APPENDIX C

CHARPY V-NOTCH PLOTS FOR EACH CAPSULE USING SYMMETRIC HYPERBOLIC TANGENT CURVE-FITTING METHOD

1

Contained in Table C-1 are the upper shelf energy values used as input for the generation of the Charpy Vnotch plots using CVGRAPH, Version 4.1. The definition for Upper Shelf Energy (USE) is given in ASTM E185-82, Section 4.18, and reads as follows:

"upper shelf energy level – the average energy value for all Charpy specimens (normally three) whose test temperature is above the upper end of the transition region For specimens tested in sets of three at each test temperature, the set having the highest average may be regarded as defining the upper shelf energy."

If there are specimens tested in set of three at each temperature Westinghouse reports the set having the highest average energy as the USE (usually unirradiated material). If the specimens were not tested in sets of three at each temperature Westinghouse reports the average of all 100% shear Charpy data as the USE. Hence, the USE values reported in Table C-1 and used to generate the Charpy V-notch curves were determined utilizing this methodology.

Table C-1 Upper Shelf Energy Values Fixed in CVGRAPH [ft-lb]				
Material	Unirradiated	Capsule X	Capsule V	Capsule Y
Lower Shell Plate C4339-1	128	122	121	111
(Longitudinal Orientation)				
Lower Shell Plate C4339-1	104	94	94	94
(Transverse Orientation)				
Weld Metal	91	71	60	58
(heat # 0227)				
HAZ Material	116	101	94	94
Correlation Monitor Material	123	103	102	100

The lower shelf energy values were fixed at 2.2 ft-lb for all cases.



UNIRR LOWER SHELL PLATE C4339-1 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C4339-1

339–1 Orientation: LT

Capsule: UNIRR Total Fluence:

Temperature	Input CVN Energy	Computed CVN Energy	Differential
40	57	54.45	2.54
40	50	54.45	-4.45
40	44.5	54.45	-9.95
75	84	78.7	5.29
75	79	78.7	29
75	76	78.7	-27
110	96	99 34	-334
110	965	99.34	-2.84
110	98	99.34	-1.34
160	119	116 92	2.07
160	119	116.92	2.07
160	121	116.92	4.07
210	126	124.14	1.85
210	129	124.14	4.85
210	127.5	124.14	3.35
		SUM of R	ESIDUALS = 142



CAPSULE X LOWER SHELL PLATE C4339-1 (LONG) Page 2 Material. PLATE SA533B1 Heat Number: C4339-1 Orientation: LT Capsule: X Total Fluence: Charpy V-Notch Data (Continued) Temperature Input CVN Energy Computed CVN Energy Differential 300 120 12114 -114 345 124 121.72 227 SUM of RESIDUALS = 1553



CAPSULE V LOWER SHELL PLATE C4339-1 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C4339-1 Orientation[.] LT

Total Fluence: Capsule: V

Charpy V-Notch Data (Continued)

Temperature 350 425

Input CVN Energy 138 109

Computed CVN Energy 118.99 120.52

Differential 19 -11.52 SUM of RESIDUALS = 13.74



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CAPSULE Y LOWER SHELL PLATE C4339-1 (LONG)

Page 2

Material: PLATE SA533B1

.

Heat Number: C4339-1

Orientation: LT

Capsule: Y Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 300 325 350 Computed CVN Energy 101.91 105.09 107.21 Differential 3.08 59 8.78 Input CVN Energy 105 111 116 SUM of RESIDUALS = 25.33



UNIRR LOWER SHELL PLATE C4339-1 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C4339-1

9-1 Orientation[.] LT

Capsule: UNIRR Total Fluence:

Temperature	Input Lateral Expansion	Computed L.E.	Differential
¹ 40	43	41.69	13
40	39	41.69	-2.69
40	36	41.69	-5.69
75	65	59.68	531
75	62	59.68	231
75	58	59.68	-1.68
110	73	73.78	78
110	72	73.78	-1.78
110	72	73.78	-1.78
160	85	84.65	34
160	84	84.65	65
160	86	84.65	134
210	87	88.69	-169
210	89	88 69	3
210	90	88 69	13
		SUM of	RESIDUALS = 81



CAPSULE X LOWER SHELL PLATE C4339-1 (LONG)

Page 2

Material: PLATE SA533B1

-

7 17.1.1 77......

Heat Number: C4339-1 Orientation: LT

Capsule: X Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 300 345

Input Lateral Expansion 87.5 90.5 Computed L.E. 90.52 91.3

Differential -3.02 SUM of RESIDUALS = 2.29



CA	PSULE V LOWER SHI	ELL PLATE C4339-1 Page 2	(LONG)
	Material: PLATE SA533B1	Heat Number: C4339–1 Orientatio	n: LT
	Capsule: V	Total Fluence:	
	Charpy V-Note	ch Data (Continued)	
Temperature 350 425	Input Lateral Expansion 89 74.5	Computed L.E. 82 62 83 07 SUM	Differential 6.37 -8.57 of RESIDUALS =09



CAPSULE Y LOWER SHELL PLATE C4339-1 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C4339-1 Orientation LT

Total Fluence: Capsule: Y

Temperature	Input Lateral Expansion	Computed L.E.	Differential
¹ 300	74	7089	3.1
325	71	73.71	-271
350	74	75.68	-168
		SUM o	f RESIDUALS =89



UNIRR LOWER SHELL PLATE C4339-1 (LONG)

Page 2

Material: PLATE SA533BI

.

Heat Number: C4339-1

Orientation[.] LT

Capsule: UNIRR Total Fluence:

Temperature	Input Percent Shear	Computed Percent Shear	Differential
40	41	4068	.31
40	40	40.68	68
40	33	4068	-7.68
75	57	59.17	-2.17
75	55	59.17	-4.17
75	52	59.17	-7.17
110	77	75.39	16
110	73	75 39	-2.39
110	67	75 39	-8.39
160	100	89.92	10 07
160	100	89.92	10 07
160	100	89 92	10 07
210	100	96.29	37
210	100	96.29	3.7
210	100	96.29	3.7
		SUM of R	ESIDUALS = 33.5



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CAPSULE X LOWER SHELL PLATE C4339-1 (LONG)

Page 2

Material: PLATE SA533B1

Capsule: X Total Fluence:

Heat Number: C4339-1

Charpy V-Notch Data (Continued)

Temperature 300 345 Input Percent Shear 100 100 Computed Percent Shear 99.54 99.89

Orientation: LT

Differential .45 .1

SUM of RESIDUALS = 13.42



CAPSULE V LOWER SHELL PLATE C4339-1 (LONG)

Page 2

Material: PLATE SA533B1

.

Heat Number: C4339-1 Orientation: LT

Capsule: V Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 350 425

Input Percent Shear 100 100

Computed Percent Shear 97.36 99.22

Differential 2.63 .77 SUM of RESIDUALS = 13.51



CAPSULE Y LOWER SHELL PLATE C4339-1 (LONG)

Page 2

Material: PLATE SA533B1

Total Fluence:

Heat Number: C4339-1 Orientation: LT

Capsule: Y

Temperature	Input Percent Shear	Computed Percent Shear	Differential
1300	100	95.68	4.31
325	100	97.73	2.26
350	100	98.82	1.17
		SUM of RE	SIDUALS = 20.83



UNIRR LOWER SHELL PLATE C4339–1 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C4339-1

C4339-1 Orientation: TL

Capsule: UNIRR Total Fluence:

Temperature	Input CVN Energy	Computed CVN Energy	Differential
40	44.5	43.68	.81
40	46	43.68	2.31
40	41	43.68	-2.68
75	62	62.35	-35
75	60.5	62.35	-1.85
75	57	62.35	-5.35
110	81	78.75	224
110	73	78.75	-5.75
110	72	78.75	-6.75
160	101	93.55	7.44
160	99	93,55	5.44
160	95	93,55	1.44
210	106	100.12	5.87
210	101.5	100.12	1.37
210	105	100.12	4.87
		SUM of F	RESIDUALS = 16.09



CAPSULE X LOWER SHELL PLATE C4339-1 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C4339-1

Capsule: X Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 300 345

Input CVN Energy 89 100

Computed CVN Energy 9248 9339 SUM of RESIDUALS = 9.96

Orientation: TL

Differential -3.48 6.6



CAPS	ULE V LOWER SHELL Page	PLATE C4339	–1 (TRANS)
-	Material: PLATE SA533B1 Heat Capsule: V	t Number: C4339–1 Or Total Fluence:	ientation: TL
	Charpy V-Notch I	Data (Continued)	
Temperature 350 425	Input CVN Energy 99 89	Computed CVN Er 92.78 93.69	hergy Differential 6.21 -4.69 SUM of RESIDUALS = 55

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CAPSULE Y LOWER SHELL PLATE C4339-1 (TRANS) Page 2 Material: PLATE SA533B1 Heat Number: C4339-1 Orientation: TL Capsule: Y Total Fluence: Charpy V-Notch Data (Continued) Temperature Input CVN Energy Computed CVN Energy Differential 300 95 84.42 10.57 325 95 87.42 7.57 350 93 89.52 3.47 SUM of RESIDUALS = 16.16



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UNIRR LOWER SHELL PLATE C4339-1 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C4339-1

-1 Orientation: TL

Capsule: UNIRR Total Fluence:

Temperature	Input Lateral Expansion	Computed LE	Differential
4 0	35	34.27	.72
40	36	34.27	1.72
40	32	34.27	-2.27
75	50	49.16	.83
75	48	49.16	-1.16
75	47	49.16	-2.16
110	65	62.32	2.67
110	59	62.32	-3.32
110	61	62.32	-1.32
160	80 •	74.32	5.67
160	75	74.32	.67
160	72	74.32	-232
210	83	79.72	327
210	77	79.72	-272
210	78	79.72	-172
		SUM of	RESIDUALS = -2.45


CAPSULE X LOWER SHELL PLATE C4339-1 (TRANS) Page 2 Material: PLATE SA533BI Material: PLATE Mathematical Continued) Temperature Input Lateral Expansion Compute Safet Colspan="2">SUM of RESIDUALS = 2.36 SUM of RESIDUALS = 2.36



CAPSULE V LOWER SHELL PLATE C4339-1 (TRANS) Page 2			
Material: PLATE SA533B1 Heat Number: C4339-1 Orientation: TL Capsule: V Total Fluence:			
	Charpy V-Notch Data (Continued)		
Temperature 350 425	Input Lateral Expansion Computed LE. Differential 74 72.93 1.06 70 73.7 -3.7 SUM of RESIDUALS = -1.51		
	·		



CAPSULE Y LOWER SHELL PLATE C4339-1 (TRANS) Page 2 Material: PLATE SA533B1 Heat Number: C4339-1 Orientation: TL Capsule: Y Total Fluence: Charpy V-Notch Data (Continued) Temperature 300 325 350 Computed L.E. 64.64 68.1 70 69 Differential 3.35 .89 Input Lateral Expansion 68 69 68 -2.69 SUM of RESIDUALS = -3.62



UNIRR LOWER SHELL PLATE C4339-1 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C4339-1

9–1 Orientation: TL

Capsule: UNIRR Total Fluence:

Temperature	Input Percent Shear	Computed Percent Shear	Differential
40	â <u>2</u> 9	32.72	-3.72
40	29	32.72	-3.72
40	29	32.72	-372
75	47	51.57	-4.57
75	47	5157	-4.57
75	42	51.57	-9.57
110	71	69.97	1.02
110	62	69.97	-7.97
110	63	69.97	-697
160	100	87.71	12.28
160	100	87.71	12.28
160	100	87.71	12.28
210	100	95.62	4.37
210	100	95.62	4.37
210	100	95.62	437
	200	SUM of RI	SIDUALS = 44.56



