April 18, 2003

Mr. J. B. Beasley, Jr.
Vice President - Farley Project
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 RE: REVIEW OF STEAM GENERATOR TUBE INSERVICE INSPECTION REPORT FOR REFUELING OUTAGE 1R17 (FALL 2001) (TAC NO. MB6476)

Dear Mr. Beasley:

By letter dated October 22, 2001, as supplemented by letter dated January 24, 2002, Southern Nuclear Operating Company submitted the 90-day inservice inspection summary report summarizing the steam generator tube inspections performed at Joseph. M. Farley Nuclear Plant, Unit 1 (Farley-1) during refueling outage 1R17 (fall 2001).

As discussed in the enclosed evaluation, the staff has concluded that you have provided the information required by the Technical Specifications for Farley-1, and that no additional follow-up is required at this time. If you have any questions, contact me at (301) 415-1447.

Sincerely,

/RA/

Frank Rinaldi, Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-348

Enclosure: As stated

cc w/encl: See next page

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NAME	FRinaldi	CHawes	JNakoski
DATE	4/16/03	4/15/03	4/17/03

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NUCLEAR REGULATORY COMMISSION STAFF'S REVIEW

OF THE STEAM GENERATOR 90-DAY REPORT

SOUTHERN NUCLEAR OPERATING COMPANY, INC., ET AL.

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

DOCKET NO. 50-348

By letter dated October 22, 2001, and supplemented by letter dated January 24, 2002, the Southern Nuclear Operating Company (the licensee) submitted summaries of the steam generators (SG) tube inspections performed at the Joseph M. Farley Nuclear Plant, Unit 1 (Farley-1) during refueling outage U1R17 (fall 2001). Also, the staff has supplemented the SG inspection summaries with information submitted by the licensee in support of a technical specifications amendment request regarding SG inspection frequency (ML020650389 and ML021960109).

The licensee has replaced Farley-1 original SGs during refueling outage U1R16 (spring 2000). The SG design now consists of three Westinghouse Model 54F recirculating SGs. Each of the SGs contain 3592 tubes fabricated from thermally treated alloy-690 material. The tubes have an outside diameter of 0.875-inch, a wall thickness of 0.050-inch, and are supported by 7 stainless steel tube support plates with quatrefoil broached holes.

The licensee implements SG inspection activities according to its technical specifications, Nuclear Energy Institute (NEI) document NEI 97-06, "Steam Generator Program Guidelines," and Electric Power Research Institute report, "PWR Steam Generator Examination Guideline, Revision 5." Refueling outage U1R17 was the first SG tube inspection following the SG replacement. The inspection scope consisted of 100 percent eddy current testing of all in-service tubes over their full length in all three SGs. Eddy current testing was performed predominantly using a bobbin probe. A rotating pancake coil probe equipped with a +point probe was used to inspect 100 percent of the low row (row 1 and 2) u-bends, 20 percent of the hot leg top of the tubesheet region, and all bobbin signals that exhibited a change since the pre-service inspection or that were not present during the pre-service inspection in each of the SG. Thirty-seven indications were identified during the inspection with the bobbin probe that required inspection with the +point probe in all three SGs.

There were no tubes plugged and there were no axial or circumferential crack-like indications reported in any of the three SGs. Also, no indication of wear or service-induced wall reduction were detected.

Enclosure

Two potential loose part signals were identified in SG-C using the +point probe during the hot leg top of the tubesheet portion of the inspection. The licensee performed visual examinations, but was unable to confirm the existence of the objects at the identified locations. Inspection of the affected tubes and the adjacent tubes, using the +point probe, did not reveal any wall loss or indications of wear.

A number of small and predominantly non-metallic parts were removed from all three SG during sludge lancing. In addition, one metal shaving and one piece of wire were removed from SG-B, while a nail was removed from SG-C. The licensee's submittal made no correlation between the nail removed from SG-C during sludge lancing and the possible-loose-parts identified during +point probe inspection.

Based on the review of the information provided by the licensee, the staff concludes that no additional follow-up is necessary at this time.

Principal Contributor: M. Murphy, DE/EMCB

Date:

Joseph M. Farley Nuclear Plant

cc:

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