 Unless a substitute value of 0.0 is installed for Process Computer point A196 RWCU FLOW (WCU), the Process Computer heat balance will use the last "good" value sensed <u>AND</u> calculate a negative value for power loss in the Cleanup Demineralizer System (QCU), whenever the system is removed from service while at power.

3.1.13

Following a RWCU automatic isolation, <u>OR</u> manually isolating sections of the RWCU System for >1 hr, "Cleanup Filter Demin Inlet" sample must be isolated by Chemistry to maintain RWCU System pressure <u>AND</u> unnecessary venting of RWCU Δ flow transmitters. [**PR 970803103**]

Hope Creek

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

3.2 Limitations

- 3.2.1 Dumping unfiltered water into the Main Condenser should be avoided unless absolutely necessary as this can introduce water-borne radioactive contaminants into Condensate System.
- 3.2.2 <u>IF</u> the valve <u>OR</u> electrical lineup CANNOT be completed as required, <u>THEN</u> the OS/CRS will determine whether the system should NOT be placed in-service <u>OR</u> standby, as required.
- 3.2.3 During blowdown operations, HV-F034, RWCU Discharge to Main Condenser, <u>AND</u> HV-F035, RWCU Discharge Drain to Waste Collector Tank, should <u>NOT</u> be simultaneously opened <u>WHEN</u> Condenser is under vacuum. Main Condenser vacuum can be lost through Radwaste piping.

3.2.4 Before opening HV-F034, RWCU to the Main Condenser, 1-RC-V005, Three-way Diverting Valve, from Sample Panel 10C251 to the Condenser <u>OR</u> CRW System, (located on the 132' EL in the Reactor Bldg. outside of Room 4402) should be turned 90° manually to the closed position to divert flow to the CRW System. This evolution is performed to reduce the risk of loosing Main Condenser vacuum during sample sink operation.

- 3.2.5 Non-Regenerative HX maximum outlet temperature is limited to 130°F for Hot Standby <u>OR</u> Startup conditions. However, for normal operation the temperature is limited to 120°F.
- 3.2.6 Non-Regenerative Heat Exchanger maximum coolant (RACS) outlet temperature is limited to 180°F for Hot Standby, Startup, OR Blowdown conditions.
- 3.2.7 RWCU Pump Cooling Water (RACS) inlet temperature must NOT exceed 110°F.
- 3.2.8 <u>WHEN</u> in Hot Standby Operation without Recirc Flow -Maximizing Bottom Head Drain Flow, <u>THEN</u> Cleanup System Outlet Temp must be maintained < 434°F to prevent thermal shock to the feedwater nozzles. [CD-389E]
- 3.2.9 During Normal operation the Filter Demins will both be in-service at approximately 148 to 150 gpm each, except as necessary to Backwash/Precoat the Filter Demins.

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

- 3.2.10 <u>WHEN</u> it becomes necessary to operate the RWCU System with 1 pump operation, <u>THEN</u> the system flow should be maintained IAW direction in Section 5.2 for single pump operation.
- 3.2.11 The Regen Heat Exchanger bypass shall only be used during Cold Shutdown, <u>OR</u> Refueling Modes <u>OR</u> Start-up Mode (Condition 2) to maintain temperature < 200°F. During Normal operation, the bypass valves will be locked closed.
- 3.2.12 DO NOT exceed 200 gpm of blowdown flow (CRIDS A2947) unless operating under emergency conditions. (Administrative limit based on conditions described in 3.1.11)
- 3.2.13 Manually operated gate valves 1-AE-V188 <u>AND</u> 1-AE-V189, on the RWCU Return Headers to the A <u>AND</u> B Feedwater Loops, when closed, allows for continued operation of the RWCU System with return flow to either the A <u>OR</u> B Feedwater Headers isolated. These alignments shall ONLY be used during outages in support of LLRT testing.

3.3 Interlocks

- 3.3.1 The RWCU System will be automatically isolated with the closure of HV-F001, Inboard Isolation Valve, <u>AND</u> HV-F004, Outboard Isolation Valve, under any of the following conditions:
 - A. Reactor water level < -38 inches (Level 2)
 - B. Non-Regenerative Heat Exchanger outlet temperature > 140°F (HV-F004 only)
 - C. Plant Leak Detection System actuation upon RWCU Pipe Chase Room 4402 (>160°F)
 <u>OR</u> RWCU Pipe Chase Room 4505 (>135°F)
 <u>OR</u> RWCU Pump Rooms
 <u>AND</u> Heat Exchangers Rooms (>140°F)
 - D. Standby Liquid Control System actuation:
 - System A isolates HV-F001
 - System B isolates HV-F004

Continued next page

Hope Creek

Page 9 of 53

DC 0001(O)

SEG Interna	l Use Or	nly	HC.OP-SO.BG-0001
.3.1 (continu	ed)	E.	Nuclear Steam Supply Shutoff System Isolation
		F.	RWCU System differential flow between influent <u>AND</u> effluent outside Containment \geq 56 gpm for 45 seconds (time delay).
		G.	RRCS Standby Liquid Control actuation:
:			Channel A or B RRCS Standby Liquid Control actuation isolates both the HV-F001 and HV-F004.
:	3.3.2	The	following conditions will trip the RWCU Recirculation Pumps:
		A.	Pump suction flow < 70 gpm (after pump start time delay of 15 minutes). This trip will only occur if the common pump suction flowpath is < 70 gpm.
		В.	HV-F001, Containment Inboard Isolation Valve, <u>AND</u> HV-F004, Containment Inboard Isolation Valve, <u>NOT</u> fully opened (time delayed 11 sec for F/D to go into Hold).
	3.3.3	HV <u>OR</u>	-F033 valve auto closes on upstream pressure ≤ 5 psig downstream pressure ≥ 140 psig.
1.0 <u>EQUI</u>	PMENT	<u>REQ</u>	UIRED
•	Pyrome	eter - 0	to 600°F or equivalent –
٠	Magnet	tic The	rmocouple (Ensure compatibility between probe and box)

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

<u>NOTE 5.0</u>

All operations are performed from Panel 10C651C unless otherwise noted.

5.0 PROCEDURE

5.1 Syste	em Fill an	Fill and Vent		
5.1.1	ENS	URE all prerequisites of Section 2.1 are satisfied.		
	А.	<u>IF</u> the system is aligned for service or inservice IAW Section 5.2 <u>THEN</u> GO TO Step 5.1.4.		
	В.	<u>IF</u> the system is removed from service <u>AND</u> is not isolated IAW Section 5.8, <u>THEN</u> GO TO Step 5.1.3.		
	C.	IF the system is removed from service and is isolated IAW Section 5.8, <u>THEN</u> GO TO Step 5.1.2		
5.1.2	2 EST Con	ABLISH a make-up water supply path from the lensate Storage and Transfer as follows:		
	А.	ENSURE BG-HV-F001 and BG-HV-F004 are CLOSED.		
	B.	CLOSE 1-BG-V024(1-BG-V030), A(B) RWCU Recirc Pmp Csg Drn Vlv.		
	C.	OPEN the following valves:		
		 1-BG-V022 (1-BG-V028), A(B) RWCU Recirc Pmp Csg Drn Vlv. 		
•		 1-BG-V023 (1-BG-V029), A(B) RWCU Recirc Pmp Csg Drn Vlv. 		
Continued next page	ge			

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5.1.2 (continued)

. 1	he fill	rate of the Pump Seal Cavity should NOT exceed 2.5 gpm.	_
:	D.	THROTTLE OPEN 1-AP-V142(1-AP-V143), Condensate and Storage Transfer to RWCU Recirc Pmp A(B) Seal Fill Isln Vlv.	
5.1.3	EST	ABLISH the vent and fill flow path as follows:	
	A.	OPEN 1-BG-V004, A RWCU Recirc Pmp Suction Vlv <u>AND</u> 1-BG-V008, B RWCU Recirc Pmp Suction Vlv.	
	В.	OPEN 1-BG-V200, A RWCU Recirc Pmp Dsch Byp Vlv <u>AND</u> 1-BG-V210, B RWCU Recirc Pmp Dsch Byp Vlv.	
· · ·	C.	THROTTLE OPEN HV-F044, Fltr Demin Byp Vlv as necessary to allow make-up water flow to the down stream piping while venting.	
<u>.</u>	- <u></u>	<u>NOTE</u> 5.1.4	

- 5.1.4 VENT using the following valves until a solid stream of water issues as seen at FG-1077(1076), A(B) RWCU Recirc Pump A(B) Vent sight glass (local):
 - A. 1-BG-V026, RWCU Recirc Pmp A Seal Flush Vent Vlv,
 <u>AND</u> 1-BG-V027, RWCU Recirc Pmp A Seal Flush Vent Vlv
 - B. 1-BG-V073, RWCU Recirc Pmp B Seal Flush Vent Vlv,
 <u>AND</u> 1-BG-V074, RWCU Recirc Pmp B Seal Flush Vent Vlv

Hope Creek

5.1.5	VENT using the following valves until piping is vented (Local):

- A. 1-BG-V173, RWCU Pmp Suct Hdr Vent, (RM 4505) AND 1-BG-V174, RWCU Pmp Suct Hdr Vent, (RM 4504D).
- B. 1-BG-V032, RHX Tube Side Inlet Hdr Vent, (RM 4508B)
 <u>AND</u> 1-BG-V033, RHX Tube Side Inlet Hdr Vent (RM 4506B).
- C. 1-BG-V037, RHX BE207 Tube Side Vent, (RM 4506B)
 <u>AND</u> 1-BG-V038, RHX BE207 Tube Side Vent, (RM 4506B).
- D. 1-BG-V041, RHX AE207 Tube Side Vent, (RM 4506B)
 <u>AND</u> 1-BG-V042, RHX AE207 Tube Side Vent, (RM 4506B).
- E. 1-BG-V049, Non-RHX AE208 Tube Side Vent, (RM 4506B) <u>AND</u> 1-BG-V050, Non-RHX AE208 Tube Side Vent (RM 4506B).
- F. 1-BG-V054, Non-RHX Outlet Hdr Vent, (RM 4506C)
 <u>AND</u> 1-BG-V055, Non-RHX Outlet Hdr Vent, (RM 4506C).
- G. 1-BG-V057, RHX AE207 Shell Side Vent, (RM 4506C) AND 1-BG-V058, RHX AE207 Shell Side Vent, (RM 4506C).
- H. 1-BG-V062, RHX BE207 Shell Side Vent, (RM 4506B)
 AND 1-BG-V063, RHX BE207 Shell Side Vent, (RM 4506B).

<u>NOTE</u> 5.1.5.1

Additional throttling of HV-F044, Fltr Demin Byp VIv may be required to ensure adequate make-up water flow to the down stream piping while venting.

I. 1-BG-V066, RHX Shell Side Outlet Hdr Vent, (RM 4506A) AND 1-BG-V067, RHX Shell Side Outlet Hdr Vent, (RM 4508B).

Hope Creek

Page 13 of 53

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

- 5.1.6 IF opened in Step 5.1.3, THEN:
 - A. CLOSE 1-BG-V004, A RWCU Recirc Pmp Suction Vlv AND 1-BG-V008, B RWCU Recirc Pmp Suction Vlv.
 - B. CLOSE 1-BG-V200, A RWCU Recirc Pmp Dsch Byp Vlv AND 1-BG-V210, B RWCU Recirc Pmp Dsch Byp Vlv.
 - C. CLOSE HV-F044, Fltr Demin Byp Vlv.

5.1.7 IF Condensate Storage and Transfer Tank was used to fill in Step 5.1.2, **RESTORE** the system as follows:

A. **CLOSE** the following valves:

- 1-AP-V142(1-AP-V143), Condensate and Storage Transfer to RWCU Recirc Pmp A(B) Seal Fill Isln Vlv.
- 1-BG-V022(1-BG-V028), A(B) RWCU Recirc Pmp Csg Drn Vlv.
- 3. 1-BG-V023(1-BG-V029), A(B) RWCU Recirc Pmp Csg Drn Vlv.
- B. OPEN 1-BG-V024(1-BG-V030), A(B) RWCU Recirc Pmp Csg Drn Vlv.

5.1.8

OPEN BG-HV-F001 and BG-HV-F004.

Hope Creek

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

<u>NOTE 5.2</u>

- A. CRIDS points B2081 and A2856, <u>OR</u> NUMAC Drawers 1SKXR-11497 <u>OR</u> 1SKXR-11499 (10C609, 10C611) may be used to obtain temperature compensated flows.
- B. In the event HV-F044, FLTR DEMIN BYPASS, is unavailable or inoperable, the operator may use HV-F104, CLEANUP BYPASS, to throttle open/closed in order to place RWCU Pumps & Demins in-service.

CAUTION 5.2

- A. Following a trip, it is recommended to start RWCU Pump immediately, if possible, to avoid thermal stresses to piping <u>AND</u> associated components.
- B. To avoid thermal stress to the Feedwater nozzles, the maximum RWCU flow should be maintained when a low flow condition exists. [CD-786D]
- 5.2 Placing RWCU System in Service [CD-937B]
 - 5.2.1 **ENSURE** all prerequisites of Section 2.2 are satisfied.

NOTE 5.2.2

Reactor Engineering may be contacted as necessary for assistance with the restoration of the Restore to Scan function.

5.2.2 IF the Process Computer heat balance (OD-3d) value for power loss in the Cleanup Demineralizer System (QCU), had a substitute value of 0.0 inserted for Computer Point A196 RWCU FLOW (WCU), when the system was removed from service,
 <u>THEN RETURN Computer Point A196 to AUTOMATIC using Attachment 3</u> - Computer Point A196 Substitute Value Instructions.

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

- 5.2.3 <u>IF</u> in two pump operation, THEN **PERFORM** the following:
 - A. CLOSE HV-F044, FLTR DEMIN BYPASS, by pressing the DECREASE push-button until CLOSE lamp is illuminated.

CAUTION 5.2.3.B

RWCU Recirc Pumps trip on low flow < 70 gpm (after pump start time delay of 15 minutes). This trip will only occur if the common pump suction flowpath is < 70 gpm.

- B. START A(B)P221, A(B) RWCU PUMP.
- C. THROTTLE OPEN HV-F044 FLTR DEMIN BYPASS UNTIL Computer point A2856 RWCU OUTLET FLOW indicates approximately 140 to 160 gpm.

CAUTION 5.2.3.D

Loss of a RWCU Pump, when in two-pump operation, can cause pump runout. Flow must be reduced immediately by closing HV-F044, FILTER DEMIN BYPASS, (if open) <u>OR</u> throttling HV-F042, REGEN HX RTN ISLN, until Chemistry can remove a Filter/Demin from service.

- D. START B(A)P221, B(A) RWCU PUMP.
- E. **THROTTLE OPEN** HV-F044, FLTR DEMIN BYPASS, <u>UNTIL</u> Computer Point A2856 RWCU OUTLET FLOW indicates approximately 300 to 320 gpm.
- F. <u>WHEN</u> the first Filter/Demin is to be placed in-service, <u>THEN</u> CONTACT Chemistry Department to place A(B) Filter/Demin in-service AND slowly raise flow.

HC.OP-SO.BG-0001(Q)

5.2.3 (continued)

PSEG Internal Use Only

- G. MONITOR Computer Point A2856 RWCU OUTLET FLOW <u>AND</u> MAINTAIN flow at or below maximum RWCU return to feedwater flow value determined using Attachment 1 by throttling closed on HV-F044, FLTR DEMIN BYPASS, <u>UNTIL</u> Computer Point B2951 (B2952) FILTER DEMIN A(B) EFL FLOW indicates a maximum of 150 gpm <u>AND</u> has stabilized. [PR 980515086]
- H. <u>WHEN</u> the second Filter/Demin is to be placed in-service, THEN PERFORM the following:
 - REQUEST Chemistry Department to place B(A) Filter/Demin in-service AND slowly raise flow.
 - MONITOR Computer Point A2856 RWCU OUTLET FLOW <u>AND</u> MAINTAIN flow at or below maximum RWCU return to feedwater flow value determined using Attachment 1 by throttling closed on HV-F044 FLTR DEMIN BYPASS until Computer Point B2952 (B2951) FILTER DEMIN B(A) EFL FLOW indicates a maximum of 150 gpm <u>AND</u> has stabilized. [PR 980515086]

<u>NOTE</u> 5.2.3.I

Compliance with UFSAR section 5.2.3.2.2.2 is required.

- I. MONITOR the following CLEANUP FILTER DEMINERALIZERS (10C650):
 - 1. CR-R601-G33, INLET //CONDUCTIVITY, <1.0 MMHO/cm
 - 2. CR-R603(Red), DEMINERALIZER A OUT CONDUCTIVITY, < 0.1 MMHO/.cm
 - CR-R603(Blue), DEMINERALIZER B OUT CONDUCTIVITY, < 0.1 MMHO/cm

5.2.3 (continued)

J. MONITOR the following parameters as required:

Table BG-001				
Computer Point Name	INST #	Computer Point		
CLEANUP SYSTEM INLET TEMP	TE-N004	A214		
CLEANUP SYSTEM OUTLET TEMP	TE-N015	A215		
RWCU REGEN HX OUTLET TEMP	TE-N006	A2945		
RWCU N-REGEN HX OUTLET TEMP	TE-N019	A2944		
RWCU REAC SUCT FLOW	FT-N036A	B2081		
RWCU SYSTEM OUTLET FLOW TO FDW	FT-N041A	A2856		

5.2.4 <u>IF in single pump operation,</u> <u>THEN PERFORM the following:</u>

A. CLOSE HV-F044 FLTR DEMIN BYPASS by pressing the DECREASE push-button until the CLOSE lamp is illuminated.

CAUTION 5.2.4.B

RWCU Recirc Pumps trip on low flow <70 gpm (after pump start time delay of 15 minutes).

- B. START A(B)P221, A(B) RWCU PUMP.
- C. THROTTLE OPEN HV-F044, FLTR DEMIN BYPASS, <u>UNTIL</u> Computer Point A2856 RWCU OUTLET FLOW indicates approximately 140 to 160 gpm.
- D. <u>WHEN RWCU OUTLET FLOW</u> is between 140 and 160 gpm, <u>THEN **REQUEST** Chemistry Department to place either one <u>OR</u> both Filter/Demin in-service <u>AND</u> slowly raise flow.</u>

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5.2.4 (continued)

CAUTION 5.2.4.E

During system operation with flow from the Reactor Bottom Head Drain <u>ONLY</u>, System flow should be maintained LESS THAN 135 GPM to preclude pump cavitation.

> E. MONITOR Computer Point A2856 RWCU OUTLET FLOW AND MAINTAIN flow at or below maximum RWCU return to feedwater flow value determined using Attachment 2 by throttling closed on HV-F044 FLTR DEMIN BYPASS <u>UNTIL</u> Computer Point B2951 (B2952) FILTER DEMIN A(B) EFL FLOW indicates a maximum of 150 gpm and has stabilized.

NOTE 5.2.4.F

Compliance with UFSAR section 5.2.3.2.2.2 is required.

- F. MONITOR the following CLEANUP FILTER DEMINERALIZERS (10C650):
 - 1. CR-R601-G33, INLET //CONDUCTIVITY, <1.0 MMHO/cm
 - CR-R603(Red), DEMINERALIZER A OUT CONDUCTIVITY, < 0.1 MMHO/.cm
 - 3. CR-R603(Blue), DEMINERALIZER B OUT CONDUCTIVITY, < 0.1 MMHO/cm

Continued next page

Hope Creek

Page 19 of 53

5.2.4 continued

G. MONITOR the following parameters as required:

Table BG-001				
Computer Point Name	INST #	Computer Point		
CLEANUP SYSTEM INLET TEMP	TE-N004	A214		
CLEANUP SYSTEM OUTLET TEMP	TE-N015	A215		
RWCU REGEN HX OUTLET TEMP	TE-N006	A2945		
RWCU N-REGEN HX OUTLET TEMP	TE-N019	A2944		
RWCU REAC SUCT FLOW	FT-N036A	B2081		
RWCU SYSTEM OUTLET FLOW TO FDW	FT-N041A	A2856		

<u>NOTE 5.3</u>

A. An idle RWCU Recirc Pump shall be warmed up IAW this procedure if temperature difference between pump casing and Reactor Vessel water is > 150°F and Reactor water temperature is > 212°F. [CD-886X]

B. All operations are performed locally unless noted otherwise.

CAUTION 5.3

The rate of RWCU Pump warm-up shall <u>NOT</u> exceed 25°F per minute to prevent pump damage

- 5.3 <u>Warm-up of RWCU Recirc Pumps</u> [CD-937B, CD-445D]
 - 5.3.1 **ENSURE** all prerequisites of Section 2.3 are satisfied.

Hope Creek

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5	.3	.2

IF fill and vent of A(B)P221, RWCU Pump, is necessary, THEN PERFORM the following:

- A. **ENSURE** the following valves are CLOSED:
 - 1. 1-BG-V004 (V008), A(B) RWCU Recirc Pmp Suction Vlv
 - 2. 1-BG-V006 (V010), A(B) RWCU Recirc Pmp Dsch Vlv
 - 3. 1-BG-V200 (V210), A(B) RWCU Recirc Pmp Dsch Byp Vlv
 - 4. 1-BG-V024 (V030), RWCU Recirc Pmp A(B) Csg Drn Vlv

B. **OPEN** the following valves:

- 1. 1-BG-V022 (V028), A(B) RWCU Recirc Pmp Csg Drn Vlv
- 2. 1-BG-V023 (V029), A(B) RWCU Recirc Pmp Csg Drn Vlv

<u>NOTE</u> 5.3.2.C

The fill rate of the Pump Seal Cavity should <u>NOT</u> exceed 2.5 gpm.

C. THROTTLE OPEN 1-AP-V142 (V143) Cond Stor & Xfr to RWCU Recirc Pmp A(B) Seal Fill Isln Vlv.

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5.3.2 (continued)

<u>NOTE</u> 5.3.2.D

The action verb VENT is defined as "To open a vent valve until a solid stream of water issues, then return the vent valve to the closed position."

	D.	VENT using the following valves <u>UNTIL</u> a solid stream of water issues from FG- V1077 (1076) A(B) RWCU Recirc Pmp A(B) Vent sight glass:				
:		 1-BG-V026 (V073), A(B) RWCU Recirc Pmp Seal Flush Vent Vlv 	<u> </u>			
		 1-BG-V027 (V074), A(B) RWCU Recirc Pmp Seal Flush Vent Vlv 				
	E.	CLOSE the following valves:				
		 1-AP-V142 (V143), Cond Stor and Xfr to A(B) RWCU Recirc Pmp Seal Fill Isln Vlv 				
		 1-BG-V022 (V028), A(B) RWCU Recirc Pmp Csg Drn Vlv 	<u> </u>			
		 1-BG-V023 (V029), A(B) RWCU Recirc Pmp Csg Drn Vlv 				
	F.	OPEN 1-BG-V024 (V030), RWCU Recirc Pmp A(B) Csg Drn Vlv.				
5.3.3	3 PERFORM the following to warm-up the first RWCU Pump: [CD-886X, CD-937B]					
•	A.	ENSURE 1-BG-V003 (1-BG-V007), A(B) RWCU Recirc Pmp Outbd Suction VIv, is OPEN.				
	B.	Slowly OPEN 1-BG-V004 (1-BG-V008), A(B) RWCU Recirc Pmp Suction Vlv.				
xt nage						

Ç.

HC.OP-SO.BG-0001(Q)

5.3.3 (continued)

PSEG Internal Use Only

<u>WHEN</u> fully open, <u>THEN</u> LOCK 1-BG-V004 (1-BG-V008), A(B) RWCU Recirc Pmp Suction Vlv.

D. ENSURE the following valves are CLOSED: [PR 970803103]

- HV-F042
- HV-F044
- HV-F104
- E. IF desired,

THEN INSTALL a magnetic thermocouple on RWCU Recirc Pump Casing by placing the magnetic probe on the inlet side of the pump casing approximately midway between the inlet flange and the outlet flange.

F. **READ** the initial temperature on RWCU Recirc Pump A(B) casing using a Pyrometer OR the installed Magnetic thermocouple.

<u>NOTE</u> 5.3.3.G

The following steps will slowly heat up the pump casing so as <u>NOT</u> to thermally shock the pump. The intent of the following steps is to allow a gradual heatup without the 15 minute low flow timer initiating. Maximum heatup rate will be attained after cracking open DISCH Bypass Valve then tapers off sharply.

CAUTION 5.3.3.G

RWCU Recirc Pump(s) trip on a low flow < 70 gpm if the common pump suction flowpath is < 70 gpm after pump start time delay of 15 minutes. The time should be marked <u>AND</u> computer point B2081 RWCU REAC SUCT FLOW should be monitored so as to establish adequate flow prior to the 15 minute timer initiation for the first pump going into service.

G. START A(B)P221, A(B) RWCU Recirc Pump.

Continued next page

Hope Creek

Page 23 of 53

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5.3.3 (continued)

- H. <u>IF</u> RWCU diff flow isolation signals alarm for >10 sec, in the following steps <u>THEN</u> THROTTLE back on HV-F044, HV-F104 OR 1BG-V200 (1-BG-V210). [PR 970803103]
- I. <u>WHEN</u> A(B)P221, A(B) RWCU Recirc Pump, has operated approximately 10 seconds at a shutoff head, <u>THEN</u> slowly CRACK OPEN 1-BG-V200 (1-BG-V210), (~ 0-4 turns) RWCU A(B) DISCH BYPASS VALVE, to pressurize system to F104 and F044. [PR 970803103]

<u>NOTE</u> 5.3.3.J

When opening HV-F044 use 10 seconds opening time initially, then 1 second intervals.

- J. MONITOR Comp Point A2950 <u>AND</u> <u>WHEN</u> pressure equals Rx pressure <u>OR</u> local pump discharge pressure, <u>THEN</u> PERFORM ONE of the following until system parameters change, indicating system pressurization or flow. [PR 970803103] ____
 - 1. Slowly THROTTLE OPEN HV-F104

<u>OR</u>

- 2. Slowly THROTTLE OPEN HV-F044 AND HV-F042
- K. THROTTLE OPEN 1-BG-V200 (1-BG-V210) RWCU "A" (B) DISCH Bypass Valve and/or HV-F044 as necessary to establish a 25°F/min. heat-up rate using pyrometer OR magnetic thermocouple. [PR 970803103]
- L. Slowly **OPEN** 1-BG-V006 (1-BG-V010), RWCU Pump A(B) Discharge Valve.
- M. CLOSE 1-BG-V200 (1-BG-V210), RWCU Recirc Pump A(B) Dsch Byp Vlv.

Continued next page

Page 24 of 53

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 PSEG Internal Use Only

HC,OP-SO.BG-0001(Q)

5.3.3 (continued)

• •

	•					
Α.	lf th to p	If the HV-F044 is unavailable in the following step, HV-F104 may be opened to provide a flowpath.				
B.	RW sup suc 000	RWCU outlet flow may be throttled to a reduced value in the following step support of Startup or Shutdown flow restrictions to prevent flashing in the suction venturi as specified in HC.OP-IO.ZZ-0003(Q), and/or HC.OP-IO.ZZ 0004(Q).				
		N.	THROTTLE OPEN HV-F044, FLTR DEMIN BYPASS, <u>UNTIL</u> Computer Point A2856 RWCU OUTLET FLOW indicates ~ 140 to 160 gpm.			
	• • •	0,	IF desired to place a RWCU F/D in-service, THEN GO TO Step 5.2.4.D.			
. 5	5.3.4	PER [CD-	FORM the following to warm-up the second RWCU Pump: 886X]			
	- :	А.	ENSURE 1-BG-V003 (1-BG-V007), A(B) RWCU Recirc Pmp Outbd Suction Vlv, is OPEN.			
		В.	PERFORM one of the following:			
			 READ the initial temperature on the RWCU Recirc Pump A(B) Casing using a Pyrometer <u>OR</u> by installing a magnetic thermocouple on RWCU Recirc Pump Casing by placing the magnetic probe on the inlet side of the pump casing approximately midway between the inlet flange <u>AND</u> the outlet flange. 			
•	·		OR			
•			2. USE the following equation to calculate a warm up time based on the use of the bypass valve only with NO monitoring of the pump casing (ALARA):	<u></u>		
			Temperature - (100°F + Ambient Rm Temp) = Warm up Time			

Hope Creek

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5.3.4 (continued)

CAUTION 5.3.4.C

The RWCU Pump must be running to cool the shaft seal for that portion of the warm-up above 200°F at the pump.

- C. Slowly OPEN 1-BG-V004 (1-BG-V008), A(B) RWCU Recirc Pmp Suction Vlv.
- D. <u>WHEN</u> fully open, <u>THEN</u> LOCK 1-BG-V004 (1-BG-V008), A(B) RWCU Recirc Pmp Suction Vlv.
- E. START A(B)P221, A(B) RWCU Recirc Pump.

<u>NOTE</u> 5.3.4.F

The following steps will slowly heat up the pump casing so as to <u>NOT</u> thermally shock the pump. Maximum heatup rate will be attained after cracking open DISCH Bypass Valve then tapers off sharply.

 F. <u>WHEN</u> A(B)P221, A(B) RWCU Recirc Pump, has run at shutoff head for approximately 10 seconds, <u>THEN</u> SLOWLY OPEN 1-BG-V200 (1-BG-V210), RWCU A(B) Disch Bypass Valve, to establish a heat up rate of less than 25°F per minute. [PR 970803103]

HC.OP-SO.BG-0001(Q)

5.3.4 (continued)

PSEG Internal Use Only

NOTE 5.3.4.G

Minimum pump to RWCU inlet differential temperature should be <100°F to complete warm-up.

- G. <u>WHEN</u> casing temperature (as monitored using pyrometer <u>OR</u> thermocouple) is within 100°F of Rx temperature, <u>OR</u> <u>WHEN</u> warm-up time (as calculated in step 5.3.4.B.2) is up, <u>THEN</u> SLOWLY OPEN 1-BG-V006 (1-BG-V010), RWCU Pump A(B) Discharge Valve, while monitoring Computer Point A2856 RWCU OUTLET FLOW <u>AND</u> maintaining between 300 to 320 gpm by throttling open on HV-F044, FLTR DEMIN BYPASS.
- H. CLOSE 1-BG-V200 (1-BG-V210), RWCU Recirc Pmp A(B) Dsch Byp Vlv.
- I. <u>IF</u> a first <u>OR</u> second Filter/Demin is to be placed in-service, <u>THEN</u> GO TO Step 5.2.3.F <u>OR</u> 5.2.3.H.

Hope Creek

PSEG Internal Use Only

HC,OP-SO,BG-0001(Q)

<u>NOTE</u> 5.4

CRIDS points B2081 and A2856, <u>OR</u> NUMAC Drawers 1SKXR-11497 <u>OR</u> 1SKXR-11499 (10C609, 10C611) may be used to obtain temperature compensated flows.

CAUTION 5.4

Loss of a RWCU Pump, when in two pump operation, can cause pump runout. Flow must be reduced immediately by closing HV-F044, FILTER DEMIN BYPASS, (if open) <u>OR</u> throttling HV-F042, REGEN HX RTN ISLN, until Chemistry can remove a Filter/Demin from service.