CAPSULE X LOWER SHELL PLATE C4339-1 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C4339-1 Orientation: TL

Capsule: X Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 300 345

Input Percent Shear 100 100

Computed Percent Shear 99 93 99.98 Differential SUM of RESIDUALS = 10.89

.06 .01



CAPSULE V LOWER SHELL PLATE C4339-1 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C4339-1 Orientation: TL

Capsule: V Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 350 425

Input Percent Shear 100 100

Computed Percent Shear 97.64 99.51 SUM of RESIDUALS = 28.54

Differential

2.35.48



CAPSULE Y LOWER SHELL PLATE C4339-1 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C4339-1 Orientation TL

Capsule: Y Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 300 325 350 Computed Percent Shear 94.23 96.98 Input Percent Shear Differential 5.76 3.01 100 100 100 98.44 1.55 SUM of RESIDUALS = 28.42



1

UNIRRADIATED WELD

Page 2

Material: WELD

Total Fluence:

Heat Number: WIRE HEAT 0227

Orientation:

Capsule: UNIRR

Temperature	Input CVN Energy	Computed CVN Energy	Differential
10	1 35	43.94	-8.94
10	53	43.94	905
10	47	43.94	305
40	50	58.22	-8.22
40	55.5	58.22	-2.72
40	53.5	58.22	-4.72
73	75	71.34	365
73	81	71.34	9.65
73	78	71.34	6.65
210	86	89.75	-3.75
210	69.5	89.75	-20.25
210	72	89.75	-17.75
300	91	90.82	.17
300	91	90.82	.17
300	91	90.82	.17
		SUM of R	ESIDUALS $=-35.92$







CAPSULE Y WELD

Page 2

Material: WELD

WELD Heat Number: WIRE HEAT 0227 Orientation:

Capsule: Y Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 375 Input CVN Energy 66 Computed CVN Energy 5729

gy Differential 8.7 SUM of RESIDUALS = 7.73



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UNIRRADIATED WELD

Page 2

Material: WELD

Heat Number: WIRE HEAT 0227

Orientation:

Capsule: UNIRR Total Fluence:

Temperature	Input Lateral Expansion	Computed L.E.	Differential
10	33	40.7	-7.7
10	47	40.7	6.29
10	40	407	7
40	50	54.36	-4.36
40	51	54.36	-3.36
40	51	54.36	-3.36
73	68	65.49	25
73	72	65.49	6.5
73	71	65.49	5.5
210	80	77.52	2.47
210	66	77.52	-11.52
210	70	77.52	-7.52
300	83	77.95	5.04
300	81	77.95	3.04
300	82	77.95	4.04
		SUM of	RESIDUALS = 0







CAPSULE Y WELD

Page 2

Material: WELD

) Heat Number: WIRE HEAT 0227 Orientation:

Capsule: Y Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 375 Input Lateral Expansion 50

Computed L.E. 45.87 Differential 4.12SUM of RESIDUALS = 13



. UNIRRADIATED WELD

Page 2

Material. WELD

Heat Number: WIRE HEAT 0227

Orientation:

Capsule: UNIRR Total Fluence

Temperature	Input Percent Shear	Computed Percent Shear	Differential
10	47	61.07	-14.07
10	68	61.07	6.92
10	58	61.07	-307
40	74	78.01	-4.01
40	74	78.01	-4.01
40	68	78.01	-10.01
73	100	89.7	10.29
73	100	89.7	10.29
73	100	89.7	10.29
210	100	99.72	27
210	100	99.72	27
210	100	9972	27
300	100	99.97	.02
300	100	99.97	.02
300	100	99.97	.02
	2	SUM of R	ESIDUALS = 15.61







CAPSULE Y WELD

Page 2

Material: WELD

Heat Number: WIRE HEAT 0227 Orientation.

> Total Fluence: Capsule: Y

Charpy V-Notch Data (Continued)

Temperature 375

Input Percent Shear 100

Computed Percent Shear 99.69

Differential

.3SUM of RESIDUALS = 6.44



UNIRRADIATED HEAT AFFECTED ZONE

Page 2

Material: HEAT AFFD ZONE

Total Fluence:

Heat Number: C4339-1 SIDE OF WELD

Orientation:

Capsule: UNIRR

Temperature	Input CVN Energy	Computed CVN Energy	Differential
10	5 0	57.57	-7.57
10	50	57.57	-7.57
10	74	57.57	16.42
40	66.5	71.96	-5.46
40	54	71.96	-17.96
40	50	71.96	-21.96
73	100.5	85.95	14.54
73	97.5	85.95	11.54
73	96	85.95	10.04
210	114	112.21	1.78
210	104	112.21	-8.21
210	130	112.21	17.78
300	120	115.16	4.83
300	100	115.16	-15.16
300	83	115.16	-32.16
	-	SUM of H	RESIDUALS = -28.26






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CAPSULE Y HEAT AFFECTED ZONE

Page 2

Material: HEAT AFFD ZONE

Capsule: Y Total Fluence:

Heat Number: C4339-1 SIDE OF WELD

Charpy V-Notch Data (Continued)

Temperature 325 Input CVN Energy 93 Computed CVN Energy Differential 86.79 SUM of RESIDUALS = 23.75

Orientation:



UNIRRADIATED HEAT AFFECTED ZONE

Page 2

Material: HEAT AFFD ZONE

Heat Number: C4339-1 SIDE OF WELD

Orientation:

Capsule: UNIRR Total Fluence:

Temperature	Input Lateral Expansion	Computed LE.	Differential
10	38	41.87	-3.87
10	40	4187	-187
10	52	41.87	10.12
40	50	52.47	-2.47
40	42	52.47	-10 47
40	40	52.47	-12.47
73	69	62.09	6.9
73	67	62.09	4.9
73	66	6209	39
210	80	77.27	2.72
210	75	77.27	-2.27
210	84	77.27	672
300	80	7855	1.44
300	78	78 55	-55
300	70	78 55	-8.55
000		SUM of	RESIDUALS = -2.81



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CAPSULE Y HEAT AFFECTED ZONE

Page 2

Material: HEAT AFFD ZONE

Capsule: Y Total Fluence:

Heat Number: C4339-1 SIDE OF WELD

Orientation:

Charpy V-Notch Data (Continued)

Temperature 325

Input Lateral Expansion 66

Computed L.E. 67.32

Differential -1.32SUM of RESIDUALS = .53



UNIRRADIATED HEAT AFFECTED ZONE

Page 2

Material: HEAT AFFD ZONE

Heat Number: C4339-1 SIDE OF WELD

Orientation:

Capsule: UNIRR Total Fluence:

Temperature	Input Percent Shear	Computed Percent Shear	Differential
¹ 10	5 3	- 56.79	-3.79
10	53	56.79	-3.79
10	58	56.79	12
40	68	70.3	-23
40	58	70.3	-12.3
40	61	70.3	-9.3
73	96	81.89	14.1
73	91	81 89	91
73	90	81 89	8.1
210	100	98.51	1.48
210	100	98 51	1.48
210	100	98 51	1.48
300	100	99.74	25
300	100	9974	25
300	100	99.74	25
		SUM of R	SIDUALS = 1804







CAPSULE Y HEAT AFFECTED ZONE

Page 2

Material: HEAT AFFD ZONE

Capsule: Y Total Fluence:

Heat Number: C4339-1 SIDE OF WELD

Charpy V-Notch Data (Continued)

Temperature 325 Input Percent Shear 100 Computed Percent Shear Differential 94.5 5.49 SUM of RESIDUALS = 7.53

Orientation:



UNIRRADIATED CORRELATION MONITOR MATERIAL

Page 2

Material: SRM SA533B1

Heat Number: HSST PLATE 02

Orientation: LT

Capsule: UNIRR Total Fluence:

Temperature	Input CVN Energy	Computed CVN Energy	Differential
4 0	35	26.74	8.25
40	22	26.74	-4.74
40	36	26.74	9.25
85	415	54.83	-13.33
85	52	54.83	-2.83
85	58.5	54.83	3.66
110	63.5	7325	-9.75
110	825	7325	924
110	85.5	7325	12.24
160	109	102.49	6.5
160	108.5	102.49	6
160	81	102.49	-21.49
210	117	116.19	.8
210	115	116.19	-1.19
210	121	116.19	4.8
300	127	122.21	4.78
300	117.5	122.21	-4.71
300	125	122.21	2.78
	•	SUM of R	FSIDUALS = -4.17







C-90

CAPSULE Y CORRELATION MONITOR MATERIAL

Page 2

Material: SRM SA533B1

....

Heat Number: HSST PLATE 02

Orientation: LT

Capsule: Y Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 375 Input CVN Energy 107 Computed CVN Energy 96.36

gy Differential 10.63 SUM of RESIDUALS = 25.26



UNIRRADIATED CORRELATION MONITOR MATERIAL

Page 2

Material: SRM SA533B1

....

Heat Number: HSST PLATE 02

Orientation: LT

Capsule: UNIRR Total Fluence:

Temperature	Input Lateral Expansion	Computed L.E.	Differential
40	32	26.31	5.68
40	23	26.31	-3.31
40	32	26.31	5.68
85	42	48.06	-6.06
85	45	4806	-3.06
85	51	48 06	2.93
110	54	59.9	-5.9
110	60	59.9	.09
110	71	59.9	11.09
160	79	76.07	2.92
160	72	76.07	-4.07
160	69	76.07	-7.07
210	84	82.91	1.08
210	88	8291	5.08
210	87	82.91	4.08
300	84	85.9	-1.9
300	83	85.9	-29
300	87	85.9	1.09
	0.	SUM of	RESIDUALS = -3.03





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CAPSULE Y CORRELATION MONITOR MATERIAL

Page 2

Material: SRM SA533B1

-

Heat Number: HSST PLATE 02

Orientation: LT

Capsule: Y Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 375 Input Lateral Expansion 69 Computed LE 72.35 $\begin{array}{rl} \text{Differential} \\ -3.35 \\ \text{SUM of RESIDUALS} = 1.73 \end{array}$



UNIRRADIATED CORRELATION MONITOR MATERIAL

Page 2

Material: SRM SA533B1

Heat Number: HSST PLATE 02

Orientation[.] LT

Capsule: UNIRR Total Fluence:

Temperature	Input Percent Shear	Computed Percent Shear	Differential
4 0	1 29	28.84	.15
40	33	28.84	4.15
40	29	28.84	.15
85	41	49.72	-8.72
85	42	49.72	-7.72
85	43	49.72	-6.72
110	55	61.88	-6.88
110	58	61.88	-388
110	67	61.88	5.11
160	87	81.39	5.6
160	84	8139	26
160	85	81.39	36
210	98	9217	582
210	98	9217	582
210	100	9217	782
300	100	98.59	14
300	100	98.59	1.1
300	100	9859	1.4
000	. 100	SUM of RI	SIDUALS = 35.18







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CAPSULE Y CORRELATION MONITOR MATERIAL

Page 2

Material: SRM SA533B1

Heat Number: HSST PLATE 02

Orientation: LT

Capsule: Y Total Fluence:

Charpy V-Notch Data (Continued)

Temperature 375

Input Percent Shear 100

Computed Percent Shear 9723

Differential 2.76 SUM of RESIDUALS = 29.02

APPENDIX D

VALIDATION OF THE RADIATION TRANSPORT MODELS BASED ON NEUTRON DOSIMETRY MEASUREMENTS

D 1 Neutron Dosimetry

Comparisons of-measured dosimetry results to both the calculated and least squares adjusted values for all surveillance capsules withdrawn from service to date at Surry Unit 2 Unit 2 are described herein. The sensor sets from these capsules have been analyzed in accordance with the current dosimetry evaluation methodology described in Regulatory Guide 1.190, "Calculational and Dosimetry Methods for Determining Pressure Vessel Neutron Fluence."^[D-1] One of the main purposes for presenting this material is to demonstrate that the overall measurements agree with the calculated and least squares adjusted values to within \pm 20% as specified by Regulatory Guide 1.190, thus serving to validate the calculated neutron exposures previously reported in Section 6.2 of this report. This information may also be useful in the future, in particular, as least squares adjustment techniques become accepted in the regulatory environment.

