

April 9, 2003

Mr. Valeri Tolstykh
Regulatory Activities Unit
Safety Assessment Section
Division of Nuclear Installation Safety
International Atomic Energy Agency
Wagramer Strasse 5
P.O. Box 100, A-1400
Vienna, Austria

Dear Mr. Tolstykh:

The following operating experience reports from United States reactors are enclosed for your consideration for including in the AIRS database:

NRC INFORMATION NOTICE 2003-01: FAILURE OF A BOILING WATER REACTOR
TARGET ROCK MAIN STEAM SAFETY/RELIEF VALVE

NRC INFORMATION NOTICE 2003-02: RECENT EXPERIENCE WITH REACTOR COOLANT
SYSTEM LEAKAGE AND BORIC ACID CORROSION

NRC INFORMATION NOTICE 2003-03: INADEQUATELY STAKED CAPSCREW RENDERS
RESIDUAL HEAT REMOVAL PUMP INOPERABLE

Each report is being submitted in the following two media: (1) a hard copy of the input file for the AIRS database; and (2) a 3.5-inch HD diskette containing the input file for the AIRS database in Wordperfect format.

Mr. Valeri Tolstykh

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April 9, 2003

If you have any questions regarding these reports, please call Jerry Dozier of my staff. He can be reached at 301-415-1014.

Sincerely,

/RA/

William D. Beckner, Program Director
Operating Reactor Improvements Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Enclosures: As stated

cc w/Enclosures:

Mr. Lennart Carlsson
Nuclear Safety Division
Nuclear Energy Agency
Organization for Economic
Cooperation and Development
Le Seine Saint Germain
12, Boulevard des Iles
92130, Issy-les-Moulineaux, France

Mr. Valeri Tolstykh

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Distribution (Transmittal Letter, Cover Sheet, and Coding Sheet Only):

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OFFICE	RSE:OES:RORP:DRIP	IMA:OES:RORP:DRIP	SC:OES:RORP:DRIP	PD:RORP:DRIP
NAME	IJDozier	KAGray	TReis	WDBeckner
DATE	04/03/2003	04/08/2003	04/09/2003	04/09/2003

OFFICIAL RECORD COPY

INCIDENT REPORTING SYSTEM

IRS NO.	EVENT DATE	4/1/2002	DATE RECEIVED
EVENT TITLE			
NRC INFORMATION NOTICE 2003-01: FAILURE OF A BOILING WATER REACTOR TARGET ROCK MAIN STEAM SAFETY/RELIEF VALVE			
COUNTRY	PLANT AND UNIT	REACTOR TYPE	
USA	Hatch	BWR	
INITIAL STATUS	RATED POWER (MWe NET)		
	N/A		
DESIGNER	1st COMMERCIAL OPERATION		
GEC	12/19/1967		

ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to alert addressees to a recent failure of a main steam safety/relief valve on a boiling water reactor (BWR).

NRC INFORMATION NOTICE 2003-01

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

1. <u>Reporting Categories:</u>	<u>1.3.1</u>	<u>1.4</u>	<u>1.3.3</u>
2. <u>Plant Status Prior to the Event:</u>	<u>2.1.3</u>	_____	_____
3. <u>Failed/Affected Systems:</u>	<u>3.AF</u>	_____	_____
4. <u>Failed/Affected Components:</u>	<u>4.2.3</u>	_____	_____
5. <u>Cause of the Event:</u>	<u>5.1.1.2</u>	<u>5.7.2</u>	_____
6. <u>Effects on Operation:</u>	<u>6.2</u>	_____	_____
7. <u>Characteristics of the Incident:</u>	<u>7.2</u>	_____	_____
8. <u>Nature of Failure or Error:</u>	<u>8.3</u>	_____	_____
9. <u>Nature of Recovery Actions:</u>	<u>9.0</u>	_____	_____