5.4 Blowdown Operation

5.4.1 ENSURE all prerequisites of Section 2.4 are satisfied.

CAUTION 5.4.2

Before opening HV-F034 (RWCU to the Main Condenser), 1-RC- V005, Three Way Diverting Valve, from Sample Panel 10C251 to the Condenser or CRW System, (located on the 132' EL in the Reactor Bldg. outside of Room 4402) should be turned 90° manually to the vertical position to divert flow to the CRW System. This evolution is performed to reduce the risk of loosing Main Condenser vacuum during sample sink operation.

5.4.2 **ENSURE** that 1-RC-V005, Three Way Diverting Valve, is in the VERTICAL position.

Hope Creek

Rev. 33

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

<u>NOTE</u> 5.4.3

This mode of operation is used to maintain Reactor water level when operating at low power levels due to CRD input <u>AND</u> Reactor coolant expansion during heatup.

CAUTION 5.4.3

- A. Non-Regenerative Hx outlet temperature is limited to < 130°F and must be observed during blowdown due to low Regenerative Hx coolant flow.
- B. Blowdown valves HV-F034 <u>AND</u> HV-F035 should <u>NOT</u> be simultaneously opened when Condenser is under a vacuum as Main Condenser vacuum can be lost through Radwaste piping.
- C. HV-F033 valve auto closes on upstream pressure \leq 5 psig <u>OR</u> on downstream pressure \geq 140 psig.
- D. The system trips on RWCU Differential Flow ≥ 56 gpm for 45 seconds (time delay).

E. A gross failure of a differential pressure transmitter 1SKXR-11497 <u>OR</u> 1SKXR-11499 (10C609, 10C611), does <u>NOT</u> cause an automatic RWCU isolation for the associated inbd/otbd isol viv (transmitter input > 21 ma <u>OR</u> < 1 ma as sensed by the monitor). In this condition, a RWCU System differential flow between influent <u>AND</u> effluent outside Containment > 56 gpm for 45 seconds (time delay) isolation function will <u>NOT</u> occur if a high flow condition exists. Tech Spec Action Statement 3.3.2, Isolation Actuation Instrumentation, shall be entered. (The same logic is programmed into the monitors for the case of a failed thermocouple unit, i.e.., <u>NO</u> isolation occurs, <u>AND</u> is intended to minimize isolations due to sensor failures).

- 5.4.3 **PERFORM** the following to establish a flow path for blowdown:
 - A. <u>IF</u> blowdown to Main Condenser is required, <u>THEN</u> **OPEN** HV-F034, RWCU RTN TO CNDSR.
 - B. <u>IF</u> blowdown into Waste Collector Tanks is required, THEN **OPEN** HV-F035, RWCU TO EQPT DRN.

Hope Creek

Page 29 of 53

Rev. 33

HC.OP-SO.BG-0001(Q) **PSEG Internal Use Only** OPEN HV-F033, HIC-R606 DRAIN FLOW CONTROL, 5.4.4 by pressing INCREASE push-button until Computer Point A2947 RWCU DISCH to COND AND EQUIP DRAIN FLOW indicates desired flow. CHECK that RWCU DIFFERENTIAL FLOW at 1SKXR-11499 5.4:5 AND 1SKXR-11497 indicates < 56 gpm (10C611, 10C609). MONITOR A2944, RWCU N-REGEN HX OUTLET TEMP, 5.4.6 for $< 130^{\circ}$ F. IF increased blowdown rate is desired, 5.4.7 THEN PRESS the INCREASE push-button for HV-F031, DRN FL ORF BYPASS. IF NRHX outlet temperature approaches 130°F, 5.4.8 THEN THROTTLE closed on HV-F033 HIC-R606 DRAIN FLOW CONTROL to reduce flow rate. (This drop in flow will lower NRHX outlet temperatures.) WHEN blowdown operation is complete, 5.4.9 THEN CLOSE the HV-F031, DRN FL ORF BYPASS. CLOSE HV-F033, HIC-R606 DRAIN FLOW CONTROL 5.4.10 VALVE. CLOSE HV-F034, RWCU RTN TO CNDSR, 5.4.11 OR HV-F035, RWCU TO EQPT DRN. IF flow to the Main Condenser is required, 5.4.12 THEN RETURN 1-RC-V005, Three Way Diverting Valve, to the horizontal position.

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

<u>NOTE</u> 5.5

CRIDS points B2081 and A2856, <u>OR NUMAC Drawers 1SKXR-11497 OR</u> 1SKXR-11499 (10C609, 10C611) may be used to obtain temperature compensated flows.

CAUTION 5.5

Loss of a RWCU Pump, when in two pump operation, can cause pump runout. Flow must be reduced immediately by closing HV-F044, FILTER DEMIN BYPASS, (if open) <u>OR</u> throttling HV-F042, REGEN HX RTN ISLN, or throttling HV-F104, CLEANUP BYPASS.

- 5.5 <u>Hot Standby Operation Without Reactor Recirc Flow Maximizing Bottom Head Drain</u> Flow
 - 5.5.1 ENSURE all prerequisites of Section 2.5 are satisfied.
 - 5.5.2 **REMOVE** both Filter Demins from service IAW Section 5.7 of this procedure.

NOTE 5.5.3

- A. HV-F102, RECIRC LOOP SUCT HDR, valve is throttled to maintain uniform Reactor Vessel temperature during Hot Standby.
- B. Steps 5.5.3 <u>AND</u> 5.5.4 should be performed concurrently.
 - 5.5.3 **PERFORM** the following to establish RWCU recirculation flow, bypassing Regenerative and Non-Regenerative HXs:
 - A. ENSURE HV-F101, SUCT FROM RPV BOT DRN, is open.

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5.5.3 (continued)

CAUTION 5.5.3.B

HV-F102, RECIRC LOOP SUCT HDR VALVE, should <u>NOT</u> be closed to the point where the RWCU Recirc Pump(s) start to cavitate on low suction pressure. This corresponds to approximately 135 gpm on computer point B2058 REAC BOTTOM HEAD DRAIN (ambient conditions <u>AND</u> normal RPV level).

> B. THROTTLE CLOSED HV-F102, RECIRC LOOP SUCT HDR, until Computer Point B2058 REAC BOTTOM HEAD DRAIN flow increases to between 115 and 125 gpm.

CAUTION 5.5.4

CLEANUP SYSTEM OUTLET TEMP must be maintained \leq 434°F to prevent thermal shock to the feedwater nozzles. [CD-389E]

- 5.5.4 **PERFORM** the following to establish RWCU recirculation flow, bypassing Regenerative and Non-Regenerative HXs:
 - A. THROTTLE CLOSE HV-F042 REGEN HX RTN ISLN by pressing the DECREASE push-button until RWCU OUTLET FLOW (Computer Point A2856) is approximately 250 gpm.
 - B. THROTTLE OPEN HV-F104 CLEANUP BYPASS by pressing the INCREASE push-button until RWCU OUTLET FLOW (Computer Point A2856) is approximately 300 gpm.
 - C. ADJUST the positions of HV-F042
 <u>AND</u> HV-F104, such that RWCU OUTLET FLOW
 (Computer Point A2856) is approximately 300 to 320 gpm
 <u>AND</u> CLEANUP SYSTEM OTLT TEMP (Computer Point A215)
 is <434°F. [CD-389E]</p>

Continued next page

Page 32 of 53

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5.5.4 (continued)

- D. <u>WHEN</u> CLEANUP SYSTEM OTLT TEMP (Computer Point A215) can be maintained ≤ 434°F <u>WITH</u> HV-F042 fully closed, <u>THEN</u> PERFORM the following: _____
 - 1. CLOSE HV-F042, REGEN HX RTN ISLN.

<u>AND</u>

2. THROTTLE HV-F104, CLEANUP BYPASS, until RWCU OUTLET FLOW (Computer Point A2856) is approximately 300 to 320 gpm.

<u>NOTE</u> 5.6

CRIDS points B2081 and A2856, <u>OR</u> NUMAC Drawers 1SKXR-11497 <u>OR</u> 1SKXR-11499 (10C609, 10C611) may be used to obtain temperature compensated flows.

CAUTION 5.6

Loss of a RWCU Pump, when in two pump operation, can cause pump runout. Flow must be reduced immediately by closing HV-F044, FILTER DEMIN BYPASS, (if open) <u>OR</u> throttling HV-F042, REGEN HX RTN ISLN, until Chemistry can remove a Filter/Demin from service.

- 5.6 Blowdown During Refueling Operation
 - 5.6.1 ENSURE all prerequisites of Section 2.6 are satisfied.
 - 5.6.2 ENSURE that 1-RC-V005, Three Way Diverting Valve, is in the VERTICAL position.

Hope Creek

Page 33 of 53

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5.6,3

IF blowdown required during Refueling, THEN **PERFORM** the following:

CAUTION 5.6.3.A & B

HV-F033, DRAIN FLOW CONTROL, valve auto closes on upstream pressure \leq 5 psig <u>OR</u> on downstream pressure \geq 140 psig.

- A. <u>IF</u> desired <u>AND</u> the Main Condenser is available, <u>THEN OPEN HV-F034, RWCU RTN TO CNDSR.</u>
- B. <u>IF</u> the Main Condenser is NOT available, <u>THEN</u>, <u>AFTER</u> notifying Radwaste of pending input, **OPEN** HV-F035, RWCU TO EQPT DRN.
- C. OPEN HV-F031, DRN FL ORF BYPASS.
- D. THROTTLE CLOSED HV-F042, REGEN HX RTN ISLN, by pressing the DECREASE push-button <u>AND</u> THROTTLE OPEN HV-F033, HIC R606 DR FL COND, by pressing INCREASE push-button until Computer Point A2947 RWCU DISCH to COND <u>AND</u> EQUIP DRAIN FLOW indicates desired flow.
- 5.6.4 <u>WHEN</u> blowdown operation is complete, <u>THEN</u> **THROTTLE OPEN** HV-F042, REGEN HX RTN ISLN, <u>AND</u> **THROTTLE CLOSED** HV-F033, HIC R606 DR FL COND.

5.6.5 **ENSURE** the following valves are closed:

HV-F034, RWCU RTN TO CNDSR

- HV-F035, RWCU TO EQPT DRN
- HV-F031, DRN FL ORF BYPASS

Hope Creek

Page 34 of 53
PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

<u>NOTE</u> 5.6.6

If 1-RC-V005, Three Way Diverting Valve, is to be left in the vertical position, then SAP/WCM should be updated to show the valve in an "off-normal" <u>OR</u> "tagged for CRS" position.

5.6.6 IF NO additional blowdowns are anticipated, <u>THEN RETURN</u> 1-RC-V005, Three Way Diverting Valve, to the horizontal position, for flow to Main Condenser.

CAUTION 5.7

Loss of a RWCU Pump, when in two pump operation, can cause pump runout. Flow must be reduced immediately by closing HV-F044, FILTER DEMIN BYPASS, (if open) <u>OR</u> throttling HV-F042 REGEN HX RTN ISLN until Chemistry can remove a Filter/Demin from service <u>OR</u> the flow is reduced through the two I/S Demineralizers.

5.7 Reducing Flow through or Bypassing the Filter/Demins

- 5.7.1 ENSURE all prerequisites of Section 2.7 are satisfied. [CD-937B, CD-445D]
- 5.7.2 <u>IF</u> it is desired to reduce flow through the RWCU Demineralizers, <u>THEN</u> **PERFORM** one or both of the following:

NOTE 5.7.3 and 5.7.4

In the event HV-F044, FLTR DEMIN BYPASS, is unavailable or inoperable, the operator may use HV-F104, CLEANUP BYPASS, <u>AND</u> Computer Point A2081 to throttle open/closed in order to bypass <u>OR</u> restore flow to Filter/Demins.

- A. THROTTLE BG-HV-F042, REGEN HX RTN ISLN, until system flow is limited to the desired value.
- B. **DIRECT** Chemistry to reduce flow through the in service Demineralizer(s) until flow is limited to the desired value.

Hope Creek

Rev. 33

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 **PSEG Internal Use Only**

HC.OP-SO.BG-0001(Q)

- 5.7.3 <u>WHEN</u> flow is no longer required to be limited <u>THEN</u> **THROTTLE** BG-HV-F042, REGEN HX RTN ISLN to restore desired system flow and/or **DIRECT** Chemistry to return the Demineralizer(s) flow controllers to their normal settings.
- 5.7.4 IF it is desired to remove a RWCU Filter/Demineralizer from service THEN **PERFORM** the following:
 - A. **DIRECT** Chemistry to slowly reduce flow on the affected Filter/Demineralizer <u>AND</u> remove it from service.
 - B. MONITOR Computer Point A2856 (or equivalent) <u>AND</u> THROTTLE OPEN on HV-F044 as necessary to maintain flow at or below maximum RWCU return to feedwater flow value determined using Attachment 1 for 2 pump operation <u>OR</u> Attachment 2 for 1 pump operation.
- 5.7.5 <u>WHEN</u> it is desired to restore flow through a RWCU Filter/Demineralizer THEN PERFORM the following:
 - A. **DIRECT** Chemistry to place the Filter/Demineralizer in service AND SLOWLY RAISE flow.
 - B. MONITOR Computer Point A2856 (or equivalent) <u>AND</u> THROTTLE CLOSED on HV-F044 as necessary to maintain flow at or below maximum RWCU return to feedwater flow value determined using Attachment 1 for 2 pump operation <u>OR</u> Attachment 2 for 1 pump operation.

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5.8	<u>Removi</u> [CD-93	<u>Removing RWCU Pump/System from Service</u> [CD-937B]			
	5.8.1	ENSURE all prerequisites of Section 2.8 are satisfied.			
	5.8.2	REQUEST Chemistry isolate the CAVS System prior to removing RWCU System from service.			
		<u>NOTE</u> 5.8.3			
Sys	stem flow	should be maintained as indicated by Computer Point A2856 \pm 20 gpm.			
	5.8.3	IF 2 RWCU Filter Demins (F/D) are in-service, <u>THEN REQUEST</u> Chemistry to remove 1 F/D from service <u>OR</u> reduce flow to maintain both Demins in-service <u>AND</u> flow between 140 - 160 gpm.			
	5.8.4	THROTTLE closed HV-F044, FLTR DEMIN BYPASS, push-button until Computer Point A2856 indicates approximately 140 to 160 gpm.			
	5.8.5	IF both RWCU Pumps are running, THEN STOP A(B)P221, A(B) RWCU PUMP. [CD-937B]			
	5.8.6	IF Reactor Vessel water temperature is > 212°F AND the pump is to be out-of-service for more than approximately 15 min, THEN PERFORM the following to isolate the pump:[CD-937B]			
•	• • •	A. CLOSE 1-BG-V006 (1-BG-V010), A(B) RWCU Recirc Pmp Dsch Vlv.			
		B. ENSURE 1-BG-V200 (1-BG-V210), A(B) RWCU Recirc Pmp Dsch Byp Vlv is closed.			
		C. CLOSE 1-BG-V004 (1-BG-V008), A(B) RWCU Recirc Pmp Suction Vlv.			

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

		<u>NOTE</u> 5.8.7	
A.	If only one RWCU Pump and F/D are to be removed from service, then the remaining steps are <u>NOT</u> required.		
В.	Sy: +/-	stem flow should be maintained as indicated by Computer Point A2856 20 gpm to prevent pump trip on low flow.	
	5.8.7	<u>IF</u> a second RWCU Pump is to be removed from service, <u>THEN</u> DIRECT Chemistry Department to perform the following:	
		A. REMOVE the remaining F/D from service.	
	•	B. ISOLATE the following[PR 970803103]:	
		• CLEAN UP DEMIN FILTER INLET.	
		CLEAN UP FILTER DEMIN. 'A' DISCH	· ·
		CLEAN UP FILTER DEMIN. 'B' DISCH	
	5.8.8	STOP B(A)P221, B(A) RWCU PUMP. [CD-937B]	
	5.8.9	IF Reactor Vessel water temperature is > 212°F	
		AND the pump is to be out-of-service for more than	
		approximately 15 min, <u>THEN</u> PERFORM the following to isolate the pump: [CD-937B]	
		A. CLOSE 1-BG-V010 (1-BG-V006), B(A) RWCU Recirc Pmp Dsch Vlv.	
		 ENSURE 1-BG-V210 (1-BG-V200), B(A) RWCU Recirc Pmp Dsch Byp Vlv is closed. 	
•		C. CLOSE 1-BG-V008 (1-BG-V004), B(A) RWCU Recirc Pmp Suction Vlv.	
·	5.8.10	CLOSE HV-F044, FLTR DEMIN BYPASS.	

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

<u>NOTE 5.8.11</u>

The following step need only be completed if RWCU System must be isolated.

5.8.11 IF RWCU System must be isolated, THEN CLOSE the following valves:

A. HV-F001, PMP SUCT CONT INBD ISOLATION VALVE

B. HV-F004, PUMP SUCT CONT OUTBD ISOLATION VALVE

C. HV-F039, RWCU RTN TO RPV ISOLATION VALVE

<u>NOTE 5.8.12</u>

If a negative value for power loss is observed in Step 5.8.12, then a substitute value of 0.0 for Computer Point A196 RWCU FLOW (WCU) (Insert Value Function) will correct the heat balance discrepancy.

5.8.12

OBSERVE the Process Computer heat balance (OD-3d)
 value for power loss in the Cleanup Demineralizer System (QCU)
 <u>AND</u>, as necessary,
 PLACE Computer Point A196 in MANUAL with a substitute value of 0.0
 using Attachment 3 - Computer Point A196 Substitute Value Instructions,

to correct the heat balance discrepancy.

Hope Creek

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

<u>NOTE 5.9</u>

The Regenerative Heat Exchangers can only be bypassed in CONDITION 4, <u>OR</u> 5, <u>OR</u> Condition 2 when maintaining temperature < 200° F.

5.9 Bypassing the Regenerative Heat Exchangers

- 5.9.1 **ENSURE** all prerequisites of Section 2.9 are satisfied.
- 5.9.2 <u>IF</u> bypass of the Regenerative Heat Exchangers is required, <u>THEN</u> **PERFORM** the following:
 - A. UNLOCK AND OPEN V231, Bypass Valve.

<u>NOTE</u> 5.9.1.B

System flow may have to be adjusted as V233 is opened.

- B. UNLOCK AND OPEN V233, Bypass Valve.
- C. **CLOSE** V230, Hx Outlet Valve.
- D. ADJUST system flowrate (for 2 pump operation) to approx. 300- 320 gpm. (flow rate may change due to the difference in system resistance due to heat exchanger being removed from service. Flow adjustments should be made using HV-F042.)

Continued next page

HC.OP-SO.BG-0001(Q)

5.9 (continued)

- 5.9.3 <u>IF</u> partially bypass of the Regenerative Heat Exchangers is required, THEN **PERFORM** the following:
 - A. UNLOCK AND OPEN V233, Bypass Valve.

<u>NOTE</u> 5.9.3.B

System flow may have to be adjusted as V231, Bypass Valve, is opened.

- B. UNLOCK AND THROTTLE OPEN V231, Bypass Valve, as necessary to maintain desired RWCU temperature.
- C. ADJUST system flowrate (for 2 pmp operation) to approx. 300 to 320 gpm. (flow rate may change due to difference in system resistance due to Heat Exchanger being bypassed. Flow adjustments should be made using HV-F042).

Continued next page

Hope Creek

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5.9 (continued)

<u>NOTE</u> 5.9.4

The following step is intended to be used during the performance HC.OP-IS.ZZ-0001(Q), Inservice System Leakage Test of the Reactor Coolant Pressure Boundary, as a temperature control method.

5.9.4 To control RWCU system outlet temperature, USE any combination of the following:

• THROTTLE BG-HV-F104, CLEANUP BYPASS -

OPEN to raise RWCU System outlet temperature (CRIDS A215) OR CLOSED to lower RWCU System outlet temperature.

- THROTTLE BG-HV-F042, HX RETURN TO VESSEL, as necessary, to maintain RWCU OUTLET FLOW (CRIDS A2856) between 300 - 320 gpm.
- THROTTLE 1-ED-V035 RWCU NRHx RACS RTN VLV -

OPEN to lower RWCU System outlet temperature (CRIDS A215) OR

CLOSED to raise RWCU System outlet temperature

- 5.9.5 <u>WHEN</u> RWCU system outlet temperature control is no longer required THEN **PERFORM** the following:
 - A. **CLOSE BG-HV-F104, CLEANUP BYPASS**
 - B. OPEN BG-HV-F042, Hx Return to Vessel.
 - C. THROTTLE 1-ED-V035 RWCU NRHx RACS RTN VLV to 43 degrees OPEN (IAW SAP WCM Blocking Point Information Page)

PSEG Internal Use Only

HC,OP-SO.BG-0001(Q)

5.9.6	PERFORM the following to secure bypassing the Regenerative Heat Exchangers:			
:	А.	<u>IF</u> Regenerative Heat Exchanger was drained, <u>THEN</u> VERIFY Heat Exchanger is filled and vented.		
	B.	ENSURE V232, Hx Inlet Valve, open.		
	C.	OPEN V230, Hx Outlet Valve.		
:	D.	CLOSE AND LOCK V231, Bypass Valve.		
:	E.	CLOSE AND LOCK V233, Bypass Valve.		
	F.	ADJUST system flowrate as required (flow rate may change due to the difference in system resistance due to securing bypassing the heat exchanger).		

Hope Creek

÷

Page 43 of 53

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

5.10 Placing the Regenerative Heat Exchangers in Service

CAUTION 5.10.1

Fill and vent Heat Exchangers slowly to avoid a RWCU Isolation.

5.10.1 <u>IF the Regenerative Heat Exchanger is to be placed in-service,</u> <u>THEN PERFORM the following:</u>

- A. **PERFORM** the following to ensure Heat Exchangers are filled and vented:
 - 1. THROTTLE OPEN BG-HV-F104 to maintain flow through the RWCU Pumps while closing BG-HV-F042, Hx Return to Vessel.
 - 2. CLOSE BG-HV-F042, Hx Return to Vessel.
 - 3. **OPEN** V230, Hx Outlet Valve.

4. CLOSE <u>AND</u> LOCK V231, Hx Bypass Valve, <u>AND</u> V233, Hx Bypass Valve.

- 5. **OPEN** V232, Hx Inlet Valve.
- 6. **VENT** using V066, Hx High Point Vent Valve, <u>AND</u> V067, Hx High Point Vent Valve.
- B. OPEN BG-HV-F042, HX Return To Vessel.
- C. CLOSE BG-HV-F104.
- D. ADJUST system flow to approximately 300 gpm.

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

<u>NOTE</u> 5.11

- A. The following steps can be used to flush a RWCU Pump prior to maintenance for the purpose of reducing Rad Levels. Rad Pro should be contacted to survey the pump after a 20 minute flush <u>OR</u> as otherwise directed by the OS/CRS.
- B. If the desired Rad levels are <u>NOT</u> achieved, then the following steps should be repeated until a sufficient dose reduction is achieved

5.11 Flushing a RWCU Pump prior to Maintenance

5.11.1 ENSURE all prerequisites of Section 2.11 are satisfied.

CAUTION 5.11.2

RWCU Pump cooldown should be limited to 25°F/min.

5.11.2	<u>IF</u> fl <u>THE</u>	lush of A RWCU Pump is required, EN PERFORM the following:	
	A.	CLOSE 1-BG-V006, A RWCU Recirc Pmp Dsch Vlv.	
	B.	CLOSE 1-BG-V200, A RWCU Recirc Pmp Dsch Byp Vlv.	
	C.	CLOSE 1-BG-V004, A RWCU Recirc Pmp Suction Vlv.	<u> </u>
	D.	OPEN 1-BG-V026, Seal Flush Vent, <u>AND</u> 1-BG-V027, Seal Flush Vent.	
•	E.	OPEN 1-BG-V194, Discharge Line Drain, <u>AND</u> 1-BG-V195, Discharge Line Drain.	
	F.	CLOSE 1-BG-V024, Casing Drain Isolation Valve.	
next nage	G.	OPEN 1-BG-V022, Casing Drain Vlv, <u>AND</u> 1-BG-V023, Casing Drain Vlv.	

Continued next page

Hope Creek

Page 45 of 53

Rev. 33

HC.OP-SO.BG-0001(Q)

5.11.2 (continued)

PSEG Internal Use Only

H.	OPEN 1-AP-V142, Condensate Transfer Isolation Valve.
I.	<u>WHEN</u> the pump has flushed for approximately 20 minutes, <u>THEN</u> CLOSE 1-AP-V142, Condensate Transfer Isolation Valve

- J. NOTIFY Radiation Protection to survey the RWCU Pump as necessary.
- K. <u>IF</u> the desired rad levels have NOT been achieved, <u>THEN **REPEAT** Steps 5.11.2 H through 5.11.2.J as</u> necessary until desired rad levels are achieved.
- 5.11.3 <u>WHEN</u> flushing is complete, <u>THEN</u> **PERFORM** the following:
 - A. CLOSE 1-AP-V142, Condensate Transfer Isolation Valve.
 - B. CLOSE 1-BG-V022, Casing Drain Vlv, AND 1-BG-V023 Casing Drain Vlv.
 - C. OPEN 1-BG-V024, Casing Drain Isolation Valve.
 - D. CLOSE 1-BG-V194, Discharge Line Drain, AND 1-BG-V195, Discharge Line Drain.
 - E. CLOSE 1-BG-V026, Seal Flush Vent, AND 1-BG-V027, Seal Flush Vent.

5.11,4

- 1.4 <u>IF</u> flush of the B RWCU Pump is required, THEN **PERFORM** the following:
 - A. CLOSE 1-BG-V010, B RWCU Recirc Pmp Dsch Vlv.

B. CLOSE 1-BG-V210, B RWCU Recirc Pmp Dsch Byp Vlv.

Continued next page

HC.OP-SO.BG-0001(Q)

PSEG Internal Use Only

5.11.4 (continued

(her		,	
icuj	C.	CLOSE 1-BG-V008, B RWCU Recirc Pmp Suction Vlv.	
	D.	OPEN 1-BG-V073, Seal Flush Vent, AND 1-BG-V074 Seal Flush Vent.	
	E.	CLOSE 1-BG-V030, Casing Drain Isolation Valve.	
	F.	OPEN 1-BG-V028, Casing Drain Vlv, <u>AND</u> 1-BG-V029, Casing Drain Vlv.	_,
•	G.	OPEN 1-AP-V143, Condensate Transfer Isolation Valve.	
	H.	<u>WHEN</u> the pump has flushed for approximately 20 minutes, <u>THEN</u> CLOSE 1-AP-V143, Condensate Transfer Isolation Valve.	
	I.	NOTIFY Radiation Protection to survey the RWCU Pump.	
·	J.	IF the desired rad levels have NOT been achieved, THEN REPEAT Steps 5.11.4. G through 5.11.4.I as necessary until desired rad levels are achieved.	
5.11.5	<u>WH</u> THI	EN flushing is complete, EN PERFORM the following:	
	А.	CLOSE 1-AP-V143, Condensate Transfer Isolation Valve.	
	B.	CLOSE 1-BG-V028, Casing Drain Vlv, AND 1-BG-V029, Casing Drain Vlv.	
÷	C.	OPEN 1-BG-V030, Casing Drain Isolation Valve.	
• •	D.	CLOSE 1-BG-V073, Seal Flush Vent, AND 1-BG-V074 Seal Flush Vent.	

Hope Creek

Page 47 of 53

÷

HC.OP-SO.BG-0001(Q)

PSEG Internal Use Only

- 6.0 REFERENCES
 - 6.1 P&ID's
 - M-25-1 M-44-1 M-45-1 M-43-1, Sht. 1 M-101

6.2 Logic Diagrams

J-44-0 Sht. 1, 2, 3, 4, 5, 6, 7, 8

6.3 Electrical Drawings

E-0021-1 Sht. 1, 5 E-0031-1 E- 0032-1

6.4 Vendor Manuals

NI-G33-386 & 387 Ecodyne Corporation - RWCU F/D NI-G33-429-2 Union Pump Co. - RWCU Recirc Pumps N1-A41-0045 (1) RWCU System

6.5 Panel Drawings

J-0650-1 Sht. 9 J-0651-1 Sht. 6

6.6 **<u>DITS</u>**

D3.31 D5.14

6.7 GE Documents

GEK 90300 Volume VI, Part 1 23A1860, RWCU Pump Instructions G.E. Drawing 166B8227

Hope Creek

Page 48 of 53

Rev. 33

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

6.8 Commitment Documents

CD-886X	(Q210.24)
CD-937B	(AID 25-80)
CD-786D	(AID 48-78)
CD-445D	(SIL 258 SUPP 2)
CD-389E	(SIL 436)

6.9 Other

DEF # DEH-92-00081

6.10 Cross References

HC.RE-SO.RJ-0010(Q), Process Computer Program Operating Instructions

6.11 Corrective Actions

PR 970803103 PR 980515086 PR 980709244 CR971127121 RWCU Differential Flow setpoint 20004666 TS990418149 20018965 80007129 DCP 4EC-3192

Hope Creek

Page 49 of 53

HC.OP-SO.BG-0001(Q)

ALL ACTIVE ON-THE-SPOT CHANGES

MUST

BE

ATTACHED

FOR

FIELD

USE

PSEG Internal Use Only



ATTACHMENT 1 MAXIMUM RWCU RETURN TO FEEDWATER FLOW - 2 PUMP OPERATION

Hope Creek

Page 50 of 53

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

ALL ACTIVE ON-THE-SPOT CHANGES

MUST

晤

ATTACHED

FOR

FIELD

USE



Maximum RWCU Return to Feed Water Flow equivalent to Total RWCU F/D Bed Flow of 150 gpm @ 120°F with one pump running



Hope Creek

Page 51 of 53

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

ATTACHMENT 3 COMPUTER POINT A196 SUBSTITUTE VALUE INSTRUCTIONS (Page 1 of 2)

NOTE

A196 (Cleanup System Flow) is placed in MANUAL with a value of "0.0" when taking RWCU out of service.

A196 (Cleanup System Flow) is placed in AUTOMATIC when returning RWCU to service.

AUTOMATIC The value of the computer point is updated based on field inputs and/or computer algorithms.

MANUAL. The value of the computer point is frozen at the current value unless a different value is manually entered.

- 1.0 To swap computer point A196 between a) AUTOMATIC and b) MANUAL (with a value of 0.0) **PERFORM** the following:
 - 1.1 **SELECT** "SYS" on top menu bar.
 - 1.2 SELECT "Change_Env" on pull down menu.
 - 1.3 **SELECT** "Maint_Env" on second pull down menu.
 - 1.4 ENTER password "HCNSSS" using keypad THEN PRESS "ENTER" key.
 - 1.5 **SELECT** "SELECT" on top menu bar.
 - 1.6 **ENSURE** that an "*" is in the "CP NAMES SEARCH" blue box.
 - 1.7 SELECT "AINMISC" on left of screen.
 (It may be necessary to repeat Steps 1.5 and 1.6 to have the list of choices displayed.)
 - **1.8 SELECT** "A196" on right of screen.
 - 1.9 VERIFY the expected "A" or "M" appears next to "MEAS" on right of screen. ("A" = AUTOMATIC, "M" = MANUAL) <u>IF</u> already is desired state <u>THEN</u> PROCEED to step 1.13.

