

April 9, 2003

Mr. Stephen A. Byrne  
Senior Vice President, Nuclear Operations  
South Carolina Electric & Gas Company  
Virgil C. Summer Nuclear Station  
Post Office Box 88  
Jenkinsville, South Carolina 29065

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE  
V.C. SUMMER NUCLEAR STATION (VCSNS), LICENSE RENEWAL  
APPLICATION - SECTION 2.3

Dear Mr. Byrne:

By letter dated August 6, 2002, South Carolina Electric & Gas Company (SCE&G) submitted, for the Nuclear Regulatory Commission's (NRC's) review, an application pursuant to 10 CFR Part 54, to renew the operating license for the VCSNS. The NRC staff is reviewing the information in the license renewal application and has identified areas where additional information is needed to complete the review. Enclosure 1 of our letter dated March 28, 2003, contained requests for additional information (RAIs) in Section 2.3. Inadvertently, the attached RAI was not included in that group of RAIs.

The staff is willing to meet with SCE&G and to clarify the RAI before SCE&G submits its response. If you have any further questions, please contact me at 301-415-1025 (e-mail: [RCA@nrc.gov](mailto:RCA@nrc.gov)).

Sincerely,

*/RA/*

Rajender Auluck, Senior Project Manager  
License Renewal Section  
License Renewal and Environmental Impacts Program  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Docket No.: 50-395

Enclosure: As stated

cc w/enclosure: See next page

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Document Name: C:\MYFILES\Copies\SummerRSBFRAI.wpd

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OFFICIAL AGENCY RECORD

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## Request for Additional Information

### 2.3 SCOPING AND SCREENING RESULTS: MECHANICAL

#### Section 2.3.1 Reactor Vessel, Internals, and Reactor Coolant System

RAI 2.3.1-1 Item 5 of FSAR Section 15.4.3.2.2, which pertains to the recovery procedure after a steam generator tube rupture accident, states, "Decrease RCS pressure by use of normal pressurizer spray until the water level returns in the pressurizer and RCS pressure and the ruptured steam generator pressure are equal, or high pressurizer water level is attained, or minimum subcooling is attained."

- a. Please provide the technical justification for not including the pressurizer spray head among the component types that are subject to aging management review (Table 2.3-6). Technical justification should address the requirements of Appendix R to 10 CFR 50, as well as 10 CFR 54.4(a).
- b. Please indicate whether the steam generator divider plates, which separate the primary-side inlet and outlet plena in each steam generator, are considered in-scope for aging management. Please provide the technical justification for your response.
- c. Please indicate whether the reactor coolant pump support structures are considered in-scope for aging management. Please provide the technical justification for your response.

S. Byrne  
South Carolina Electric & Gas Company

VIRGIL C. SUMMER NUCLEAR STATION

cc:

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