

CF

November 15, 1982

ALL BOILING WATER REACTOR LICENSEES OF OPERATING REACTORS (EXCEPT LACROSSE, BIG ROCK POINT, DRESDEN 1, HUMBOLDT BAY) APPLICANTS FOR AN OPERATING LICENSE AND HOLDERS OF CONSTRUCTION PERMITS. (GENERIC LETTER NO. 82-27 )

Gentlemen:

SUBJECT: TRANSMITTAL OF NUREG-0763, "GUIDELINES FOR CONFIRMATORY IN-PLANT TESTS OF SAFETY-RELIEF VALVE DISCHARGES FOR BWR PLANTS," AND NUREG-0783, "SUPPRESSION POOL TEMPERATURE LIMITS FOR BWR CONTAINMENTS."

Enclosed are copies of NUREGs-0763 and 0783. These documents are being issued to provide guidance that the NRC staff believes should be followed to meet the requirements of General Design Criteria 16 and 29 of Appendix A to 10 CFR Part 50. These NUREGs are not a substitute for the regulations, and compliance is not a requirement. However, an approach or method different from the guidance contained herein will be accepted if the substitute approach or method provides a basis for determining that the requirements of General Design Criteria 16 and 29 have been met.

NUREG-0763 provides guidance for: (1) determining if in-plant safety relief valve (SRV) tests are required, (2) formulating appropriate test matrices, (3) establishing test procedures, (4) selecting necessary instrumentation, and (5) reporting test results. Implementation of the guidelines is described in Section 9 of the report. It should be noted, however, that the guidelines are intended for providing information for SRV in-plant tests which are still in the planning stage.

If the licensees/applicants have performed or installed instrumentation for SRV in-plant tests in accordance with the acceptance criteria specified in NUREG-0661, "Mark I Containment Long Term Program," we find this approach acceptable.

NUREG-0783 provides: (1) acceptance criteria related to the suppression pool temperature limits, (2) the events required to analyze the suppression pool temperature response, (3) assumptions used for the analyses, and (4) requirements for the suppression pool temperature monitoring system. Implementation of this issue is described in Section 6 of the report. The acceptance criteria specified in the report can be considered a relaxation of the existing suppression pool temperature limit criteria which are specified in NUREG-0661 (Mark I) and NUREG-0487 "Mark II Containment Lead Program Load Evaluations and Acceptance Criteria."

The application, reporting and record-keeping requirements contained in NUREG-0763 have been approved by the Office of Management and Budget: OMB approval No. 3150-0074 which expires July 31, 1983.

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The application, reporting and record-keeping requirements contained in NUREG-0783 have been approved by the Office of Management and Budget; OMB Approval No. 3150-0082 which expires December 31, 1982.

Original signed by  
Darrell G. Eisenhut

Darrell G. Eisenhut, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

Enclosures as stated *see jacket*  
cc Service List w/o enclosures

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Shanover  
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SURNAME	SNORTIS	BStegbe:POB:MC	DVassallo	GLainas	DEEisenhut	AD:L	AD:SA
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