Continued Next Page

Hope Creek

Page 52 of 53

PSEG Internal Use Only

HC.OP-SO.BG-0001(Q)

ATTACHMENT 3 COMPUTER POINT A196 SUBSTITUTE VALUE INSTRUCTIONS (Page 2 of 2)

1.0. (Continued)

- 1.10 SELECT "A/M" block on bottom of screen to swap.
- 1.11 VERIFY the desired "A" or "M" appears next to "MEAS" on right of screen.
- 1.12 IF placing A196 in MANUAL with a value of 0.0 THEN PERFORM the following:
 - A. **SELECT** "PNT" on right side of screen and move mouse to the middle of the screen (current value will be outlined in a yellow box along with "PNT").
 - B. **SELECT** the space between the up and down arrows at bottom of the screen (a blue box will appear).
 - C. ENTER "0.0" in the blue box <u>THEN</u> PRESS "ENTER" key (box will turn gray).

D. **VERIFY** that "0.0" appears in the gray box.

1.13 SELECT "SYS" on top menu bar.

1.14 SELECT "Change_Env" on pull down menu.

- 1.15 SELECT "Oper_Env" on second pull down menu.
- 1.16
 ENTER password "Operator" using keypad

 THEN
 PRESS "ENTER" key.
- 1.17 SELECT "DISP" on top menu bar followed by "OD_3d THEN VERIFY the following:
 - "WCU" is colored blue (cyan) when in MANUAL
 - "WCU" shows a value of 0.0 when in MANUAL, with A196 set to "0.0".
 - "QCU" shows a value of 0.0 with A196 set to "0.0".
 - "WCU" is colored black when in AUTOMATIC
- If placing A196 in MANUAL with a value of "0.0"

 THEN VERIFY that CRIDS point "B2081" is in alarm (red), with a value of "0.0".

Hope Creek

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 PSEG Internal Use Only

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

4/8/02

040402

APPROVED:

Manager - Hope Creek Operations

CATEGORY II

Effective Date

SHUTDOWN COOLING

ALARMS A6 - A1RHR LOGIC A OUT OF SERVICE A7 - A1RHR LOGIC B OUT OF SERVICE A6 – B1 RHR LOOP A TROUBLE A7 – B1 RHR LOOP B TROUBLE A6 – D5 RHR HX CLG WTR OUTLET TEMP HI A4 – F5 COMPUTER PT IN ALARM C6 – E1 APRM/RBM FLOW REF OFF NORMAL

INDICATIONS

- Trip of RHR pump in Shutdown Cooling
- Isolation of Shutdown Cooling Flowpath
- Reduced or stopped RHR Shutdown Cooling flow to the Jet Pumps
- Lowering Core flow
- Rising Reactor coolant temperature/pressure
- Rising Recirc pump loop flow

Hope Creek

Page 1 of 45

Rev. 0

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

ADDITIONAL INFORMATION:

Procedures:

HC.OP-GP.SM-0001(Q) DEFEATING NSSSS ISOLATIONS FOR SHUTDOWN COOLING

Valves:

- BC-HV-F008 SHUTDOWN COOLING OUTBD ISOLATION
- BC-HV-F009 SHUTDOWN COOLING INBD ISOLATION
- BC-HV-F015A RHR LOOP A RET TO RECIRC
- BC-HV-F015B RHR LOOP B RET TO RECIRC
- BC-HV-F022 RHR LOOP B HEAD SPRAY INBD ISOLATION
- BC-HV-F023 RHR LOOP B HEAD SPRAY OUTBD ISOLATION
- BC-HV-F006A RHR PMP A SUCT FROM RECIRC
- BC-HV-F006B RHR PMP B SUCT FROM RECIRC

Hope Creek

IMMEDIATE OPERATOR ACTIONS

NONE

AUTOMATIC ACTIONS

	THEN THE REPORT OF	
Reactor Pressure > 82 psig	The following valves cannot be opened from the Control Room <u>OR</u> their Remote Shutdown controls:	
	 HV-F008 HV-F009 HV-F015A HV-F015B HV-F022 HV-F023 	
Reactor Pressure > 82 psig <u>OR</u> Reactor Level < 12.5" <u>OR</u> Loss of <u>EITHER</u> RPS Bus <u>AND</u> RSP Takeover Switches in NORMAL	The following valves will isolate: • HV-F008* • HV-F009* • HV-F015A* • HV-F015B* • HV-F022 • HV-F023	
RHR Pump in Shutdown Cooling AND Closure of ANY of the following:	RHR pump trips.	
• HV-F008		
• HV-F009		
Associated HV-F006		

*If GP.SM-0001 has been performed, these isolations are bypassed.

Hope Creek

Rev. 0

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

This Page Intentionally Left Blank

LIST OF CONDITIONS

A.	Loss of the RHR system providing Shutdown Cooling7
B.	Alternate Loop of RHR is required to restore Shutdown Cooling
C.	RHR S/D Cooling CANNOT be established within 1 hour
D.	Loss of S/D Cooling AND RCS temp. < 200°F AND RCS temp. is anticipated to reach \geq 200°F9
E.	Forced Circulation CANNOT be established using preferred RHR loops or Reactor Recirculation.11
F.	RWCU is required for Alternate Decay Heat Removal
G.	Condensate Transfer is required for S/D cooling
H.	RPV Temperature and Pressure CANNOT be maintained using Normal OR Alternate Decay Heat Removal

ALL ACVIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 SEG INTERNAL USE UNIV

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

ADDITIONAL INFORMATION:

Procedures:

- HC.OP-SO.BC-0002 DECAY HEAT REMOVAL OPERATION.
- HC.OP-SO.BB-0002 REACTOR RECIRCULATION SYSTEM OPERATION.

Valves:

- BC-HV-F015A RHR LOOP A RET TO RECIRC
- BC-HV-F015B RHR LOOP B RET TO RECIRC
- BC-HV-F008 SHUTDOWN COOLING OUTBD ISOLATION
- BC-HV-F009 SHUTDOWN COOLING INBD ISOLATION

FIGURE 1





Time from Rx Shutdown (hours)

SUBSEQUENT OPERATOR ACTIONS

CONDITION	ACTION
A. Loss of the RHR system providing Shutdown Cooling	□ A.1 <u>IF</u> below 200°F, <u>THEN</u> PERFORM the following:
	a. ESTIMATE time to reach 200°F IAW Figure 1.
Time:	b. IF RCS temperature is expected to rise abo 200°F,
	THEN IMPLEMENT Condition D.
Estimated time RCS	A.2 IF the Shutdown Cooling suction line was isolated, THEN PERFORM the following:
	 <u>PRIOR</u> to establishing the suction flow path from the vessel, FILL & VENT the suction line IAW SO.BC-0002. [CD-891D]
	• EVALUATE the need to implement Condition C (Suction line venting may take more than one hour).
	A.3 <u>IF</u> necessary, dispatch operators to manually open HV-F008 and/or HV-F009. [CD-065X]
	A.4 Restore the tripped RHR pump as follows:
	a. CLOSE F015A(B).
	b. RESTART RHR Pump A(B).
	C. IMMEDIATELY THROTTLE OPEN F015A(B) to establish Shutdown Cooling flow between 3000 gpm and 10,000 gpm.
B. Alternate Loop of RHR is required to restore Shutdown Cooling	B.1 PLACE RHR loop A(B) in Shutdown Cooling IAW SO.BC-0002.
Time:	
C. RHR S/D Cooling CANNOT be established within 1 hour. [T/S 3.4.9.1, 3.4.9.2]	 C.1 MONITOR Reactor Coolant temperature and pressure at least once per hour. [3.4.9.1.b, 3.4.9.2.b]
[T/S 3.9.11.1, 3.9.11.2] Time:	 C.2 ENSURE forced circulation in the core utilizing Reactor Recirc IAW SO.BB-0002 OB on alternate method

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 200301223EG Internal Use Only

HC,OP-AB.RPV-0009(Q) SHUTDOWN COOLING

ADDITIONAL INFORMATION:

Valves:

- REACTOR HEAD VENT DRW INBD ISLN BB-HV-F001 .
- BB-HV-F002

REACTOR HEAD VENT DRW OUTBD ISLN

TABLE D.2 - RED	UNDANT REACTOR VI	ESSEL PRESSURE INDICATIONS
NOMENCLATURE	RANGE	DESCRIPTION
	CONTROL ROOM PA	NEL 10-C-650
PI-5824A	0 - 50	LOW RANGE REACTOR PRESS
PI-5824B	0 - 50	LOW RANGE REACTOR PRESS
PI-R605-C32	0 - 1200	REACTOR PRESSURE
PR-R623A-B21	0 - 1500	PAMS
PR-R623B-B21	0 - 1500	PAMS
PI-3684A	0 - 1500	PAMS
PI-3684B	0 - 1500	PAMS
	LOWER RELAY ROOM	4 PANEL 10-C-617
1FDPISL-N658A-E41	0 - 200	HPCI TRIP UNIT
1FDPISL-N658E-E41	0 - 200	HPCI TRIP UNIT
	LOWER RELAY ROOM	1 PANEL 10-C-641
IEDPISL-N658C-E41	0 - 200	HPCI TRIP UNIT
1FDPISL-N658G-E41	0 - 200	HPCI TRIP UNIT
	LOWER RELAY ROOM	I PANEL 10-C-618
1ECPISL-N658B-E51	0 - 200	RCIC TRIP UNIT
IFCPISL-N658F-E51	0 - 200	RCIC TRIP UNIT
	LOWER RELAY ROOM	I PANEL 10-C-631
IECPISL-N658D-E51	0 - 200	RCIC TRIP UNIT
IFCPISL-N658H-E51	0 - 200	RCIC TRIP UNIT

٠

SUBSEQUENT OPERATOR ACTIONS

and the second

CONDITION	ACTION
D. Loss of S/D Cooling <u>AND</u> RCS temp. < 200°F	 D.1 EVALUATE the need to establish Primary <u>AND</u> Secondary Containment.
$\frac{\text{AND}}{\text{RCS temp. is anticipated to reach}} > 200^{\circ}\text{F}$	D.2 EVALUATE the following for indications of entry into OPCON 3:
<pre>[T/S 3.6.1.1, 3.6.5.1] Time:</pre>	 An increase in steam dome pressure could be indicative that boiling, to some degree, occurring in the Reactor Core. This parameter should be monitored using the redundant Reactor pressure indications given in Table D.2, particularly those with the lowest ranges such as Reactor Low Range Pressure (0 – 50 psig on Panel 10C650A) or, the HPCI and RCIC trip units.
	 An increase in Reactor Head Vent temperature could be indicative of the ons or verification of the presence of boiling is the vessel. This temperature is monitored by TE-N064 (when BB-HV-F001 and BB-HV-F002 are open), and can be read the Control Room recorder B21-TRR614 point 24, Reactor Head Vent.
	 An unexplained increase in Drywell leaka could be indicative of the presence of stea flow out of the vessel head and into the Drywell. This parameter is monitored at RMS and includes primarily the Drywell Equipment Drain flow, however, steamin into the Equipment Drain Sump could als be indicated by total Drywell leakage and Drywell Cooler condensate flow, therefor all of these points should be monitored for unexplained increases.

CAUTIONS:

1. Apply temperature compensation as necessary. Main Steam Line flooding occurs at 118 inches.

NOTES:

1. <u>IF</u> in IO-0004, IO-0005, or IO-0009, the associated attachment for "PLACING THE PLANT IN ALTERNATE DECAY HEAT REMOVAL MODE OF OPERATION" <u>MAY</u> identify an effective mode of Alternate Decay Heat Removal.

1

ADDITIONAL INFORMATION:

Procedures:

- HC.OP-IO.ZZ-0004(Q) SHUTDOWN FROM RATED POWER TO COLD SHUTDOWN
- HC.OP-IO.ZZ-0005(Q) COLD SHUTDOWN TO REFUELING
- HC.OP-IO.ZZ-0009(Q) REFUELING OPERATION
- HC.OP-DL.ZZ-0026(Q) SURVEILLANCE LOG
- HC.OP-SO.EC-0001(Q) FUEL POOL COOLING AND CLEANUP SYSTEM
- HC.OP-SO.BG-0001(Q) REACTOR WATER CLEANUP SYSTEM OPERATION

Valves:

• 1-ED-V035

RWCU NRHX RACS RTN PLUG. (Rm 4504E).

ç

SUBSEQUENT OPERATOR ACTIONS (continued)

CONDITION		ACTION		
E.	Forced Circulation CANNOT be		** <u>NOTE 1</u> **	
	established using preferred RHR loops or Reactor Recirculation.		E.1 MONITOR temperatures IAW DL-0026 Attachment 3s.	
	CD-100A, CD-076B, CD-065X]		CAUTION 1	
	Time:		E.2 MAINTAIN RPV LVL \geq 80 inches, BUT < 90 inches.	
			E.3 <u>IF RPV LVL reaches 90 inches,</u> <u>THEN</u> CLOSE the MSIV's.	
			E.4 ENSURE T.S. cool down limits are not exceeded. [T/S 3.4.6.1.b]	
			E.5 EVALUATE the following systems for alternate decay heat removal:	
			• RWCU (Subsequent F) [CD-900E]	
			• "C" RHR (Attachment 1)	
			• "D" RHR (Attachment 2)	
		a	 CONDENSATE TRANSFER (Subsequent G) 	
			E.6 <u>IF</u> the vessel head is removed, <u>AND</u> the Reactor Cavity is flooded, <u>THEN</u> maximize Fuel Pool Cooling:	
			• ENSURE two Fuel Pool Cooling pump are in service. (EC)	
			• ENSURE SACS flow aligned through BOTH Fuel Pool Cooling heat exchange	
F.	RWCU is required for Alternate Decay		F.1 ENSURE RWCU is in service. (BG)	
	Heat Removal. [CD-900E]		F.2 FULLY OPEN ED-V035.	
	Time:		F.3 <u>IF</u> necessary, <u>THEN</u> Bypass the Regenerative heat exchanger to maximize decay heat remova (BG)	

Hope Creek

Rev. 0

ALL ACFIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 200301223EG Internal Use Only

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

ADDITIONAL INFORMATION:

Procedures:

- HC.OP-SO.AP-0001(Q) CONDENSATE TRANSFER SYSTEM OPERATION
- HC.OP-SO.BG-0001(Q) REACTOR WATER CLEANUP SYSTEM OPERATION

Valves:

TABLE G: Condensate Transfer Injection Flowpaths					
SYSTEM	CS XFR DISCH ISLN	OUTBD ISOL VLV	INJECTION VALVE		
'A' RHR	1-AP-V044 (Rm. 4328)	N/A	BC-HV-F017A		
'C' RHR	1-AP-V047 (Rm. 4328)	N/A	BC-HV-F017C		
'B' RHR	1-AP-V056 (Rm 4322B)	N/A	BE-HV-F017B		
'D' RHR	1-AP-V059 (Rm. 4322B)	N/A	BE-HV-F017D		
'A' CORE SPRAY	1-AP-V041 (Rm. 4331)	BE-HV-F004A	BE-HV-F005A		
'B' CORE SPRAY	1-AP-V062 (Rm. 4322B)	BE-HV-F004B	BE-HV-F005B		
HPCI	1-AP-V038 (Rm. 4331)	N/A	BJ-HV-8278		
RCIC	1-AP-V052 (Rm. 4315)	N/A	BD-HV-F013		

SUBSEQUENT OPERATOR ACTIONS (continued)

.

CONDITION	ACTION
G. Condensate Transfer is required for S/D cooling.	G.1 ENSURE Condensate Transfer is in-service. (AP)
	G.2 ENSURE RWCU is in-service. (BG)
Time:	G.3 REFER to Table G and align one of the following flowpaths to feed the vessel from condensate transfer as follows:
	G.4 <u>IF</u> an RHR injection line will be used, <u>THEN</u> PERFORM the following:
	a. OPEN the desired loops INJECTION VALVE listed in Table G.
	b. THROTTLE OPEN the associated CS XFR DISCH ISLN listed in Table G to obtain the desired injection rate.
	□ G.5 <u>IF</u> a Core Spray injection line will be used, <u>THEN</u> PERFORM the following:
	a. CLOSE the desired loops OUTBD ISOL VLV listed in Table G.
	b. OPEN the associated INJECTION VALVE listed in Table G.
	c. RE-OPEN the OUTBD ISOL VLV.
	d. THROTTLE OPEN the associated CS XFR DISCH ISLN listed in Table G to obtain the desired injection rate.
	A *** Continued on Page 15 ***

NOTES:

2. Reactor pressure must be below the Low Pressure Isolation setpoint for the system to prevent auto-closure of the injection valve.

ADDITIONAL INFORMATION:

Procedures:

HC.OP-SO.BG-0001(Q) REACTOR WATER CLEANUP SYSTEM OPERATION

Valves:

- AE-HV-F032A FW INLET CHECK VALVE
- AE-HV-F032B FW INLET CHECK VALVE

TABLE G: Condensate Transfer Injection Flowpaths				
SYSTEM	CS XFR DISCH ISLN	OUTBD ISOL VLV	INJECTION VALVE	
'A' RHR	1-AP-V044 (Rm. 4328)	N/A	BC-HV-F017A	
'C' RHR	1-AP-V047 (Rm. 4328)	N/A	BC-HV-F017C	
'B' RHR	1-AP-V056 (Rm 4322B)	N/A	BE-HV-F017B	
'D' RHR	1-AP-V059 (Rm. 4322B)	N/A	BE-HV-F017D	
'A' CORE SPRAY	1-AP-V041 (Rm. 4331)	BE-HV-F004A	BE-HV-F005A	
'B' CORE SPRAY	1-AP-V062 (Rm. 4322B)	BE-HV-F004B	BE-HV-F005B	
HPCI	1-AP-V038 (Rm. 4331)	N/A	BJ-HV-8278	
RCIC	1-AP-V052 (Rm. 4315)	N/A	BD-HV-F013	

CONDITION	ACTION		
*** Continued from Page 13 ***	*** Continued from Page 13 ***		
 G. Condensate Transfer is required for S/D cooling. 	G.6 <u>IF</u> the RCIC injection line will be used, <u>THEN</u> PERFORM the following:		
-	□ ** <u>NOTE 2</u> **		
<u>.</u>	□ a. VERIFY reactor pressure < 64 psig		
	b. ENSURE the HV-F032B is open.		
	c. PLACE the NORM/BYPASS keylock switch for the HV-F013 in BYPASS.		
	d. OPEN the HV-F013.		
	• c. THROTTLE OPEN the 1-AP-V052 to obtain the desired injection rate.		
	G.7 IF the HPCI injection line will be used, THEN PERFORM the following:		
	••• <u>NOTE 2</u> **		
	a. VERIFY reactor pressure < 100 psig		
	b. ENSURE the HV-F032A is open.		
	c. PLACE the NORM/BYPASS keylock switch for the HV-8278 in BYPASS.		
	d. OPEN the HV-8278.		
	e. THROTTLE OPEN the 1-AP-V038 to obtain the desired injection rate.		
	G.8 MAINTAIN RPV LVL using RWCU Blowdown. (BG)		

Page 15 of 45

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 200301225E INTERNAL USE UNLY

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

NOTES:

3. The goal of the following steps is to establish water flow through two SRV's with at least a 50# d/p. It is desirable to maintain less than a 160# d/p across the SRV's to ensure a single LPCI or Core Spray pump can deliver sufficient flow to remove decay heat. Any injection source may be used to flood the vessel, but it is desirable to use the torus as a suction source for recirculation of water through the vessel and SRVs. This will prevent unnecessary water addition to the torus from outside sources.

ADDITIONAL INFORMATION:

Procedures:

HC.OP-AB.ZZ-0001(Q) TRANSIENT PLANT CONDITIONS.

Valves:

- BB-HV-F001 REACTOR HEAD VENT DRW INBD ISLN
- BB-HV-F002 REACTOR HEAD VENT DRW OUTBD ISLN
- BB-HV-F005 REACTOR HEAD VENT STM LINE A
- AB-HV-F016 CTMT INBD STM LINE DRAIN HDR ISLN OUTBOARD MOV
 AB-HV-F019 CTMT INBD STM LINE DRAIN HDR ISLN INBOARD MOV
 - FD-HV-F002 HPCI STM INBD ISLN VLV
- FD-HV-F003 HPCI STM OUTBD ISLN VLV
- FD-HV-F100 HPCI STM WARMUP VLV
- FC-HV-F007 RCIC STM INBD ISLN VLV
- FC-HV-F008 RCIC STM OUTBD ISLN VLV
- FC-HV-F076 RCIC STM WARMUP VLV

Hope Creek

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 **PSEG Internal Use Only**

.

SUBSEQUENT OPERATOR ACTIONS (continued)

	CONDITION			ACTION
H.	RPV Temperature and Pressure	a		** <u>NOTE 3</u> **
	CANNOT be maintained using Normal OR Alternate Decay Heat Removal.	a	H.1	INITIATE Suppression Pool Cooling IAW AB-0001.
	[CD-973B, CD-110E, CD-950B] *** Continued on Page 19 ***		H.2	ENSURE the following Valves are closed.
			٠	RPV Head Vents.
	Timer		•	MSIV's.
	1 mic		•	Main Steam Line Drain valves.
			٠	HPCI Steam Isolations.
			٠	RCIC Steam Isolations.
			H.3	PLACE the control switches for two SRV's OPEN (SRVs will not open until a 50# d/p i established).
			H.4	SLOWLY RAISE RPV water level to establish a flow path through the OPEN SRV's.
-			H.5	SECURE all injection into the RPV except from CRD.
			H.6	START one Core Spray Subsystem or one LPCI Pump with suction from the Suppress Pool.
			H.7	RAISE Core Spray <u>OR</u> LPCI injection into the RPV to maximum.
			H.8	IF RPV pressure does not stabilize at least 5 psig above suppression chamber pressure, <u>OR</u> two SRVs are NOT open, <u>THEN START</u> and additional Core Spray Subsystem or LPCI pump <u>AND</u> continue to RAISE injection flow to establish stated conditions.
	•		H.9	IF RPV pressure stabilizes at more than 160 psig above Suppression Chamber
				pressure, THEN OPEN another SRV.
				*** Continued on page 19 ***

Hope Creek

Page 17 of 45
ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122SEG Internal Use Only

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

CAUTIONS:

2. Maintain at least 50 psig above Suppression Chamber pressure with at least two SRV's open.

.

Hope Creek

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 **PSEG Internal Use Only**

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

SUBSEQUENT OPERATOR ACTIONS

CONDITION	ACTION	
*** Continued from Page 17 ***		*** Continued from Page 17 ***
H RPV Temperature and Pressure		
CANNOT be maintained using Normal OR Alternate Decay Heat Removal.		H.10 <u>IF</u> the cooldown rate exceeds 90°F/hr, <u>THEN</u> REDUCE Core Spray or LPCI injection into the RPV until <u>EITHER</u> of the following occurs:
		• The cooldown rate drops below 90°F/hr.
		• RPV pressure decreases to within 50 psig of Suppression Chamber pressure.
· · ·		 H.11 CONTROL Suppression Pool temperature to maintain RPV water temperature above 79°F. [T/S 3.4.6.1.d]

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

This Page Intentionally Left Blank

4

Hope Creek

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 **PSEG Internal Use Only**

HC.OP-AB:RPV-0009(Q) SHUTDOWN COOLING

ATTACHMENT 1 ALTERNATE DECAY HEAT REMOVAL USING C TO A CROSS-TIE (Page 1 of 6)

<u>NOTE</u> 1.0

Operation of the C to A Cross-Tie is restricted to conditions when reactor coolant temperature is < 300°F (67 psia). [PR 970602179, PR 960927120]

1.0 Establishing Alternate Decay Heat Removal Using the C to A Cross-tie [CD-609G]

- 1.1 **ENSURE** the AP System is available providing keepfill for all A Loop ECCS Systems required for operability. (1-AP-V041, CS Xfr to A Core Spray Dsch (Rm. 4331)
- 1.2 ENSURE that the Jockey Pump CP228 is secured <u>AND</u> tagged IAW NC.NA-AP.ZZ-0015(Q); Safety Tagging Program. (Breaker 52-232074)
- 1.3 CLOSE 1BC-V203 ECCS Jock Pmp C Suct Vlv (Rm. 4114C) AND TAG IAW NC.NA-AP.ZZ-0015(Q).
- 1.4 **CLOSE** the following valves:
 - HV-F004A RHR PMP A SUPP POOL SUCT MOV
 - HV-F004C RHR PMP C SUPP POOL SUCT MOV

1.5 **TAG** the following IAW NC.NA-AP.ZZ-0015(Q):

- HV-F004A RHR PMP A SUPP POOL SUCT MOV (52-212031)
- HV-F004C RHR PMP C SUPP POOL SUCT MOV (52-232031)

CAUTION 1.6

Manual or automatic opening of HV-F007 A(C) RHR PMP A(C) MIN FL MOV will drain the Reactor Vessel to the Suppression Pool.

- 1.6 **ENSURE** the following valves are closed:
 - HV-F007A RHR PMP A MIN FL VLV
 - HV-F007C RHR PMP C MIN FL VLV.

Hope Creek

Page 21 of 45

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 200301225EG Internal Use Only

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

This Page Intentionally Left Blank

e,

Hope Creek

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 **PSEG Internal Use Only**

ATTACHMENT 1 ALTERNATE DECAY HEAT REMOVAL USING C TO A CROSS-TIE (Page 2 of 6)

1.7 **VERIFY** the following LPCI Injection Valves are closed, <u>THEN DE-ENERGIZE AND</u> TAG their power sources to ensure these valves do not open if a LPCI signal is received during cross-tie operating mode:

- HV-F017A RHR LOOP A LPCI INJ MOV (52-212052)
- HV-F017C RHR LOOP C LPCI INJ MOV (52-232052)

1.8 **CLOSE** AND TAG the following valves:

- HV-F010A RHR LOOP C TEST RET MOV (52-232044)
- HV-F024A RHR LOOP A TEST RET MOV (52-212192)
- HV-F021A RHR LOOP A SPRAY ISLN MOV (52-451062)
- HV-F027A RHR LOOP A SUPP CHAMBER SPRAY HDR ISLN MOV (52-212083)
- 1.9 Fully OPEN 1BC-V133 RHR Pmp C Suct Frm Recir Loop B (Rm 4227E) AND TAG in the open position IAW NC.NA-AP.ZZ-0015(Q).
- 1.10 ENSURE F077 RECIRC LOOP B TO RHR SUP MAN VLV is open.
- 1.11 IF the Shutdown Cooling suction line was isolated, <u>THEN</u> **PERFORM** a fill and vent IAW HC.OP-SO.BC-0002(Q), Decay Heat Removal Operation.
- 1.12 ENSURE the following valves are open:
 - HV-F008 SHUTDOWN COOLING OUTBD ISLN MOV
 - HV-F009 SHUTDOWN COOLING INBD ISLN MOV.
- 1.13 UNLOCK AND OPEN 1BC-V571 RHR Cross-Tie Isolation Valve for C Loop BC-HV-11673. (Rm 4114A)
- 1.14 **OPEN** the following valves until a solid stream of water issues, THEN **CLOSE** (Local):
 - 1BC-V578 AND 1BC-V579 RHR Vent Valves (Rm. 4113A)
 - 1BC-V580 AND 1BC-V581 RHR Vent Valves (Rm. 4114A)

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 DECIMIENTAL USE UNIV

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

This Page Intentionally Left Blank

1

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 PSEG Internal Use Only

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

ATTACHMENT 1 ALTERNATE DECAY HEAT REMOVAL USING C TO A CROSS-TIE (Page 3 of 6)

1.15 OPEN 1BC-V570 RHR Cross-Tie Isolation Valve for A Loop (Rm. 4113A)

<u>NOTE</u> 1.16

- A. The interlock override will allow C RHR Pump to operate when the pump is aligned to the alternate suction from the RPV, when and if required.
- B. In the cross-tie mode, C RHR Pump will not be automatically protected against the loss of suction from the RPV.
 - 1.16 **OBTAIN** key for the 1-BC-HS-11496 Keylock Switch from Work Control key cabinet.
 - 1.17 At Panel 10C641 perform the following:
 - A. **INSERT** the key in the 1-BC-HS-11496 Keylock Switch.
 - B. **OVERRIDE** the HV-F004C Valve/Pump C Interlock using the 1-BC-HS-11496 Keylock Switch.
 - C. LOG in CRS Log the position of the Keylock Switch.
 - 1.18 <u>IF</u> during the cross-tie operation mode, the HV-F008 or HV-F009 close (e.g., on RPV Low Level 3 signal), then immediately STOP the C RHR Pump <u>AND</u> TAKE corrective action.
 - 1.19 IF HV-F015A RHR LOOP A RET TO RECIRC LOOP A ISLN MOV does NOT open immediately to establish flow, <u>THEN SECURE</u> the RHR Pump.

Hope Creek

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

This Page Intentionally Left Blank

÷¢

Hope Creek

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 **PSEG Internal Use Only**

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

ATTACHMENT 1 ALTERNATE DECAY HEAT REMOVAL USING C TO A CROSS-TIE (Page 4 of 6)

CAUTION 1.20

Manual or automatic opening of HV-F007A(C) RHR PMP A(C) MIN FL MOV will drain the Reactor Vessel to the Suppression Pool.

1.20 START RHR PUMP CP202 and immediately THROTTLE OPEN HV-F015A RHR LOOP A RET TO RECIRC LOOP A ISLN MOV until FI-R603C LOOP C FLOW indicates 3000 gpm.

- **OBSERVE** AI-6358C PUMP C MOT AMPS.
- MAINTAIN flow of 3000 gpm for at least 10 minutes.

1.21 OPEN HV-F015A RHR LOOP A RET TO RECIRC LOOP A ISLN MOV until FI-R603C LOOP C FLOW indicates 10,000 gpm. ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122- INTERNAL USE UNITY

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

This Page Intentionally Left Blank

-2

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 PSEG Internal Use Only

ATTACHMENT 1 ALTERNATE DECAY HEAT REMOVAL USING C TO A CROSS-TIE (Page 5 of 6)

2.0 Securing Alternate Decay Heat Removal When C to A Cross-tie was used.

- 2.1 CLOSE HV-F015A RHR LOOP A RET TO RECIRC LOOP A ISLN MOV.
- 2.2 <u>WHEN</u> the HV-F015A RHR LOOP A RET TO RECIRC LOOP A ISLN MOV is closed, THEN **STOP** the C RHR Pump CP202.
- 2.3 **REMOVE** 1-BC-HS-11496 Keylock Switch from the OVERRIDE position.
 - A. **REMOVE** the key from 1-BC-HS-11496 Keylock Switch AND RETURN the key to the Work Control key cabinet.
 - B. LOG in CRS Log the position of the keylock switch.
- 2.4 CLOSE 1BC-V570 RHR Cross-Tie Isolation Valve for A Loop (Rm. 4113A).
- 2.5 CLOSE AND LOCK 1BC-V571 RHR Cross-Tie Isolation Valve for C Loop BC-HV-11673. (Rm 4114A)
- 2.6 **RELEASE** tags from 1BC-V203 ECCS Jock Pmp C Suct Vlv (Rm. 4114C) AND **OPEN** valve.
- 2.7 RELEASE tags from Jockey Pump CP228.
- 2.8 RELEASE tags from 1BC-V133 RHR Pmp C Suc Frm Recir Loop B (Rm 4227E)
- 2.9 CLOSE AND LOCK 1BC-V133 RHR Pmp C Suc Frm Recir Loop B (Rm 4227E)
- 2.10 **CLOSE** the following valves:
 - HV-F008 SHUTDOWN COOLING OUTBD ISLN MOV
 - HV-F009 SHUTDOWN COOLING INBD ISLN MOV

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 SEC INTERNAL USE UNIV

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

This Page Intentionally Left Blank

4

Hope Creek

Page 30 of 45

ALL ACVIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

ATTACHMENT 1 ALTERNATE DECAY HEAT REMOVAL USING C TO A CROSS-TIE (Page 6 of 6)

2.11 **RELEASE** the tags from the following valves:

- HV-F017A RHR LOOP A LPCI INJ MOV
- HV-F017C RHR LOOP C LPCI INJ MOV
- HV-F010A RHR LOOP C TEST RET MOV
- HV-F024A RHR LOOP A TEST RET MOV
- HV-F021A RHR LOOP A SPRAY ISLN MOV
- HV-F027A RHR LOOP A SUPP CHAMBER SPRAY HDR ISLN MOV
- 2.12 RELEASE tags from HV-F004A AND HV-F004C RHR PMP SUPP POOL SUCT MOVs.
- 2.13 ALIGN system as plant conditions require.