D.1.1 Sensor Reaction Rate Determinations

In this section, the results of the evaluations of the five neutron sensor sets withdrawn to date as a part of the Surry Unit 2 Reactor Vessel Materials Surveillance Program are presented. The capsule designation, location within the reactor, and time of withdrawal of each of these dosimetry sets were as follows:

Capsule ID	Equivalent Azimuthal Location	Withdrawal Time	Irradiation Time [EFPY]
X	15°	End of Cycle 1	1.2
W	25°	End of Cycle 4	3.8
V	15°	End of Cycle 8	8.4
S	45°	End of Cycle 13	15.0
Y	25°/15°	End of Cycle 17	20.3

The azimuthal locations included in the above tabulation represent the first octant equivalent azimuthal angle of the geometric center of the respective surveillance capsules.

The passive neutron sensors included in the evaluations of Surveillance Capsules X, W, V, S, and Y are summarized as follows:

Sensor Material	Reaction Of Interest
Copper	⁶³ Cu(n,α) ⁶⁰ Co
Iron	⁵⁴ Fe(n,p) ⁵⁴ Mn
Nickel	⁵⁸ Ni(n,p) ⁵⁸ Co
Titanium	⁴⁶ Ti(n,p) ⁴⁶ Sc
Uranium-238	²³⁸ U(n,f) ¹³⁷ Cs
Neptunium-237	²³⁷ Np(n,f) ¹³⁷ Cs
Cobalt-Aluminum	⁵⁹ Co(n,γ) ⁶⁰ Co

Pertinent physical and nuclear characteristics of the passive neutron sensors are listed in Table D-1. The use of passive monitors such as those listed above does not yield a direct measure of the energy dependent neutron flux at the point of interest. Rather, the activation or fission process is a measure of the integrated effect that the time and energy dependent neutron flux has on the target material over the course of the irradiation period. An accurate assessment of the average neutron flux level incident on the various monitors may be derived from the activation measurements only if the irradiation parameters are well known. In particular, the following variables are of interest:

- the measured specific activity of each monitor,
- the physical characteristics of each monitor,
- the operating history of the reactor,
- the energy response of each monitor, and
- the neutron energy spectrum at the monitor location.

The radiometric-counting of the neutron sensors from Capsules X and W was carried out by the Battelle Memorial Institute. The radiometric counting of the sensors from Capsules V, S, and Y was completed at the Pace Analytical Services Laboratory located at the Westinghouse Waltz Mill Site. In all cases, the radiometric counting followed established ASTM procedures. Following sample preparation and weighing, the specific activity of each sensor was determined by means of a high-resolution gamma spectrometer. For the copper, iron, nickel, and cobalt-aluminum sensors, these analyses were performed by direct counting of each of the individual samples In the case of the uranium fission sensors, the analyses were carried out by direct counting preceded by dissolution and chemical separation of cesium from the sensor material.

The irradiation history of the reactor over the irradiation periods experienced by Capsules X, W, V, S, and Y was based on the reported monthly power generation of Surry Unit 2 from initial reactor criticality through the end of the dosimetry evaluation period. For the sensor sets utilized in the surveillance capsules, the half-lives of the product isotopes are long enough that a monthly histogram describing reactor operation has proven to be an adequate representation for use in radioactive decay corrections for the reactions of interest in the exposure evaluations.

Having the measured specific activities, the physical characteristics of the sensors, and the operating history of the reactor, reaction rates referenced to full-power operation were determined from the following equation:

$$R = \frac{A}{N_o F Y \sum \frac{P_j}{P_{ref}} C_j [I - e^{-\lambda t_j}] [e^{-\lambda t_d}]}$$

where.

- R = Reaction rate averaged over the irradiation period and referenced to operation at a core power level of P_{ref} (rps/nucleus).
- A = Measured specific activity (dps/gm).
- N_0 = Number of target element atoms per gram of sensor.
- F = Weight fraction of the target isotope in the sensor material.
- Y = Number of product atoms produced per reaction.
- P_1 = Average core power level during irradiation period j (MW).
- P_{ref} = Maximum or reference power level of the reactor (MW)
- C_j = Calculated ratio of sensor reaction rate during irradiation period j to the time weighted average sensor reaction rate over the entire irradiation period.
- λ = Decay constant of the product isotope (1/sec).
- $t_j =$ Length of irradiation period j (sec).
- t_d = Decay time following irradiation period j (sec).

and the summation is carried out over the total number of monthly intervals comprising the irradiation period.

For capsules remaining in a single location for the entire irradiation period, the spectrum averaged reaction cross-section is essentially constant and, therefore, the cycle dependent neutron flux (E > 1.0 MeV) can be substituted for individual reaction rates in the computation of the C₁ term. However, for cases such as
Capsule Y where relocation of the capsule resulted in significant changes in the relative neutron energy spectrum, the explicit sensor reaction rates must be used to compute the time history corrections.

In the equation describing the reaction rate calculation, the ratio $[P_j]/[P_{ref}]$ accounts for month-by-month variation of reactor core power level within any given fuel cycle as well as over multiple fuel cycles. The ratio C_j , which was calculated for each fuel cycle using the transport methodology discussed in Section 6.2, accounts for the change in sensor reaction rates caused by variations in flux level induced by changes in core spatial power distributions from fuel cycle to fuel cycle. For a single-cycle irradiation, C_j is normally taken to be 1.0. However, for multiple-cycle irradiations, particularly those employing low leakage fuel management, the additional C_j term should be employed. The impact of changing flux levels for constant power operation can be quite significant for sensor sets that have been irradiated for many cycles in a reactor that has transitioned from non-low leakage to low leakage fuel management or for sensor sets contained in surveillance capsules that have been moved from one capsule location to another. The fuel cycle specific neutron flux values used in the time history corrections for Capsules X, W, V, and S as well as the individual sensor reaction rates used in the time history corrections for Capsule Y are listed in Table D-2. These values represent the cycle dependent results at the radial and azimuthal center of the respective capsules at the axial elevation of the active fuel midplane.

Prior to using the measured reaction rates in the least-squares evaluations of the dosimetry sensor sets, corrections were made to the ²³⁸U measurements to account for the presence of ²³⁵U impurities in the sensors as well as to adjust for the build-in of plutonium isotopes over the course of the irradiation Corrections were also made to the both the ²³⁸U and ²³⁷Np sensor reaction rates to account for gamma ray induced fission reactions that occurred over the course of the capsule irradiations. The correction factors applied to the Surry Unit 2 fission sensor reaction rates are summarized as follows:

Correction	Capsule X	Capsule W	Capsule V	Capsule S	Capsule Y
²³⁵ U Impurity/Pu Build-in	0.873	0.860	0.843	0 813	0.785
²³⁸ U(γ,f)	0.957	0.961	0.959	0.957	0.961
Net ²³⁸ U Correction	0.835	0.826	0.808	0.778	0.754
²³⁷ Np(γ,f)	0.983	0.984	0.982	0.983	0 984

These factors were applied in a multiplicative fashion to the decay corrected uranium fission sensor reaction rates.

Results of the sensor reaction rate determinations for Capsules X, W, V, S, and Y are given in Table D-3. In Table D-3, the computed reaction rates for each sensor indexed to the radial center of the capsule are listed The fission sensor reaction rates as listed include the applied corrections for ²³⁸U impurities, plutonium build-in, and gamma ray induced fission effects.

D 1.2 Least Squares Evaluation of Sensor Sets

Least squares adjustment methods provide the capability of combining the measurement data with the corresponding neutron transport calculations resulting in a Best Estimate neutron energy spectrum with associated uncertainties. Best Estimates for key exposure parameters such as $\phi(E > 1.0 \text{ MeV})$ or dpa/s along with their uncertainties are then easily obtained from the adjusted spectrum In general, the least squares methods, as applied to surveillance capsule dosimetry evaluations, act to reconcile the measured sensor reaction rate data, dosimetry reaction cross-sections, and the calculated neutron energy spectrum within their respective uncertainties. For example,

$$R_{i} \pm \delta_{R_{i}} = \sum_{g} (\sigma_{ig} \pm \delta_{\sigma_{ig}}) (\phi_{g} \pm \delta_{\phi_{g}})$$

relates a set of measured reaction rates, R_i , to a single neutron spectrum, ϕ_g , through the multigroup dosimeter reaction cross-section, σ_{ig} , each with an uncertainty δ . The primary objective of the least squares evaluation is to produce unbiased estimates of the neutron exposure parameters at the location of the measurement.

For the least squares evaluation of the Surry Unit 2 surveillance capsule dosimetry, the FERRET code^[D-2] was employed to combine the results of the plant specific neutron transport calculations and sensor set reaction rate measurements to determine best-estimate values of exposure parameters ($\phi(E > 1.0 \text{ MeV})$ and dpa) along with associated uncertainties for the two in-vessel capsules withdrawn to date.

The application of the least squares methodology requires the following input:

- 1 The calculated neutron energy spectrum and associated uncertainties at the measurement location
- 2 The measured reaction rates and associated uncertainty for each sensor contained in the multiple foil set.
- 3 The energy dependent dosimetry reaction cross-sections and associated uncertainties for each sensor contained in the multiple foil sensor set.

For the Surry Unit 2 application, the calculated neutron spectrum was obtained from the results of plant specific neutron transport calculations described in Section 6.2 of this report. The sensor reaction rates were derived from the measured specific activities using the procedures described in Section D.1.1. The dosimetry reaction cross-sections and uncertainties were obtained from the Sandia National Laboratory Radiation Metrology Laboratory (SNLRML) dosimeter cross-section library^[D-3]. The SNLRML library is

an evaluated dosimetry reaction cross-section compilation recommended for use in LWR evaluations by ASTM Standard E1018, "Application of ASTM Evaluated Cross-Section Data File, Matrix E 706 (IIB)".

The uncertainties associated with the measured reaction rates, dosimetry cross-sections, and calculated neutron spectrum were input to the least squares procedure in the form of variances and covariances. The assignment of the input uncertainties followed the guidance provided in ASTM Standard E 944, "Application of Neutron Spectrum Adjustment Methods in Reactor Surveillance."

The following provides a summary of the uncertainties associated with the least squares evaluation of the Surry Unit 2 surveillance capsule sensor sets.

Reaction Rate Uncertainties

The overall uncertainty associated with the measured reaction rates includes components due to the basic measurement process, irradiation history corrections, and corrections for competing reactions. A high level of accuracy in the reaction rate determinations is assured by utilizing laboratory procedures that conform to the ASTM National Consensus Standards for reaction rate determinations for each sensor type.

After combining all of these uncertainty components, the sensor reaction rates derived from the counting and data evaluation procedures were assigned the following net uncertainties for input to the least squares evaluation:

Reaction Contraction	Uncertainty March
⁶³ Cu(n,α) ⁶⁰ Co	5%
⁵⁴ Fe(n,p) ⁵⁴ Mn	5%
⁵⁸ Ni(n,p) ⁵⁸ Co	5%
⁴⁶ Ti(n,p) ⁴⁶ Sc	5%
²³⁸ U(n,f) ¹³⁷ Cs	10%
²³⁷ Np(n,f) ¹³⁷ Cs	10%
⁵⁹ Co(n,γ) ⁶⁰ Co	5%

These uncertainties are given at the 1σ level.

