**Dominion Nuclear Connecticut, Inc.** Millstone Power Station Rope Ferry Road Waterford, CT 06385



MAR 27 2003

Docket No. 50-336 B18866

RE: 10 CFR 50.46(a)(3)(ii)

U.S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, DC 20555

> Millstone Power Station, Unit No. 2 2002 Annual Reporting of Changes to and Errors in Emergency Core Cooling System Models or Applications

In accordance with 10 CFR 50.46(a)(3)(ii), Dominion Nuclear Connecticut, Inc. (DNC) hereby submits the annual summary of changes to and errors identified in the Emergency Core Cooling System (ECCS) evaluation models or applications of those models for Millstone Unit No. 2

Attachment 1 transmits the annual report for the period January 2002 through December 2002. The following is a synopsis of the information provided in Attachment 1.

#### Small Break Loss of Coolant Accident (SBLOCA)

1. DNC has previously reported the changes in the SBLOCA analysis to the U.S. Nuclear Regulatory Commission (NRC) in a letter dated September 5, 2002, (1) to meet the 30-day reporting criterion of 10 CFR 50.46(a)(3)(ii).

## Large Break Loss of Coolant Accident (LBLOCA)

- 1. As a result of additional validation and verification activities, Framatome ANP (FRA-ANP) identified and corrected a number of minor errors in the RFPAC computer code. The RFPAC code performs the refill and reflood calculations for the LBLOCA event in the SEM/PWR-98 evaluation model. This item resulted in a 16°F increase in Peak Cladding Temperature (PCT).
- 2. FRA-ANP identified an incorrect Reactor Coolant Pump junction area used in the RELAP4 LBLOCA blowdown analysis. Correction of this error resulted in a small change in the pump exit junction area. This item resulted in a 0°F change in PCT.

J. A. Price letter to U.S. Nuclear Regulatory Commission, "Millstone Power Station, Unit No. 2, 30 Day Reporting of Changes to, and Errors in, Emergency Core Cooling System Models or Applications," dated September 5, 2002 (B18734).

U.S. Nuclear Regulatory Commission B18866/Page 2

- 3. FRA-ANP identified an error in the clad thermal expansion calculation in the TOODEE2 computer code. The TOODEE2 code performs the fuel rod heatup calculations for the LBLOCA event in the SEM/PWR-98 evaluation model. This item resulted in a 1°F reduction in PCT.
- 4. FRA-ANP identified an error in the biasing of the accumulator line losses in the LBLOCA ECCS model when applied to the SEM/PWR-98 methodology. This item resulted in a 1°F reduction in PCT.

Considering the changes summarized in Attachment 1, the corrected PCTs for the limiting SBLOCA (1808°F) and LBLOCA (1826°F) remain below the 2200°F limit specified by 10 CFR 50.46(b)(1).

This information satisfies the 2002 annual reporting requirements of 10 CFR 50.46(a)(3)(ii). In addition, no reanalysis or other actions are necessary to demonstrate compliance with 10 CFR 50.46 requirements.

There are no regulatory commitments contained within this letter.

If you should have any questions concerning this submittal, please contact Mr. Ravi Joshi at (860) 440-2080.

Very truly yours,

DOMINION NUCLEAR CONNECTICUT, INC.

J. Alan Price

Site Vice President - Millstone

Attachment (1)

cc: H. J. Miller, Region I Administrator

R. B. Ennis, NRC Senior Project Manager, Millstone Unit No. 2 Millstone Senior Resident Inspector

## Attachment 1

Millstone Power Station, Unit No. 2

2002 Annual Reporting of 10 CFR 50.46 Margin Utilization

## 10 CFR 50.46 Margin Utilization Small Break Loss of Coolant Accident (SBLOCA)

	llstone Unit No. 2	<u> </u>	
Utility Name: Do	minion Nuclear Connecticut, I	nc.	
<b>Analysis Information</b>			
EM:	PWR Small Break LOCA S-RELAP5 Based	Limiting Break Size:	0.08 ft <sup>2</sup>
Analysis Date:	01/02		
Vendor:	Framatome ANP		
Peak Linear Power:	15.1 kW/ft		
Notes:	None		
		Clad Temp (°F)	Notes
LICENSING BASIS  Analysis of Record PCT		1941	(1)
MARGIN ALLOCATIONS (Delta PCT)  A. Prior Permanent ECCS Model Assessments  1. None		0	
<ul><li>B. Planned Plant Change Evaluations</li><li>1. None</li></ul>		0	
<ul> <li>C. 2002 Permanent ECCS Model Assessments</li> <li>1. Decay Heat Model Error</li> <li>2. Revised SBLOCA Guideline</li> </ul>		-133 0	(2)
<ul><li>D. Temporary ECCS Model Issues</li><li>1. None</li></ul>		0	
E. Other Margin Allo 1. None	ocations	0	
LICENSING BASIS PCT	PCT = 1808		

## NOTES:

- (1) New Analysis of Record using S-RELAP5 based methodology.
- (2) Assessment of this change resulted in a PCT differential of +66°F. Framatome ANP provided this assessment for information. The +66°F assessment does not apply since the current Analysis of Record incorporates the revised SBLOCA guideline.

## 10 CFR 50.46 Margin Utilization Large Break Loss of Coolant Accident (LBLOCA)

Plant Name:		Millstone Unit No. 2			
Utility Name:		Dominion Nuclear Connecticut, Inc.			
<u>Analy</u>	ysis Information				
El	M:	SEM/PWR-98 Li	miting Break Size:	1.0 [	DECLG
	nalysis Date:	11/98			
	endor:	Framatome ANP			
Pe	eak Linear Power:	15.1 kW/ft			
N	otes:	None			
•			Clad Temp (	<u>°F)</u>	<u>Notes</u>
LICE	NSING BASIS				
Analysis of Record Po		PCT	18	314	(1)
	GIN ALLOCATION				
A.		ECCS Model Assessments			
		Corrosion Enhancement Factor	r	-1	
		oding Errors		0	
		PAC Fuel Temperatures at Sta	irt of Reflood	-2	
		ujun98 Code Error	-0	0	
		w Blockage Model in TOODEE		0	
		TOODEE2-Calculation of QMA	X.	0	
		Gadolinia Modeling		0	
		OCA Split Break Modeling		0	
		Iculation Error	•	0 0	
		te Heat Transfer in TOODEE2 ass Prediction by TEOBY		0	
	<i>-</i> • • • • • • • • • • • • • • • • • • •	write of Junction Inertia		0	
		unction Inertia Multipliers		1	
		overed During RODEX2 V&V		ò	
		ken Loop SG Tube Exit Juncti	on Inertia	Ô	
	io. Liforni bic	Men 200p 00 Tube Exit bullot	on mora	·	
В.	Planned Plant Cl	nange Evaluations			
	1. None	90		0	
C.	2002 Permanent	ECCS Model Assessments			
	1. RFPAC Re	fill and Reflood Calculation Co	de Errors	16	
		ump Junction Area Used in RE		0	
	3. Error in TC	ODEE2 Clad Thermal Expans	ion	-1	
		or Line Loss Error		-1	
_					
D.	Temporary ECCS	6 Model Issues		^	
	1. None			0	

## -U.S. Nuclear Regulatory Commission B18866/Attachment 1/Page 3

# 10 CFR 50.46 Margin Utilization Large Break Loss of Coolant Accident (LBLOCA)

## E. Other Margin Allocations

1. None

0

## **LICENSING BASIS PCT + MARGIN ALLOCATIONS**

**PCT = 1826** 

#### Notes:

(1) New Analysis of Record with SEM/PWR-98 LOCA Evaluation Model