

April 2, 2003

LICENSEE: Entergy Nuclear Operations, Inc.  
FACILITY: Indian Point Nuclear Generating Unit No. 2  
SUBJECT: SUMMARY OF FEBRUARY 13, 2003, MEETING REGARDING BEST  
ESTIMATE SMALL-BREAK LOSS-OF-COOLANT ACCIDENT METHODOLOGY  
(TAC NO. MB7274)

On February 13, 2003, the U.S. Nuclear Regulatory Commission (NRC) staff held a meeting at its offices in Rockville, Maryland, with representatives from Entergy Nuclear Operations, Inc., the licensee for the Indian Point Nuclear Generating Unit No. 2 (IP2). The list of meeting attendees is included as Enclosure 1. Enclosures 2 is a copy of the slides presented at the meeting by the licensee. The presentations closely followed information presented in the slides.

The licensee, with support from its contractor, Westinghouse Electric Company, presented information discussing the background and overview of its proposed methodology that will be used to perform small-break loss-of-coolant accident (LOCA) analysis. This analysis is part of the evaluation of the calculated emergency core cooling systems (ECCS) performance following postulated accidents. The licensee stated that it has requested NRC review and approval of the proposed methodology in order to apply the methodology to support a possible power increase to about 3216 megawatts thermal (MWt) from the current licensed power level of 3071.4 MWt. The licensee also discussed its schedule leading to the implementation of a power uprate after refueling outage 16.

The NRC staff stated that a review of this type of methodology generally takes about 1½ to 2 years if the computer code is provided with the application. If not provided, the review time could increase to 3 years. Since the proposed methodology will use a number of the existing code models, the NRC staff will also need to verify that the models are appropriate for this application. The staff stated that it will be conducting the review without contractor support. In addition, a meeting to discuss the approach and the results of the staff's review would probably be scheduled with the Advisory Committee on Reactor Safety.

Upon completion of the overview and scheduler discussions, the meeting was closed to the public in order to discuss aspects of a proprietary nature to the proposed methodology. In this regard, Westinghouse presented information related to its analytical codes, plant-specific application to IP2, uncertainty approach, and the phenomena identification and ranking table (PIRT) process. Westinghouse stated that the PIRT was independently reviewed by an external review team.

The NRC staff noted that the licensee needs to be clear about the application of this methodology beyond IP2 and that questions may arise during the review dealing with generic applicability.

*/RA/*

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Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosures: As stated

cc w/encls: See next page

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Enclosures: As stated

cc w/encls: See next page

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Official Record Copy

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INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

FEBRUARY 13, 2003

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Other Participants:

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