

April 24, 2003

Mr. David L. Wilson
Site Vice President
Monticello Nuclear Generating Plant
Nuclear Management Company, LLC
2807 West County Road 75
Monticello, MN 55362-9637

SUBJECT: MONTICELLO NUCLEAR GENERATING PLANT - REQUEST FOR ADDITIONAL
INFORMATION RELATED TO RELIEF REQUEST NO. 5 (TAC NO. MB6956)

Dear Mr. Wilson:

Nuclear Management Company's letter of December 6, 2002, submitted Relief Request No. 5 associated with the fourth 10-year interval inservice inspection examination plan for the Monticello Nuclear Generating Plant. The U.S. Nuclear Regulatory Commission staff finds that it needs additional information to complete its review, as indicated in the enclosure to this letter.

I discussed the enclosed request for additional information with Mr. J. Fields of your organization on April 22, 2003. We agreed that you would respond to this request by April 28, 2003. If you have questions or need to revise this date, please contact me at (301) 415-1423.

Sincerely,

/RA/

L. Mark Padovan, Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosure: Request for Additional Information

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION
RELATED TO INSERVICE INSPECTION RELIEF REQUEST NO. 5
NUCLEAR MANAGEMENT COMPANY (NMC), LLC
MONTICELLO NUCLEAR GENERATING PLANT
DOCKET NO. 50-263

1. As a basis for Request for Relief No. 5 dated December 6, 2002, NMC stated that the control rod drive housing joints are VT-2 examined as part of the periodic reactor pressure vessel leakage and hydrostatic pressure tests. These tests are conducted with the vessel temperature much less than the design operating temperature. For a typical test, the vessel temperature would be <212 degrees F, as compared to a normal operating temperature of about 540 degrees F.

Please confirm that NMC is following the American Society of Mechanical Engineers Code, Section XI, IWA-5000, IWB-5240 temperature requirements when performing periodic reactor pressure vessel leakage and hydrostatic pressure tests.

2. Please provide the numbers and results of your inspections of the control rod drive mechanism (CRDM) bolting. How many of new-type bolts, as required by General Electric Services Information Letter No. 483, were installed in the CRDMs?
3. Please provide the material of the bolts and environmental conditions that these bolts are subject to during operation? Also, what is the degradation mechanism of these bolts?
4. How does NMC determine that leakage from the CRDMs has stopped when the operating pressure and temperature is reached?

Monticello Nuclear Generating Plant

cc:

Jonathan Rogoff, Esquire
General Counsel
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March 2003