Hope Creek

Page 31 of 45

This Page Intentionally Left Blank

e,

Hope Creek

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 **PSEG Internal Use Only**

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

ATTACHMENT 2 ALTERNATE DECAY HEAT REMOVAL USING D TO B CROSS-TIE (Page 1 of 7)

<u>NOTE</u> 1.0

Operation of the D to B Cross-Tie is restricted to conditions when reactor coolant temperature is < 300°F (67 psia). [PR 970602179, PR 960927120]

1.0 <u>Establishing Alternate Decay Heat Removal Using the D to B Cross-tie</u> [CD-609G]

- 1.1 ENSURE the AP System is available providing keepfill for all B Loop ECCS Systems required for operability. (1-AP-V062 CS Xfr to B Core Spray Dsch Isln [Rm, 4322B])
- 1.2 ENSURE that the Jockey Pump DP228 is secured <u>AND</u> tagged IAW NC.NA-AP.ZZ-0015(Q); Safety Tagging Program. (Breaker 52-242074)
- 1.3 CLOSE 1BC-V261 ECCS Jockey Pump D Suction Valve AND TAG IAW NC.NA-AP.ZZ-0015(Q).
- 1.4 **CLOSE** the following valves:
 - HV-F004B RHR PMP B SUPP POOL SUCT MOV
 - HV-F004D RHR PMP D SUPP POOL SUCT MOV

1.5 **TAG** the following IAW NC.NA-AP.ZZ-0015(Q):

- HV-F004B RHR PMP B SUPP POOL SUCT MOV Breaker 52-222031
- HV-F004D RHR PMP D SUPP POOL SUCT MOV Breaker 52-242031

Page 33 of 45

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

.

Hope Creek

Page 34 of 45

e,

This Page Intentionally Left Blank

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 **PSEG Internal Use Only**

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

ATTACHMENT 2 ALTERNATE DECAY HEAT REMOVAL USING D TO B CROSS-TIE (Page 2 of 7)

CAUTION 1.6

Manual or automatic opening of HV-F007B(D) RHR PMP B(D) MIN FL MOV will drain the Reactor Vessel to the Suppression Pool.

1.6 ENSURE the following valves are closed:

- HV-F007B RHR PUMP B MIN FLOW VLV
- HV-F007D RHR PUMP D MIN FLOW VLV.

1.7 **VERIFY** the following LPCI Injection Valves are closed, <u>THEN DE-ENERGIZE AND</u> TAG their power sources to ensure these valves do not open if a LPCI signal is received during cross-tie operating mode:

- HV-F017B RHR LOOP B LPCI INJ MOV (52-222052)
- HV-F017D RHR LOOP D LPCI INJ MOV (52-242052)
- 1.8 CLOSE <u>AND</u> TAG the following valves:
 - HV-F010B RHR LOOP D TEST RET MOV (52-242044)
 - HV-F024B RHR LOOP B TEST RET MOV (52-222063)
 - HV-F021B RHR LOOP B SPRAY ISLN MOV (52-222062)
 - HV-F027B RHR LOOP B SUPP CHAMBER SPRAY HDR ISLN MOV (52-222083)
- 1.9 <u>FULLY</u> OPEN 1BC-V043 RHR Pmp D Suc Frm Recir Loop B (Rm. 4227D; Az 150 above catwalk) AND TAG in the open position IAW NC.NA-AP.ZZ-0015(Q).
- 1.10 ENSURE F077 RECIRC LOOP B TO RHR SUP MAN VLV is open.

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

This Page Intentionally Left Blank

Hope Creek

ALL ACFIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 PSEG Internal Use Only

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

ATTACHMENT 2 ALTERNATE DECAY HEAT REMOVAL USING D TO B CROSS-TIE (Page 3 of 7)

- 1.11 IF the Shutdown Cooling suction line was isolated, <u>THEN</u> **PERFORM** a fill and vent IAW HC.OP-SO.BC-0002(Q), Decay Heat Removal Operation.
- 1.12 ENSURE the following valves are open:
 - HV-F008 SHUTDOWN COOLING OUTBD ISLN MOV
 - HV-F009 SHUTDOWN COOLING INBD ISLN MOV.
- 1.13 UNLOCK AND OPEN 1BC-V601 RHR Crosstie Iso Vlv B LOOP BC-HV-11680 MOV (Rm. 4107A)
- 1.14 **OPEN** the following valves until a solid stream of water issues, <u>THEN</u> **CLOSE** (Local).
 - 1BC-V610 and 1BC-V611 RHR B&D Loop X-Tie Vent Vlvs (Rm. 4107A)
 - 1BC-V608 and 1BC-V609 RHR B&D Loop X-Tie Vent Vlvs (Rm. 4107A)
- 1.15 CRACK OPEN 1BC-V600 RHR X-Tie Iso Vlv Loops B&D. (Rm. 4107A)
- 1.16 OPEN 1BC-V608 and 1BC-V609 RHR B&D Loop X-Tie Vent Vlvs until a solid stream of water issues, THEN CLOSE (Rm. 4107A)
- 1.17 OPEN 1BC-V600 RHR X-Tie Iso Vlv Loops B&D. (Rm. 4107A)

Hope Creek

Page 37 of 45

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 DE INTERNAL USE UNIV

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

This Page Intentionally Left Blank

4

ATTACHMENT 2 ALTERNATE DECAY HEAT REMOVAL USING D TO B CROSS-TIE (Page 4 of 7)

<u>NOTE</u> 1.18

- A. The interlock override will allow D RHR Pump to operate when the pump is aligned to the alternate suction from the RPV, when and if required.
- B. In the cross-tie mode, D RHR Pump will not be automatically protected against the loss of suction from the RPV.
 - 1.18 **OBTAIN** key to the 1-BC-HS-11682 Keylock Switch from Work Control key cabinet.
 - 1.19 **PERFORM** the following at Panel 10C640:
 - A. INSERT the key in the 1-BC-HS-11682 Keylock Switch.
 - B. **OVERRIDE** the HV-F004D Valve/Pump D Interlock using the 1-BC-HS-11682 Keylock Switch.
 - C. LOG in CRS Log the position of the Keylock Switch.
 - 1.20 <u>IF</u> during the cross-tie operation mode, the HV-F008 or HV-F009 close (e.g., on RPV Low Level 3 signal), then immediately **STOP** the D RHR Pump AND **TAKE** corrective action.
 - 1.21 IF HV-F015B RHR LOOP B RET TO RECIRC LOOP B ISLN MOV does not open immediately to establish flow, THEN SECURE the RHR Pump.

Hope Creek

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

This Page Intentionally Left Blank

-

• • • • • •

.

Hope Creek

Page 40 of 45

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 2003@122 PSEG Internal Use Only

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

ATTACHMENT 2 ALTERNATE DECAY HEAT REMOVAL USING D TO B CROSS-TIE (Page 5 of 7)

(Page 5 of 7)

CAUTION 1.22

Manual or automatic opening of HV-F007B(D) RHR PMP B(D) MIN FL MOV will drain the Reactor Vessel to the Suppression Pool.

1.22 START RHR PUMP DP202 and immediately THROTTLE OPEN HV-F015B RHR LOOP B RET TO RECIRC LOOP B ISLN MOV <u>UNTIL</u> FI-R603D LOOP D FLOW indicates 3000 GPM.

- OBSERVE AI-6358D PUMP D MOT AMPS.
- MAINTAIN flow of 3000 gpm for at least 10 minutes.
- 1.23 OPEN HV-F015B RHR LOOP B RET TO RECIRC LOOP B ISLN MOV UNTIL FI-R603D LOOP D FLOW indicates 10,000 gpm.

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

This Page Intentionally Left Blank

.

ATTACHMENT 2 ALTERNATE DECAY HEAT REMOVAL USING D TO B CROSS-TIE (Page 6 of 7)

- 2.0 Securing Alternate Decay Heat Removal When D to B Cross-tie was used.
 - 2.1 CLOSE HV-F015B RHR LOOP B RET TO RECIRC LOOP B ISLN MOV.
 - 2.2 <u>WHEN</u> the HV-F015B RHR LOOP B RET TO RECIRC LOOP B ISLN MOV is closed, THEN STOP the D RHR Pump DP202.
 - 2.3 **REMOVE** 1-BC-HS-11682 Keylock Switch from the OVERRIDE position.

- B. LOG in NSS Log the position of the Keylock Switch.
- 2.4 CLOSE 1BC-V600 RHR X-Tie Iso Vlv Loops B&D. (Rm. 4107A)
- 2.5 CLOSE AND LOCK 1BC-V601 RHR Crosstie Iso Vlv B LOOP BC-HV-11680 MOV (Rm. 4107A)
- 2.6 **RELEASE** tags from 1BC-V261 ECCS Jockey Pmp D Suct Vlv (Rm. 4107D) AND OPEN valve.
- 2.7 **RELEASE** tags from Jockey Pump DP228.
- 2.8 **RELEASE** tags from 1BC-V043 RHR Pmp D Suc Frm Recir Loop B. (Rm. 4227D; Az 150 above catwalk)
- 2.9 CLOSE AND LOCK 1BC-V043 RHR Pmp D Suc Frm Recir Loop B. (Rm. 4227D; Az 150 above catwalk)
- 2.10 **CLOSE** the following valves:
 - HV-F008 SHUTDOWN COOLING OUTBD ISLN MOV
 - HV-F009 SHUTDOWN COOLING INBD ISLN MOV

Hope Creek

Page 43 of 45

A. **REMOVE** the key from 1-BC-HS-11682 Keylock Switch AND RETURN to the Work Control key cabinet.

HC.OP-AB.RPV-0009(Q) SHUTDOWN COOLING

÷

This Page Intentionally Left Blank

Hope Creek

Page 44 of 45

ALL ACTIVE ON THE SPOT CHANGES MUST BE ATTACHED FOR FIELD USE

ATTACHMENT 2 ALTERNATE DECAY HEAT REMOVAL USING D TO B CROSS-TIE (Page 7 of 7)

- 2.11 **RELEASE** the tags from the following valves:
 - HV-F017B RHR LOOP B LPCI INJ MOV
 - HV-F017D RHR LOOP D LPCI INJ MOV
 - HV-F010B RHR LOOP D TEST RET MOV
 - HV-F024B RHR LOOP B TEST RET MOV
 - HV-F021B RHR LOOP B SPRAY ISLN MOV
 - HV-F027B RHR LOOP B SUPP CHAMBER SPRAY HDR ISLN
 MOV
- 2.12 RELEASE tags from HV-F004B AND HV-F004D RHR PMP SUPP POOL SUCT MOV.

2.13 ALIGN system as plant conditions require.

Hope Creek

Page 45 of 45

REVISION SUMMARY

<u>Rev.</u> 0

This procedure has been re-formatted into a two-column format as part of the Hope Creek Abnormal Operating Procedure upgrade project. This procedure supercedes HC.OP-AB.ZZ-0142(Q). The sequence of steps have been re-arranged to provide better flow of the steps. The information previously contained in the discussion section has been moved to the Bases Document for this procedure. Revision bars have been omitted from this revision due to the number of changes that have been made.

The following list of Editorial changes has been made to this procedure:

- 1. Added new Alarms and Indications which are indicative of an RHR pump trip, shutdown cooling suction line isolation, and shutdown cooling flow bypass event.
- 2. Added Automatic Actions for Shutdown Cooling valves.
- 3. Incorporated former Attachment 1 into the body of the procedure.
- 4. Deleted former Note 4.8. The intent of this note is satisfied by the Condition statement for this section.
- 5. Reworded sub-steps of former step 4.8 and added a note for clarity.(80023233)
- 6. Changed "Alternate Shutdown Cooling" in Attachments 1 and 2 to "Alternate Decay Heat Removal" IAW SOR Reviewer comments.

The following list of Non-Editorial changes has been made to this procedure:

- 1. Changed former Note 4.0 to indicate the respective Integrating Operating procedure may identify which mode of Alternate Decay Heat Removal would be effective as determined by Engineering. Reference to HC.OP-SO.BC-0002(Q) was deleted. The abnormal action steps direct the operators to this procedure at the appropriate times.
- 2. Deleted former step 4.1. The CRS will determine when conditions are appropriate for exiting the abnormal.
- 3. Former step 4.4 was expanded to specific corrective actions throughout the abnormal.
- 4. Former step 4.5 was modified to include a one hour time limit. This is consistent with T/S 3.9.11.1 action b.
- 5. Added guidance to bypass the regenerative heat exchanger IAW SOP if necessary to provide additional decay heat removal.
- 6. Expanded guidance on using Condensate Transfer for feed and bleed cooling. Existing guidance was inadequate and required entry into High Radiation areas to manually throttle MOVs.
- 7. Deleted former step 4.7.c to use head spray. For Hope Creek, head spray operation depends on having an RHR pump in Shutdown Cooling. If I have an RHR pump operating in Shutdown Cooling, I would not be in this abnormal. Additionally, head spray injects into the steam dome. Hope Creek Core Spray and RHR inject inside the core shroud, which is more effective at providing core circulation and cooling.
- 8. Former step 4.8.12 was deleted.
- 9. Based on 50.59 reviewer input, the maximum RPV to torus d/p when performing Alternate Shutdown Cooling IAW Condition 'H' was raised from 140 to 160 psi.

Hope Creek

Page 1 of 1

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

STATION:	HOPE CREEK				
SYSTEM:	234000 Fuel Handling Equip	ment			
TASK:	Bent Mast IAW HC.OP-SO.K (Alternate Path)	E-0001(Q) Atta	achment 2		
TASK NUMBER:	234000 A2.03				
JPM NUMBER:	2003-NRC-S3				
ALTERNATE PAT	H: X	K/	A NUMBER:	234000	A2.03
APPLICABILITY: EO		IMPORTANC		2.8 RO	3.1 SRO
EVALUATION SE	TTING/METHOD: REFUELIN	IG PLATFORI	/I – PERFORM /	SIMULATE	
REFERENCES:	HC.OP-SO.KE-0001 Rev. 28	; HC.RE-FR.Z	Z-0001 Attachm	ent 1	
TOOLS AND EQU	JIPMENT: Refueling Platforn	n, Dummy Bun	dle, Move Shee	t	
VALIDATED JPM		10 min			
TIME PERIOD ID	ENTIFIED FOR TIME CRITICA	AL STEPS:	<u>N/A</u>		
APPROVED:					
N/A BARGAININ REPRESEN	G UNIT TRAINING S	SUPERVISOR	OF	PERATIONS or Desig	MANAGER
CAUTION:	 No plant equipment shall be ope Permission from the OS or b Direct oversight by a qualified based on plant conditions). Verification of the "as left" control 	erated during the Unit CRS; ed individual (de ondition by a qu	e performance of a termined by the in alified individual.	i JPM without dividual granti	the following: ng permission
		min.			
		 N/A	min.		
				SAT	UNSAT
REASON, IF UNS					
EVALUATOR'S S	IGNATURE:		_ DATE:		

Ś

5

÷

NC.TQ-WB.ZZ-0310(Z)

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME:	
DATE:	

SYSTEM: 234000 Fuel Handling Equipment

TASK: Bent Mast IAW HC.OP-SO.KE-0001(Q) Attachment 2 (Alternate Path)

TASK 234000 A2.03

INITIAL CONDITIONS:

- You are the Refueling Bridge operator.
- The Reactor is in Operational Condition 4 preparing for refueling.
- The Refuel Platform is in a standby lineup, powered up, and warmed up > ½ hour.
- A Spotter and Refueling SRO are standing by.
- The Dummy bundle is being used to simulate an irradiated fuel bundle.
- The Dummy bundle is full up on the Main Hoist being moved IAW Move Sheet Step #2.
- You are to continue HC.OP-SO.KE-0001 at step 5.8.14.

INITIATING CUE:

Place the bundle in its designated storage location IAW the move sheet step #2.

Successful Completion Criteria:

- 1. All critical steps completed.
- 2. All sequential steps completed in order.
- 3. All time-critical steps completed within allotted time.
- 4. JPM completed within validated time. Completion time may exceed the validated time if satisfactory progress is being made (and NRC concurrence is obtained).

NAME: ______ DATE: _____

JOB PERFORMANCE MEASURE

SYSTEM: 234000 Fuel Handling Equipment

TASK: Bent Mast IAW HC.OP-SO.KE-0001(Q) Attachment 2 (Alternate Path)

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
		Operator obtains procedure HC.OP-SO.KE- 0001.	Operator obtains the correct procedure.		
		Operator reviews prerequisites, precautions and limitations.	Operator reviews prerequisites, precautions and limitations. Examiner Cue: If excessive time is taken reviewing prerequisites, precautions and limitations, inform operator that all are satisfied.		
		Operator determines beginning step of the procedure.	Operator determines correct beginning step to be 5.8.14 of HC.OP-SO.KE-0001(Q)		

NC.TO .ZZ-0310(Z)

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: ______ DATE: _____

SYSTEM: 234000 Fuel Handling Equipment

 TASK:
 Bent Mast IAW HC.OP-SO.KE-0001(Q) Attachment 2 (Alternate Path)

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
	5.8	<u>Fuel/Blade Guide Movement – Within Spent</u> <u>Fuel Storage Pool (other than Refuel Mode</u>)	 Examiner Note: All operations for this JPM are performed on the Refueling Platform. Initialing steps is not critical A generic Fuel Movement sheet is provided where MOVE Step #1 is the DUMMY BUNDLE from the normal storage location to an empty spare Fuel Pool location. Move Step #2 is from the spare location back to the normal storage location. Examiners Cues proceeded by a \$ are given ONLY if the evolution is SIMULATED. 		

NC.TC .ZZ-0310(Z)

NAME: _____ DATE: _____

JOB PERFORMANCE MEASURE

SYSTEM: 234000 Fuel Handling Equipment

TASK: Bent Mast IAW HC.OP-SO.KE-0001(Q) Attachment 2 (Alternate Path)

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
*	5.8.14	START TIME: USE the Refuel Platform <u>AND</u> Trolley Position Controls <u>AND</u> Position Indicating System to maneuver the Fuel Grapple to the target location listed on the Fuel Movement Sheet(s).	Operator uses the PLATFORM AND TROLLEY control operators to maneuver the Fuel Grapple using position indication cameras so that it is positioned over Fuel Pool location AD-28 in accordance to the fuel movement sheet. \$ Examiner Cue: "The controls respond to the directions stated." \$ Examiner Cue: "The grapple is positioned over the location stated."		
	5.8.15	ENSURE the Fuel Grapple is at the correct Core <u>OR</u> Fuel Pool Coordinates. (REFER TO Note 5.8)	Operator ensures the grapple is positioned over Fue! Pool location AD-28 in accordance to the fuel movement sheet. \$ Examiner Cue: "The grapple is positioned over the location stated."		
*	5.8.16	ROTATE the Fuel Grapple <u>AND</u> attached fuel assembly/blade guide to attain direct alignment <u>AND</u> orientation with the target location. [CD-396Y]	Operator rotates the Fuel Grapple and attached fuel assembly to attain direct alignment and orientation with the target location. \$ Examiner Cue: "The grapple and fuel assembly are oriented and aligned over the location stated."		

NAME: ______ DATE: _____

JOB PERFORMANCE MEASURE

SYSTEM: 234000 Fuel Handling Equipment

TASK: Bent Mast IAW HC.OP-SO.KE-0001(Q) Attachment 2 (Alternate Path)

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
*	5.8.17	LOWER the Fuel Grapple to approximately one foot above the fuel assembly/blade guide to be removed by using the Fuel Grapple Hoist control in the LOWER position <u>AND</u> OBSERVE the GRAPPLE NORMAL UP light off.	Operator lowers the Fuel Grapple using the Fuel Grapple Hoist control in the LOWER position. \$ Examiner Cue: "The grapple is lowering; the GRAPPLE NORMAL UP light is off." Examiner Cue: <i>Wait 10 seconds and cue</i> "Stop Hoist Travel" Examiner Cue: After the hoist is stopped "The grapple has stopped suddenly. The SLACK CABLE light is lit with the hoist at position 105. The load cell is reading 25 pounds."		

NC.TQ .ZZ-0310(Z)

NAME: ______ DATE: _____

JOB PERFORMANCE MEASURE

SYSTEM: 234000 Fuel Handling Equipment

TASK: Bent Mast IAW HC.OP-SO.KE-0001(Q) Attachment 2 (Alternate Path)

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
			Operator notifies Refueling SRO or CRS/OS. Operator recognizes mast is bent or binding by the valid SLACK CABLE light on with the grapple loaded and not in contact with any structure. Examiner Note: Allow the operator reasonable time to determine cause. If the operator does not recognize symptoms of a bent mast or diagnoses problem as a load cell malfunction, then Cue: "Carry out actions for a bent mast."		
	HC.OP- SO.KE- 0001 Attach- ment 2		Operator determines correct procedure is HC.OP-SO.KE-0001 Attachment 2 beginning step to be 2.0 Bent Mast.		
	2.0	Bent Mast			

NC.TQ ZZ-0310(Z)
OPERATOR TRAINING PROGRAM

NAME: _____

JOB PERFORMANCE MEASURE

SYSTEM: 234000 Fuel Handling Equipment

TASK: Bent Mast IAW HC.OP-SO.KE-0001(Q) Attachment 2 (Alternate Path)

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
	Note 2.0	DO NOT attempt to lower or release the bundle without SRO approval if a bent Mast is encountered.	Operator reads Note 2.0		
			Operator reads step 2.0		
*		IF while lowering the mast, an unexpected slack cable <u>or</u> pre-mature weight transfer occurs, stop lowering, raise the load to clear the slack cable (if one exists), and notify the cognizant Refuel SRO or OS/CRS before attempting any corrective action.	Operator raises main fuel hoist to clear SLACK CABLE light using the Fuel Grapple Hoist control in the RAISE position Examiner Cue: SLACK CABLE light is off.		
			Operator notifies Refueling SRO and/or CRS/OS to initiate a Notification. Examiner Cue: Acknowledge the request for corrective action.		
		STOP TIME:	Termination Cue: This JPM is complete. A different refueling crew will troubleshoot the Mast problem.		

NC.TQ .ZZ-0310(Z)

JOB PERFORMANCE MEASURE

INITIAL CONDITIONS:

- You are the Refueling Bridge operator.
- The Reactor is in Operational Condition 4 preparing for refueling.
- The Refuel Platform is in a standby lineup, powered up, and warmed up > 1/2 hour.
- A Spotter and Refueling SRO are standing by.
- The Dummy bundle is being used to simulate an irradiated fuel bundle.
- The Dummy bundle is full up on the Main Hoist being moved IAW Move Sheet Step #2.
- You are to continue HC.OP-SO.KE-0001 at step 5.8.14.

INITIATING CUE:

Place the bundle in its designated storage location IAW the move sheet step #2.

ALL*ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122

PSEG Internal Use Only

Page 1 of 1

PSEG NUCLEAR L.L.C.

HOPE CREEK GENERATING STATION

HC.OP-SO.KE-0001(Q) - Rev. 28

REFUELING PLATFORM AND FUEL GRAPPLE OPERATION

USE CATEGORY: II

А.		Biennial Review perfor	med Yes	No N/A	✓
B. Change Package(s) an	d Affected Document	Number(s) incorporated int	o this revision.		
• CP No	CP Rev. No.	AD No	AD Rev. No.	or None	√
C. OTSC(s) incorporated	into this revision:				
OTSC No(s)				or None	<u> </u>

REVISION SUMMARY

- This following change is part of the Abnormal Upgrade Project. The following change involves 1. renaming of the applicable abnormal and is editorial in nature:
 - Changed HC.OP-AB.ZZ-0101(Q) to HC.OP-AB.CONT-0005(Q) on pages 20 and 78.

IMPLEMENTATION REQUIREMENTS

Effective date _ 4/8/02

Implementation of HC.OP-AB.CONT-0005(Q)

APPROVED:

4/3/02 Date

Manager - Hope Creek Operations

-

HC.OP-SO.KE-0001(Q)

REFUELING PLATFORM AND FUEL GRAPPLE OPERATION

TABLE OF CONTENTS

Section	<u>Title</u>		Page
1.0	PURPOS	Ε	3
2.0	PREREQ	UISITES	3
3.0	PRECAU	TIONS & LIMITATIONS	22
4.0	EQUIPM	ENT REQUIRED	28
5.0	PROCED	URE	29
	5.1	Refuel Platform Start-up and Shutdown	29
	5.2	Fuel Transfer - Reactor Core to Spent Fuel Storage Pool	31
	5.3	Fuel Transfer - Spent Fuel Storage Pool to Reactor Core	37
	5.4	Blade Guide Transfer - Spent Fuel Storage Pool to Reactor Core	43
	5.5	Blade Guide Transfer - Reactor Core to Spent Fuel Storage Pool	46
	5.6	Fuel Movement - Within Reactor Vessel	50
	5.7	Blade Guide Movement - Within Reactor Vessel	. 54
	5.8	Fuel/Blade Guide Movement - Within Spent Fuel Storage Pool (other than Refuel Mode)	. 59
	5.9	Auxiliary Hoist Tool Installation	. 64
6.0	REFERE	NCES	. 77

4

REFUELING PLATFORM AND FUEL GRAPPLE OPERATION

TABLE OF CONTENTS

<u>Section</u>	<u>Title</u>		<u>Page</u>
АТТАСНМ	ENTS		
Attachment	1	ANALYZED LOADS Permitted Over IRRADIATED FUEL Not Requiring SECONDARY CONTAINMENT INTEGRITY	. 79
Attachment 2	2	Instructions for Manually Lowering a Fuel Bundle	80
Attachment	3	Refueling Platform Interlocks	. 84

HC.OP-SO.KE-0001(Q)

REFUELING PLATFORM AND FUEL GRAPPLE OPERATION

1.0 **PURPOSE**

- 1.1 This procedure provides detailed instructions to accomplish the following Refueling tasks:
 - 1.1.1 Transfer fuel to and from the Spent Fuel Storage Pool and Reactor Core. [CD-736D].
 - 1.1.2 Transfer blade guides to and from the Spent Fuel Storage Pool and Reactor Core.
 - 1.1.3 Transfer fuel within Reactor Vessel.
 - 1.1.4 Transfer blade guides within Reactor Vessel.
 - 1.1.5 Transfer Fuel and Blade Guides within the Spent Fuel Storage Pool (other than Refuel Mode). [CD-736D]
 - 1.1.6 Install necessary tools on Refueling Platform Auxiliary Hoists.
- 1.2 In addition this procedure provides the prerequisites, precautions and limitations required to maintain fuel integrity during Refueling Operations.

2.0 **PREREQUISITES**

2.1 Refuel Platform Start-up and Shutdown

- 2.1.1 Breaker 52-254061 is closed to provide power to Refuel Platform.
- 2.1.2 Applicable Precautions and Limitations have been reviewed by each procedure user.

2.2 Fuel Transfer - Reactor Core to Spent Fuel Storage Pool

- 2.2.1 The Reactor Building Secondary Containment Ventilation System RBVS & FRVS automatic isolation dampers shall be operable IAW T.S. 3.6.5.2.
- 2.2.2 Reactor Building integrity shall be maintained IAW T.S. 3.6.5.1.b.
- 2.2.3 The Fuel Grapple Hoist shall be verified operable IAW T.S. 3.9.6 by performing procedure HC.OP-ST.KE-0001(Q), Refuel Interlock Operability Functional Test - Section 5.4.

HC.OP-SO.KE-0001(Q)

2.2.4	At least 23 feet of water shall be maintained over the top of irradiated Fuel Assemblies seated in the Spent Fuel Storage Racks IAW T.S. 3.9.9	
2.2.5	The Filtration, Recirculation and Ventilation System shall be operable IAW T.S. 3.6.5.3.	

2.2.6 Procedure HC.OP-ST.KE-0001(Q); Refuel Interlock Operability Test must be performed prior to CORE ALTERATIONS, IAW T.S. 3.9.1. [CD-182C]

2.2.7 The Reactor Mode Switch shall be operable <u>AND</u> locked in the REFUEL <u>OR</u> the SHUTDOWN position IAW T.S. 3.9.1.

2.2.8 At least two Source Range Monitors (SRM) shall be operable <u>AND</u> inserted to the normal operating level IAW T.S. 3.9.2 <u>AND</u>:

- A. Annunciation and continuous visual indication of these monitors is available in the Control Room.
- B. One of the required SRM detectors is located in the Reactor Core quadrant where CORE ALTERATIONS are being performed
 <u>AND</u> the other SRM detector shall be located in an adjacent quadrant.
- 2.2.9 All control rods are fully inserted IAW T.S. 3.9.3.
- 2.2.10 The Reactor shall be subcritical for a minimum of 24 hours IAW T.S. 3.9.4.
- 2.2.11 Establish and maintain direct communication between the Control Room and Refueling Floor personnel IAW T.S. 3.9.5.
- 2.2.12 At least 22 feet 2 inches of water shall be maintained over the top of the Reactor Pressure Vessel flange IAW T.S. 3.9.8.
- 2.2.13 All CORE ALTERATIONS shall be observed <u>AND</u> directly supervised by either a Senior Reactor Operator <u>OR</u> a Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation IAW T.S. 6.2.2.C.
- 2.2.14 Breaker 52-254061 is closed to provide power to Refuel Platform.

