C. Files

2/3/81

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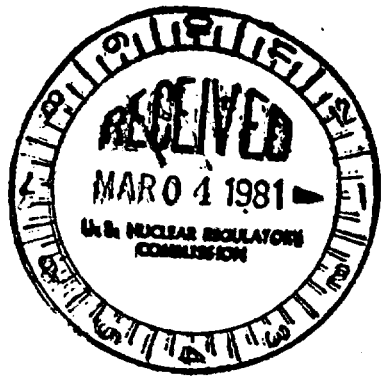

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 3, 1981

TO ALL LICENSEES OF OPERATING PLANTS AND APPLICANTS FOR OPERATING LICENSES AND HOLDERS OF CONSTRUCTION PERMITS*

SUBJECT: CONTROL OF HEAVY LOADS (Generic Letter 81-07)

Gentlemen:

By our letter dated December 22, 1980, you were requested to review your controls of the handling of heavy loads to determine the extent to which the guidelines of NUREG-0612 are presently satisfied at your facility and to identify the changes and modifications that would be required in order to fully satisfy these guidelines.

To expedite your review, three enclosures were included with the letter. One of the enclosures was Request for Additional Information on Control of Heavy Loads (Enclosure 3). We have found that five pages from Enclosure 3 were missing due to a reproduction error. The missing pages are enclosed with this letter. In addition the December 22, 1980, letter on Page 2 in Item 1 required that information identified in Section 2.1 through 2.4 of Enclosure 3 be included in a report documenting the results of your review. This requirement should be modified to read: "Sections 2.1 through 2.4 for PWR plants and Sections 2.1 through 2.3 for BWR plants."

Because of these errors we are extending the Enclosure 2 90-day implementation requirement to May 15, 1981.

Sincerely,

Darrell G. Eisenhut, Director
Division of Licensing

Enclosure:
"Enclosure 3" missing pages

*With the exception of licensees for Indian Point 2 and 3, Zion 1 and 2 and Three Mile Island 1

ANALYSIS OF PLANT STRUCTURES

The following information should be provided for analyses conducted to demonstrate compliance with Criteria III and IV of NUREG 0612, Section 5.1.

1. INITIAL CONDITIONS/ASSUMPTIONS

Discuss the assumptions used in the analysis, including:

a. Weight of heavy load
b. Impact area of load
c. Drop height
d. Drop location
e. Assumptions regarding credit taken in the analysis for the action of impact limiters
f. Thickness of walls or floor slabs impacted
g. Assumptions regarding drag forces caused by th environment
h. Load combinations considered
i. Material properties of steel and concrete

2. METHOD OF ANALYSIS

Provide the method of analysis used to demonstrate that sufficient load-carrying capability exists within the wall(s) or floor slab(s). Identify any computer codes employed, and provide a description of their capabilities. If test data was employed, provide it and describe its applicability.

3. CONCLUSION

Provide an evaluation comparing the results of this analysis with Criteria III and IV of NUREG 0612, Section 5.1. Where safe-shutdown equipment has a ceiling or wall separating it from an overhead handling system, provide an evaluation to demonstrate that postulated load drops do not penetrate the ceiling or cause secondary missiles that could prevent a safe-shutdown system from performing its safety function.

(3) A description of any Engineered Safety Feature filter system which includes information sufficient to demonstrate compliance with the guidelines of USNRC Regulatory Guide 1.52, "Design, Testing, and Maintenance Criteria for Engineered Safety Feature Atmosphere Cleanup System Air Filtration and Absorption Units of Light-Water-Cooled Nuclear Power Plants."

(4) A discussion of any initial conditions (e.g., manual valves locked shut, containment airlocks or equipment hatches shut) necessary to ensure that releases will be terminated or mitigated upon Engineered Safety Feature actuation and the measures employed (i.e., Technical Specification and administrative controls) to ensure that these initial conditions are satisfied and that Engineered Safety Feature systems are operable prior to the load lift.

2. METHOD OF ANALYSIS

Discuss the method of analysis used to demonstrate that post-accident dose will be well within 10CFR100 limits. In presenting methodology used in determining the radiological consequences, the following information should be provided.

a. A description of the mathematical or physical model employed.

b. An identification and summary of any computer program used in this analysis.

c. The consideration of uncertainties in calculational methods, equipment performance, instrumentation response characteristics, or other indeterminate effects taken into account in the evaluation of the results.

3. CONCLUSION

Provide an evaluation comparing the results of the analysis to Criterion I of NUREG 0612, Section 5.1. If the postulated heavy-load-drop accident analyzed bounds other postulated heavy-load drops, a list of these bounded heavy loads should be provided.

bounds other postulated heavy-load drops, a list of these bounded heavy loads should be provided.

SHIELDED SHIPPING CASKS CERTIFICATED FOR NUCLEAR POWER PLANTS

II - Waste

CERT.	MODEL	PRIMARY LICENSEE	GROSS LOT IN LBS. (APPROX.)	SECONDARY LICENSEE*
6744	Poly Tiger	Nuclear Engineering Co.	35,000	APL, BEC, CPC, DLP, MEC, NPP, SMU, VEP
6771	SN-1	Nuclear Engineering Co.	60,000	APL, CPC, DLP, NPP, SMU, VEP
9074	AP-100		28,000	DLC
9079	EN-100 Ser. 2	Hittman Nuclear and Development Corp.	98,000	APL, BGE, CEC, CWE, DLP, IME, JCP, MYA, MEC, NPP, PEC
9080	EN-600	Hittman Nuclear and Development Corp.	42,000	BGE, CWE, CEC, DLP, IME, IEL, JCP, MYA, MEC, NPP, PEC, YAC
9086	EN-100 Ser. 1	Hittman Nuclear and Development Corp.	46,000	APL, BGE, CWE, DLP, IME, JCP, MYA, MEC, NPP, NNE, PEC, RGE, VYC
9089	EN-100S	Hittman Nuclear and Development Corp.	36,500	BGE, CWE, CEC, IME, JCP, MYA, NPP, PEC
9092	EN-300	Hittman Nuclear and Development Corp.	43,000	MYA
9093	EN-400	Hittman Nuclear and Development Corp.	43,000	MYA
9094	CNSI-14-195-H	Chem-Nuclear Systems, Inc.	56,500	APC, APL, BEC, CPL, CWE, CYA, CEC, CPC, DPC, FPL, FPC, GPC, JCP, MEC, NMP, NNE, NSP, OPP, PGE, PEC, PGC, PNY, PEG, TVA, VEP
9096	CNSI-21-300	Chem-Nuclear Systems, Inc.	57,450	APC, APL, CPL, CEC, DPC, FPL, FPC, GPC, JCP, MEC, NMP, NNE, PNY, PEG, VEP

*See attached list of abbreviations.

SHIELDED SHIPPING CASKS CERTIFICATED FOR NUCLEAR POWER PLANTS

III - Byproducts

CERT.	MODEL	PRIMARY LICENSEE	GROSS LOT IN LBS. (APPROX.)	SECONDARY LICENSEE*
5971	GE-200		10,000	PEC
5980	GE-600		18,500	NNE, NSP
6275	LL-28-4	Chem-Nuclear Systems, Inc.	30,000	APC, CPL, DPC, FPL, FPC, NPP, VEP
9081	CNS-1600	Chem-Nuclear Systems, Inc.	26,000	APC, BGE, CPL, DPC, FPL, FPC, GPC, NSP, TVA, VEP

*See attached list of abbreviations.