

April 1, 2003

MEMORANDUM TO: File

FROM: Girija S. Shukla, Project Manager, Section 2 **/RA/**  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

SUBJECT: SUMMARY OF CONFERENCE CALL WITH PACIFIC GAS AND  
ELECTRIC COMPANY REGARDING THE 2002 STEAM GENERATOR  
INSPECTION RESULTS AT DIABLO CANYON UNIT 1  
(TAC NO. MB5152)

On May 20, 2002, the NRC staff participated in a conference call with Pacific Gas and Electric Company regarding the ongoing steam generator tube inspection activities at Diablo Canyon Unit 1. Attached is a summary of the conference call.

Docket No. 50-275

Attachment: Summary of Conference Call

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**Attachment: Accession No. ML030550720**

**Package: Accession No. ML030910575**

**Memo: Accession No. ML030910542**

**NRR-106**

OFFICE	PDIV-2/PM	PDIV-2/LA	PDIV-2/SC
NAME	GShukla	EPeyton	SDembek
DATE	2/26/03	2/26/03	4/01/03

DOCUMENT NAME: C:\ORPCheckout\FileNET\ML030910542.wpd

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SUMMARY OF CONFERENCE CALL  
BETWEEN THE NRC STAFF AND PACIFIC GAS AND ELECTRIC COMPANY  
REGARDING THE 2002 STEAM GENERATOR INSPECTION RESULTS

DIABLO CANYON UNIT 1

DOCKET NO. 50-275

The NRC staff participated in a conference call with Pacific Gas and Electric Company (PG&E/ the licensee) representatives on May 20, 2002, to discuss the ongoing steam generator (SG) inspection activities at Diablo Canyon Unit 1. The issues discussed included those listed in the enclosure to the letter documenting the arrangement of this conference call, which was sent to PG&E on May 17, 2002 (Accession #ML021750581). The licensee provided supporting information which is attached.

At the time of the call, data acquisition and analyses were complete. Details of the licensee's initial inspection scope and expansion criteria are described in the licensee's written materials. Several of these topics were discussed during the conference call.

- In the U-bend region, 100% of the tubes in Rows 1 and 2, plus 20% of the tubes in Row 3, were inspected with a rotating probe coil (RPC) equipped with a Plus point (+Pt) coil. The expansion of the inspection to Row 3 of the U-bends is due to a recent report of low-row U-bend indications found in other plants.
- Of the approximately 150 tubes unplugged during the outage, no tubes exhibited any swelling.
- Although minor primary-to-secondary leakage was observed during the cycle, no through-wall flaws were detected nor were any leaking plugs identified during the visual inspection.
- The scope of the plus-point coil inspections of dents at the tube support plate differ from steam generator to steam generator since the extent of denting and the cracking observed varies from steam generator to steam generator.
- The licensee indicated that all tubes which were not expanded in the tubesheet (i.e., NTEs) were plugged.
- The selection of which dings to inspect was random (i.e., the size of the ding was not used in the selection criteria).

The licensee provided a table in the attached material summarizing the inspection findings. The table is attached. Several of the items in the table were clarified during the conference call, as discussed below.

W\* ARC (Axial indications in the WEXTEx region inside tubesheet):

There was one axial primary water stress corrosion cracking (PWSCC) indication which failed to meet the W\* criteria. This was a new indication (i.e., was not detected or present in the previous inspections).

During this outage, no in-situ testing was performed to support the W\* leakage model. This is due to the fact that none of the flaws met the W\* in-situ testing selection criteria.

GL 95-05, "Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking"

One indication was detected in a tube at a tube support plate location with a tube support plate ligament indication. This indication will be addressed in the licensee's response to GL 95-05.

U-Bend Noise

The licensee inspected the U-bend region of the tubes from Row 1 to Row 3 with a mid-range plus point coil. U-bends with high noise levels were also inspected with high frequency plus point coils.

Axial PWSCC ARC

Mixed-mode indications were detected at two dented support plate intersections. One indication consisted of an inside-diameter axial flaw and an outside-diameter (OD) circumferential flaw separated by 0.65 inches. The other mixed-mode indication consisted of an OD axial flaw and an OD circumferential flaw. Both of the tubes were plugged. The indications were small. They will be further evaluated and will be discussed in the licensee's 90-day report.

In-Situ Testing

There was no in-situ testing performed during this outage. The licensee stated that none of the flaws detected during the outage met the in-situ testing criteria. The licensee further stated that, based on the evaluation, the flaws were not even close to meeting the in-situ testing criteria.

The NRC staff did not have any further questions and determined a followup call was not required.

Attachments: Material Provided by Licensee (ADAMS Accession No. ML030550720)