٦

2.2.15	Prior to moving the Refueling Platform ensure the following conditions exist:
	A. Refuel Platform tracks are clear of equipment or materials which would impede platform motion.
	B. Cables, ropes or other devices are not tied to the Refuel Platform or Trolley.
	C. The Jib Cranes and Channel Handling Boom have been positioned to prevent collisions with the Refuel Platform.
	D. The Fuel Grapple, Monorail Hoist and Auxiliary Hoist are not carrying any loads.
	 E. Underwater obstructions which may interfere with fuel transfer are identified and repositioned <u>IF</u> possible. CD-396Y
	F. PERFORM HC.OP-FT.KE-0001(Q); Refuel Platform and Fuel Grapple Operability Test - Refueling:
	1. <u>IF</u> maintenance has been completed on the Refueling Platform and/or Hoists.
	2. Monthly during fuel handling operations.
	G. PERFORM HC.OP-FT.KE-0002(Q); Fuel Grapple Full Down Travel Functional Test – Refueling <u>IF</u> not performed within 30 days of starting fuel handling operations.
2.2.16	No testing, maintenance or other activity is in progress on the Refuel Platform, Spent Fuel Storage Pool, New Fuel Vault, Reactor Cavity or Shipping Cask Area which would impede the movement of the Refuel Platform.
2.2.17	Permission to move irradiated nuclear fuel OR PERFORM CORE ALTERATIONS has been granted by the OS/CRS.
2.2.18	OBTAIN the Fuel Movement Sheet(s) developed by Reactor Engineering IAW HC.RE-FR.ZZ-0001(Q).
2.2.19	All personnel that will operate the Refuel Platform and Fuel Grapple have completed training and are qualified. [CD-124B]

- 2.2.20 <u>WHEN</u> transferring irradiated fuel, or any other irradiated Reactor internal, between Reactor Vessel and Spent Fuel Storage Pool the Shielded Fuel Transfer Chute shall be installed.
- 2.2.21 NOTIFY the Radiation Protection Department prior to starting any fuel transfer operation. The Radiation Protection Department shall ensure that personnel leave the upper level of the Drywell prior to fuel transfer. [CD-069A]
- 2.2.22 Power to Refuel Platform Control Console energized.
- 2.2.23 Position indication cameras for the Main Hoist are selected for display.
- 2.2.24 No equipment or tools, with the exception of; the Refueling Platform, the Polar Crane or any <u>ANALYZED LOADS</u> (<u>ANALYZED LOADS</u> are listed in Attachment 1), SHALL be moved over IRRADIATED FUEL <u>UNLESS</u> SECONDARY CONTAINMENT INTEGRITY is in effect IAW T.S. 1.39.
- 2.2.25 During fuel movement and control blade movement, the Service Pole Caddy is to be locked in its storage position on the East end of the rail using the two locking brakes to the handrail provided on the pole caddy.
- 2.2.26 Applicable Precautions and Limitations have been reviewed by each procedure user.
- 2.2.27 Underwater lights have been installed in the Reactor Vessel and Cavity and energized as necessary to support Refueling activities.

•

HC.OP-SO.KE-0001(Q)

2.3.1	The Reactor Building Secondary Containment Ventilation System RBVS & FRVS automatic isolation dampers shall be operable IAW T.S. 3.6.5.2.
2.3.2	Reactor Building integrity shall be maintained IAW T.S. 3.6.5.1.b.
2.3.3	The Fuel Grapple Hoist shall be verified operable IAW T.S. 3.9.6 by performing procedure HC.OP-ST.KE-0001(Q), Refuel Interlock Operability Functional Test - Section 5.4.
2.3.4	At least 23 feet of water shall be maintained over the top of irradiated Fuel Assemblies seated in the Spent Fuel Storage Racks IAW T.S. 3.9.9
2.3.5	The Filtration, Recirculation and Ventilation System shall be operable IAW T.S. 3.6.5.3.
2.3.6	Procedure HC.OP-ST.KE-0001(Q); Refuel Interlock Operability Test must be performed prior to CORE ALTERATIONS IAW T.S. 3.9.1. [CD-182C]
2.3.7	The Reactor Mode Switch shall be operable <u>AND</u> locked in the REFUEL <u>OR</u> the SHUTDOWN position IAW T.S. 3.9.1.
2.3.8	At least two Source Range Monitors (SRM) shall be operable <u>AND</u> inserted to the normal operating level IAW T.S. 3.9.2 <u>AND</u> :
	A. Annunciation <u>AND</u> continuous visual indication of these monitors is available in the Control Room.
	B. One of the required SRM detectors is located in the Reactor core quadrant where CORE ALTERATIONS are being performed <u>AND</u> the other SRM detector shall be located in an adjacent quadrant quadrant and the other SRM detector shall be located in an adjacent quadrant qua
2.3.9	All control rods are fully inserted IAW T.S. 3.9.3.
2.3.10	The Reactor shall be subcritical for a minimum of 24 hours IAW T.S. 3.9
2.3.11	Establish and maintain direct communication between the Control Room

•

:

2.3.12	At least 22 feet 2 inches of water shall be maintained over the top of the Reactor Pressure Vessel flange IAW T.S. 3.9.8.	_
2.3.13	All CORE ALTERATIONS shall be observed <u>AND</u> directly supervised by either a Senior Reactor Operator <u>OR</u> a Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation IAW T.S. 6.2.2.c.	
2.3.14	Breaker 52-254061 is closed to provide power to Refuel Platform.	_
2.3.15	Prior to moving the Refueling Platform ensure the following conditions exist:	_
	A. Refuel Platform tracks are clear of equipment or materials which would impede platform motion.	
	B. Cables, ropes or other devices are not tied to the Refuel Platform or Trolley.	
	C. The Jib Cranes and Channel Handling Boom have been positioned to prevent collisions with the Refuel Platform.	_
	D. The Fuel Grapple, Monorail Hoist and Auxiliary Hoist are not carrying any loads.	_
	 E. Underwater obstructions which may interfere with fuel transfer are identified and repositioned <u>IF</u> possible. [CD-396Y] 	
	F. PERFORM HC.OP-FT.KE-0001(Q); Refuel Platform and Fuel Grapple Operability Test - Refueling:	
	1. <u>IF maintenance has been completed on the Refueling</u> Platform and/or Hoists.	
	2. Monthly during fuel handling operations.	
	 G. PERFORM HC.OP-FT.KE-0002(Q); Fuel Grapple Full Down Travel Functional Test – Refueling <u>IF</u> not performed within 30 days of starting fuel handling operations. 	

HC.OP-SO.KE-0001(Q)

- 2.3.16 No testing, maintenance or other activity is in progress on the Refuel Platform, Spent Fuel Storage Pool, New Fuel Vault, Reactor Cavity or Shipping Cask Area which would impede the movement of the Refuel Platform.
- 2.3.17 Permission to move irradiated nuclear fuel OR perform CORE ALTERATIONS has been granted by the OS/CRS.
- 2.3.18 **OBTAIN** the Fuel Movement Sheet(s) developed by Reactor Engineering IAW HC.RE-FR.ZZ-0001(Q).
- 2.3.19 All personnel that will operate the Refuel Platform and Fuel Grapple have completed training and are qualified. [CD-124B]
- 2.3.20 <u>WHEN</u> transferring irradiated fuel, or any other irradiated Reactor internal, between Reactor Vessel and Spent Fuel Storage Pool the Shielded Fuel Transfer Chute shall be installed.
- 2.3.21 **NOTIFY** the Radiation Protection Department prior to starting any fuel transfer operation. The Radiation Protection Department shall ensure that personnel leave the upper level of the Drywell prior to fuel transfer. **[CD-069A]**
- 2.3.22 Power to Refuel Platform Control Console energized.
- 2.3.23 Position indication cameras for the Main Hoist are selected for display.
- 2.3.24 No equipment or tools, with the exception of; the Refueling Platform, the Polar Crane or any <u>ANALYZED LOADS</u> (<u>ANALYZED LOADS</u> are listed in Attachment 1), SHALL be moved over IRRADIATED FUEL <u>UNLESS</u> SECONDARY CONTAINMENT INTEGRITY is in effect IAW T.S. 1.39.
- 2.3.25 During fuel movement and control blade movement, the Service Pole Caddy is to be locked in its storage position on the East end of the rail using the two locking brakes to the handrail provided on the paddy caddy.
- 2.3.26 Applicable Precautions and Limitations have been reviewed by each procedure user.
- 2.3.27 Underwater lights have been installed in the Reactor Vessel and Cavity and energized as necessary to support Refueling activities.

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 HC.OP-SO.KE-0001(Q) **PSEG Internal Use Only** Blade Guide Transfer - Spent Fuel Storage Pool to Reactor Core 2.4 The Fuel Grapple Hoist shall be verified operable IAW T.S. 3.9.6 by 2.4.1 performing procedure HC.OP-ST.KE-0001(Q), Refuel Interlock Operability Functional Test - Section 5.4. At least 23 feet of water shall be maintained over the top of 2.4.2 irradiated Fuel Assemblies seated in the Spent Fuel Storage Racks IAW T.S. 3.9.9. Procedure HC.OP-ST.KE-0001(Q); Refuel Interlock Operability Test 2.4.3 must be performed prior to CORE ALTERATIONS IAW T.S. 3.9.1. [CD-182C] The Reactor Mode Switch shall be operable 2.4.4 AND locked in the REFUEL OR the SHUTDOWN position IAW T.S. 3.9.1. The Reactor shall be subcritical for a minimum of 24 hours IAW 2.4.5 T.S. 3.9.4. At least 22 feet 2 inches of water shall be maintained over the top 2.4.6 of the Reactor Pressure Vessel flange IAW T.S. 3.9.8. Breaker 52-254061 is closed to provide power to Refuel Platform. 2.4.7 Prior to moving the Refueling Platform ensure the following 2.4.8 conditions exist: Refuel Platform tracks are clear of equipment A. or materials which would impede platform motion. Cables, ropes or other devices are not tied to the Refuel Platform Β. OR Trolley. The Jib Cranes and Channel Handling Boom have been С. positioned to prevent collisions with the Refuel Platform. The Fuel Grapple, Monorail Hoist and Auxiliary Hoist D. are not carrying any loads. Continued next page

HC.OP-SO.KE-0001(Q)

2.4.8 (continued)

•

nued)		
,	E.	Underwater obstructions which may interfere with fuel transfer are identified and repositioned <u>IF</u> possible. [CD-396Y]
	F.	PERFORM HC.OP-FT.KE-0001(Q); Refuel Platform and Fuel Grapple Operability Test - Refueling:
		1. <u>IF maintenance has been completed on the Refueling</u> Platform and/or Hoists.
		2. Monthly during fuel handling operations.
	G.	PERFORM HC.OP-FT.KE-0002(Q); Fuel Grapple Full Down Travel Functional Test – Refueling <u>IF</u> not performed within 30 days of starting fuel handling operations.
2.4.9	No t Refi or S Refi	esting, maintenance or other activity is in progress on the sel Platform, Spent Fuel Storage Pool, New Fuel Vault, Reactor Cavity hipping Cask Area which would impede the movement of the sel Platform.
2.4.10	OB Read	FAIN the Fuel Movement Sheet(s) developed by ctor Engineering IAW HC.RE-FR.ZZ-0001(Q).
2.4.11	All j have	personnel that will operate the Refuel Platform and Fuel Grapple e completed training and are qualified. [CD-124B]
2.4.12	NO any ensu fuel	TIFY the Radiation Protection Department prior to starting fuel transfer operation. The Radiation Protection Department shall are that personnel leave the upper level of the Drywell prior to transfer. [CD-069A]
2.4.13	Pow	ver to Refuel Platform Control Console energized.
2.4.14	Posi	ition indication cameras for the Main Hoist are selected for display.
2.4.15	No o Pola in A SEC	equipment or tools, with the exception of; the Refueling Platform, the ar Crane or any <u>ANALYZED LOADS</u> (<u>ANALYZED LOADS</u> are listed attachment 1), SHALL be moved over IRRADIATED FUEL <u>UNLESS</u> CONDARY CONTAINMENT INTEGRITY is in effect IAW T.S. 1.39.
2.4.16	Dur is to two	ing fuel movement and control blade movement, the Service Pole Caddy be locked in its storage position on the East end of the rail using the locking brakes to the handrail provided on the paddy caddy.

Hope Creek

Rev. 28

- 2.4.17 Applicable Precautions and Limitations have been reviewed by each procedure user.
- 2.4.18 Underwater lights have been installed in the Reactor Vessel and Cavity and energized as necessary to support Refueling activities.

2.5 Blade Guide Transfer - Reactor Core to Spent Fuel Storage Pool

- 2.5.1 The Fuel Grapple Hoist shall be verified operable IAW T.S. 3.9.6 by performing procedure HC.OP-ST.KE-0001(Q), Refuel Interlock Operability Functional Test - Section 5.4.
- 2.5.2 At least 23 feet of water shall be maintained over the top of irradiated Fuel Assemblies seated in Spent Fuel Storage Racks IAW T.S. 3.9.9.
- 2.5.3 Procedure HC.OP-ST.KE-0001(Q); Refuel Interlock Operability Test must be performed prior to CORE ALTERATIONS IAW T.S. 3.9.1. [CD-182C]
- 2.5.4 The Reactor Mode Switch shall be operable <u>AND</u> locked in the REFUEL OR the SHUTDOWN position IAW T.S. 3.9.1.
- 2.5.5 The Reactor shall be subcritical for a minimum of 24 hours IAW T.S. 3.9.4.
- 2.5.6 At least 22 feet 2 inches of water shall be maintained over the top of the Reactor Pressure Vessel flange IAW T.S. 3.9.8.
- 2.5.7 Breaker 52-254061 is closed to provide power to Refuel Platform.

-

1

HC.OP-SO.KE-0001(Q)

2.5.8	Prior to moving the Refueling Platform ensure the following conditions exist:
	A. Refuel Platform tracks are clear of equipment or materials which would impede platform motion.
	B. Cables, ropes or other devices are not tied to the Refuel Platform or Trolley.
	C. The Jib Cranes and Channel Handling Boom have been positioned to prevent collisions with the Refuel Platform.
	D. The Fuel Grapple, Monorail Hoist <u>AND</u> Auxiliary Hoist are not carrying any loads.
	 E. Underwater obstructions which may interfere with fuel transfer are identified <u>AND</u> repositioned <u>IF</u> possible. [CD-396Y]
	F. PERFORM HC.OP-FT.KE-0001(Q); Refuel Platform and Fuel Grapple Operability Test - Refueling:
	1. IF maintenance has been completed on the Refueling Platform and/or Hoists.
	2. Monthly during fuel handling operations.
	G. PERFORM HC.OP-FT.KE-0002(Q); Fuel Grapple Full Down Travel Functional Test – Refueling <u>IF</u> not performed within 30 days of starting fuel handling operations.
2.5.9	No testing, maintenance or other activity is in progress on the Refuel Platform, Spent Fuel Storage Pool, New Fuel Vault, Reactor Cavity or Shipping Cask Area which would impede the movement of the Refuel Platform.
2.5.10	OBTAIN the Fuel Movement Sheet(s) developed by Reactor Engineering IAW HC.RE-FR.ZZ-0001(Q).
2.5.11	All personnel that will operate the Refuel Platform and Fuel Grapple have completed training and are qualified. [CD-124B]

HC.OP-SO.KE-0001(Q)

2.5.12	NOTIFY the Radiation Protection Department prior to starting any fuel transfer operation. The Radiation Protection Department shall ensure that personnel leave the upper level of the Drywell prior to fuel transfer. [CD-069A]	
2.5.13	Power to Refuel Platform Control Console energized.	
2.5.14	Position indication cameras for the Main Hoist are selected for display.	

2.5.15 No equipment or tools, with the exception of; the Refueling Platform, the Polar Crane or any <u>ANALYZED LOADS</u> (<u>ANALYZED LOADS</u> are listed in Attachment 1), SHALL be moved over IRRADIATED FUEL <u>UNLESS</u>

SECONDARY CONTAINMENT INTEGRITY is in effect IAW T.S. 1.39.

- 2.5.16 During fuel movement and control blade movement, the Service Pole Caddy is to be locked in its storage position on the East end of the rail using the two locking brakes to the handrail provided on the paddy caddy.
- 2.5.17 Applicable Precautions and Limitations have been reviewed by each procedure user.
- 2.5.18 Underwater lights have been installed in the Reactor Vessel and Cavity and energized as necessary to support Refueling activities.

2.6 Fuel Movement - Within Reactor Vessel

- 2.6.1 The Reactor Building Secondary Containment Ventilation System RBVS & FRVS automatic isolation dampers shall be operable IAW T.S. 3.6.5.2.
- 2.6.2 Reactor Building integrity shall be maintained IAW T.S. 3.6.5.1.b.
- 2.6.3 The Fuel Grapple Hoist shall be verified operable IAW T.S. 3.9.6 by performing procedure HC.OP-ST.KE-0001(Q), Refuel Interlock Operability Functional Test - Section 5.4.
- 2.6.4 At least 23 feet of water shall be maintained over the top of irradiated Fuel Assemblies seated in the Spent Fuel Storage Racks IAW T.S. 3.9.9.
- 2.6.5 The Filtration, Recirculation and Ventilation System shall be operable IAW T.S. 3.6.5.3.
- 2.6.6 Procedure HC.OP-ST.KE-0001(Q); Refuel Interlock Operability Test must be performed prior to CORE ALTERATIONS IAW T.S. 3.9.1. [CD-182C]

HC.OP-SO.KE-0001(Q)

2.6.7	The I <u>ANE</u> <u>OR</u> S	Reactor Mode Switch shall be operable locked in the REFUEL SHUTDOWN position IAW T.S. 3.9.1.	
2.6.8	At le <u>ANI</u> <u>ANI</u>	ast two Source Range Monitors (SRM) shall be operable inserted to the normal operating level IAW T.S. 3.9.2 :	
	۸	Annunciation	
	л.	AND continuous visual indication of these	
		monitors is available in the Control Room.	
			<u> </u>
	В.	One of the required SRM detectors is located in the Reactor Core quadrant where CORE ALTERATIONS are being performed <u>AND</u> the other SRM detector shall be located in an adjacent quadrant	nt
2.6.9	All c	ontrol rods are fully inserted IAW T.S. 3.9.3.	
2.6.10	The T.S.	Reactor shall be subcritical for a minimum of 24 hours IAW 3.9.4.	
2.6.11	Estal and I	blish and maintain direct communication between the Control Room Refueling Floor personnel IAW T.S. 3.9.5.	·
2.6.12	At le of th	ast 22 feet 2 inches of water shall be maintained over the top e Reactor Pressure Vessel flange IAW T.S. 3.9.8.	

- 2.6.13 All CORE ALTERATIONS shall be observed <u>AND</u> directly supervised by either a Senior Reactor Operator <u>OR</u> a Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation IAW T.S. 6.2.2.c.
- 2.6.14 Breaker 52-254061 is closed to provide power to Refuel Platform.

`

HC.OP-SO.KE-0001(Q)

2.6.15	Prior to moving the Refueling Platform ensure the following conditions exist:		
	A. Refuel Platform tracks are clear of equipment or materials which would impede platform motion.		
	B. Cables, ropes or other devices are not tied to the Refuel Platform or Trolley.		
	C. The Jib Cranes and Channel Handling Boom have been positioned to prevent collisions with the Refuel Platform.		
	D. The Fuel Grapple, Monorail Hoist and Auxiliary Hoist are not carrying any loads.		
	 E. Underwater obstructions which may interfere with fuel transfer are identified and repositioned <u>IF</u> possible. [CD-396Y] 		
	F. PERFORM HC.OP-FT.KE-0001(Q); Refuel Platform and Fuel Grapple Operability Test - Refueling:		
	1. IF maintenance has been completed on the Refueling Platform and/or Hoists.		
	2. Monthly during fuel handling operations.		
	G. PERFORM HC.OP-FT.KE-0002(Q); Fuel Grapple Full Down Travel Functional Test – Refueling <u>IF</u> not performed within 30 days of starting fuel handling operations.		
2.6.16	No testing, maintenance or other activity is in progress on the Refuel Platform, Spent Fuel Storage Pool, New Fuel Vault, Reactor Cavity or Shipping Cask Area which would impede the movement of the Refuel Platform.		
2.6.17	Permission to move irradiated nuclear fuel or perform CORE ALTERATIONS has been granted by the OS/CRS.		
2.6.18	OBTAIN the Fuel Movement Sheet(s) developed by Reactor Engineering IAW HC.RE-FR.ZZ-0001(Q).		
2.6.19	All personnel that will operate the Refuel Platform and Fuel Grapple have completed training and are qualified. [CD-124B]		

HC.OP-SO.KE-0001(Q)

- 2.6.20 **NOTIFY** the Radiation Protection Department prior to starting any fuel transfer operation. The Radiation Protection Department shall ensure that personnel leave the upper level of the Drywell prior to fuel transfer. **[CD-069A]**
- 2.6.21 Power to Refuel Platform Control Console energized.
- 2.6.22 Position indication cameras for the Main Hoist are selected for display.
- 2.6.23 No equipment or tools, with the exception of; the Refueling Platform, the Polar Crane or any <u>ANALYZED LOADS</u> (<u>ANALYZED LOADS</u> are listed in Attachment 1), SHALL be moved over IRRADIATED FUEL <u>UNLESS</u> SECONDARY CONTAINMENT INTEGRITY is in effect IAW Tech Spec 1.39.
- 2.6.24 During fuel movement and control blade movement, the Service Pole Caddy is to be locked in its storage position on the East end of the rail using the two locking brakes to the handrail provided on the paddy caddy.
- 2.6.25 Applicable Precautions and Limitations have been reviewed by each procedure user.
- 2.6.26 Underwater lights have been installed in the Reactor Vessel and Cavity and energized as necessary to support Refueling activities.

2.7 Blade Guide Movement - Within Reactor Vessel

- 2.7.1 The Fuel Grapple Hoist shall be verified operable IAW T.S. 3.9.6 by performing procedure HC.OP-ST.KE-0001(Q), Refuel Interlock Operability Functional Test - Section 5.4.
- 2.7.2 At least 23 feet of water shall be maintained over the top of irradiated Fuel Assemblies seated in the Spent Fuel Storage Racks IAW T.S. 3.9.9.
- 2.7.3 Procedure HC.OP-ST.KE-0001(Q); Refuel Interlock Operability Test must be performed prior to CORE ALTERATIONS IAW T.S. 3.9.1. [CD-182C]
- 2.7.4 The Reactor Mode Switch shall be operable <u>AND</u> locked in the REFUEL <u>OR</u> the SHUTDOWN position IAW T.S. 3.9.1.
- 2.7.5 The Reactor shall be subcritical for a minimum of 24 hours IAW T.S. 3.9.4.

.

•

HC.OP-SO.KE-0001(Q)

2.7.6	At least 22 feet 2 inches of water shall be maintained over the top of the Reactor Pressure Vessel flange IAW T.S. 3.9.8.			
2.7.7	Breaker 52-254061 is closed to provide power to Refuel Platform.			
2.7.8	Prior to moving the Refueling Platform ensure the following conditions exist:			
	A. Refuel Platform tracks are clear of equipment or materials which would impede platform motion.			
	B. Cables, ropes or other devices are not tied to the Refuel Platform or Trolley.			
	C. The Jib Cranes and Channel Handling Boom have been positioned to prevent collisions with the Refuel Platform.			
	D. The Fuel Grapple, Monorail Hoist and Auxiliary Hoist are not carrying any loads.			
	 E. Underwater obstructions which may interfere with fuel transfer are identified and repositioned <u>IF</u> possible. [CD-396Y] 			
	F. Perform HC.OP-FT.KE-0001(Q); Refuel Platform and Fuel Grapple Operability Test - Refueling:			
	1. <u>IF</u> maintenance has been completed on the Refueling Platform and/or Hoists.			
	2. Monthly during fuel handling operations.			
	 G. PERFORM HC.OP-FT.KE-0002(Q); Fuel Grapple Full Down Travel Functional Test – Refueling <u>IF</u> not performed within 30 days of starting fuel handling operations. 			
2.7.9	No testing, maintenance or other activity is in progress on the Refuel Platform, Spent Fuel Storage Pool, New Fuel Vault, Reactor Cavity or Shipping Cask Area which would impede the movement of the Refuel Platform.			
2.7.10	OBTAIN the Fuel Movement Sheet(s) developed by Reactor Engineering IAW HC.RE-FR.ZZ-0001(Q).			

- 2.7.11 All personnel that will operate the Refuel Platform and Fuel Grapple have completed training and are qualified. [CD-124B]
- 2.7.12 **NOTIFY** the Radiation Protection Department prior to starting any fuel transfer operation. The Radiation Protection Department shall ensure that personnel leave the upper level of the Drywell prior to fuel transfer. **[CD-069A]**
- 2.7.13 Power to Refuel Platform Control Console energized.
- 2.7.14 Position indication cameras for the Main Hoist are selected for display.
- 2.7.15 An SRO shall be on the Refuel Bridge to ensure no CORE ALTERATIONS take place.
- 2.7.16 No equipment or tools, with the exception of; the Refueling Platform, the Polar Crane or any <u>ANALYZED LOADS</u> (<u>ANALYZED LOADS</u> are listed in Attachment 1), SHALL be moved over IRRADIATED FUEL <u>UNLESS</u> SECONDARY CONTAINMENT INTEGRITY is in effect IAW Tech Spec 1.39.
- 2.7.17 During fuel movement and control blade movement, the Service Pole Caddy is to be locked in its storage position on the East end of the rail using the two locking brakes to the handrail provided on the paddy caddy.
- 2.7.18 Applicable Precautions and Limitations have been reviewed by each procedure user.
- 2.7.19 Underwater lights have been installed in the Reactor Vessel and Cavity and energized as necessary to support Refueling activities.

2.8 <u>Fuel/Blade Guide Movement - Within Spent Fuel Storage Pool (other than Refuel</u> <u>Mode)</u>

- 2.8.1 The Reactor Building Secondary Containment Ventilation System RBVS & FRVS automatic isolation dampers shall be operable IAW T.S. 3.6.5.2.
- 2.8.2 Reactor Building integrity shall be maintained IAW T.S. 3.6.5.1.b.
- 2.8.3 The Fuel Grapple Hoist shall be verified operable by performing procedure HC.OP-ST.KE-0001(Q), Refuel Interlock Operability Functional Test - Section 5.4. [CD-178H]
- 2.8.4 At least 23 feet of water shall be maintained over the top of irradiated Fuel Assemblies seated in spent Fuel Storage Racks IAW T.S. 3.9.9.

ì

HC.OP-SO.KE-0001(Q)

Hope Creek			Page 20 of 84	Rev. 28
	2.8.13	OBT Engir	AIN the Fuel Movement Sheet(s) developed by Reactor neering IAW HC.RE-FR.ZZ-0001(Q).	
	2.8.12	Perm by the	ission to move irradiated nuclear fuel has been granted e OS/CRS.	
	2.8.11	Irradi <u>AND</u> <u>OR</u> a other since IAW a fuel	ated Fuel movements in Spent Fuel Storage Pool shall be observe directly supervised by either a Senior Reactor Operator Senior Reactor Operator Limited to Fuel Handling who has no concurrent responsibilities during this operation. This is required the SRO may have to evacuate Refuel floor HC.OP-AB.CONT-0005(Q), Irradiated Fuel Damage, in the even bundle is dropped and radiation levels increase. [CD-168A]	d t
	0.0.11	E.	Underwater obstructions which may interfere with fuel transfer are identified and repositioned <u>IF</u> possible.	
		D.	The Fuel Grapple, Monorail Hoist and Auxiliary Hoist are not carrying any loads.	······
		C.	The Jib Cranes and Channel Handling Boom have been positioned to prevent collisions with the Refuel Platform.	
		B.	Cables, ropes or other devices are not tied to the Refuel Platform or Trolley.	
		А.	Refuel Platform tracks are clear of equipment or materials which would impede platform motion.	·
	2.8.10	Prior cond	to moving the Refueling Platform ensure the following itions exist:	<u> </u>
	2.8.9	Brea	ker 52-254061 is closed to provide power to Refuel Platform.	
	2.8.8	The	Refuel Platform is in a standby line up.	
	2.8.7	Proc Func Refu	edure HC.OP-ST.KE-0001(Q), Refuel Interlock Operability tional Test - Section 5.4); has been completed to demonstrate el Platform load cell setpoint IAW T.S. 3.9.6. [CD-178H]	
	2.8.6	Proc Grap	edure HC.OP-FT.KE-0001(Q); Refuel Platform and Fuel ple Operability Test - Refueling has been completed. [CD-182C]	
	2.8.5	The l	Filtration, Recirculation and Ventilation System shall be able IAW T.S. 3.6.5.3.	

HC.OP-SO.KE-0001(Q)

- 2.8.14 All personnel that will operate the Refuel Platform and Fuel Grapple have completed training and are qualified. [CD-124B]
- 2.8.15 **NOTIFY** the Radiation Protection Department prior to starting any fuel transfer operation.
- 2.8.16 Power to Refuel Platform Control Console energized.
- 2.8.17 Position indication cameras for the Main Hoist are selected for display.
- 2.8.18 Spent Fuel Pool gates are verified installed per HC.MD-FR.KE-0013(Q), Refuel Floor-Shield and Pool Plugs Removal and Replacement.
- 2.8.19 No equipment or tools, with the exception of; the Refueling Platform, the Polar Crane or any <u>ANALYZED LOADS</u> (<u>ANALYZED LOADS</u> are listed in Attachment 1), SHALL be moved over IRRADIATED FUEL <u>UNLESS</u> SECONDARY CONTAINMENT INTEGRITY is in effect IAW T/S 1.39.
- 2.8.20 During fuel movement and control blade movement, the Service Pole Caddy is to be locked in its storage position on the East end of the rail using the two locking brakes to the handrail provided on the paddy caddy.
- 2.8.21 Applicable Precautions and Limitations have been reviewed by each procedure user.

2.9 Auxiliary Hoist Tool Installation

Applicable Precautions and Limitations have been reviewed by each procedure user.

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

3.0 PRECAUTIONS AND LIMITATIONS

3.1 **Precautions**

3.1.1 The normal up stop on Refueling Platform Main Hoist shall NOT be bypassed when handling Fuel Assemblies. The normal up stop on the Refueling Platform Auxiliary Hoist shall not be bypassed when handling irradiated components. (**T.S. 3.9.6**)

3.1.2 To prevent damage to Fuel Assemblies:

- DO NOT attempt to release a grappled Fuel Assembly <u>UNTIL</u> it is fully seated in a storage
 <u>OR</u> Reactor Core location. [CD-736D, CD-123E, CD-182C]
- B. DO NOT reposition the Refuel Platform or Trolley while the Fuel Assembly is grappled and not fully withdrawn from the storage or Reactor Core location.
- C. DO NOT twist a grappled Fuel Assembly while it is held fast in a storage or Reactor Core location.
- 3.1.3 <u>WHEN</u> transferring irradiated fuel between the Reactor Cavity <u>AND</u> Spent Fuel Pool.
 - A. Minimize the time the fuel is in the Drywell Bellows Area. [CD-612X, CD-069A]
 - B. Restrict fuel transfer to most direct path between the Reactor Vessel <u>AND</u> Spent Fuel Pool. [CD-612X]
- 3.1.4 While positioning irradiated fuel in the Reactor Vessel or Spent Fuel Pool, maximize the amount of water shielding between the Fuel Assembly <u>AND</u> Reactor Vessel wall or Spent Fuel Pool wall. [CD-069A, CD-612X, CD-719A]
- 3.1.5 DO NOT remove a blade guide from its Reactor Core location IF the respective control rod will be left unsupported.