INCIDENT REPORTING SYSTEM

IRS NO.	EVENT DATE	12/26/2002	DATE RECEIVED
EVENT TITLE			
NRC INFORMATION NOTICE 2003-02: RECENT EXPERIENCE WITH REACTOR COOLANT SYSTEM LEAKAGE AND BORIC ACID CORROSION			
COUNTRY	PLANT AND UNIT	REACTOR TYPE	
USA	Sequoyah Unit 2	PWR	
INITIAL STATUS	RATED POWER (MWe NET)		
	N/A		
DESIGNER	1st COMMERCIAL OPERATION		
Westinghouse	4/18/1968		

ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to inform addressees of recently observed reactor coolant leakage at two pressurized water reactor facilities, one of which resulted in the subsequent degradation of the reactor pressure vessel head.

NRC INFORMATION NOTICE 2003-02

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

- | | | | | |
|----|---|----------------|-------------------|-------------------|
| 1. | <u>Reporting Categories:</u> | <u>1.2</u> | <u>1.2.2</u> | <u> </u> |
| 2. | <u>Plant Status Prior to the Event:</u> | <u>2.1.1</u> | <u> </u> | <u> </u> |
| 3. | <u>Failed/Affected Systems:</u> | <u>3.AC</u> | <u> </u> | <u> </u> |
| 4. | <u>Failed/Affected Components:</u> | <u>4.2.5</u> | <u> </u> | <u> </u> |
| 5. | <u>Cause of the Event:</u> | <u>5.1.1.1</u> | <u>5.1.1.6</u> | <u> </u> |
| 6. | <u>Effects on Operation:</u> | <u>6.0</u> | <u> </u> | <u> </u> |
| 7. | <u>Characteristics of the Incident:</u> | <u>7.2</u> | <u> </u> | <u> </u> |
| 8. | <u>Nature of Failure or Error:</u> | <u>8.0</u> | <u> </u> | <u> </u> |
| 9. | <u>Nature of Recovery Actions:</u> | <u>9.0</u> | <u> </u> | <u> </u> |

INCIDENT REPORTING SYSTEM

IRS NO.	EVENT DATE	10/22/2002	DATE RECEIVED
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EVENT TITLE

NRC INFORMATION NOTICE 2003-03: INADEQUATELY STAKED CAPSCREW RENDERS RESIDUAL HEAT REMOVAL PUMP INOPERABLE

COUNTRY

USA

PLANT AND UNIT

Vogtle Unit 2

REACTOR TYPE

PWR

INITIAL STATUS

RATED POWER (MWe NET)

N/A

DESIGNER

Westinghouse

1st COMMERCIAL OPERATION

2/13/1973

ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to inform addressees of recently observed inadequately staked capscrews on casing wear rings of residual heat removal pumps.

NRC INFORMATION NOTICE 2003-03

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

- | | | | | |
|----|---|----------------|-------------------|-------------------|
| 1. | <u>Reporting Categories:</u> | <u>1.4</u> | <u>1.2.5</u> | <u> </u> |
| 2. | <u>Plant Status Prior to the Event:</u> | <u>2.3.2.2</u> | <u> </u> | <u> </u> |
| 3. | <u>Failed/Affected Systems:</u> | <u>3.BE</u> | <u> </u> | <u> </u> |
| 4. | <u>Failed/Affected Components:</u> | <u>4.2.1</u> | <u> </u> | <u> </u> |
| 5. | <u>Cause of the Event:</u> | <u>5.1.1.8</u> | <u>5.7.2</u> | <u> </u> |
| 6. | <u>Effects on Operation:</u> | <u>6.0</u> | <u> </u> | <u> </u> |
| 7. | <u>Characteristics of the Incident:</u> | <u>7.4</u> | <u>7.8</u> | <u> </u> |
| 8. | <u>Nature of Failure or Error:</u> | <u>8.3</u> | <u> </u> | <u> </u> |
| 9. | <u>Nature of Recovery Actions:</u> | <u>9.0</u> | <u> </u> | <u> </u> |