Dosimetry Cross-Section Uncertainties

The reaction rate cross-sections used in the least squares evaluations were taken from the SNLRML library. This data library provides reaction cross-sections and associated uncertainties, including covariances, for 66 dosimetry sensors in common use. Both cross-sections and uncertainties are provided in a fine multigroup structure for use in least squares adjustment applications. These cross-sections were compiled from the most recent cross-section evaluations and they have been tested with respect to their accuracy and consistency for least squares evaluations. Further, the library has been empirically tested for use in fission spectra determination as well as in the fluence and energy characterization of 14 MeV neutron sources.

Reaction 3	Uncertainty data
⁶³ Cu(n,α) ⁶⁰ Co	4.08-4.16%
⁵⁴ Fe(n,p) ⁵⁴ Mn	3.05-3.11%
⁵⁸ Ni(n,p) ⁵⁸ Co	4.49-4.56%
⁴⁶ Ti(n,p) ⁴⁶ Sc	4.51-4.87%
²³⁸ U(n,f) ¹³⁷ Cs	0.54-0.64%
237 Np(n,f) 137 Cs	10.32-10.97%
⁵⁹ Co(n,γ) ⁶⁰ Co	0.79-3.59%

For sensors included in the Surry Unit 2 surveillance program, the following uncertainties in the fission spectrum averaged cross-sections are provided in the SNLRML documentation package.

These tabulated ranges provide an indication of the dosimetry cross-section uncertainties associated with the sensor sets used in LWR irradiations.

Calculated Neutron Spectrum

The neutron spectra input to the least squares adjustment procedure were obtained directly from the results of plant specific transport calculations for each surveillance capsule irradiation period and location. The spectrum for each capsule was input in an absolute sense (rather than as simply a relative spectral shape). Therefore, within the constraints of the assigned uncertainties, the calculated data were treated equally with the measurements.

While the uncertainties associated with the reaction rates were obtained from the measurement procedures and counting benchmarks and the dosimetry cross-section uncertainties were supplied directly with the SNLRML library, the uncertainty matrix for the calculated spectrum was constructed from the following relationship:

$$M_{gg'} = R_n^2 + R_g * R_{g'} * P_{gg}$$

where R_n specifies an overall fractional normalization uncertainty and the fractional uncertainties R_g and $R_{g'}$ specify additional random groupwise uncertainties that are correlated with a correlation matrix given by:

$$P_{gg'} = [1 - \theta] \delta_{gg'} + \theta e^{-H}$$

where

$$H = \frac{(g - g')^2}{2\gamma^2}$$

The first term in the correlation matrix equation specifies purely random uncertainties, while the second term describes the short-range correlations over a group range γ (θ specifies the strength of the latter term). The value of δ is 1.0 when $g = g^2$, and is 0.0 otherwise.

The set of parameters defining the input covariance matrix for the Surry Unit 2 calculated spectra was as follows:

Flux Normalization Uncertainty (Rn)	15%
Flux Group Uncertainties (Rg, Rg)	
(E > 0.0055 MeV)	15%
(0.68 eV < E < 0.0055 MeV)	29%
(E < 0.68 eV)	52%
Short Range Correlation (θ)	
$(E > 0 \ 0055 \ MeV)$	0.9
$(0.68 \text{ eV} \le E \le 0.0055 \text{ MeV})$	0.5
(E < 0.68 eV)	0.5
Flux Group Correlation Range (y)	
(E > 0.0055 MeV)	6
(0.68 eV < E < 0.0055 MeV)	3
(E < 0.68 eV)	2

D.1.3 Comparisons of Measurements and Calculations

Results of the least squares evaluations of the dosimetry from the Surry Unit 2 surveillance capsules withdrawn to date are provided in Tables D-4 and D-5. In Table D-4, measured, calculated, and best-estimate values for sensor reaction rates are given for each capsule. Also provided in this tabulation are ratios of the measured reaction rates to both the calculated and least squares adjusted reaction rates. These ratios of M/C and M/BE illustrate the consistency of the fit of the calculated neutron energy spectra to the measured reaction rates both before and after adjustment. Also included in the tabulation are the results of the X²/Degree of freedom statistical test associated with each of the least squares evaluations. In Table D-5, comparison of the calculated and best estimate values of neutron flux (E > 1.0 MeV) and iron atom displacement rate are tabulated along with the BE/C ratios observed for each of the capsules.

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The data comparisons provided in Tables D-4 and D-5 show that the adjustments to the calculated spectra are relatively small and well within the assigned uncertainties for the calculated spectra, measured sensor reaction rates, and dosimetry reaction cross-sections. Further, these results indicate that the use of the least squares evaluation results in a reduction in the uncertainties associated with the exposure of the surveillance capsules. From Section 6.4 of this report, it may be noted that the uncertainty associated with the unadjusted calculation of neutron fluence (E > 1.0 MeV) and iron atom displacements at the surveillance capsule locations is specified as 12% at the 1σ level. From Table D-5, it is noted that the corresponding uncertainties associated with the least squares adjusted exposure parameters have been reduced to 6-7% for neutron flux (E > 1.0 MeV) and 6-8% for iron atom displacement rate. Again, the uncertainties from the least squares evaluation are at the 1σ level.

Further comparisons of the measurement results with calculations are given in Tables D-6 and D-7. These comparisons are given on two levels. In Table D-6, calculations of individual threshold sensor reaction rates are compared directly with the corresponding measurements. These threshold reaction rate comparisons provide a good evaluation of the accuracy of the fast neutron portion of the calculated energy spectra. In Table D-7, calculations of fast neutron exposure rates in terms of $\phi(E > 1.0 \text{ MeV})$ and dpa/s are compared with the best estimate results obtained from the least squares evaluation of the capsule dosimetry results. These two levels of comparison yield consistent and similar results with all measurement-to-calculation comparisons falling well within the 20% limits specified as the acceptance criteria in Regulatory Guide 1.190.

In the case of the direct comparison of measured and calculated sensor reaction rates, the M/C comparisons for fast neutron reactions range from 0.80–1.16 for the 23 samples included in the data set. The overall average M/C ratio for the entire set of Surry Unit 2 data is 0.97 with an associated sample standard deviation of 10.5%.

In the comparisons of best estimate and calculated fast neutron exposure parameters, the corresponding BE/C comparisons for the capsule data sets range from 0.84-1.01 for neutron flux (E > 1.0 MeV) and from 0.85-0.98 for iron atom displacement rate. The overall average BE/C ratios for neutron flux (E > 1.0 MeV) and iron atom displacement rate are 0.95 with a sample standard deviation of 6.9% and 0.94 with a samole standard deviation of 6.0%, respectively.

Based on these comparisons, it is concluded that the calculated fast neutron exposures provided in Section 6.2 of this report are validated for use in the assessment of the condition of the materials comprising the beltline region of the Surry Unit 2 reactor pressure vessel.

Table D-1

Nuclear Parameters Used In The Evaluation Of Neutron Sensors

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		Target	90% Response		Fission
Monitor	Reaction of	Atom	Range	Product	Yield
Material	Interest	Fraction	<u>(MeV){</u> &&}	Half-life	
Copper	⁶³ Cu (n,α)	0.6917	5.0 - 12.0	5.271 y	
Iron	⁵⁴ Fe (n,p)	0.0585	2.4 - 8.8	312.3 d	
Nickel	⁵⁸ Ni (n,p)	0.6808	1.7 – 8.4	70.82 d	
Uranium-238	²³⁸ U (n,f)	1 0000	1.5 – 7.9	30.07 y	
Neptunium-237	²³⁷ Np (n,f)	1.0000	0.45 - 5.0	30.07 y	6.02
Cobalt-Aluminum	⁵⁹ Co (n,y)	0 0017	non-threshold	5.271 y	6 17

Notes: The 90% response range is defined such that, in the neutron spectrum characteristic of the Surry Unit 2 surveillance capsules, approximately 90% of the sensor response is due to neutrons in the energy range specified with approximately 5% of the total response due to neutrons with energies below the lower limit and 5% of the total response due to neutrons with energies above the upper limit.

Table D-2

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Cycle	Capsule V	Capsule S
1	8.01e+10	2.79 c+ 10
2	8.24e+10	3.03 c+ 10
4	8.06e+10	3.01e+10
5	8.09e+10	2.81e+10
6	6.58e+10	2.44e+10
7	6.46 c+ 10	2.34e+10
8	6.63e+10	2.29e+10
9	5.490+10	2.200+10
10		1.96e+10
11		1.99e+10
12		1.89c+10
13		1.65e+10
14		1.000 10
15		
16		
17		
Average	7.10e+10	2.27e+10

$\phi(E > 1.0 \text{ MeV})$ [n/cm²-s] at the Surveillance Capsule Center Core Midplane Elevation

Table D-2 (Continued)

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Sensor Reaction Rates [rps/a] at the Surveillance Capsule Center Core Midplane Elevation

	Bare	Bare	Bare	Cd Cov.	Cd Cov.	Bare	Cd Cov.
Cycle	Cu-63(n,a)	Fe-54(n,p)	Ni-58(n,p)	U-238(n,f)	Np-237(n,f)	Co-59(n,g)	Co-59(n,g)
1	3.978E-17	4.133E-15	5.621E-15	1.901E-14	1.356E-13	2.048E-12	1.023E-12
2	4.114E-17	4.278E-15	5.819E-15	1.969E-14	1.406E-13	2.125E-12	1.062E-12
3	4.188E-17	4.362E-15	5.935E-15	2.010E-14	1.436E-13	2.171E-12	1.085E-12
4	4.018E-17	4.179E-15	5.684E-15	1.924E-14	1.374E-13	2.076E-12	1.037E-12
5	3.416E-17	3.457E-15	4.691E-15	1.566E-14	1.107E-13	1.654E-12	8.256E-13
6	3.428E-17	3.488E-15	4.736E-15	1.586E-14	1.123E-13	1.680E-12	8.390E-13
7	3.200E-17	3.222E-15	4.371E-15	1.457E-14	1.029E-13	1.539E-12	7.675E-13
8	3.037E-17	3.051E-15	4.139E-15	1.378E-14	9.724E-14	1.449E-12	7.234E-13
9	2.886E-17	2.887E-15	3.915E-15	1.301E-14	9.170E-14	1.365E-12	6.811E-13
10	3.047E-17	3.060E-15	4.151E-15	1.383E-14	9.759E-14	1.455E-12	7.266E-13
11	2.544E-17	2.519E-15	3.413E-15	1.130E-14	7.935E-14	1.177E-12	5.876E-13
12	2.611E-17	2.589E-15	3.509E-15	1.162E-14	8.165E-14	1 211E-12	6.045E-13
13	3.565E-17	3.756E-15	5.130E-15	1.771E-14	1.315E-13	2.155E-12	1.070E-12
14	3.305E-17	3.455E-15	4.716E-15	1.622E-14	1.201E-13	1.961E-12	9.728E-13
15	3.196E-17	3.346E-15	4.568E-15	1.573E-14	1.165E-13	1.906E-12	9.458E-13
16	3.211E-17	3.336E-15	4.551E-15	1.561E-14	1.153E-13	1.877E-12	9.312E-13
17	3.133E-17	3.247E-15	4.428E-15	1.517E-14	1.118E-13	1.813E-12	9.001E-13
Avg. 1-17	3.274E-17	3.356E-15	4.565E-15	1.541E-14	1.108E-13	1.708E-12	8.512E-13