- 3.1.6 The Service Pole Caddy which rides on the same rail as the Monorail Auxiliary Hoist is for the removal and installation of the Moisture Separator and miscellaneous in vessel work. During the use of this procedure for fuel movement and control blade movement it is to be locked in its storage position on the East end of the rail using the two locking brakes to the handrail provided on the paddy caddy.
 REFER to procedures HC.MD-FR.KE-0006(Q)
 AND HC.MD-FR.KE-0026(Q) for operation of the Pole Caddy.
- Any of the Refueling Bridge Mast Sections (10", 7", or 5"), can bind 3.1.7 while being extended or retracted. The Encoder readout (distance traveled), used in conjunction with the Load Cell readout (weight on main mast cable), can be used to effectively verify whether binding is occurring. For example, IF you start from a full up position (10", 7", and 5" sections fully retracted), and start lowering the mast sections while observing the Encoder and Load Cell indications, the weight of the 10" section should transfer from the main mast cable to the 12" section at approximately 180" on the Encoder reading (the Load Cell indication will decrease by approximately 105#). Continuing down, the weight of the 7" section should transfer from the main mast cable to the 10" section at approximately 380" on the Encoder reading (the Load Cell indication will decrease by approximately 88#). This methodology can also be applied for use in the opposite direction. IF while lowering the mast, an unexpected slack cable OR pre-mature weight transfer occurs, stop lowering, raise the load to clear the slack cable (if one exists), and notify the cognizant Refuel SRO OR OS/CRS before attempting any corrective action. [CD-619G]
- 3.1.8 <u>WHEN</u> handling fuel out of the Fuel Prep Machine VERIFY the channel fastener is clearly visible and not enclosed by the grapple head. This is the only place that the operator can see the channel fastener while grabbing a fuel bundle. [CD-739F]
- 3.1.9 To prevent damage to underwater lamps ensure that the lamps remain fully submerged while they are energized. [CD-673D]
- 3.1.10 Operation of the Refuel Platform (Bridge), Trolley or Fuel Grapple with the variable speed control operators should be accomplished by gradual accelerations and decelerations, rather than full on and full off deflections of the operators. This will produce less wear on the equipment and better operator control.
- 3.1.11 The Refueling Platform should never be left unattended with the "POWER ON" (as indicated by the STOP light on the Left Hand Controller). An electrical failure could cause the platform to move on its own accord. Turning off the power for short durations while the operator leaves the platform should be a standard practice.

Hope Creek

Rev. 28

HC.OP-SO.KE-0001(Q)

- 3.1.12 Abnormalities that may occur during operation, such as spurious high load readings or audible indications should not be ignored. Such occurrences must be investigated and the cause ascertained to prevent possible subsequent damage to the platform or handled fuel.
- 3.1.13 Should an emergency stop occur during lowering of auxiliary or monorail hoists, the safety brake of that hoist will set without delay and take the hoist load on its ratchet-pawl arrangement.
 - Prior to attempting to releasing the brake, the cause of the emergency stop should be understood. <u>IF</u> possible, the grapple should be moved away from irradiated fuel prior to performing the release.
 - In the case where a full load is being lowered at the time of the stop, this load may prevent the safety brake from immediately releasing on a subsequent hoist operation, unless the hoist is commanded to raise as an initial operation, thereby taking the load off the safety brake.
 - <u>IF</u> the brake does not release because of a lowering command, it will emit a loud chattering noise, alerting the operator to raise the load. In most cases, no harm will occur as the result of an operator not following this procedure. The only consequence is that the safety brake will prevent the hoist from lowering.
- 3.1.14 Should an emergency stop occur during the lowering of the Main Hoist, the hoist safety brake will set without delay and take the hoist load.
 - Prior to attempting to release the brake, the cause of the emergency stop should be understood. <u>IF</u> possible, the grapple should be moved away from irradiated fuel prior to performing the release.
 - In the case where a full load is being loaded at the time of the stop, this load may prevent the safety brake from immediately releasing on a subsequent hoist operation, unless the hoist is commanded to raise as an initial operation, thereby taking the load off the safety brake.
 - In most cases,

IF the brake does not release because of a lowering command, no harm will occur as the result of an operator not following this procedure. The only consequence is that the safety brake will prevent the hoist from lowering.

3.1.15 An indication that the Grapple has missed completely is a Full Grapple Down Light ON with NO Slack Cable Light.

3.2 Limitations

- 3.2.1 Operate the Refuel Platform, Trolley and Fuel Grapple Hoist variable speed controls in a manner which results in gradual accelerations or decelerations.
- 3.2.2 The Fuel Grapple is the only hoist used to transfer Fuel Assemblies in the Spent Fuel Pool and Reactor Cavity.
- 3.2.3 The fuel may be transferred only to those devices and storage locations specified on the Fuel Movement Sheet(s) developed by Reactor Engineering IAW HC.RE-FR.ZZ-0001(Q). [CD-719A]
- 3.2.4 DO NOT move a Fuel Assembly within one foot of any assembly stored in a fuel preparation machine, shipping canister or special test fixture. [CD-719A]
- 3.2.5 A designated "spotter" is required for all bridge activities which require any grapple to be loaded.
- 3.2.6 No individual should perform bridge activities for greater than six consecutive hours.
- 3.2.7 <u>IF</u> the Technical Specification requirements listed in Section 2.0 of this procedure cannot be maintained:
 - A. **COMPLETE** the transfer of any fuel loaded on the Fuel Grapple OR **RETURN** the fuel to its original location.
 - B. **TERMINATE** all further fuel transfers <u>AND</u> CORE ALTERATIONS <u>UNTIL</u> the conditions specified in the Technical Specification requirements listed in Section 2.0 are restored.

`

HC.OP-SO.KE-0001(Q)

3

3.2.8	IF Refuel Platform or Fuel Grapple hoist equipment failure occurs while fuel is loaded on the Fuel Grapple: [CD-736D, CD-442A]	
	A. PERFORM corrective actions IAW Attachment 2.	
	B. REPORT the nature and extent of the equipment damage to the Refuel Floor Supervisor or Operations Superintendent.	
3.2.9	IF underwater lights are used for illumination ensure the lamps are de-energized prior to removing them from the water. [CD-673D]	
3.2.10	The Radiation Protection Department should be notified when fuel transfers are completed or are to be delayed for greater than one hour.	
3.2.11	Operate the Refuel Platform, Trolley and Fuel Grapple in a manner to avoid collisions with underwater obstructions that can not be repositioned.	
3.2.12	At the discretion of the Refuel Bridge SRO, and, with concurrence from Station Management, operation of the bridge and mast in concurrent multiple directions is permissible (e.g., moving the mast up/down with bridge motion, etc.). Individual responsibilities (SRO and spotter) regarding collision-avoidance should be briefed prior to any multiple dimensional fuel moves. [70007921]	
3.2.13	<u>WHEN</u> inserting and removing Fuel Assemblies in the Reactor Core prevent contact between the Fuel Assembly and incore detectors and installed neutron sources. [CD-365D]	
3.2.14	ENSURE the Refuel Bridge handrails remain in place whenever the camera cart is on the bridge. [CD-711F]	
3.2.15	Pre-operational assembly requirements for the Fuel Support Grapple is dependent upon the configuration of the control rod and drive assembly at the site of the fuel support.	
3.2.16	The control blade grapple, the combined CRB/FSP grapple and the jet pump grapple are the only grapples to be used to handle control blades within the reactor pressure vessel.	
3.2.17	All torque wrenches used in the performance of this procedure should be of the appropriate range to read the reference value or the last recorded torque value between 20% and 100% of range. [PR 960128076]	

HC.OP-SO.KE-0001(Q)

- 3.2.18 IF alternate RHR-FPCC assist mode is in service, up to 9000 gpm may flow through the spent fuel pool to the reactor well. This will cause increased drag loads on the refuel mast when a load is transported through the fuel pool gates. It may be necessary to reduce bridge speed, when passing through the gate, to control vertical swing on the mast.
- 3.2.19 The refueling mast should not be permitted to swing against the trolley. This may cause bending or damage to the mast.

3.3 Interlocks

<u>NOTE</u> 3.3.1

Main hoist loaded relays actuate at 535 pounds and 550 pounds.

- 3.3.1 The following conditions will cause a Rod Block:
 - A. Main Hoist loaded, the Refuel Bridge over the Reactor Core, <u>AND</u> Mode Switch in REFUEL.
 - B. The Refuel Bridge over the Reactor Core <u>AND</u> Mode Switch in STARTUP.
- 3.3.2 The following conditions will prevent the Refueling Bridge from traveling in the reverse direction:
 - A. Main Hoist loaded, the Refuel Bridge over the Core, and any control rod withdrawn.
 - B. Refuel Bridge over the Core <u>AND</u> Mode Switch in STARTUP.

The following conditions will prevent Main Hoist up motion:		
A. Normal up stop (can be bypassed).		
B. Backup hoist limit.		
C. Hoist jammed as indicated by 1200 pounds load.		
D. Grapple not closed and the Main Hoist weight over 550 lbs.		
E. Fuel Hoist Interlock:		
 Rod out signal from Control Room RS1 Limit Switch actuated Main Hoist Loaded 		
The following conditions will prevent Main Hoist down motion:		
A. Slack cable as indicated by (< 60 lbs) and the Slack Cable Light on. (Actual weight indication will vary between Fuel Pool and In Core operation due to the length of cable used and the weight of that cable).		
B. Full down limit reached at approximately 4 inches below the elevation of a fuel bundle in the core (as indicated by the Full Grapple Down Light on).		
C. Fuel Hoist Interlock:		
 Rod out signal from Control Room RS1 Limit Switch actuated Main Hoist Loaded 		
L and Float function is disconnected per ECA (HE 0021		

• Calibrated M&TE Torque Wrench (25 - 30 ft-lbs) for a 3/4" nut.

Hope Creek

4.0

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

5.0 **PROCEDURE**

<u>NOTE</u> 5.0

All movements of the Refuel Platform, Trolley and Fuel Grapple hoist described in this procedure are performed at the Fuel Grapple Operator's Console. <u>WHEN</u> moving the auxiliary hoist(s) to set the mechanical jam, the hoist is operated from the applicable hoist pendant.

5.1 Refuel Platform Start-up and Shutdown

5.1.1 **ENSURE** that all prerequisites have been satisfied IAW Section 2.1.

<u>NOTE</u> 5.1.2

<u>IF</u> the AC power to the Refuel Platform was open, the Refuel Platform should be warmed up for a minimum of 1/2 hour prior to use.

5.1.2	Ref	uel Platform Start-up Instructions		
	А.	At the West end of the Refuel Platform VERIFY the following:		
		1. The Main Power Disconnect Switch is in the CLOSED position.		
		2. The Monorail Hoist Disconnect Switch is in the CLOSED position.		
	B.	At the West end of the Refuel Platform (Bridge), VERIFY the Main Hoist Air Isolation Valve AND Monorail Hoist Air Isolation Valve are open.		
	C.	At the West end of the Refuel Platform PLACE the Air Compressor Disconnect Switch is in the CLOSED position.		
Continued next page				

HC.OP-SO.KE-0001(Q)

5.1.2 (continued)

- D. PRESS the ON pushbutton for each of the 5 position indication cameras (2 cameras overhead behind Trolley Cab, 3 cameras on the West End Truck ground level).
- E. **PLACE** the position indication CRTs to the ON position in the Trolley Cab.
- F. **PLACE** each of the 3 Digital Position Readout power supply toggle switches to the ON position in the Trolley Cab. West side.

<u>NOTE</u> 5.1.3

AC power should remain available to the Refuel Platform. Breaker 52-254061 should remain closed to ensure adequate warm-up prior to use. The Control Panels heaters are required to be energized to prevent condensation between periods of use (outages).

5.1.3	Refuel Platform Shutdown/Standby Instructions
-------	---

- A. **PRESS** the Refuel Platform STOP pushbutton <u>AND</u> **VERIFY** the red STOP pushbutton light is off.
- B. **PLACE** the Air Compressor Disconnect Switch is in the OPEN position at the West end of the Refuel Platform.
- C. ENSURE the Main Power Disconnect Switch <u>AND</u> the Monorail Hoist Disconnect Switch remain in the CLOSED position until maintenance is to be performed.
- **PRESS** the OFF pushbutton for each of the 5 position indication cameras (2 cameras overhead behind Trolley Cab, 3 cameras on the West End Truck ground level).
- E. **PLACE** the position indication CRTs to the OFF position in the Trolley Cab.
- F. **PLACE** each of the 3 Digital Position Readout power supply toggle switches to the OFF position In the Trolley Cab. West side.

•

5.2.1	ENSURE that all prerequisites have been satisfied IAW Section 2.2 of this procedure.	
5.2.2	USE the Trolley position control to position the Fuel Grapple to attain direct alignment with the center of the fuel transfer path. [CD-396Y]	
5.2.3	USE the Refuel Platform position control in the REVERSE position to maneuver the Refuel Platform over the Reactor Core.	
5.2.4	Using the Refuel Platform <u>AND</u> Trolley position controls <u>AND</u> position indication system MANEUVER the Fuel Grapple to the coordinates listed on the Fuel Movement Sheet(s).	
5.2,5	LOWER the Fuel Grapple to approximately one foot above the Fuel Assembly to be removed by using the Fuel Grapple Hoist control in the LOWER position.	
5.2.6	ROTATE the Fuel Grapple so that the grapple hooks are in line with the Fuel Assembly handle.	
5.2.7 PLACE the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position to ensure the Fuel Grapple hooks are of OBSERVE the grapple ENGAGE light off.		
5.2.8	Slowly LOWER the Fuel Grapple until the Fuel Grapple is resting on the Fuel Assembly handle. Grapple may be gently twisted to seat on handle.	
	A. OBSERVE Fuel Grapple Hoist downward motion automatically stops.	
	B. OBSERVE the SLACK CABLE light on. [CD-736D]	
	C. ENSURE Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.	
	D DOTATE grapple in both directions (CD 730E)	

Hope Creek

Rev. 28

v

HC.OP-SO.KE-0001(Q)

1

5.2.10	PLACE the GRAPPLE ENGAGE/RELEASE Switch in the ENGAGE position to grapple the Fuel Assembly.			
	A. OBSERVE the grapple ENGAGE light is on.			
	B. ROTATE grapple in both directions to ensure proper engagement. [CD-739F]			
5.2.11	NOTIFY Control Room that assembly is being removed from Core <u>AND</u> that the SRM's should be observed UNTIL assembly is out of the core.			
5.2.12	Slowly RAISE the grappled Fuel Assembly using the Fuel Grapple Hoist Control in the RAISE position.			
	A. OBSERVE the SLACK CABLE light off.			
	B. OBSERVE the HOIST LOADED light on. [CD-736D]			
	C. OBSERVE the load cell indicator to ensure the assembly has been grappled.			
	D. Visually VERIFY assembly is being lifted.			

Hope Creek

Rev. 28

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

CAUTION 5.2.13

- A. DO NOT attempt to release the Fuel Assembly unless it is fully seated in a storage or Reactor Core location. [CD-123E, CD-736D, CD-182C]
- B. DO NOT reposition the Refuel Platform or Trolley while the Fuel Assembly is grappled and not fully withdrawn from the storage or Reactor Core location.
- C. DO NOT twist a grappled fuel assembly while it is held fast in a Storage or Reactor Core location.
- D. While positioning irradiated fuel in the Reactor Vessel or Spent Fuel Pool maximize the amount of water shielding between the Fuel Assembly and Reactor Vessel or Fuel Pool wall. [CD-612X, CD-069A]
 - 5.2.13 **RAISE** <u>AND</u> CONTINUOUSLY **OBSERVE** the Fuel Assembly during its upward travel. **NOTIFY** Control Room when assembly is out of the Core.
 - 5.2.14 **ENSURE** hose and cable reels are taking up slack by observing cables and air hoses remain taught within the grapple.

CAUTION 5.2.15

DO NOT bypass the Main Hoist normal up stop when handling Fuel Assemblies or control rod blades in order to comply with T.S. 3.9.6.

- 5.2.15 WHEN the Fuel Assembly has reached its upward limit:
 - A. **OBSERVE** the GRAPPLE NORMAL UP light on. **[CD-736D]**
 - B. Fuel Grapple upward motion automatically stops.
PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

CAUTION 5.2.16

A. Minimize the time the fuel is in the Drywell Bellows Area. [CD-612X, CD-069A]

B. Restrict fuel transfer to the most direct path between the Reactor Vessel and Spent Fuel Pool. [CD-612X]

- 5.2.16 USE the Trolley position control to position the Fuel Grapple to attain direct alignment with the center of the fuel transfer path to the Fuel Storage Pool. [CD-396Y]
- 5.2.17 **ENSURE** the Refuel Platform position control is in the FORWARD position to maneuver the Refuel Platform to the Spent Fuel Pool.
- 5.2.18 USE the Refuel Platform and Trolley position controls to maneuver the Fuel Grapple to the Spent Fuel Pool Storage Rack location listed on the Fuel Movement Sheet(s).

5.2.19 **ROTATE** the Fuel Grapple <u>AND</u> attached Fuel Assembly to attain direct alignment and correct orientation with the Fuel Storage Rack location.

Hope Creek

Rev. 28

PSEG Internal Use Only

~

•

HC.OP-SO.KE-0001(Q)

!

		CAUTION 5.2.20
Α.	DO N in a s	OT attempt to release the Fuel Assembly unless it is fully seated to rage or Reactor Core location. [CD-123E, CD-736D, CD-182C]
В.	DO N Asse Reac	OT reposition the Refuel Platform or Trolley while the Fuel mbly is grappled and not fully withdrawn from the storage or tor Core location.
C.	DO N Stora	OT twist a grappled fuel assembly while it is held fast in a ige or Reactor Core location.
D.	While maxi and F	e positioning irradiated fuel in the Reactor Vessel or Spent Fuel Pool mize the amount of water shielding between the Fuel Assembly Reactor Vessel or Fuel Pool wall. [CD-612X, CD-069A]
	5.2.20	ENSURE the Fuel Grapple is at the correct Core or Fuel Pool Coordinates.
	5.2.21	Slowly LOWER the Fuel Assembly into the storage rack location using the Fuel Grapple Hoist position control in the LOWER position.
	5.2.22	<u>WHEN</u> the SRO has ensured that the Fuel Assembly is fully seated in the correct Fuel Storage Rack location:
		A. OBSERVE the SLACK CABLE light on. [CD-736D]
		B. OBSERVE the HOIST LOADED light off. [CD-736D]
		C. OBSERVE no load indicated (< 60 lbs) on the load indicator.
		D. OBSERVE Fuel Grapple downward motion automatically stops
		E. ENSURE Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.
	5.2.23	PLACE the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position AND OBSERVE the grapple ENGAGE light off. [CD-736D]

HC.OP-SO.KE-0001(Q)

- 5.2.24 Slowly **RAISE** the Fuel Grapple by using the hoist position control in the RAISE position <u>AND</u>:
 - A. **OBSERVE** the SLACK CABLE light off. [CD-736D]
 - B. **OBSERVE** the HOIST LOADED light remains off. **[CD-736D]**
 - C. **OBSERVE** the load cell indicator to ensure that no load has been inadvertently grappled. **[CD-442A]**
 - D. **ENSURE** that hose and cable reels are taking up slack by observing air hoses remain taught within the Grapple.
- 5.2.25 **RAISE** the Fuel Grapple fully to avoid collision with any underwater obstructions. **[CD-396Y]**
- 5.2.26 **INITIAL** the appropriate box on the Fuel Movement Sheet(s) indicating the fuel transfer has been completed.
- 5.2.27 **NOTIFY** the Control Room personnel that the fuel move has been completed.
- 5.2.28 Control Room personnel should update the fuel location tracking tool in use.

HC.OP-SO.KE-0001(Q)

5.3 Fuel	Transfer - Spent	Fuel Storage Pool	to Reactor Core
-----------------	------------------	-------------------	-----------------

- 5.3.1 **ENSURE** that all prerequisites have been satisfied IAW Section 2.3 of this procedure.
- 5.3.2 USE the Refuel Platform and Trolley position controls to maneuver the Fuel Grapple to the Spent Fuel Pool location listed on the Fuel Movement Sheets.
- 5.3.3 **LOWER** the Fuel Grapple to approximately one foot above the Fuel Assembly to be removed by using the Fuel Grapple Hoist control in the LOWER position.
- 5.3.4 IF possible, visually ENSURE correct serial number of Fuel Assembly.

SRO

<u>NOTE</u> 5.3.5

The Fuel Assembly should be grappled in the Spent Fuel Pool so that the Fuel Assembly channel fastener is in the proper orientation for Fuel Assembly installation in the Reactor Core. The Fuel Movement Sheet(s) provides correct core channel fastener orientation.

- 5.3.5 **ROTATE** the Fuel Grapple so that the grapple hooks are in line with the Fuel Assembly handle.
- 5.3.6 PLACE the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position to ensure the Fuel Grapple hooks are open, OBSERVE the grapple ENGAGE light off.
- 5.3.7 Slowly **LOWER** the Fuel Grapple UNTIL the Fuel Grapple is resting on the Fuel Assembly handle. Grapple may be gently twisted to seat on handle <u>AND</u>:
 - A. **OBSERVE** Fuel Grapple downward motion automatically stops.
 - B. OBSERVE the SLACK CABLE light is on. [CD-736D]
 - C. **ENSURE** Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.
 - D. ROTATE grapple in both directions. [CD-739F]

Hope Creek

Page 37 of 84

Rev. 28

ALL ACTIVE ON 20030122	-THE-SPOT CHAN	GES M	JST BE ATTACHED FOR FIELD USE	
PSE	<u>G Internal Use Or</u>	HC.OP-SO.KE-00	HC.OP-SO.KE-0001(Q)	
	5.3.8	ENS <u>OR</u>	SURE the Fuel Grapple is at the correct Core Fuel Pool Coordinates.	SRO
	5.3.9	PLA ENG	CE the GRAPPLE ENGAGE/RELEASE Switch in the GAGE position to grapple the Fuel Assembly.	
		A.	OBSERVE the GRAPPLE ENGAGED light is on. [CD-736D]	
		B.	ROTATE grapple in both directions to ensure proper engagement. [CD-739F]	
	5.3.10	Slov Graj	vly RAISE the grappled Fuel Assembly using the Fuel ople hoist control in the RAISE position.	
		А.	OBSERVE the SLACK CABLE light is off. [CD-736D]	
		B.	OBSERVE the HOIST LOADED light is on. [CD-736D]	
		C.	OBSERVE the load cell indicator to ensure the assembly has been grappled.	
		D.	Visually ENSURE assembly is being lifted.	
\smile		E.	ENSURE that hose and cable reels are taking up slack by observing air hoses remain taught within the Grapple.	

١

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

CAUTION 5.3.11

- A. DO NOT attempt to release the Fuel Assembly unless it is fully seated in a Storage or Reactor Core location. [CD-123E, CD-736D, CD-182C]
- B. DO NOT reposition the Refuel Platform or Trolley while the Fuel Assembly is grappled and not fully withdrawn from the storage or Reactor Core location.
- C. DO NOT twist a grappled fuel assembly while it is held fast in a storage or Reactor Core location.
- D. While positioning irradiated fuel in the Reactor Vessel or Spent Fuel Pool maximize the amount of water shielding between the Fuel Assembly and Reactor Vessel or Fuel Pool wall. [CD-612X, CD-069A]
 - 5.3.11 **RAISE** AND CONTINUOUSLY **OBSERVE** the Fuel Assembly during its upward travel.

CAUTION 5.3.12

DO NOT bypass the Main Hoist normal up stop when Handling Fuel Assemblies or control rod blades in order to comply with T.S. 3.9.6.

5.3.12 <u>WHEN</u> the Fuel Assembly has reached its upper limit of travel THEN **OBSERVE**:

- A. GRAPPLE NORMAL UP light on. [CD-736D]
- B. Fuel Grapple upward motion automatically stops.

PSEG Internal Use Only

L,

HC.OP-SO.KE-0001(Q)

		<u>CAUTION</u> 5.3.13		
Α.	Minir [CD-6	nize the time the fuel is in the Drywell Bellows area. 312X, CD-069A]		
В.	Restrict fuel transfer to the most direct path between the Reactor Vesse and Spent Fuel Pool.			
	5.3.13	USE the Trolley position control to position the Fuel Grapple in direct alignment with the center of the fuel transfer path to the Reactor Well. [CD-396Y]		
	5.3.14	USE the Refuel Platform Position Control in the REVERSE position to maneuver the Refuel Platform to the Reactor Cavity area.		
	5.3.15	USE the Refuel Platform and Trolley Position Controls and Position Indication System to maneuver the Fuel Grapple to the Reactor Core location listed on the Fuel Movement Sheet(s).		
	5.3.16	NOTIFY Control Room that platform is moving over the Core.	·	
	5.3.17	ENSURE the Fuel Grapple is at the correct Core <u>OR</u> Fuel Pool Coordinates.		
	5.3.18	ROTATE the Fuel Grapple <u>AND</u> attached Fuel Assembly in direct alignment with the Reactor Core location and corresponding channel fastener orientation listed on the Fuel Movement Sheet(s).	5KU	
	5.3.19	NOTIFY the Control Room that the assembly is about to enter the Core and that SRM's must be observed until assembly is seated.	. <u></u>	

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

CAUTION 5.3.20

- A. DO NOT attempt to release the Fuel Assembly unless it is fully seated in a storage or Reactor Core location. [CD-123E, CD-736D, CD-182C]
- B. DO NOT reposition the Refuel Platform or Trolley while the Fuel Assembly is grappled and not fully withdrawn from the storage or Reactor Core location.
- C. DO NOT twist a grappled fuel assembly while it is held fast in a storage or Reactor Core location.
- D. While positioning irradiated fuel in the Reactor Vessel or Spent Fuel Pool maximize the amount of water shielding between the Fuel Assembly and Reactor Vessel or Fuel Pool wall. [CD-612X, CD-069A]
 - 5.3.20 Slowly LOWER the Fuel Assembly into the Reactor Core location using the Fuel Grapple Hoist Position Control in the LOWER position.
 - 5.3.21 <u>WHEN</u> the SRO has ensured that the Fuel Assembly is fully seated in the correct Reactor Core location:
 - A. NOTIFY Control Room that assembly is seated in it proper Core location.
 - B. **OBSERVE** the SLACK CABLE light on. [CD-736D]
 - C. OBSERVE the HOIST LOADED light off. [CD-736D]
 - D. OBSERVE no load indicated (< 60 lbs) on the load cell indicator.
 - E. **OBSERVE** Fuel Grapple downward motion automatically stops.
 - F. **ENSURE** Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.
 - 5.3.22 PLACE the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position <u>AND</u> OBSERVE the grapple ENGAGE light off. [CD-736D]

٦

HC.OP-SO.KE-0001(Q)

5.3.23	Slowly RAISE the Fuel Grapple by using the Hoist Position Control in the RAISE position <u>AND</u> :		
	A. OBSERVE the SLACK CABLE light off. [CD-736D]		
	B. OBSERVE the HOIST LOADED light remains off. [CD-736D]		
	C. OBSERVE the Load Cell Indicator to ensure that no load has inadvertently been grappled. [CD-442A]		
5.3.24	ENSURE that hose and cable reels are taking up slack by observing air hoses remain taught within the Grapple.		
5.3.25	RAISE the Fuel Grapple fully to avoid collision with any underwater obstructions. [CD-396Y]		
5.3.26	INITIAL the appropriate box on the Fuel Movement Sheet(s) indicating the fuel transfer has been completed.		
5.3.27	NOTIFY the Control Room personnel that the fuel move has been completed.		
5.3.28	Control Room personnel should UPDATE the fuel location tracking tool in use.		

÷

			/
5.4	<u>Blade G</u>	uide Transfer - Spent Fuel Storage Pool to Reactor Core	
	5.4.1	ENSURE that all prerequisites have been satisfied IAW Section 2.4 of this procedure.	
	5.4.2	USE the Refuel Platform and Trolley position controls to maneuver the unloaded Fuel Grapple to the blade guide location in the Spent Fuel Pool.	
	5.4.3	ENSURE the Fuel Grapple is at the correct Core OR Fuel Pool Coordinates.	
			SRO
	5.4.4	LOWER the Fuel Grapple to approximately one foot above the blade guide to be grappled by using the Fuel Grapple Hoist Control in the LOWER position.	
	5.4.5	ROTATE the Fuel Grapple so that the grapple hooks are in line with the blade guide handle.	
	5.4.6	PLACE the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position to ensure the Fuel Grapple hooks are open, OBSERVE the grapple ENGAGE light off.	
	5.4.7	Slowly LOWER the Fuel Grapple UNTIL the Fuel Grapple is resting on the blade guide handle. Grapple may be gently twisted to seat on handle.	
		A. OBSERVE Fuel Grapple Hoist downward motion automatically stops.	
		B. OBSERVE the SLACK CABLE light on. [CD-736D]	
		C. ENSURE Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.	
	5.4.8	PLACE the GRAPPLE ENGAGE/RELEASE Switch in the ENGAGE position to grapple the blade guide AND OBSERVE the GRAPPLE ENGAGE light on. [CD-736D]	

HC.OP-SO.KE-0001(Q)

- 5.4.9 Slowly **RAISE** the grappled blade guide using the Fuel Grapple hoist control in the RAISE position <u>AND</u>:
 - A. **OBSERVE** the SLACK CABLE light off. [CD-736D]
 - B. **OBSERVE** the Load Cell Indicator to ensure that blade guide has been grappled.
 - C. Visually VERIFY blade guide is being lifted.
- 5.4.10 **RAISE** <u>AND</u> CONTINUOUSLY **OBSERVE** the blade guide during its upward travel.
- 5.4.11 ENSURE hose and cable reels are taking up slack by observing cables and air hoses remain taught within the grapple.

<u>NOTE</u> 5.4.12

HOIST LOADED light may come on when hoist is almost full up.

- 5.4.12 <u>WHEN</u> the blade guide has reached its upper limit <u>AND</u> **OBSERVE**:
 - A. GRAPPLE NORMAL UP light on. [CD-736D]
 - B. Fuel Grapple upward motion automatically stops.
- 5.4.13 USE the Trolley Position Control to position the Fuel Grapple in direct alignment with the center of the fuel transfer path to the Reactor Well. [CD-396Y]
- 5.4.14 **USE** the Refuel Platform Control in the Reverse position to maneuver the Refuel Platform to the Reactor Cavity area.
- 5.4.15 **USE** the Refuel Platform and Trolley Position Controls to maneuver the Fuel Grapple to the Reactor Core location listed on the Fuel Movement Sheet(s). Position Indication System should be used for single blade guides.
- 5.4.16 **NOTIFY** Control Room that platform is moving over the core.

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 **PSEG Internal Use Only** HC.OP-SO.KE-0001(Q) 5.4.17 **ENSURE** the Fuel Grapple is at the correct Core **OR Fuel Pool Coordinates.** SRO 5.4.18 **ROTATE** the Fuel Grapple AND attached blade guide in direct alignment with the Reactor Core location AND orientation listed on the Fuel Movement Sheet(s). **CAUTION 5.4.19** Α. DO NOT attempt to release the blade guide unless it is fully seated in the Reactor Core location. [CD-123E] Β. DO NOT reposition the Refuel Platform or Trolley while the blade guide is grappled and not fully withdrawn from the Reactor Core location. C. DO NOT twist a grappled blade guide while it is held fast in a Reactor Core location. Slowly LOWER the blade guide into the Reactor Core 5.4.19 location(s) using the Fuel Grapple hoist in the LOWER position. 5.4.20 WHEN the blade guide is fully seated in the Reactor Core location(s): A. **OBSERVE** the SLACK CABLE light on. Β. **OBSERVE** the HOIST LOADED light off. **C**. **OBSERVE** no load indicated (< 60 lbs) on the load cell indicator. D. **OBSERVE** Fuel Grapple downward motion automatically stops E. ENSURE Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout. 5.4.21 **NOTIFY** the Control Room that the blade guides are in the Core location(s). 5.4.22 PLACE the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position AND **OBSERVE** the grapple ENGAGE light off.

÷

.

HC.OP-SO.KE-0001(Q)

1

	5.4.23	Slowly RAISE the Fuel Grapple by using the Hoist Position Control in the RAISE position.	
		A. OBSERVE the SLACK CABLE light off.	
		B. OBSERVE the HOIST LOADED light remains off.	
		C. OBSERVE the Load Cell Indicator to ensure that no load has been inadvertently been grappled. [CD-442A]	
	5.4.24	ENSURE that hose and cable reels are taking up slack by observing cables and air hoses remain taught within the grapple.	
	5.4.25	RAISE the Fuel Grapple fully to avoid collision with any underwater obstructions. [CD-396Y]	
	5.4.26	INITIAL the appropriate box on the Fuel Movement Sheet(s) indicating the completion of the blade guide transfer step.	
	5.4.27	Control Room personnel should UPDATE the location tracking tool in use.	
5.5	<u>Blade G</u> i	uide Transfer - Reactor Core to Spent Fuel Storage Pool	
	5.5.1	ENSURE that all prerequisites have been satisfied IAW Section 2.5 of this procedure.	
	5.5.2	USE the Refuel Platform and Trolley Position Controls to maneuver the Fuel Grapple to the coordinates listed on the Fuel Movement Sheet(s). Digital Position Indication System should be used for single blade guides.	
	5.5.3	ENSURE the Fuel Grapple is at the correct Core OR Fuel Pool Coordinates.	
	5.5.4	LOWER the Fuel Grapple to approximately one foot above the blade guide to be removed by using the Fuel Grapple Hoist control in the LOWER position.	SRO
	5.5.5	ROTATE the Fuel Grapple so that the grapple hooks are in line with the blade guide handle.	

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

- 5.5.6 **PLACE** the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE to ensure the Fuel Grapple hooks are open, **OBSERVE** the grapple ENGAGE light off.
- 5.5.7 Slowly LOWER the Fuel Grapple UNTIL the Fuel Grapple is resting on the blade guide handle. Grapple may be gently twisted to seat on handle.
 - A. **OBSERVE** Fuel Grapple Hoist downward motion automatically stops.
 - B. **OBSERVE** the SLACK CABLE light on.
 - C. **ENSURE** Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.
- 5.5.8 **PLACE** the GRAPPLE ENGAGE/RELEASE Switch in the ENGAGE position to grapple the blade guide <u>AND</u> **OBSERVE** the GRAPPLE ENGAGE light on.

CAUTION 5.5.9

DO NOT remove a blade guide from its Reactor Core location if the respective control rod will be left unsupported.

- 5.5.9 Slowly **RAISE** the grapple blade guide using the Fuel Grapple Hoist Control in the RAISE position.
 - A. **OBSERVE** the SLACK CABLE light off.
 - B. **OBSERVE** the load cell indicator to ensure the blade guide has been grappled.
 - C. Visually **VERIFY** blade guide is being lifted.

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

CAUTION 5.5.10

- A. DO NOT attempt to release the blade guide unless it is fully seated in the Reactor Core location. [CD-123E]
- B. DO NOT reposition the Refuel Platform or Trolley while the blade guide is grappled and not fully withdrawn from the Reactor Core location.
- C. DO NOT twist a grappled blade guide while it is held fast in a Reactor Core location.
 - 5.5.10 **RAISE** <u>AND</u> CONTINUOUSLY **OBSERVE** the blade guide during its upward travel.
 - 5.5.11 **ENSURE** that hose and cable reels are taking up slack by observing cables and air hoses remain taught within the grapple.

<u>NOTE</u> 5.5.12

HOIST LOADED light may come on when hoist is almost full up.

- 5.5.12 <u>WHEN</u> the blade guide has reached its upper limit <u>THEN</u> OBSERVE:
 - A. GRAPPLE NORMAL UP light on.
 - B. Fuel Grapple upward motion automatically stops.
- 5.5.13 **NOTIFY** the Control Room that blade guide is out of the core.
- 5.5.14 USE the Trolley position control to position the Fuel Grapple in direct alignment with the center of the fuel transfer path to the Fuel Pool. [CD-396Y]
- 5.5.15 **USE** the Refuel Platform position control in the FORWARD position to maneuver the Refuel Platform to the Spent Fuel Pool.
- 5.5.16 **NOTIFY** the Control Room that the platform is over the Fuel Pool.

PSEG Internal Use Only

•

×

HC.OP-SO.KE-0001(Q)

!

5.5.17	USE the Refuel Platform <u>AND</u> Trolley Position Controls to maneuver the Fuel Grapple to an available blade guide storage location.
5.5.18	ENSURE the Fuel Grapple is at the correct Core OR Fuel Pool Coordinates.
5.5.19	ROTATE the Fuel Grapple SKO AND attached blade guide so the blade guide cross beam is directly over the storage location support listed on the Fuel Movement Sheet(s).
5.5.20	Slowly LOWER the blade guide UNTIL it is fully seated on the support.
5.5.21	<u>WHEN</u> the blade guide is fully seated on the support <u>THEN</u> :
	A. OBSERVE the SLACK CABLE light on.
	B. OBSERVE the HOIST LOADED light off.
	C. OBSERVE no load indicated (< 60 lbs) on the load cell indicator.
	D. ENSURE Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.
5.5.22	PLACE the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position AND OBSERVE the grapple ENGAGE light off.
5.5.23	Slowly RAISE the Fuel Grapple using the Hoist Control in the RAISE position.
	A. OBSERVE the SLACK CABLE light off.
	B. OBSERVE the HOIST LOADED light remains off.
	C. OBSERVE the load cell indicator to verify that a load was not inadvertently grappled. [CD-442A]
	D. ENSURE that hose and cable reels are taking up slack by observing air hoses remain taught within the Grapple.

5.6

~

		(()
5.5.24	RAISE the Fuel Grapple fully to avoid collision with any underwater obstructions. [CD-396Y]	
5.5.25	INITIAL the appropriate block on the Fuel Movement Sheet(s) indicating that the blade guide transfer has been completed.	
5.5.26	NOTIFY Control Room personnel that the blade guide transfer has been completed.	
<u>Fuel M</u>	ovement - Within Reactor Vessel	
5.6.1	ENSURE that all prerequisites have been satisfied IAW Section 2.6 of this procedure.	
5.6.2	USE the Trolley Position Control to position the Fuel Grapple to attain direct alignment with the center of the fuel transfer path. [CD-396Y]	
5.6.3	USE the Refuel Platform Position Control in the REVERSE position to maneuver the Refuel Platform over the Reactor Core.	
5.6.4	Using the Refuel Platform and Trolley Position Controls <u>AND</u> Position Indication System, MANELIVER the Fuel Grapple to the coordinates listed	
	on the Fuel Movement Sheet(s).	_
5.6.5	LOWER the Fuel Grapple to approximately one foot above the Fuel Assembly to be transferred by using the Fuel Grapple Hoist control in the LOWER position.	
5.6.6	ENSURE the Fuel Grapple is at the correct Core Coordinates.	
5.6.7	ROTATE the Fuel Grapple so that the grapple hooks are in line with the fuel assembly handle.	SRO
5.6.8	PLACE the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position to ensure the Fuel Grapple hooks are open, OBSERVE the grapple ENGAGE light off.	

~

HC.OP-SO.KE-0001(Q)

5.6.9	Slowly LOWER the Fuel Grapple UNTIL the Fuel Grapple resting on the Fuel Assembly handle. Grapple may be gently twisted to seat on handle.			
	А.	OBSERVE Fuel Grapple Hoist downward motion automatically stops.		
	B. OBSERVE the SLACK CABLE light on. [CD-736D]			
	C.	ENSURE Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.		
	D.	ROTATE grapple in both directions. [CD-739F]		
5.6.10	PLA in th	ACE the GRAPPLE ENGAGE/RELEASE Switch he ENGAGE position to grapple the fuel assembly <u>AND</u> :		
	А.	OBSERVE the grapple ENGAGE light is on.		
	B.	ROTATE grapple in both directions to ensure proper engagement. [CD-739F]		
5.6.11	NO' ANI	TIFY Control Room the assembly is being removed from Core D that the SRM's should be observed until assembly is out of the Core	<u>.</u>	
5.6.12	Slov Graj	wly RAISE the grappled Fuel Assembly using the Fuel pple Hoist Control in the RAISE position.		
	А.	OBSERVE the SLACK CABLE light off.		
	B.	OBSERVE the HOIST LOADED light on. [CD-736D]		
	C.	OBSERVE the Load Cell Indicator to ensure the assembly has been grappled.		
	D.	Visually VERIFY assembly is being lifted.		

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

CAUTION 5.6.13

- A. DO NOT attempt to release the Fuel Assembly unless it is fully seated in a storage or Reactor Core location. [CD-123E, CD-736D, CD-182C]
- B. DO NOT reposition the Refuel Platform or Trolley while the Fuel Assembly is grappled and not fully withdrawn from the storage or Reactor Core location.
- C. DO NOT twist a grappled fuel assembly while it is held fast in a storage or Reactor Core location.
- D. While positioning irradiated fuel in the Reactor Vessel or Spent Fuel Pool maximize the amount of water shielding between the Fuel Assembly and Reactor Vessel or Fuel Pool wall. [CD-612X, CD-069A]
 - 5.6.13 **RAISE** <u>AND</u> CONTINUOUSLY **OBSERVE** the Fuel Assembly during its upward travel. **NOTIFY** Control Room when assembly is out of the core.
 - 5.6.14 **ENSURE** that hose and cable reels are taking up slack by observing cables and air hoses remain taught within the grapple.

<u>NOTE</u> 5.6.15

Safe working height is the height of the Fuel Assembly so that the top guide is cleared by approximately 5 feet.

- 5.6.15 <u>WHEN</u> the Fuel Assembly has reached its safe working height as indicated with a hoist grapple elevation digital readout (< 300 inches) <u>THEN</u> **NOTIFY** Control Room that Fuel Assembly is out of its old core location.
- 5.6.16 USE the Refuel Platform <u>AND</u> Trolley Position Controls <u>AND</u> Digital Position Indication System to maneuver the Fuel Grapple to the Reactor Core location listed on the Fuel Movement Sheet(s).

.

•

HC.OP-SO.KE-0001(Q)

5.6.17	NOTIFY Control Room that Fuel Assembly is over its new core location.	
5.6.18	ENSURE the Fuel Grapple is at the correct Core OR Fuel Pool Coordinates.	
		SRO
5.6.19	ROTATE the Fuel Grapple <u>AND</u> attached Fuel Assembly in direct alignment with the Reactor Core location and corresponding channel fastener orientation listed on the Fuel Movement Sheet(s).	
5.6.20	NOTIFY the Control Room that the assembly is about to enter the Core <u>AND</u> that SRM's must be observed until assembly is seated.	
5.6.21	Slowly LOWER the Fuel Assembly into the Reactor Core location using the Fuel Grapple Hoist Position Control in the LOWER position.	
5.6.22	<u>WHEN</u> the SRO has ensured that the Fuel Assembly is fully seated in the correct Reactor Core location:	
	A. NOTIFY Control Room that assembly is seated in its proper Core location.	
	B. OBSERVE the SLACK CABLE light on. [CD-736D]	
	C. OBSERVE the HOIST LOADED light off. [CD-736D]	
	D. OBSERVE no load (< 60 lbs) indicated on the load cell indicator.	
	E. OBSERVE Fuel Grapple downward motion automatically stops.	
	F. ENSURE Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.	
5.6.23	PLACE the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position AND OBSERVE the grapple ENGAGE light off. [CD-736D]	

_

~

HC.OP-SO.KE-0001(Q)

	5.6.24	Slowly RAISE the Fuel Grapple by using the Hoist Position Control in the RAISE position.	
		A. OBSERVE the SLACK CABLE light off. [CD-736D]	
		B. OBSERVE the HOIST LOADED light remains off. [CD-736D]	
		C. OBSERVE the load cell indicator to ensure that no load has been inadvertently grappled. [CD-442A]	
	5.6.25	ENSURE that hose and cable reels are taking up slack by observing cables and air hoses remain taught within the grapple.	
	5.6.26	RAISE the Fuel Grapple fully to avoid collision with any underwater obstructions. <u>IF</u> next Fuel Assembly to be handled is nearby, RAISE the Fuel Grapple at least one foot. [CD-396Y]	
	5.6.27	INITIAL the appropriate box on the Fuel Movement Sheet(s) indicating the fuel transfer has been completed.	<u></u>
	5.6.28	NOTIFY the Control Room personnel that the fuel move has been completed.	
	5.6.29	Control Room personnel should UPDATE the fuel location tracking tool in use.	<u> </u>
5.7	<u>Blade G</u>	uide Movement - Within Reactor Vessel	
	5.7.1	ENSURE that all prerequisites have been satisfied IAW Section 2.7 of this procedure.	
	5.7.2	USE the Refuel Platform and Trolley Position Controls to maneuver the Fuel Grapple to the coordinates listed on the Fuel Movement Sheet(s) Digital Position Indication System should be used for single blade guides). 3
	5.7.3	ENSURE the Fuel Grapple is at the correct Core <u>OR</u> Fuel Pool Coordinates.	
	5.7.4	LOWER the Fuel Grapple to approximately one foot above the blade guide to be transferred by using the Fuel Grapple hoist control in the LOWER position.	SRO
	5.7.5	ROTATE the Fuel Grapple so that the grapple hooks are in line with the blade guide handle.	
Hope Creek		Page 54 of 84 R	ev. 28

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

- 5.7.6 **PLACE** the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position to ensure the Fuel Grapple hooks are open, **OBSERVE** the ENGAGE light off.
- 5.7.7 Slowly LOWER the Fuel Grapple UNTIL the Fuel Grapple is resting on the blade guide handle. Grapple may be gently twisted to seat on handle.
 - A. **OBSERVE** Fuel Grapple hoist downward motion automatically stops.
 - B. **OBSERVE** the SLACK CABLE light on.
 - C. **ENSURE** Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.
- 5.7.8 **PLACE** the GRAPPLE ENGAGE/RELEASE Switch in the ENGAGE position to grapple the blade guide <u>AND</u> **OBSERVE** the GRAPPLE ENGAGE light on.

<u>CAUTION</u> 5.7.9

DO NOT remove a blade guide from its Reactor Core location if the respective control rod will be left unsupported.

- 5.7.9 Slowly **RAISE** the grapple blade guide using the Fuel Grapple Hoist Control in the RAISE position.
 - A. **OBSERVE** the SLACK CABLE light off.
 - B. **OBSERVE** the Load Cell Indicator to ensure that blade guide has been grappled.
 - C. Visually **VERIFY** blade guide is being lifted.

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

CAUTION 5.7.10

- A. DO NOT attempt to release the blade guide unless it is fully seated in the Reactor Core location. [CD-123E]
- B. DO NOT reposition the Refuel Platform or Trolley while the blade guide is grappled and not fully withdrawn from the Reactor Core location.
- C. DO NOT twist a grappled blade guide while it is held fast in a Reactor Core location.
 - 5.7.10 **RAISE** <u>AND</u> CONTINUOUSLY **OBSERVE** the blade guide during its upward travel.
 - 5.7.11 **ENSURE** hose and cable reels are taking up slack by observing cables and air hoses remain taught within the grapple.

<u>NOTE 5.7.12</u>

Safe working height is the height of the Fuel Assembly so that the top guide is cleared by approximately 5 feet.

- 5.7.12 <u>WHEN</u> the blade guide has reached its safe working height as indicated with a hoist grapple elevation digital readout (< 300 inches) <u>THEN</u> NOTIFY Control Room that blade guide is out of the core.
- 5.7.13 USE the Refuel Platform <u>AND</u> Trolley position controls to maneuver the Fuel Grapple to the Reactor Core location listed on the Fuel Movement Sheet(s). Digital Position Indication System should be used for single blade guides.
- 5.7.14 **NOTIFY** Control Room that platform is moving to new location over the core.
- 5.7.15 **ENSURE** the Fuel Grapple is at the correct Core OR Fuel Pool Coordinates.

SRO

HC.OP-SO.KE-0001(Q)

5.7.16 **ROTATE** the Fuel Grapple <u>AND</u> attached blade guide in direct alignment with the Reactor Core location <u>AND</u> orientation listed on the Fuel Movement Sheet(s).

CAUTION 5.7.17

- A. DO NOT attempt to release the blade guide unless it is fully seated in the Reactor Core location. [CD-123E]
- B. DO NOT reposition the Refuel Platform or Trolley while the blade guide is grappled and not fully withdrawn from the Reactor Core location.
- C. DO NOT twist a grappled blade guide while it is held fast in a Reactor Core location.
 - 5.7.17 Slowly **LOWER** the blade guide into the Reactor Core location(s) using the Fuel Grapple hoist in the LOWER position.
 - 5.7.18 <u>WHEN</u> the blade guide is fully seated on the support <u>THEN</u>:
 - A. **OBSERVE** the SLACK CABLE light on.
 - B. **OBSERVE** the HOIST LOADED light off.
 - C. **OBSERVE** no load indicated (< 60 lbs) on the load cell indicator.
 - D. **OBSERVE** Fuel Grapple downward motion automatically stops.
 - E. **ENSURE** Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.
 - 5.7.19 **NOTIFY** the Control Room that the blade guides are in the core location(s).
 - 5.7.20 **PLACE** the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position <u>AND</u> **OBSERVE** the grapple ENGAGE light off.

Ń

HC.OP-SO.KE-0001(Q)

5.7.21	Slowly RAISE the Fuel Grapple using the Hoist Position Control in the RAISE position <u>AND</u> :	
	A. OBSERVE the SLACK CABLE light off.	
	B. OBSERVE the HOIST LOADED light remains off.	
	C. OBSERVE the Load Cell Indicator to ensure that no load has been inadvertently been grappled. [CD-442A]	
5.7.22	ENSURE that hose and cable reels are taking up slack by observing cables and air hoses remain taught within the grapple.	
5.7.23	RAISE the Fuel Grapple fully to avoid collision with any underwater obstructions. <u>IF</u> next blade guide to be handled is nearby RAISE the Fuel Grapple at least one foot. [CD-396Y]	
5.7.24	INITIAL the appropriate box on the Fuel Movement Sheet(s) indicating the completion of the blade guide transfer step.	

<u>NOTE</u> 5.7.25

Control Room personnel should update the location tracking tool in use.

5.7.25 **NOTIFY** Control Room personnel that the blade guide transfer has been completed.

PSEG Internal Use Only

5.8 <u>Fuel/Blade Guide Movement - Within Spent Fuel Storage Pool (other than Refuel</u> <u>Mode</u>)

<u>NOTE</u> 5.8

All irradiated fuel moves or core alterations must be directly supervised by a licensed SRO or SRO limited to fuel-handling. Non-irradiated fuel handling not involving core alterations and blade guide movement do not require direct supervision by an SRO and can be annotated by the spotter directly involved with the evolution. [CD-168A]

This procedure Section describes any combination of fuel/blade guide in Spent Fuel Storage Pool movements between any of the following fuel storage locations:

- Fuel storage racks
- Fuel preparation machines
- Fuel sipping canisters
- Special test and measurement devices

This Section uses the term "source" location as the location where a fuel/blade guide assembly is initially grappled and "target" location as the destination of the fuel/blade guide assembly.

Fuel Assembly channel fastener orientation and location must be accomplished and documented in accordance with the Fuel Movement Sheets.

- 5.8.1 **ENSURE** that all prerequisites have been satisfied IAW Section 2.8 of this procedure.
- 5.8.2 Using the Refuel Platform
 <u>AND</u> Trolley Position Controls
 <u>AND</u> Position Indication System
 <u>MANEUVER</u> the Fuel Grapple to the coordinates listed
 for the source location on the Fuel Movement Sheet(s).
- 5.8.3 ENSURE the Fuel Grapple is at the correct Fuel Pool Coordinates. (REFER TO Note 5.8)
- 5.8.4 LOWER the Fuel Grapple to approximately one foot above the fuel assembly/blade guide to be removed by using the Fuel Grapple Hoist control in the LOWER position <u>AND</u> OBSERVE the GRAPPLE NORMAL UP light off.

۰.

HC.OP-SO.KE-0001(Q)

1

5.8.5	<u>IF</u> possible <u>WHEN</u> fuel is being moved, SRO should visually ENSURE correct serial number of Fuel Assembly. (REFER TO Note 5.8)	
5.8.6	ROTATE the Fuel Grapple so that the grapple hooks are in line with the fuel assembly/blade guide handle.	
5.8.7	PLACE the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position to ensure the Fuel Grapple hooks are open, OBSERVE the ENGAGE light off.	
5.8.8	Slowly LOWER the Fuel Grapple UNTIL the Fuel Grapple is resting on the fuel assembly/blade guide handle. Grapple may be gently twisted to seat on handle.	
	A. OBSERVE Fuel Grapple downward motion automatically stops.	
	B. OBSERVE the SLACK CABLE light on.	
	C. ENSURE Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.	
	D. ROTATE grapple in both directions. [CD-739F]	
5.8.9	PLACE the GRAPPLE ENGAGE/RELEASE Switch in the ENGAGE position to grapple the fuel assembly/blade guide.	
	A. OBSERVE the grapple ENGAGE light on.	
	B. ROTATE grapple in both directions to ensure proper engagement. [CD-739F]	
5.8.10	Slowly RAISE the grappled fuel assembly/blade guide using the Fuel Grapple hoist control in the RAISE position.	
	A. OBSERVE the SLACK CABLE light off	
	B. OBSERVE the HOIST LOADED light:	
	Light ON for Fuel Assembly	<u>_</u>
Continued next page	• Light OFF for Blade Guide	. <u></u>

Hope Creek

Page 60 of 84

5.8.10 (continued)

PSEG Internal Use Only

- C. **OBSERVE** the Load Cell Indicator to ensure the assembly has been grappled.
- D. Visually VERIFY assembly/blade guide is being lifted.

CAUTION 5.8.11

- A. DO NOT attempt to release the fuel assembly/blade guide unless it is fully seated in storage location. [CD-182C, CD-123E]
- B. DO NOT reposition the Refuel Platform or Trolley while the fuel assembly/blade guide is grappled and not fully withdrawn from the storage location.
- C. DO NOT twist a grappled fuel assembly/blade guide while it is held fast in a storage location. [CD-069A CD-612X]
- D. While positioning Irradiated fuel in the Spent Fuel Pool maximize the amount of water shielding between the Fuel Assembly and Fuel Pool wall.
 - 5.8.11 **RAISE** <u>AND</u> CONTINUOUSLY **OBSERVE** the fuel assembly/blade guide during its upward travel.
 - 5.8.12 **ENSURE** hose and cable reels are taking up slack by observing cables and air hoses remain taught within the grapple.

CAUTION 5.8.13

DO NOT bypass the Main Hoist normal up stop when handling Fuel Assemblies or control rod blades in order to comply with T.S. 3.9.6.

- 5.8.13 <u>WHEN</u> the fuel assembly/blade guide has reached its upper limit:
 - A. **OBSERVE** the GRAPPLE NORMAL UP light on.
 - B. Fuel Grapple upward motion automatically stops.

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

- 5.8.14 USE the Refuel Platform <u>AND</u> Trolley Position Controls <u>AND</u> Position Indicating System to maneuver the Fuel Grapple to the target location listed on the Fuel Movement Sheet(s).
- 5.8.15 ENSURE the Fuel Grapple is at the correct Core OR Fuel Pool Coordinates. (REFER TO Note 5.8)
- 5.8.16 **ROTATE** the Fuel Grapple <u>AND</u> attached fuel assembly/blade guide to attain direct alignment <u>AND</u> orientation with the target location. **[CD-396Y]**

CAUTION 5.8.17

- A. DO NOT attempt to release the fuel assembly/blade guide unless it is fully seated in a storage location. [CD-182C, CD-123E]
- B. DO NOT reposition the Refuel Platform or Trolley while the fuel assembly is grappled and not fully withdrawn from the storage or Reactor Core location.
- C. DO NOT twist a grappled fuel assembly while it is held fast in a storage location.
 - 5.8.17 Slowly LOWER the fuel assembly/blade guide into the target location using the Fuel Grapple hoist position control in the LOWER position.
 - 5.8.18 <u>WHEN</u> the fuel assembly/blade guide is fully seated in the target location AND:
 - A. **OBSERVE** the SLACK CABLE light on.
 - B. **OBSERVE** the HOIST LOADED light off.
 - C. **OBSERVE** no load indicated (< 60 lbs) on the load cell indicator.
 - D. **OBSERVE** Fuel Grapple downward motion automatically stops.
 - E. ENSURE Fuel Grapple is at required depth by observing HOIST GRAPPLE ELEVATION Digital Readout.

Page 62 of 84

5

HC.OP-SO.KE-0001(Q)

: •

į

5.8.19	PRES PLACE the GRAPPLE ENGAGE/RELEASE Switch to the RELEASE position <u>AND</u> OBSERVE the grapple ENGAGE light off.	
5.8.20	Slowly RAISE the Fuel Grapple by using the hoist position control in the RAISE position.	
	A. OBSERVE the SLACK CABLE light off.	
	B. OBSERVE the HOIST LOADED light remains off.	
	C. OBSERVE the Load Cell Indicator to ensure that no load has been inadvertently grappled. [CD-442A]	u
5.8.21	RAISE the Fuel Grapple fully to avoid collisionwith any underwater obstructions.IF next Fuel AssemblyOR blade guide to be moved is nearby,RAISE the Fuel Grapple at least one foot.	
5.8.22	ENSURE hose and cable reels are taking up slack by observing cables and air hoses remain taught within the grapple.	
5.8.23	IF Fuel Assemblies were moved, INITIAL the appropriate box on the Fuel Movement Sheet(s) indicating the fuel transfer has been completed.	
5.8.24	NOTIFY the Control Room that the fuel transfer has been completed.	
5.8.25	Control Room personnel should UPDATE the fuel location tracking tool in use.	

Hope Creek

Page 63 of 84

Rev. 28

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

5.9 Auxiliary Hoist Tool Installation [CD-117G]

<u>NOTE</u> 5.9

- A. The installation steps in this section will be performed by a qualified Maintenance Technician under the direction of the operations department.
- B. The fitting on the end of the auxiliary and monorail hoist cable has 1/2" -13 UNC threads. The Control Rod Grapple, Jet pump Grapple, Fuel Support Piece Grapple, Combined CRB/FSP Grapple and the General Purpose Grapple have 7/16" -14 UNC threads in the mounting stud (threaded hole on top of tool). Failure to use a cable terminal/adaptor, which threads between the hoists cable fitting and the tool mounting stud can cause failure due to thread mismatch. [CD-217E, CD-949A]
 - 5.9.1 The following tools may be mounted to the Auxiliary Hoist's (Monorail OR Frame mounted) cable terminal, using the cable terminal/adaptor:
 - A. Jet Pump Grapple
 - B. Control Rod Grapple
 - C. Fuel Support Grapple
 - D. Instrument Handling Tool
 - E. General Purpose Grapple
 - F. Combined CRB/FSP Grapple

5.9.2 **CONNECT** the Instrument Handling Tool to a Refueling Platform Auxiliary Hoist as follows:

<u>NOTE 5.9.2.A</u>

To assure preload, thread sealant should NOT be used on the jam nuts at both ends of the cable terminal/adapter.

- A. APPLY a thread sealant, either Loctite Thread Sealant
 2432 or 5772, on both the male cable terminal threads and the female cable terminal/adaptor threads. [CD-949A]
- B. **THREAD** the cable terminal/adaptor onto the Auxiliary Hoist cable terminal. **[CD-949A]**
- C. APPLY a thread sealant, either Loctite Thread Sealant 2432 or 5772, to the male cable terminal threads and the female mounting stud on top of the tool. [CD-949A]
- D. **SUPPORT** the instrument handling tool in the vertical position <u>AND</u> **THREAD** the cable terminal/adaptor into the top of the mounting stud.
- E. VERIFY proper thread depth of insertion, both visually
 <u>AND</u> by point insertion at the sight hole in the square mounting stud. [CD-949A]
- F. **TIGHTEN** the jam nuts, by torquing each jam nut to 25 30 ft-lbs [CD-949A].

<u>NOTE</u> 5.9.2.G

Radioactive water may be present in the air lines. Radiation Protection should monitor performance of Step 5.9.2.G.

 G. Carefully BLEED Refuel Platform dual airlines into rag to remove any moisture, <u>THEN</u> ATTACH the air supply connections to the tool quick connect devices.

Continued next page

Hope Creek

Page 65 of 84

5.9.2 (continued)

- H. <u>IF</u> the hoist must be removed from the water <u>AND</u> irradiated components are not being handled, **LOOSEN** the bolts on the mechanical jam <u>AND</u> **SLIDE** the jam toward the grapple to allow full withdrawal of the grapple. **TIGHTEN** the bolts.
- I. <u>IF</u> the hoist to be set is used for handling irradiated components, **PERFORM** the following: [CD-451X]

<u>NOTE</u> 5.9.2.1.1

Performance of Step 5.9.2.1.1 will meet the requirements for the Instrument Handling tool. [80033517]

- VERIFY the electrical stops for the applicable Refueling Platform Auxiliary Hoist have been set IAW HC.OP-ST.KE-0001(Q), Refuel Interlock Operability Functional Test - Section 5.4 for the Jet Pump Grapple.
- 2. **LOWER** the grapple to greater than 7 feet below the surface of the water to clear the hoist override.
- 3. **RAISE** the grapple to reach the electrical stops. (The mechanical jam may have to be lowered to allow this).
- 4. **LOOSEN** the bolts on the mechanical jam <u>AND</u> **SLIDE** the jam up UNTIL it just touches the limit switch LS4 (LS4 is a finger switch just below the mechanical jam entry point).

5. **TIGHTEN** the bolts.

HC.OP-SO.KE-0001(Q)

5.9.3 **CONNECT** a Control Rod Grapple to a Refueling Platform Auxiliary Hoist as follows:

<u>NOTE</u> 5.9.3.A

To assure preload, thread sealant should NOT be used on the jam nuts at both ends of the cable terminal/adapter.

- A. APPLY a thread sealant, either Loctite Thread Sealant
 2432 or 5772, on both the male cable terminal threads and the female cable terminal/adaptor threads. [CD-949A]
- B. **THREAD** the cable terminal/adaptor onto the Auxiliary Hoist cable terminal. **[CD-949A**]
- C. APPLY a thread sealant, either Loctite Thread Sealant 2432 or 5772, to the male cable terminal threads and the female mounting stud on top of the tool. [CD-949A]
- D. **SUPPORT** the Control Rod Grapple tool in the vertical position <u>AND</u> **THREAD** the cable terminal/adaptor into the top of the mounting stud.
- E. **VERIFY** proper thread depth of insertion, both visually <u>AND</u> by point insertion at the sight hole in the square mounting stud. **[CD-949A]**.
- F. TIGHTEN the jam nuts, by torquing each jam nut to 25 30 ft-lbs. [CD-949A].

<u>NOTE</u> 5.9.3.G

Radioactive water may be present in the air lines. Radiation Protection should monitor performance of Step 5.9.3.G.

 G. Carefully BLEED Refuel Platform dual airlines into rag to remove any moisture, <u>THEN</u> ATTACH the air supply connections to the tool quick connect devices.

Continued next page

HC.OP-SO.KE-0001(Q)

5.9.3 (continued)

- H. <u>IF</u> the hoist must be removed from the water <u>AND</u> irradiated blades are not being handled, **LOOSEN** the bolts on the mechanical jam <u>AND</u> SLIDE the jam toward the grapple to allow full withdrawal of the grapple. **TIGHTEN** the bolts.
- I. <u>IF</u> the hoist is to be set to be used for handling irradiated blades, **PERFORM** the following: [CD-451X]
 - VERIFY the electrical stops for the applicable Refueling Platform Auxiliary Hoist have been set IAW HC.OP-ST.KE-0001(Q), Refuel Interlock Operability Functional Test - Section 5.4 for the Control Rod Grapple.
 - 2. **LOWER** the grapple to greater than 7 feet below the surface of the water to clear the hoist override.
 - 3. **RAISE** the grapple to reach the electrical stops. (The mechanical jam may have to be lowered to allow this).
 - LOOSEN the bolts on the mechanical jam <u>AND</u> SLIDE the jam up UNTIL it just touches the limit switch LS4 (LS4 is a finger switch just below the mechanical jam entry point).
 - 5. **TIGHTEN** the bolts.