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Table D-3

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	XX * • • 5	Measured Reaction Rate [rps/nucleus]					
Sensor Reaction	Capsule X	Capsule W	Capsule V	Capsule S	Capsule Y		
Cu-63(n,a)Co-60	5.87E-17	4.70E-17	4.86E-17	2.20E-17	3.21E-17		
Fe-54(n,p)Mn-54	6.56E-15	4.39E-15	4.99E-15	1.69E-15	3 04E-15		
Ni-58(n,p)Co-58	8.26E-15	5.92E-15	6.36E-15	2 29E-15			
U-238(n,f)Cs-137 (Cd)	2.75E-14	1.79E-14	2.36E-14	6.81E-15	1.36E-14		
Np-237(n,f)Cs-137 (Cd)	1.89E-13		1.75E-13	5.04E-14	1.28E-13		
Co-59(n,y) Co-60	3.94E-12		3.12E-12	7.08E-13	1.53E-12		
Co-59(n,γ) Co-60 (Cd)					1.06E-12		

Measured Sensor Activities And Reaction Rates

Table D-4

Comparison of Measured, Calculated, and Best Estimate Reaction Rates At The Surveillance Capsule Center

	Reac	tion Rate [rps/:	atom]	2477. [#] **	
Reaction	Measured	Calculated	Best Estimate	. M/C	M/BE
⁶³ Cu(n,α) ⁶⁰ Co	5.87E-17	5.23E-17	5.84E-17	1.12	1.01
⁵⁴ Fe(n,p) ⁵⁴ Mn	6.56E-15	5.79E-15	6.29E-15	1.13	1.04
⁵⁸ Ni(n,p) ⁵⁸ Co	8.26E-15	7.96E-15	8.42E-15	1.04	0.98
238 U(n,f) 137 Cs (Cd)	2.75E-14	2.82E-14	2.90E-14	0.98	0.95
²³⁷ Np(n,f) ¹³⁷ Cs (Cd)	1.89E-13	2.14E-13	2.01E-13	0.88	0.94
⁵⁹ Co(n,γ) ⁶⁰ Co	3.94E-12	3.53E-12	3.93E-12	1.12	1.00

Capsule X (χ^2 /DOF = 0.25)

Capsule W (χ^2 /DOF = 0.27)

t ser san san ta	North Reac	tion Rate [rps/	atom]	·	las' 🖉 🛣 ng
Reaction	Measured	Calculated	Best Estimate	M/C	M/BE
⁶³ Cu(n,α) ⁶⁰ Co	4.70E-17	4.05E-17	4.56E-17	1.16	1.03
54 Fe(n,p) 54 Mn	4.39E-15	4.21E-15	4.40E-15	1.04	1.00
⁵⁸ Ni(n,p) ⁵⁸ Co	5.92E-15	5.74E-15	5.95E-15	1.03	0.99
²³⁸ U(n,f) ¹³⁷ Cs (Cd)	1.79E-14	1.95E-14	1.95E-14	0.92	0.92

Capsule V (χ^2 /DOF = 0.16)

à Càng tại đàn đảo đão đão đão đão đão đão đão đão đão đã	Reac	tion Rate [rps/:	atom]	<u>). Mar i -</u>	į,,,,,,, įš,,,,,,,,,,,,,,,,,,,,,,,,,,,,
Reaction	Measured	Calculated	Best Estimate	M/C	M/BE
⁶³ Cu(n,α) ⁶⁰ Co	4.86E-17	4.76E-17	4.75E-17	1.02	1.02
⁵⁴ Fe(n,p) ⁵⁴ Mn	4.99E-15	5.19E-15	4.94E-15	0.96	1.01
⁵⁸ Ni(n,p) ⁵⁸ Co	6.36E-15	7.12E-15	6.63E-15	0.89	0.96
²³⁸ U(n,f) ¹³⁷ Cs (Cd)	2.36E-14	2.51E-14	2.33E-14	0.94	1 01
237 Np(n,f) 137 Cs (Cd)	1.75E-13	1.90E-13	1.75E-13	0.92	1.00
⁵⁹ Co(n,γ) ⁶⁰ Co	3.12E-12	3.10E-12	3.12E-12	1.01	1.00

Table D-4 cont'd

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Comparison of Measured, Calculated, and Best Estimate Reaction Rates At The Surveillance Capsule Center

	React	ion Rate [rps/	atôm] 🚺 🐑	î,~	1: "p., {}``
Reaction	Measured	Calculated	Best Estimate	M/C	M/BE
⁶³ Cu(n,α) ⁶⁰ Co	2.20E-17	2.14E-17	2.09E-17	1.03	1.05
⁵⁴ Fe(n,p) ⁵⁴ Mn	1 69E-15	1.98E-15	1.74E-15	0.85	0.97
⁵⁸ Ni(n,p) ⁵⁸ Co	2.29E-15	2.66E-15	2.33E-15	0.86	0.98
²³⁸ U(n,f) ¹³⁷ Cs (Cd)	6.81E-15	8.47E-15	7.22E-15	0.80	0.94
237 Np(n,f) 137 Cs (Cd)	5.04E-14	5.73E-14	4.93E-14	0.88	1.02
⁵⁹ Co(n,γ) ⁶⁰ Co	7.08E-13	7.88E-13	7.09E-13	0.90	1.00

<u>Capsule S (χ^2 /DOF = 0.31)</u>

Capsule Y (χ^2 /DOF = 0.36)

	Reac	tion Rate [rps/:	ătom] 💥 👯	," ;«,,.»	ite (te te t
Reaction	Measured	Calculated	Best Estimate	M/C	M/BE
⁶³ Cu(n,α) ⁶⁰ Co	3.21E-17	3.27E-17	3.15E-17	0.98	1.02
⁵⁴ Fe(n,p) ⁵⁴ Mn	3.04E-15	3.35E-15	3.12E-15	0 91	0.97
²³⁸ U(n,f) ¹³⁷ Cs (Cd)	1.36E-14	1.54E-14	1.46E-14	0.88	0.93
237 Np(n,f) 137 Cs (Cd)	1.28E-13	1.11E-13	1.17E-13	1.15	1.09
⁵⁹ Co(n,γ) ⁶⁰ Co	1.53E-12	1.68E-12	1.57E-12	0 91	0.97
⁵⁹ Co(n, γ) ⁶⁰ Co (Cd)	1.06E-12	8.19E-13	1.03E-12	1.29	1.03

Table D-5

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· A. A. A.		<u>`</u>	eV) [n/cm ² -s])))((), (); (%
Capsule ID	Calculated	Best Estimate	Uncertainty (10)	BE/C
X	8.01E+10	8.11E+10	6%	1.01
W	5.37E+10	5.31E+10	7%	0.99
V	7.10E+10	6.59E+10	6%	0.93
S	2.27E+10	1.91E+10	6%	0.84
Y	4 27E+10	4.10E+10	6%	0.96

Comparison of Calculated and Best Estimate Exposure Rates At The Surveillance Capsule Center

	, , , , , , , , , , × , , , × , , , , ,	Iron Atom Displac	ement Rate [dpa/s]	
Capsule ID	Calculated	Best Estimate	Uncertainty (10)	BE/C
Х	1.34E-10	1.32E-10	7%	0.98
W	8.75E-11	8.52E-11	8%	0.97
V	1.19E-10	1.09E-10	7%	0 92
S	3.64E-11	3.08E-11	6%	0 85
Y	6.99E-11	6.72E-11	7%	0.96

Table D-6

	i e in in in i		M/C Ratio	<u> Iddittion óf /</u>	* - 3 ^{**} ,*** * `****
Reaction	Capsule X	Capsule W	Capsule V	Capsule S	Capsule Y
63 Cu(n, α) 60 Co (Cd)	1.12	1.16	1.02	1.03	0.98
⁵⁴ Fe(n,p) ⁵⁴ Mn	1.13	1.04	0.96	0.85	0.91
⁵⁸ Ni(n,p) ⁵⁸ Co (Cd)	1.04	1.03	0.89	0.86	
²³⁸ U(n,f) ¹³⁷ Cs (Cd)	0.98	0.92	0.94	0.80	0.88
²³⁷ Np(n,f) ¹³⁷ Cs (Cd)	0.88		0.92	0.88	1.15
Average	1.03	· 1.04	0.95	0.89	0.98
Sample	10.2	9.5	5.1	9.6	12.4
% Standard Deviation					

Comparison of Measured/Calculated (M/C) Sensor Reaction Rate Ratios Including all Fast Neutron Threshold Reactions

Notes:

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1. The overall average M/C ratio for the set of 23 sensor measurements is 0.97 with an associated sample standard deviation of 10.5%.

Table D-7

Comparison of Best Estimate/Calculated (BE/C) Exposure Rate Ratios

	BE/C Ratio						
Capsule ID	φ(E > 1.0 MeV)	dpa/s					
X	1.01	0.98					
W	0.99	0.97					
V	0.93	0.92					
S	0.84	0.85					
Y	0.96	0.96					
Average	0.95	0.94					
% Standard Deviation	6.9	6.0					

Appendix D References

- D-1. Regulatory Guide RG-1.190, "Calculational and Dosimetry Methods for Determining Pressure Vessel Neutron Fluence," U. S. Nuclear Regulatory Commission, Office of Nuclear Regulatory Research, March 2001.
- D-2. A Schmittroth, *FERRET Data Analysis Core*, HEDL-TME 79-40, Hanford Engineering Development Laboratory, Richland, WA, September 1979.
- D-3. RSIC Data Library Collection DLC-178, "SNLRML Recommended Dosimetry Cross-Section Compendium", July 1994.

ATTACHMENT 2

EVALUATION OF APPLICATION OF SURRY UNIT 2 CAPSULE Y SURVEILLANCE DATA TO SURRY UNIT 2 LOWER SHELL PLATE MATERIAL C4339-1 AND INTERMEDIATE TO LOWER SHELL CIRCUMFERENTIAL WELD MATERIAL R3008

ATTACHMENT 2

EVALUATION OF APPLICATION OF SURRY UNIT 2 CAPSULE Y SURVEILLANCE DATA TO SURRY UNIT 2 LOWER SHELL PLATE MATERIAL C4339-1 AND INTERMEDIATE-TO-LOWER SHELL CIRCUMFERENTIAL WELD MATERIAL R3008

BACKGROUND

Surry Unit 2 surveillance Capsule Y was withdrawn from the Surry Unit 2 reactor vessel on March 31, 2002. Capsule Y contains Surry Unit 2 lower shell plate material C4339-1 and intermediate-to-lower shell circumferential weld material R3008 (weld wire heat 0227).

This evaluation provides revised Surry Unit 2 data tables for the NRC's Reactor Vessel Integrity Database (RVID) and an evaluation of changes relative to the previous RVID update for Surry Unit 2 (1). The evaluation considers the impact of Surry Unit 2 Capsule Y basis reactor coolant system (RCS) licensing surveillance data on (a) pressure/temperature (P/T) limit curves, (b) the associated Low Temperature Overpressure Protection System (LTOPS) setpoints and enabling temperature, (c) 10 CFR 50.61 Pressurized Thermal Shock (PTS) screening calculations, and (d) Charpy Upper Shelf Energy (CvUSE) values. The evaluation was performed in a manner consistent with applicable regulatory guidance. Specifically, the calculation of the Reference Temperature for the Nil Ductility Transition (RTNDT) is performed in accordance with Regulatory Guide 1.99 Revision 2 (3), and the regulatory guidance provided in the meeting minutes from the November 12, 1997 NRC/Industry meeting on reactor vessel integrity (5). PTS screening calculations were performed in accordance with 10 CFR 50.61 (2). CvUSE values were developed in accordance with Regulatory Guide 1.99 Revision 2 (3). Evaluation results are presented in a format consistent with the data requirements of the NRC's Reactor Vessel Integrity Database (RVID).

DISCUSSION OF CHANGES TO PREVIOUSLY REPORTED INFORMATION

Surry Unit 2 revised RVID data tables are presented in Appendix A. Shaded cells in Appendix A indicate a changed value relative to those currently presented in RVID (Version 2.0.1, dated 7/6/00). The following changes have been incorporated into the revised tables:

Surry Unit 2 Lower Shell Plate material C4339-1 and Intermediate-to-Lower Shell Circumferential Weld Material R3008

 The RG 1.99 Revision 2 Position 2.1 chemistry factor (CF) calculation includes consideration of the capsule Y analysis results. The Capsule Y data are documented in Table 5-12 of Reference (4).

- Because the surveillance capsules were irradiated in a single reactor and the surveillance material was derived from a single source, irradiation temperature and chemistry corrections are not applied in the credibility determination.
- The surveillance data applicable to C4339-1were determined to be non-credible. However, the data were within 2σ of the RG 1.99 Rev. 2 Position 1.1 curve based on a CF for the average surveillance material chemical composition. Therefore, the Position 1.1 CF value was applied to the C4339-1 beltline material with a full margin term. The surveillance data for the R3008 material were determined to be credible, so the RG 1.99 Rev. 2 Position 2.1 CF values were applied for this material.

EVALUATION OF EXISTING P/T LIMITS AND LTOPS SETPOINTS

The existing Surry Units 1 and 2 P/T limits and LTOPS setpoints (7)(8) are based on a limiting ¼-thickness (¼-T) RTNDT of 228.4°F. When the P/T limits and LTOPS setpoints were developed, this value of RTNDT was determined to bound all Surry Units 1 and 2 reactor vessel beltline materials at fluences corresponding to 28.8 EFPY and 29.4 EFPY for Surry Units 1 and 2, respectively (7)(8). RTNDT calculations have been performed for all Surry Unit 2 reactor vessel beltline materials at a neutron fluence value corresponding to 30.1 EFPY (1). The results are presented in Appendix A. After consideration of the aforementioned changes to previously reported information, the most limiting ¼-T RTNDT value for Surry Unit 2 is 208.8°F at the fluence value corresponding to a cumulative core burnup of 30.1 EFPY (1). However, the P/T Limits and LTOPS Setpoints are based on a limiting ¼-T RTNDT value of 228.4°F. Therefore, the existing P/T Limits and LTOPS Setpoints remain valid and conservative.

EVALUATION OF PTS SCREENING CALCULATIONS

PTS screening calculations have been performed for all Surry Unit 2 reactor vessel beltline materials at a neutron fluence value corresponding to 30.1 EFPY (1). The results of these calculations are presented in Appendix A. After consideration of the aforementioned changes to previously reported information, it is concluded that all Surry Unit 2 beltline materials continue to meet the 10 CFR 50.61 screening criteria.

REPORT OF CVUSE VALUES

CvUSE data and calculations are presented in Appendix A. Although surveillance Capsule Y only contained Surry Unit 2 lower shell plate material C4339-1 and intermediate-to-lower shell circumferential weld material R3008 (weld wire heat 0227), the CvUSE table reflects the following changes:

- The current listing in the RVID for weld material L737/4275 (nozzle-to-intermediate shell circumferential weld) identifies the material as Linde 80 material when in fact it consists of SAF 89 material.

- The current listing in the RVID for weld material R3008/0227 (intermediate-to-lower shell circumferential weld) identifies the material as Linde 80 material when in fact it consists of Grau Lo material.
- ¼-T USE fluence values are calculated using the RG 1.99 Rev. 2 Position 1.1 attenuation equation. The RG 1.99 Rev. 2 Position 1.1 methodology includes the vessel clad material thickness in the fluence calculation (i.e., "...depth into the vessel wall measured from the inner (wetted) surface."):

 $f/f_{surf} = \exp(-0.24x), x = [0.25*vessel thickness] + [clad thickness]$

For Surry Unit 2, the maximum allowable f/f_{surf} would be:

 $x = [0.25*8.079 \text{ in}] + [0.157 \text{ in}] = 2.177 \text{ in}, f/f_{surf} = 0.593$

However, Surry Unit 2 attenuation was calculated by the more conservative approach of calculating wall depth by taking 25% of the total including the clad thickness:

 $x = [0.25^{*}(8.079 \text{ in} + 0.157 \text{ in})] = 2.059 \text{ in}, f/f_{surf} = 0.610$

The higher f/f_{surf} produces a higher \triangle RTNDT, so the ¼-T fluences calculated for Surry Unit 2 are conservative with respect to the RG 1.99 Rev. 2 Position 1.1 methodology.

- The values for unirradiated USE for materials C4331-2 and C4339-1 appeared to have been mistyped in the RVID. Corrected values have been provided.
- The percentage drops in CvUSE values were calculated using the RG 1.99 Rev. 2 Position 1.2 methodology. CvUSE data obtained from surveillance capsules compares favorably with predictions. For those Rotterdam and Linde 80 materials that are below 50 ft-lbs, equivalent margin analyses (EMAs) have been previously approved in References (9) and (10).

CONCLUSIONS

After consideration of the aforementioned changes to previously reported information, the most limiting ¼-T RTNDT value for Surry Unit 2 is 208.8°F at a fluence value corresponding to a cumulative core burnup of 30.1 EFPY. The existing Surry Unit 2 Technical Specification RCS P/T limits, LTOPS setpoints, and LTOPS enabling temperature are based upon a ¼-T RTNDT value of 228.4°F (7) (8). Therefore, the analyses supporting the Surry Unit 2 RCS P/T limits, LTOPS setpoints, and LTOPS enabling temperature remain valid and conservative (7) (8). In addition, after consideration of the aforementioned changes to previously reported information, all Surry Unit 2 reactor vessel beltline materials continue to meet the 10 CFR 50.61 PTS screening criteria for cumulative core burnups up to 30.1 EFPY. Finally, calculated Surry Unit 2 CvUSE values continue to be greater than the 50 ft-lb 10CFR50 Appendix G criterion.

NRC REACTOR VESSEL INTEGRITY DATABASE

Virginia Power requests that information presented in Appendix A be used to update the NRC Reactor Vessel Integrity Database (RVID).

FUTURE CAPSULE EXTRACTION PLANS

The currently docketed reactor vessel materials surveillance program includes withdrawal of the final Surry Unit 2 surveillance capsule at a fluence value corresponding to a cumulative core burnup of 30.1 EFPY. As a result of Dominion's license renewal efforts for Surry Unit 2, a submittal is planned to change the capsule withdrawal schedule to reflect the recommendations of the Generic Aging Lessons Learned (GALL) report (6).

REFERENCES

- (1) Letter from L. N. Hartz to USNRC, "Virginia Electric and Power Company, North Anna Power Station Units 1 and 2, Surry Power Station Units 1 and 2, Evaluation of Reactor Vessel Materials Surveillance Data," Serial Number 99-452A dated November 19, 1999.
- (2) Title 10, Code of Federal Regulations, Part 50.61, "Fracture Toughness Requirements for Protection Against Pressurized Thermal Shock Events."
- (3) Regulatory Guide 1.99 Revision 2, "Radiation Embrittlement of Reactor Vessel Materials," dated May, 1988.
- (4) WCAP-16001, "Analysis of Capsule Y from Dominion Surry Unit 2 Reactor Vessel Radiation Surveillance Program," dated February 2003.
- (5) Memorandum from K. R. Wichman to E. J. Sullivan, "Meeting Summary for November 12, 1997 Meeting with Owners Group Representatives and NEI Regarding Review of Responses to Generic Letter 92-01, Revision 1, Supplement 1 Responses," dated November 19, 1997.
- (6) NUREG-1801, "Generic Aging Lessons Learned (GALL) Report," dated July 2001.
- (7) Letter from R. F. Saunders to USNRC, "Virginia Electric and Power Company, Surry Power Station Units 1 and 2, Request for Exemption - ASME Code Case N-514, Proposed Technical Specifications Change, Revised Pressure/Temperature Limits and LTOPS Setpoint," Serial No. 95-197, June 8, 1995.
- (8) Letter from B. C. Buckley to J. P. O'Hanlon, "Surry Units 1 and 2 Issuance of Amendments Re: Surry Units 1 and 2 Reactor Vessel Heatup and Cooldown

Curves (TAC Nos. M92537 and M92538)," Serial No. 96-020, dated December 28, 1995.

- (9) BAW-2178PA, "Low Upper Shelf Toughness Fracture Mechanics Analysis of Reactor Vessels of B&W Owners Reactor Vessel Working Group for Level C & D Service Loads," dated April 1994.
- (10) BAW-2192PA, "Low Upper Shelf Toughness Fracture Analysis of Reactor Vessels of B&W Owners Group Reactor Vessel Working Group for Level A & B Conditions," dated April 1994.

APPENDIX A

REACTOR VESSEL MATERIALS DATA TABLES FOR SURRY UNIT 2

(11 pages)

Facility: Surry Unit 1 Vessel Manufacturer: B&W and Rotterdam Dockyard

RPV Weld Wire Heat or Material ID	Location	Best- Estimate Copper (wt%)	Best- Estimate Nickel (wt%)	ID Fluence (x1E19)	Assigned Material Chemistry Factor	Method of Determining CF	Initial RT(NDT)	Sigma(I)	Sigma(delta)	Margin	Inner Surf. ART or RT(PTS)	1/4-T ART*
122V109VA1	Nozzle Shell Forging	0 1 1 0	0 740	0 307	76 1	Tables	40	00	17 0	34 0	125 5	116 1
C4326-1	Intermediate Shell	0 110	0 550	3 530	73 5	Tables	10	00	17 0	34 0	1416	132 8
C4326-2	Intermediate Shell	0 1 1 0	0 550	3 530	73 5	Tables	0	00	17 0	34 0	131 6	122 8
4415-1	Lower Shell	0 102	0 493	3 530	85 0	Surv Data	20	00	85	17 0	149 9	139 7
4415-2	Lower Shell	0 1 1 0	0 500	3 530	73 0	Tables	0	00	17 0	34 0	131 0	122 2
J726/25017	Nozzle to Int Shell Circ Weld	0 330	0 100	0 307	152 0	Tables	0	20 0	28 0	68 8	1716	153 0
SA-1585/72445	Int to Low Sh Circ (ID 40%)	0 220	0 540	3 200	131 4	Surv Data	-5	197	28 0	68 5	235 1	218 9
SA-1650/72445	Int to Low Sh Circ (OD 60%)	0 220	0 540	3 200	131 4	Surv Data	-5	197	28 0	68 5	235 1	218 9
SA-1494/8T1554	Int Shell Long Welds L3 & L4	0 160	0 570	0 600	143 9	Tables	-5	197	28 0	68 5	186 8	167 4
SA-1494/8T1554	Lower Shell Long Weld L1	0 160	0 570	0 540	143 9	Tables	-5	197	28 0	68 5	182 6	163 4
SA-1526/299L44	Lower Shell Long Weld L2	0 340	0 680	0 540	220 6	Tables	-7	20 6	28 0	69 5	245 1	215 7

* 1/4-T ART value of 228 4 F was used in the determination of P/T limits

Note Shaded cells indicate a changed value relative to the NRC's Reactor Vessel Integrity Database (RVID) Version 2 0 1 (Data Update on 7/6/00)

Facility: Surry Unit 2 Vessel Manufacturer: B&W and Rotterdam Dockyard

RPV Weld Wire Heat or Material ID	Location	Best- Estimate Copper (wt%)	Best- Estimate Nickel (wt%)	ID Fluence (x1E19)	Assigned Material Chemistry Factor	Method of Determining CF	Initial RT(NDT)	Sigma(I)	Sıgma(delta)	Margin	Inner Surf ART or RT(PTS)	1/4-T ART*
123V303VA1	Nozzle Shell Forging	0 1 1 0	0 720	0 298	75 8	Tables	30	00	17 0	34 0	1147	105 5
C4331-2	Intermediate Shell	0 120	0 600	3 520	83 0	Tables	-10	00	17 0	34 0	134 2	124 2
C4339-2	Intermediate Shell	0 1 1 0	0 540	3 520	73 4	Tables	•20	0.0	170	34 0	111.5	_102 6
C4208-2	Lower Shell	0 150	0 550	3 520	107 3	Tables	-30	00	17 0	34 0	146 4	133 5
C4339-1	Lower Shell	0 107	0 530	3 520	70 8	Tables	-10	00	17 0 🐣	34 0	1180	109 5
L737/4275	Nozzle to Int Shell Circ Weld	0 350	0 100	0 298	160 5	Tables	0	20 0	28 0	68 8	176 1	156 6
R3008/0227	Int to Lower Shell Circ Weld	0 187	0 545	3 520	132 4	Surv Data	0	20 0	14 0	48 8	224 7	_208 8
WF-4/8T1762	Int Shell Long L4 (ID 50%)	0 190	0 570	0 697	152 4	Tables	-5	197	28 0	68 5	200 4	179 6
SA-1585/72445	Int Sh L3 (100%), L4 (OD 50)	_0 220	0 540	0 697	131 4	Surv Data	-5	197	28 0	68 5	181 6	163 7
WF-4/8T1762	LS L2 (ID 63%), L1 (100)	0 190	0 570	0 697	152 4	Tables	-5	197	28 0	68 5	200 4	1796
WF-8/8T1762	LS Long Weld L2 (OD 37%)	0 190	0 570	0 697	152 4	Tables	-5	19.7	28 0	68 5	200 4	179 6

* 1/4-T ART value of 228 4 F was used in the determination of P/T limits

Note Shaded cells indicate a changed value relative to the NRC's Reactor Vessel Integrity Database (RVID) Version 2 0 1 (Data Update on 7/6/00)

CvUSE Values Facility: Surry Unit 2 Vessel Manufacturer: B&W and Rotterdam Dockyard

RPV Weld Wıre Heat or Material ID	Location	Forging or Flux Type	USE @ 1/4 T	1/4-T Fluence (x1E19)	Unirradiated USE	Unirradiated USE Method	%Drop in USE @ 1/4 T	%Drop in USE Method	Cu %
123V303VA1	Nozzle Shell Forging	SA508, CI 2	89 5	0 182	104 0	Measured/MTEB 5-2	14 0%	Pos 1 2	0 11
C4331-2	Intermediate Shell	SA533, Gr B1	63 0	2 147	84 0 🗠	Measured/MTEB 5-2	25 0%	Pos 1 2	0 12
C4339-2	Intermediate Shell	SA533, Gr. B1	63 2	2,147	83 0	Measured/MTEB 5-2	23 9%	Pos 1 2	0 11
C4208-2	Lower Shell	SA533, Gr B1	67 3	2 147	94 0	Measured/MTEB 5-2	28 4%	Pos 1 2	0 15
C4339-1	Lower Shell	SA533, Gr B1	80 2	2 147	105 0	Measured	· 23 6%	Pos 1 2	0 11
L737/4275	Nozzle to Int Shell Circ Weld	SAF 89	EMA	0 182	EMA	Estimate	EMA	EMA	0 35
R3008/0227	Int to Lower Shell Circ Weld	Grau Lo	55 0	2 147	90 0	Measured	38 9%	Pos 1.2	0 19
WF+4/8T1762	Int, Shell Long L4 (ID 50%)	Linde 80	EMA	0 425	EMA	Estimate	EMA	EMA	0 19
SA-1585/72445	Int Sh L3 (100%), L4 (OD 50)	Linde 80	54 0	0 425	77 0	Measured	29 9%	Pos 1 2	0 22
WF+4/8T1762	LS L2 (ID 63%), L1 (100)	Linde 80	EMA	0 425	EMA	Estimate	EMA	EMA	0 19
WF+8/8T1762	LS Long Weld L2 (OD 37%)	Linde 80	EMA	0 425	EMA	Estimate	EMA	EMA	0 19

Note Shaded cells indicate a changed value relative to the NRC's Reactor Vessel Integrity Database (RVID) Version 2 0 1 (Data Update on 7/6/00)

Surry Unit 2 Plate Material C4339-1 (Combined Longitudinal and Transverse Data)

Capsule ID (Including Source)	Copper (wt%)	Nickel (wt%)	Irradiation Temperature (F)	Fluence (x1E19)	Measured Delta-RT(NDT) (F)	Data Used In Assessing Vessei? (Yes or No)
Surry Unit 2 Capsule X (Long)	0 104	0 520	534 9	0 297	, 59	Yes
Surry Unit 2 Capsule V (Long)	0 104	0 520	540 1	1 890	· · · 79 · · ·	Yes
Surry Unit 2 Capsule X (Trans)	0 104	0 520	534 9	0 297	49 ,	Yes
Surry Unit 2 Capsule V (Trans)	0.104	0 520	540 1	··· 1 890 ·	64	Yes
Surry Unit 2 Capsule Y (Long)	0 104	0 520	543 7 🔸	2 730	114	· Yes
Surry Unit 2 Capsule Y (Trans)	0 104	0 520	. 543 7	2 730	- 107	Yes
-	•	•	•	-	-	•
•	•	-	-	-	-	-
-	•	-	•	•	•	•
-		•	•	-	-	•
•	•	•	•	*	-	•
•	•	-	•	•	•	-

Table 3:

Table 2:

Surry Unit 2 Plate Material C4339-1 (Combined Longitudinal and Transverse Data)

Capsule ID (Including Source)	Copper (wt%)	Nickel (wt%)	Irradiation Temperature (F)	Fluence Factor	Measured Delta-RT(NDT) (F)	Adjusted Delta- RT(NDT) (F) *	Adjusted - Predicted Delta- RT(NDT) (F) *
Surry Unit 2 Capsule X (Long)	0 104	0 520	534 9	5 0 6677 · · ·		1, 59 ^{m 1}	5 1 9
Surry Unit 2 Capsule V (Long)	0 104	0 520	540 1	1.1743	79	79	• • 1 0
Surry Unit 2 Capsule X (Trans)	0 104	0 520	534 9	0 6677	, i • 4 9	49	-2
Surry Unit 2 Capsule V (Trans)	0 104	0 520	540 1	^{**} - 1 1743	64	64	-25
Surry Unit 2 Capsule Y (Long)	0 104	0 520	- 543 7	1 2679	~` 114	114	··· · 18
Surry Unit 2 Capsule Y (Trans)	0 104	0 520	543 7	<u>``12679</u>	- 107	107	11 .
•	-	-	•	•	•	-	•
•	•	•	•	•	•	•	•
-		•	•	•	•	•	•
-	•	•	•	•	•	-	•
•	•	•	•	-	•	-	•
•	-	•		-	•	•	•

* For credibility check, measured shift values are adjusted to average surveillance material chemistry and irradiation temperature as required. See Table 4

.

Surry Unit 2 Plate Material C4339-1 (Combined Longitudinal and Transverse Data)

CF Determination

						Surveillance Data	If Surv Data Non-Credible,	
	Beltline Material	Beltline Material	Irradiation			Credible or Non-	Verify Conservatism of	Chemistry Factor Applied to Beltline
Beltline Material ID	Copper (wt%)	Nickel (wt%)	Temperature (F)	Position 1 1 Chemistry Factor	Position 2 1 Chemistry Factor	Credible?	Position 1.1 CF *	Matenal **
Plate C4339-1	0 107	0 530	543.7	70 8	75 7 ·	Non-Credible	0.0	70.8

* Measured shift values are adjusted to the average surveillance material chemistry and irradiation temperature, and are verified to be within 2 sigma of the trend curve based on RG 1 99 Rev 2 Position 1 1

** If surveillance data are non-credible but the Pos. 1 1 CF is shown to be conservative, the lower of the Pos 1.1 and Pos 2.1 chemistry factors is applied to the beltline material with a full margin term.

If surveillance data are non-credible and the Pos 11 CF is shown to be non-conservative, the greater of the Pos 1.1 and Pos 21 chemistry factors is applied to the beltime material with a full margin term

Credibility Assessment

						Conservatism Chec	k for Pos 1.1 CF when Surv.	Data Non-Credible
Capsule ID (Including Source)	(1) Temperature Correction Applied for Credibility?	(2) Chemistry Correction Applied for Credibility?	Survelllance Data Credible or Non- Credible?	(3) Temperature Correction Applied to Surv Data for Application to Beltline Material?	(4) Chemistry Correction Applied to Surv Data for Application to Beltline Material?	Adjusted Deita- RT(NDT) (F) *	Adjusted - Predicted Delta- RT(NDT) (F) *	Are adjusted surveillance data within 2 sigma of the applied CF trend curve? *
Surry Unit 2 Capsule X (Long)	No	No	Credible	No	No	· 59 ′	13	Conservative *
Surry Unit 2 Capsule V (Long)	No	No	Credible	No	No		- 1 -	- Conservative
Surry Unit 2 Capsule X (Trans)	No	No	Credible	No	No	49	1 × 3 * 2	Conservative
Surry Unit 2 Capsule V (Trans)	No	No	Non-Credible	No	No	64	- 17	Conservative
Surry Unit 2 Capsule Y (Long)	No	¹ No	Non-Credible	No 🤟	No a	114	28 [.]	Conservative
Surry Unit 2 Capsule Y (Trans)	No	No 🎽	Credible ·	No 🔎	це т No * » с	<u>_</u> 107	20 .	Conservative
-	-	-	-	•	-	-	-	-
-	-	-	•	•	•	•	•	•
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(1) For the credibility determination, a temperature correction is not applied to measured values of transition temperature shift if applicable surveillance data were irradiated in a single reactor (i.e., were irradiated at a similar temperature)

(2) For the credibility determination, a chemistry correction is not applied to measured values of transition temperature shift if applicable surveillance data were obtained from a single source (i e, were machined from the same block of material)

(3) For determination of the beltline material chemistry factor, a temperature correction is not applied to measured values of transition temperature shift if applicable surveillance data were irradiated in the reactor vessel which is being evaluated (i e, were irradiated at a similar temperature). A temperature correction is applied only in the conservative direction

(4) For determination of the beltline material chemistry factor, a chemistry correction (i e, ratio procedure) is not applied to measured values of transition temperature shift if the chemical composition of applicable surveillance data is essentially identical to the best-estimate chemical composition of the beltline material being evaluated

Table 4:

Capsule ID (Including Source)	Fluence (x1E19)	Copper (wt%)	CvUSE (ft-lb)	CvUSE - Drop Measured %	CVUSE - Drop Predicted %	Delta-CvUSE %
Unirradiated Surv Material (Long)	0 00	0 10	128 00	-	-	•
Unirradiated Surv Material (Trans)	0 00	0 10	104 00	-	•	•
Surry Unit 2 Capsule X (Long)	. 0 30	0 10	122 00	4.7%	14 8%	-10 1%
Surry Unit 2 Capsule V (Long)	1 89 🧭	° (010 ∞	- 121 00	5 5%	· · · · · 22.6%	17 1% 🕴
Surry Unit 2 Capsule X (Trans)	0 30	° 010 →	94 00	96%	14 8%	-5 1%
Surry Unit 2 Capsule V (Trans)	1 89	0 10	94 00	9 6%	22 6%	⊸-12 9%
Surry Unit 2 Capsule Y (Long)	2 73	0 10	111 00	13 3%	24 4%	-11.1%
Surry Unit 2 Capsule Y (Trans)	2 73	0.10	94 00	96%	24 4%	-14 8%
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Table 5: CvUSE Data Surry Unit 2 Plate Material C4339-1 (Combined Longitudinal and Transverse Data)

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Table 2:

Surry Unit 2 Weld Material R3008

Capsule ID (Including Source)	Copper (wt%)	Nickel (wt%)	Irradiation Temperature (F)	Fluence (x1E19)	Measured Delta-RT(NDT) (F)	Data Used In Assessing Vessel? (Yes or No)
Surry Unit 2 Capsule X	0 187	0 545	534 9	0 297 .	96	Yes
Surry Unit 2 Capsule V	0 187	0 545	540 1	- 1 890	140	Yes
Surry Unit 2 Capsule Y	0 187	0 545	543 7	2.730	178	Yes
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-	•	-	•	•	-	-
-	•	-	•	•	-	-
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Table 3:

Surry Unit 2 Weld Material R3008

Capsule ID (Including Source)	Copper (wt%)	Nickel (wt%)	Irradiation Temperature (F)	Fluence Factor	Measured Delta-RT(NDT) (F)	Adjusted Deita- RT(NDT) (F) *	Adjusted - Predicted Delta- RT(NDT) (F) *
Surry Unit 2 Capsule X	0 187	0 545	534 9	0 6677	· 96	96	· 7 ·
Surry Unit 2 Capsule V	0 187	0 545	540 1	<u> </u>	140	<u> </u>	15
Surry Unit 2 Capsule Y	0 187	0 545	543 7	1 2679	178	178	10
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* For credibility check, measured shift values are adjusted to average surveillance material chemistry and irradiation temperature as required. See Table 4

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Surry Unit 2 Weld Material R3008

CF Determination

						Surveillance Data	If Surv Data Non-Credible,	
	Beltline Material	Beltline Material	Irradiation			Credible or Non-	Verify Conservatism of	Chemistry Factor Applied to Beltline
Beltline Material ID	Copper (wt%)	Nickel (wt%)	Temperature (F)	Position 1 1 Chemistry Factor	Position 2 1 Chemistry Factor	Credible?	Position 1 1 CF *	Material **
R3008	0 187	0 545	543 7	147 5	- 132.4	Credible	•	132 4

* Measured shift values are adjusted to the average surveillance material chemistry and irradiation temperature, and are verified to be within 2 sigma of the trend curve based on RG 1 99 Rev 2 Position 1 1

** If surveillance data are non-credible but the Pos 1.1 CF is shown to be conservative, the lower of the Pos 1.1 and Pos 2.1 chemistry factors is applied to the beltline material with a full margin term.

If surveillance data are non-credible and the Pos 11 CF is shown to be non-conservative, the greater of the Pos 11 and Pos 21 chemistry factors is applied to the beltline material with a full margin term

Credibility Assessment

Conservatism Check for Pos 1 1 CF when Surv Data Non-Credible											
Capsule ID (Including Source)	(1) Temperature Correction Applied for Credibility?	(2) Chemistry Correction Applied for Credibility?	Surveillance Data Credible or Non- Credible?	(3) Temperature Correction Applied to Surv. Data for Application to Bettline Material?	(4) Chemistry Correction Applied to Surv. Data for Application to Beltline Material?	Adjusted Delta- RT(NDT) (F) *	Adjusted - Predicted Delta- RT(NDT) (F) *	Are adjusted surveiliance data within 2 sigma of the applied CF trend curve? *			
Surry Unit 2 Capsule X	No	No	Credible	No	No	-	•	÷.			
Surry Unit 2 Capsule V	No	No	Credible	No	No	•	•				
 Surry Unit 2 Capsule Y 	No	No	Credible	No ***	in No	-	-	-			
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(1) For the credibility determination, a temperature correction is not applied to measured values of transition temperature shift if applicable surveillance data were irradiated in a single reactor (i.e., were irradiated at a similar temperature)

(2) For the credibility determination, a chemistry correction is not applied to measured values of transition temperature shift if applicable surveillance data were obtained from a single source (i e, were machined from the same block of material).

(3) For determination of the beltiline material chemistry factor, a temperature correction is not applied to measured values of transition temperature shift if applicable surveillance data were irradiated in the reactor vessel which is being evaluated (i e, were irradiated at a similar temperature). A temperature correction is applied only in the conservative direction

(4) For determination of the beltline material chemistry factor, a chemistry correction (i e, ratio procedure) is not applied to measured values of transition temperature shift if the chemical composition of applicable surveillance data is essentially identical to the best-estimate chemical composition of the beltline material being evaluated

Table 4:

Table 5: CvUSE Data Surry Unit 2 Weld Material R3008

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Capsule ID (Including Source)	Fluence (x1E19)	Copper (wt%)	CVUSE (ft-lb)	CVUSE - Drop Measured %	CvUSE - Drop Predicted %	Delta-CvUSE %
Unirradiated Surv Material	0 00	0 19	91 00	•	•	•
Surry Unit 2 Capsule X	0 30	0 19	71.00	22 0%	24 6%	-2 7%
Surry Unit 2 Capsule V	1 89	0 19	60 00	34 1%	37 8%	- 37%
Surry Unit 2 Capsule Y	2 73	0 19	58 00	36 3%	40 9% -	-4 6%
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•	•	-	•	-	-	•
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Surry Unit 1 and 2 Weld Material SA-1585 (Point Beach 1 Data Only)

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Capsule ID (Including Source)	Copper (wt%)	Nickel (wt%)	Irradiation Temperature (F)	Fluence (x1E19)	Measured Delta-RT(NDT) (F)	Data Used In Assessing Vessel? (Yes or No)
Point Beach Unit 1 Capsule T	0 230	0 615	533 4	2 230	181	Yes
Point Beach Unit 1 Capsule R	0 230	0 615	541 6	2 190	155	Yes
Point Beach Unit 1 Capsule S	0 230	0 615	542 0	0 829	165	Yes
Point Beach Unit 1 Capsule V	0 230	0 615	542 0	0 634	107	Yes
Capsule CR3-LG2 (BWOG CR-3 Irrad)	0 220	0 590	556 0	1 670	164	No
Capsule CR3-LG1 (BWOG CR-3 Irrad)	0 220	0 590	556 0	0 510	139	No
Capsule W-1 (ANO-1 NBD)	0 220	0 590	546 3	0 660	138	No
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Table 3:

Table 2:

Surry Unit 1 and 2 Weld Material SA-1585 (Point Beach 1 Data Only)

Capsule ID (Including Source)	Copper (wt%)	Nickel (wt%)	Irradiation Temperature (F)	Fluence Factor	Measured Delta-RT(NDT) (F)	Adjusted Delta- RT(NDT) (F) *	Adjusted - Predicted Delta- RT(NDT) (F) *
Point Beach Unit 1 Capsule T	0 230	0 615	533 4	1 2173	181	181	7
Point Beach Unit 1 Capsule R	0 230	0 615	541 6	1 2126	155	155	-18
Point Beach Unit 1 Capsule S	0 230	0 615	542 0	0 9474	165	165	30
Point Beach Unit 1 Capsule V	0 230	0 615	542 0	0 8723	107	107	•17
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• For credibility check, measured shift values are adjusted to average surveillance material chemistry and irradiation temperature as required See Table 4.

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Surry Unit 1 and 2 Weld Material SA-1585 (Point Beach 1 Data Only)

CF Determination

						Surveillance Data	If Surv Data Non-Credible,	
	Beltline Material	Beltline Material	Irradiation			Credible or Non-	Verify Conservatism of	Chemistry Factor Applied to Beltline
Beltine Material ID	Copper (wt%)	Nickel (wt%)	Temperature (F)	Position 1 1 Chemistry Factor	Position 2.1 Chemistry Factor	Credible?	Position 1.1 CF*	Material **
SA-1585/72445	0 220	0 540	542 0	158 0	131 4	Non-Credible	00	131.4

* Measured shift values are adjusted to the average surveillance material chemistry and irradiation temperature, and are venfied to be within 2 sigma of the trend curve based on RG 1 99 Rev. 2 Position 1.1.

** If surveillance data are non-credible but the Pos 1 1 CF is shown to be conservative, the lower of the Pos 1.1 and Pos 2.1 chemistry factors is applied to the beltline material with a full margin term.

If surveillance data are non-credible and the Pos 1 1 CF is shown to be non-conservative, the greater of the Pos 1.1 and Pos 2.1 chemistry factors is applied to the beltline material with a full margin term

Credibility Assessment

Conservatism Check for Pos 11 CF when Surv Data Non-Credible									
Capsule ID (Including Source)	(1) Temperature Correction Applied for Credibility?	(2) Chemistry Correction Applied for Credibility?	Surveillance Data Credible or Non- Credible?	(3) Temperature Correction Applied to Surv Data for Application to Beltline Material?	(4) Chemistry Correction Applied to Surv Data for Application to Beitline Material?	Adjusted Delta- RT(NDT) (F) *	Adjusted - Predicted Delta- RT(NDT) (F) *	Are adjusted surveillance data within 2 sigma of the applied CF trend curve? *	
Point Beach Unit 1 Capsule T	No	No	Credible	Yes	Yes	181	-28	Conservative	
Point Beach Unit 1 Capsule R	No	No	Credible	Yes	Yes	155	-53	Conservative	
Point Beach Unit 1 Capsule S	No	No	Non-Credible	Yes	Yes	165	2	Conservative	
Point Beach Unit 1 Capsule V	No	No	Credible	Yes	Yes	107	-43	Conservative	
Capsule CR3-LG2 (BWOG CR-3 Irrad)	•	-	-	•	•	-	•	-	
Capsule CR3-LG1 (BWOG CR-3 Irrad)	•	-		•	•	-	-	•	
Capsule W-1 (ANO-1 NBD)	-	-	•	•	•	•	-	-	
•	•	-	•	•	•	-	-	•	
-	-	•	•	•	•	-	•	-	
•	•	•	•	•	•	•	•	•	
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(1) For the credibility determination, a temperature correction is not applied to measured values of transition temperature shift if applicable surveillance data were irradiated in a single reactor (i e , were irradiated at a similar temperature)

(2) For the credibility determination, a chemistry correction is not applied to measured values of transition temperature shift if applicable surveillance data were obtained from a single source (i e , were machined from the same block of material)

(3) For determination of the bettine material chemistry factor, a temperature correction is not applied to measured values of transition temperature shift if applicable surveillance data were irradiated in the reactor vessel which is being evaluated (i e, were irradiated at a similar temperature). A temperature correction is applied only in the conservative direction

(4) For determination of the beltline material chemistry factor, a chemistry correction (i e, ratio procedure) is not applied to measured values of transition temperature shift if the chemical composition of applicable surveillance data is essentially identical to the best-estimate chemical composition of the beltline material being evaluated.

Table 4:

Capsule ID (Including Source)	Fluence (x1E19)	Copper (wt%)	C√USE (ft-lb)	CvUSE - Drop Measured %	CVUSE - Drop Predicted %	Delta-CvUSE %
Unirradiated Surv Material (Long) SA-						
1263	0 00	0 23	66 00	•	•	-
Unirradiated Surv Material (Long) SA-						
1585	0 00	0 23	79 00	•	-	•
Point Beach Unit 1 Capsule T	2 23	0 23	55 00	16 7%	44 3%	-27.7%
Point Beach Unit 1 Capsule R	2.19	0 23	52 00	21 2%	44 2%	-23 0%
Point Beach Unit 1 Capsule S	0 83	0 23	52 00	21 2%	35 3%	-14 1%
Point Beach Unit 1 Capsule V	0 63	0 23	53 00	19 7%	33 3%	-13 6%
Capsule CR3-LG2 (BWOG CR-3 Irrad)	1 67	0 22	53 00	32 9%	41 4%	-8 5%
Capsule CR3-LG1 (BWOG CR-3 Irrad)	0 51	0 22	56 00	29 1%	31 8%	-2 7%
Capsule W-1 (ANO-1 NBD)	0 66	0 22	51 00	35 4%	33 6%	1 8%
-	-	-	•	-	-	•
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Table 5: CvUSE Data Surry Unit 1 and 2 Weld Material SA-1585 (Point Beach 1 Data Only)

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