5.9.4 **CONNECT** the Jet Pump Grapple to a Refueling Platform auxiliary hoist as follows:

<u>NOTE</u> 5.9.4.A

To assure preload, thread sealant should NOT be used on the jam nuts at both ends of the cable terminal/adapter.

- A. APPLY a thread sealant, either Loctite Thread Sealant 2432 or 5772, on both the male cable terminal threads and the female cable terminal/adaptor threads. [CD-949A]
- B. **THREAD** the cable terminal/adaptor onto the Auxiliary Hoist cable terminal. **[CD-949A]**.
- C. APPLY a thread sealant, either Loctite Thread Sealant 2432 or 5772, to the male cable terminal threads and the female mounting stud on top of the tool. [CD-949A]
- D. **SUPPORT** the Jet Pump Grapple in the vertical position <u>AND</u> **THREAD** the cable terminal/adaptor into the top of the mounting stud.
- E. VERIFY proper thread depth of insertion, both visually
 <u>AND</u> by point insertion at the sight hole in the square mounting stud.
 [CD-949A].
- F. **TIGHTEN** the jam nuts, by torquing each jam nut to 25 30 ft-lbs. [CD-949A].

<u>NOTE</u> 5.9.4.G

Radioactive water may be present in the air lines. Radiation Protection should monitor performance of Step 5.9.4.G.

G. Carefully **BLEED** Refuel Platform dual airlines into rag to remove any moisture, <u>THEN</u> **ATTACH** the air supply connections to the tool quick connect devices.

Continued next page
ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122 **PSEG Internal Use Only** HC.OP-SO.KE-0001(Q) 5.9.4 (continued) H. PRESS the GRAPPLE RELEASE pushbutton AND VERIFY the keeper on the jet pump grapple hook is open. I. PRESS the GRAPPLE ENGAGE pushbutton AND VERIFY the keeper closes over the grapple hook. PRESS the GRAPPLE RELEASE pushbutton to open the keeper. J. IF the hoist must be removed from the water AND irradiated components are not being handled, LOOSEN the bolts on the mechanical jam AND SLIDE the jam toward the grapple to allow full withdrawal of the grapple. **TIGHTEN** the bolts. K. IF the hoist is to be set to be used for handling irradiated blades, PERFORM the following: [CD-451X] VERIFY the electrical stops for the applicable 1. Refueling Platform Auxiliary Hoist have been set IAW HC.OP-ST.KE-0001(Q), Refuel Interlock **Operability Functional Test - Section 5.4** 2. LOWER the grapple to greater than 7 feet below the surface of the water to clear the hoist override. 3. **RAISE** the grapple to reach the electrical stops. (The mechanical jam may have to be lowered to allow this). 4. LOOSEN the bolts on the mechanical jam AND SLIDE the jam up UNTIL it just touches the limit switch LS4 (LS4 is a finger switch just below the mechanical jam entry point). 5. TIGHTEN the bolts.

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

<u>NOTE 5.9.5</u>

<u>IF</u> the control rod is partially extended through the cruciform slot in the support, and the protrusion is observed to exceed the internal dimension of the slot in the grapple, the assembly is prepared using the two-legged top frame.

<u>IF</u> no obstructions are observed to exist, the grapple is assembled with the grid guide, a device provided to hold the grapple in proper orientation with respect to the top guide structure.

<u>IF</u> the location of the support is readily accessible, the hoist connection may be made directly to the stud and plate assembly.

5.9.5 **CONNECT** the Fuel Support Grapple to a Refueling Platform auxiliary hoist as follows:

- A. <u>IF</u> necessary, **INSTALL** the top frame or the top guide.
- B. **REMOVE** the lockwire from the 4 caps on the bolts AND SCREW out the bolts to take off the stud plate.
- C. <u>WITH</u> the components supported to avoid torsional stress, BOLT the top assembly to the grapple top plate <u>AND</u> REWIRE the caps to specification.

<u>NOTE</u> 5.9.5.D

To assure preload, thread sealant should NOT be used on the jam nuts at both ends of the cable terminal/adapter.

- D. APPLY a thread sealant, either Loctite Thread Sealant 2432 or 5772, on both the male cable terminal threads and the female cable terminal/adaptor threads. [CD-949A]
- E. **THREAD** the cable terminal/adaptor onto the Auxiliary Hoist cable terminal. **[CD-949A]**.

Continued next page

Hope Creek

Page 71 of 84

Rev. 28

HC.OP-SO.KE-0001(Q)

5.9.5 (continued)

- F. APPLY a thread sealant, either Loctite Thread Sealant 2432 or 5772, to the male cable terminal threads and the female mounting stud on top of the tool. [CD-949A]
- G. **SUPPORT** the Fuel Support Grapple in the vertical position <u>AND</u> **THREAD** the cable terminal/adaptor into the top of the mounting stud.
- H. VERIFY proper thread depth of insertion, both visually <u>AND</u> by point insertion at the sight hole in the square mounting stud. [CD-949A].
- I. **TIGHTEN** the jam nuts, by torquing each jam nut to 25 30 ft-lbs. [CD-949A].

<u>NOTE</u> 5.9.5.J

Radioactive water may be present in the air lines. Radiation Protection should monitor performance of Step 5.9.5.J.

- J. Carefully **BLEED** Refuel Platform dual airlines into rag to remove any moisture, <u>THEN</u> **ATTACH** the air supply connections to the tool quick connect devices.
- K. <u>IF</u> the hoist must be removed from the water and fuel support pieces are NOT being handled, LOOSEN the bolts on the mechanical jam <u>AND</u> SLIDE the jam toward the grapple to allow full withdrawal of the grapple. TIGHTEN the bolts.

Continued next page

5.9.5 (continued)

L. <u>IF</u> the hoist is to be set to be used for handling fuel support pieces, **PERFORM** the following: **[CD-451X]**

<u>NOTE</u> 5.9.5.L.1

IF the electrical stops are set in accordance with Step 5.9.5.L.1, this will meet the requirement for the DEI or the GE Fuel Support Piece Grapple.

- 1. **VERIFY** the electrical stops for the applicable Refueling Platform Auxiliary Hoist have been set for either the Control Rod Grapple or the Jet Pump Grapple IAW HC.OP-ST.KE-0001(Q), Refuel Interlock Operability Functional Test - Section 5.4
- 2. LOWER the grapple to greater than 7 feet below the surface of the water to clear the hoist override.
- 3. **RAISE** the grapple to reach the electrical stops. (The mechanical jam may have to be lowered to allow this).
- LOOSEN the bolts on the mechanical jam <u>AND</u> SLIDE the jam up UNTIL it just touches the limit switch LS4 (LS4 is a finger switch just below the mechanical jam entry point).

5. **TIGHTEN** the bolts.

HC.OP-SO.KE-0001(Q)

5.9.6 **CONNECT a Combined CRB/FSP** Grapple to a Refueling Platform Auxiliary Hoist as follows:

<u>NOTE</u> 5.9.6.A

To assure preload, thread sealant should NOT be used on the jam nuts at both ends of the cable terminal/adapter.

- A. APPLY a thread sealant, either Loctite Thread Sealant
 2432 or 5772, on both the male cable terminal threads and the female cable terminal/adaptor threads. [CD-949A]
- B. **THREAD** the cable terminal/adaptor onto the Auxiliary Hoist cable terminal. **[CD-949A]**
- C. APPLY a thread sealant, either Loctite Thread Sealant 2432 or 5772, to the male cable terminal threads and the female mounting stud on top of the tool. [CD-949A]
- D. **SUPPORT** the Combined CRB/FSP Grapple tool in the vertical position <u>AND</u> **THREAD** the cable terminal/adaptor into the top of the mounting stud.
- E. VERIFY proper thread depth of insertion, both visually <u>AND</u> by point insertion at the sight hole in the square mounting stud. [CD-949A].
- F. TIGHTEN the jam nuts, by torquing each jam nut to 25 30 ft-lbs. [CD-949A].

<u>NOTE</u> 5.9.6.G

Radioactive water may be present in the air lines. Radiation Protection should monitor performance of Step 5.9.6.G.

G. Carefully **BLEED** Refuel Platform dual airlines into rag to remove any moisture, <u>THEN</u> **ATTACH** the air supply connections to the tool quick connect devices.

Continued next page

HC.OP-SO.KE-0001(Q)

5.9.6 (continued)

- H. <u>IF</u> the hoist must be removed from the water <u>AND</u> irradiated blades are NOT being handled, **LOOSEN** the bolts on the mechanical jam <u>AND</u> SLIDE the jam toward the grapple to allow full withdrawal of the grapple. **TIGHTEN** the bolts.
- I. <u>IF</u> the hoist is to be set to be used for handling irradiated blades, **PERFORM** the following: **[CD-451X]**
 - VERIFY the electrical stops for the applicable Refueling Platform Auxiliary Hoist have been set IAW HC.OP-ST.KE-0001(Q), Refuel Interlock Operability Functional Test - Section 5.4 for the Combined CRB/FSP Grapple.
 - 2. **LOWER** the grapple to greater than 7 feet below the surface of the water to clear the hoist override.
 - 3. **RAISE** the grapple to reach the electrical stops. (The mechanical jam may have to be lowered to allow this).
 - LOOSEN the bolts on the mechanical jam <u>AND</u> SLIDE the jam up UNTIL it just touches the limit switch LS4 (LS4 is a finger switch just below the mechanical jam entry point).

5. **TIGHTEN** the bolts.

Continued next page

HC.OP-SO.KE-0001(Q)

5.9.6 (continued)

<u>NOTE</u> 5.9.6.J

Overhead Crane should be used for lifting Combined Grapple with grid guide installed.

- J. To install Combined Grapple Grid Guide **PERFORM** the following:
 - 1. **PLACE** guide on mounts at top of grapple WITH the "L" latches pointing towards hoist cable.
 - SEAT guide on mounts
 <u>AND</u> while pushing down on T-handle,
 <u>TURN each latch 90° (one turns clockwise</u>
 and one turns counter clockwise).
 - 3. **LIFT-UP** on guide to verify it is latched to grapple.
 - PLACE hoist cable within the rollers on the top of the guide <u>THEN</u> CLOSE <u>AND</u> PIN gate with ball lock pin.

S

HC.OP-SO.KE-0001(Q)

6.0 **<u>REFERENCES</u>**

6.1 <u>GE Documents</u>:

GEK 75563, Refueling Platform Equipment Assembly GEK 33076A GEK 33147C GEK 33221A GEK 33143D GEK 33073B GEK 103853A

6.2 <u>Commitment Documents</u>

CD-069A	IE BULLETIN 78-08
CD-442A	INPO SER 59.81
CD-719A	GE SIL 152
CD-124B	SER 31-83
CD-182C	NRC C-77-12
CD-281C	INPO SER 28-84
CD-365D	GE SIL 409
CD-673D	INPO OE 1239
CD-736D	NRC IN 85-12
CD-789D	GE AID 55
CD-123E	INPO OE 1470
CD-711F	H-1-KE-SEE-0332
CD-739F	GE SIL 486
CD-443X	F09-0025-00
CD-612X	F01-0091-01
CD-267Y	FSAR Question 410.59
CD-396Y	DSER 142 A&B
CD-217E	INPO OE 1556
CD-117G	NHO INCI 354/91-166
CD-619G	NHO INCI 354/94-049
CD-451X	FSAR - Mechanical Jam, Stop Block - Backups to Normal Up Limit
	Section 9.1
CD-949A	GE SIL 82 R1 S1
CD-168A	IE Cir 80-21

6.3 DCP No. 4-HM-0643 DCP 80016207

Continued next page

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

6.0 (continued)

6.4 **Procedures**

HC.OP-ST.KE-0001(Q) - Refuel Interlock Operability Test HC.OP-FT.KE-0002(Q); Fuel Grapple Full Down Travel Functional Test – Refueling HC.OP-FT.KE-0001(Q) - Refuel Platform and Fuel Grapple Operability Test HC.RE-FR.ZZ-0001(Q) - Refuel Handling Controls HC.OP-AP.ZZ-0046(Q) - Radiological Access Control Program HC.OP-AB.CONT-0005(Q) - Irradiated Fuel Damage NC.NA-AP.ZZ-0049(Q) - Conduct of Fuel Handling

6.5 Letters

HCGS-L-95-036

6.6 Other Documents

PR 960821103 GE SIL 82 R1 S1 PR 960128076 Order 70007921 Order 70006867

ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

ATTACHMENT 1 Page 1 of 1 ANALYZED LOADS PERMITTED OVER IRRADIATED FUEL NOT REQUIRING SECONDARY CONTAINMENT INTEGRITY

NOTE

70 feet relates to 17 feet above the surface of the water.

1. Fuels analysis is for an object ≤ 117 lbs. <u>WITH</u> a maximum distance of 70 ft from the load to the top of irradiated fuel in the Reactor Vessel.

REFERENCE: NFS 2000-069 Calculation File T03.6-068, Light loads limits over irradiated fuel 14 Dec 1990 NFU-90-734

HC.OP-SO.KE-0001(Q)

ATTACHMENT 2 Page 1 of 4 INSTRUCTIONS FOR MANUALLY LOWERING A FUEL BUNDLE

During fuel handling, various scenarios depict the need to lower a fuel bundle to a safe location in the Core or Fuel Pool. These scenarios may include the loss of power to the Bridge or malfunctions in other parts of the Bridge or the plant.

Except in emergency (evacuation, injury, etc.) <u>IF</u> the Bridge malfunctions while fuel is grappled, **NOTIFY** the cognizant Refuel SRO <u>OR</u> OS/CRS <u>before</u> attempting any corrective action.

The purpose of this Attachment is to place the Fuel Assembly in the nearest appropriate storage location to allow the Bridge malfunction to be safely corrected. The cognizant SRO <u>AND</u> Reactor Engineer will determine the appropriate storage location.

1.0 In the event of a Loss of Power to the Bridge or Hoist Motor Malfunction (burnout, short)

1.1 IF the power is lost, the Hoist electric motor brake and emergency brake will engage.

<u>NOTE</u> 1.2

The Bridge/Trolley movement should be the minimum required to reach an appropriate location.

- 1.2 **MOVE** the Bridge/Trolley to an appropriate cell location using the handwheels provided for the Bridge Drive and Trolley Drive.
- 1.3 **VERIFY** the bundle is over an appropriate cell location.

<u>NOTE</u> 1.4

DO NOT drop the handwheel while installing it on the shaft.

1.4 **ATTACH** the handwheel on the West end of the Hoist Motor shaft AND **SECURE** it. ALL ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

ATTACHMENT 2 Page 2 of 4 INSTRUCTIONS FOR MANUALLY LOWERING A FUEL BUNDLE

1.5 <u>WITH</u> one person holding the handwheel <u>AND</u> ready to lower the bundle, another person should release the emergency (safety) brake using the manual release on the brake

<u>NOTE</u> 1.6

The handwheel is marked with a direction indicator.

1.6 **TAKE UP** on the handwheel until the bundle starts to raise (approximately 3" bundle upward travel). This assures the handwheel has the load.

CAUTION 1.7

Step 1.7 will immediately transfer the bundle weight to the handwheel.

- 1.7 While holding the handwheel, release the motor brake by pushing the lever located inside the East end of the motor housing down and to the side. This permits lowering the bundle with the handwheel.
- 1.8 The cover is left off for the motor brake housing to preclude the need for tools in the case of an emergency.

ĂLC ACTIVE ON-THE-SPOT CHANGES MUST BE ATTACHED FOR FIELD USE 20030122

PSEG Internal Use Only

HC.OP-SO.KE-0001(Q)

ATTACHMENT 2 Page 3 of 4 INSTRUCTIONS FOR MANUALLY LOWERING A FUEL BUNDLE

<u>NOTE</u> 2.0

DO NOT attempt to lower or release the bundle without SRO approval if a bent Mast is encountered.

2.0 Bent Mast

<u>IF</u> the Mast is bent while it is loaded with a Fuel Bundle, its ability to extend and safely "put down" the Fuel Bundle may be impaired. Attempting to lower the load electrically may unspool the cable and allow an uncontrolled descent. A "slack cable" indication may also be received do to Mast Section(s) binding. Unless in the case of an emergency, SRO permission is required prior to attempting to move after binding the mast.

Any of the refueling bridge mast sections (10", 7", or 5"), can bind while being extended or retracted. The Encoder readout (distance traveled), used in conjunction with the Load Cell readout (weight on main mast cable), can be used to effectively verify whether binding is occurring. For example, IF you start from a full up position (10", 7", and 5" sections fully retracted), and start lowering the mast sections while observing the Encoder and Load Cell indications, the weight of the 10" section should transfer from the main mast cable to the 12" section at approximately 180" on the Encoder reading (the Load Cell indication will decrease by approximately 105#). Continuing down, the weight of the 7" section should transfer from the main mast cable to the 10" section at approximately 380" on the Encoder reading (the Load Cell indication will decrease by approximately 88#). This methodology can also be applied for use in the opposite direction. IF while lowering the mast, an unexpected slack cable or pre-mature weight transfer occurs, stop lowering, raise the load to clear the slack cable (if one exists), and notify the cognizant Refuel SRO or OS/CRS before attempting any corrective action. [CD-619G]

HC.OP-SO.KE-0001(Q)

ATTACHMENT 2 Page 4 of 4 INSTRUCTIONS FOR MANUALLY LOWERING A FUEL BUNDLE

<u>NOTE</u> 3.0

DO NOT attempt to lower or release the bundle without SRO approval if Platform failure is encountered.

3.0 Platform Failure

A Platform failure includes Hoist Gear Train failure, Coupling failure or related structure failure. As these components affect the ability to lower a Fuel Bundle (either electrically or manually), unless in the case of an emergency, SRO approval is required prior to an attempt to move or lower the Fuel Bundle.





Hope Creek

.

Rev. 28



ATTACHMENT 1 FUEL MOVEMENT SHEET

REFUEL NO. <u>N/A</u> SHEET NO. <u>1</u>

CONTINUED ON SHEET _____N/A

Step	I.D.	FROM	Orientation	Time	TO		Time	Operator	Verified By
	Number	Location	Orientation	Date	Location	Orientation	Date	Initials	Initiais
01	DUMMY	SFP	ANY			SFP			
		AD - 28				AH - 50			
02	DUMMY	SFP	ANY			SFP			
		AH - 50				AD - 28			

Prepared By

Date

Independently Verified By

Date

NC.TQ-WB	.ZZ-0310(Z)
----------	-------------

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

STATION:	HOPE CREEK	
SYSTEM:	Refueling Platform	
JPM NUMBER:	2003-NRC-S4	
TASK:	Perform the Monorail Auxiliary Hoist Controls Functional Tes	t
TASK NUMBER:	N2340010204	
ALTERNATE PA	TH: N/A K/A NUMBER: IMPORTANCE FACTOR:	234000 A3.01 2.6 3.6
APPLICABILITY	EO RO STA SROX LSROX]
EVALUATION S	ETTING/METHOD: Walkthrough/Simulate (Perform)	
REFERENCES:	HC.OP-FT.KE-0001(Q), Rev. 15	
TOOLS AND EG	UIPMENT: Refueling Platform/None	
VALIDATED JP	M COMPLETION TIME: 20 Min.	
	DENTIFIED FOR TIME CRITICAL STEPS: N/A Min	n.
APPROVAL:	N/A	OPERATIONS
	BARGAINING UNIT TRAINING SUPERVISOR REPRESENTATIVE	MANAGER or Designee
CAUTION:	 No plant equipment shall be operated during the perform following: 1. Permission from the OS or Unit CRS; 2. Direct oversight by a qualified individual (determined permission based on plant conditions). 3. Verification of the "as left" condition by a qualified individual (determined permission based on plant condition by a qualified individual). 	mance of a JPM without the d by the individual granting ndividual.
ACTUAL JPM	COMPLETION TIME: Min.	
ACTUAL TIME	CRITICAL COMPLETION TIME: N/A Min.	
JPM PERFORM	IED BY: GRADE:	
REASON, IF UI	NSATISFACTORY:	
ا بذvaluator's	SIGNATURE:	DATE:

ŝ.

3

ŝ

NC.TQ-WB.ZZ-0310(Z)

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: _____

DATE: _____

SYSTEM: Refueling Platform

JPM NUMBER: 2003-NRC-S4

TASK: Perform the Monorail Auxiliary Hoist Controls Functional Test

TASK NUMBER: N2340010204

INITIAL CONDITIONS:

- 1. HC.OP-FT.KE-0001 is in progress.
- 2. Provide operator with marked up copy of HC.OP-FT.ZZ-0001.

INITIATING CUE:

Perform the Monorail Auxiliary Hoist Controls Functional Test IAW HC.OP-FT.KE-0001 Section 5.4.

Successful Completion Criteria:

All critical steps completed.

All sequential steps completed in order.

All time-critical steps completed within allotted time.

JPM completed within validated time. Completion time may exceed the validated time if satisfactory progress is being made.

JPM NUMBER: 2003-NRC-S4

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: ____ DATE: ____

SYSTEM: Refueling Platform

TASK: Perform the Monorail Auxiliary Hoist Controls Functional Test IAW HC.OP-FT.KE-0001.

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
		Operator obtains/locates procedure HC.OP-FT.KE-0001	Operator obtains the correct procedure		
		Operator reviews precautions and limitations	Operator views precautions and limitations Examiner Cue: If excessive time is taken reviewing precautions and limitations, inform operator that all are satisfied.		
		Operator determines beginning step of procedure	The operator determines the correct beginning step of the procedure to be 5.4.1.		
		<u>NOTE 5.4.</u> IF defective or inoperative equipment is discovered while conducting this test a Notification should be generated which directs equipment repair.	Operator reads Note.		

_-0310(Z)

NC. Q-W



OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: ______ DATE: _____

NC.TQ-V

_-0310(Z)

SYSTEM: Refueling Platform

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
		<u>CAUTION</u> 5.4 During movement of the Refuel Platform ensure that the Fuel Grapple is not damaged by impact with the Fuel Pool walls or underwater obstructions.	Operator reads Caution.		
*	5.4.1	START TIME At the Right Hand Controller, TURN the CONSOLE SELECT switch to MONO.	 Examiner Note: Initialing Attachment 2 in the following steps is not critical. Operator turns the Right Hand Controller CONSOLE SELECT switch to MONO. \$ Examiner Cue: The switch that you indicated is in the position stated. 		
*	5.4.2	On the Monorail Pendant, PRESS the BRIDGE FORWARD pushbutton, VERIFY the following <u>AND</u> INITIAL Attachment 2.	Operator presses the Monorail Pendant Bridge Forward pushbutton to maneuver the Monorail Hoist. \$ Examiner Cues: The pushbutton indicated is in the position stated .		
*		A. Refuel Platform moves North	Refuel Platform moves in direction indicated.		
		B. Speed control is smooth and continuous.	Speed control is smooth and continuous.		
		C. Refuel Platform movement alarm is sounding.	Refuel Platform movement alarm is sounding.		
		D. VERIFY Bridge motion from only this station.	Bridge motion from only this station.		

JPM NUMBER: 2003-NRC-S4

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: _____ DATE: _____

NC.TQ-W

O DALLENITO

_-0310

SYSTEM: Refueling Platform

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	(Required for UNSAT evaluation)
*	5.4.3	RELEASE the BRIDGE FORWARD	Operator releases FORWARD pushbutton.		
		VERIFY the following AND INITIAL Attachment 2.	\$ Examiner Cues:		
		A. Refuel Platform movement stops.	Bridge stops.		
		B. Refuel Platform movement alarm is silent.	Alarm is quiet.		
*	5.4.4	PRESS the BRIDGE REVERSE pushbutton, VERIFY the following <u>AND</u> INITIAL Attachment 2.	Operator presses the Monorail Pendant Bridge REVERSE pushbutton. \$ Examiner Cues: The pushbutton indicated is in the position stated.		
*		A. Refuel Platform moves south.	Refuel Platform moves in direction indicated.		
		B. Speed control is smooth and continuous.	Speed control is smooth and continuous.		
		C. Refuel Platform movement alarm is sounding.	Refuel Platform movement alarm is sounding.		
*	5.4.5	RELEASE the BRIDGE REVERSE pushbutton, VERIFY the following <u>AND</u> INITIAL Attachment 2.	Operator releases REVERSE pushbutton. \$ Examiner Cues: The pushbutton indicated is in the position stated.		
-		A. Refuel Platform motion stops.	Refuel Platform motion stops.		



OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: ______ DATE: _____

SYSTEM: Refueling Platform

TASK: Perform the Monorail Auxiliary Hoist Controls Functional Test IAW HC.OP-FT.KE-0001.

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
		B. Refuel Platform movement alarm is silent.	Refuel Platform movement alarm is silent.		
*	5.4.6	PRESS the TROLLEY RIGHT pushbutton, VERIFY the following AND INITIAL Attachment 2.	Operator presses the TROLLEY RIGHT pushbutton. \$ Examiner Cues: The pushbutton		
			indicated is in the position stated.		
*		A. Trolley moves west.	Refuel Platform moves in direction indicated.		
		B. Speed control is smooth and continuous.	Speed control is smooth and continuous.		
*	5.4.7	RELEASE the TROLLEY RIGHT pushbutton, VERIFY the Trolley motion stops AND INITIAL Attachment 2.	Operator releases the TROLLEY RIGHT pushbutton. \$ Examiner Cue: Trolley motion stops.		
*	5.4.8	PRESS the TROLLEY LEFT pushbutton, VERIFY the following AND INITIAL Attachment 2. A. Trolley moves east.	Operator presses the TROLLEY LEFT pushbutton. \$ Examiner Cues: The pushbutton indicated is in the position stated. Refuel Platform moves in direction indicated.		

NC.TQ-W __-0310(Z)

JPM NUMBER: 2003-NRC-S4

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: ______ DATE: _____

SYSTEM: Refueling Platform

TASK: Perform the Monorail Auxiliary Hoist Controls Functional Test IAW HC.OP-FT.KE-0001.

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
		B. Speed control is smooth and continuous.	Speed control is smooth and continuous.		
*	5.4.9	RELEASE the TROLLEY LEFT pushbutton, VERIFY the Trolley motion stops AND INITIAL Attachment 2.	Operator releases the TROLLEY LEFT pushbutton. \$ Examiner Cue: Trolley motion stops		
*	5.4.10	PRESS the HOIST LOWER pushbutton, AND LOWER the grapple to greater than 7 feet below the surface of the water to clear the hoist override.	Operator presses the HOIST LOWER pushbutton. The hoist cable moves downward at least 6 inches below the Normal Up stop.		
		VERIFY the following AND INITIAL Attachment 2	\$ Examiner Cues: The pushbutton indicated is in the position stated.		
*		A. Monorail Hoist cable moves downward.	Monorail Hoist cable moves in the direction indicated.		
		B. Speed control has high and low speed and is smooth and continuous.	Speed control has high and low speed and is smooth and continuous.		
		A slight depression on the pushbutton will produce a slow speed.	Hoist runs slow.		
		• A full depression on the pushbutton will produce a slow speed for a short period of time then switch to fast speed.	Hoist runs slow then fast.		

NC.TQ-W _-0310(Z)



OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: _____ DATE: _____

NC.TQ-W

_-0310

SYSTEM: Refueling Platform

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
*	5.4.11	RELEASE the HOIST LOWER pushbutton, VERIFY the hoist cable motion stops AND INITIAL Attachment 2.	Operator releases HOIST LOWER pushbutton. \$ Examiner Cue: Hoist cable motion stops.		
*	5.4.12	PRESS the HOIST RAISE pushbutton, VERIFY the following AND INITIAL Attachment 2.	Operator presses HOIST RAISE pushbutton. \$ Examiner Cues: The pushbutton indicated is in the position stated.		
*		A. Monorail Hoist cable moves upward.	Monorail Hoist cable moves in direction indicated.		
		B. Speed control has high and low speed and is smooth and continuous	Speed control has high and low speed and is smooth and continuous.		
		 A slight depression on the pushbutton will produce a slow speed. 	Hoist moves at a slow speed.		
		 A full depression on the pushbutton will produce a slow speed for a short period of time then switch to fast speed. 	Hoist moves at a slow speed for a short period of time then switch to fast speed.		



OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: ______ DATE: _____

NC.TQ-W

∠-0310(Z)

SYSTEM: Refueling Platform

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
*	5.4.13	RELEASE the HOIST RAISE pushbutton before it reaches the Normal Up Rotary electrical stop, VERIFY the hoist cable motion stops AND INITIAL Attachment 2.	Operator releases HOIST RAISE pushbutton before the hoist reaches the Normal Up stop. \$ Examiner Cue: Hoist cable motion stops below the Normal Up stop.		
*	5.4.14	PRESS the HOIST RAISE pushbutton to raise the hoist cable to the Normal Up Rotary electrical stop. VERIFY the hoist cable motion automatically stops AND INITIAL Attachment 2.	Operator presses HOIST RAISE pushbutton until hoist automatically stops. \$ Examiner Cue: The pushbutton indicated is in the position stated. Hoist cable rises until automatically stops.		

JPM NUMBER: 2003-NRC-S4

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: ______

NC.TQ-W

_-0310(Z)

SYSTEM: Refueling Platform

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
		NOTE 5.4.15 The Mechanical Jam Stop may be moved down on the cable to check the MAX UP ROTARY Switch. HC.OP-SO.KE-0001(Q) should be referred to for stop setup.	Operator reads Note. Examiner Cue: Moving the Mechanical Jam Stop is not required. Examiner Note: Depending where the Mechanical Jam Stop is positioned, simulation of step 5.4.15 may be required.		
*	5.4.15	Simultaneously PRESS the HOIST LIMIT OVERRIDE AND HOIST RAISE pushbuttons. VERIFY the Monorail Hoist cable moves upward AND automatically stop at the Maximum Up Rotary limit switch cut-out AND INITIAL Attachment 2.	Operator simultaneously presses the HOIST LIMIT OVERRIDE and HOIST RAISE pushbutton until hoist automatically stops. \$ Examiner Cue: The pushbutton indicated is in the position stated. The Hoist cable rises until automatically stops. Examiner Cue: Another refueling crew will finish the remaining steps of the procedure.		

JPM NUMBER: 2003-NRC-S4

OPERATOR TRAINING PROGRAM JOB PERFORMANCE MEASURE

NAME: _____

NC.TQ-W

-0310

SYSTEM: Refueling Platform

# *	STEP NO.	STEP (*Denotes a Critical Step) (#Denotes a Sequential Step)	STANDARD	EVAL S/U	COMMENTS (Required for UNSAT evaluation)
		STOP TIME:	Terminating Cue: Repeat back message from the operator on the status of the JPM, and then state " This JPM is complete ".		

NC.TQ-WB.ZZ-0310(Z)

JOB PERFORMANCE MEASURE STUDENT INFO SHEET

ITIAL CONDITIONS:

1. HC.OP-FT.KE-0001 is in progress.

INITIATING CUE: