



Tennessee Valley Authority, Post Office Box 2000, Spring City, Tennessee 37381-2000

MAR 24 2003

10 CFR 50, App E.

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Gentlemen:

In the Matter of) Docket No. 50-390
Tennessee Valley Authority)

WATTS BAR NUCLEAR PLANT (WBN) - EMERGENCY PLAN IMPLEMENTING
PROCEDURE (EPIP) REVISION

In accordance with the requirements of 10 CFR Part 50, Appendix E,
Section V, the enclosure provides the following EPIP.

<u>EPIP</u>	<u>Rev</u>	<u>Title</u>	<u>Effective Date</u>
EPIP-1	21	Emergency Plan Classification Flowchart	3-3-2003

There are no regulatory commitments in this letter. If you should
have any questions, please contact me at (423) 365-1824.

Sincerely,

P. D. Pace
Manager, Site Licensing and Industry Affairs

Enclosure
cc: See Page 2

A045

U.S. Nuclear Regulatory Commission
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PLP:JES

Enclosure

cc (Enclosure):

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TENNESSEE VALLEY AUTHORITY

WATTS BAR NUCLEAR PLANT

EMERGENCY PLAN IMPLEMENTATING
PROCEDURES

EPIP-1

EMERGENCY PLAN CLASSIFICATION FLOWCHART

Revision 21

Unit 0

NON-QUALITY RELATED

PREPARED BY: James F. Hagy
(Type Name)

SPONSORING ORGANIZATION: Emergency Planning

APPROVED BY: Frank L. Pavlechko

EFFECTIVE DATE: 03/03/2003

LEVEL OF USE: REFERENCE

REVISION LOG

Revision Number	Implementation Date	Description of Revision
0	04/13/90	New WBN-EPIP. Supersedes IP-1.
1	02/04/91	Revised to separate RCS leak and identified S/G tube leak initiating conditions. Clarified initiating condition in fire. Updated ODS telephone numbers.
2	11/28/91	Add initiation conditions. Clarify reference to Attachment 1 Definitions. Define Protected Area, Owner Controlled Area, and Vital Areas throughout procedures. Clarify NOUE declaration for Uncontrolled Shutdown.
3	03/04/92	Change all Technical Specification references to reflect new "Merit" Tech Specs and ODCM references.
4	02/10/93	Procedure revised to reflect the new methodology for development of Emergency Action Levels per: NUMARC/NESP-007, Rev. 3, 1/92, endorsed by REG GUIDE 1.101 Emergency Planning and Preparedness For Nuclear Power Reactors Rev. 3, 8/92.
5	09/15/93	Editorial (non-intent) and formal changes. Text changes made to EALs to meet review comments identified by the NRC.
6	01/01/94	Procedure revised to reflect new 10 CFR 20 changes.
7	05/27/94	Procedure revised to reflect changes to System 90 (Radmonitoring) and establish site perimeter monitoring points.
8	01/10/95	FPBM, EAL 1.3.4, CNTMT, Bypass, Loss (1), revised to eliminate potential for misclassification. Maps revised to reference north and wind direction. Table 7-2, Alert, Radiation Levels enhanced to provide Operators additional information.
9	4/28/98	Revised Revision Log to include page numbers. References added to the document. Fission Product Barrier Matrix revised to reflect information found in the EOP Set Point Verification Document (WBN-OS64-188). Reference to AOI-27 revised to AOI-30.2. Phone numbers to the National Weather Service changed due to their reorganization. Annunciator window references for the earthquake corrected to match Main Control Room alignment. All references to RM were changed to RE to make it consistent with site description documents. Tables in section seven revised to reflect the following: System 90 changes, monitor efficiencies, default flow rates, release time durations, and annual meteorological data enhancements.

Revision Number	Implementation Date	Pages Affected	Description of Revision
CN-1	09/28/95	10, 14, 26	The following non-intent enhancements were made: (CCP) Acronym added to the Fission Product Barrier Matrix in 1.2 RCS Barrier, (2. RCS Leakage LOCA), to enhance description. New SI reference number for Reactor Coolant System Water Inventory Balance identified in event 2.5 (RCS Unidentified Leakage) and 2.6 (RCS Identified Leakage). Area code and phone number in event 5.2 (Tomado) revised to new number.
CN-2	11/10/95	3, 6, 34	The following non-intent enhancements were made: Corresponding ERFDS system identifiers were added next to the rad monitors on Table 7-1; Table 7-1 was realigned to improve its usability; an enhanced description for RE-404 was provided in Note 3 of Table 7-1; the ERFDS Operators Manual was added to the Reference section.

REVISION LOG(Continued)

Revision Number	Implementation Date	Pages Affected	Description of Revision
CN-3	05/24/96	8, 11, 16, 19, 23, 24, 26, 29, 32, 34	The following non-intent enhancements were made: Due to revisions made to AOI-27, it was added back to the EALs in event 4.5 "Control Room Evacuation"; The Assessment Method on Table 7-1 was enhanced to correspond with the note at the top of the table. In addition, the reference to TI-30 was removed since this procedure will be terminated due to the enhancements being made to EPIP-16 and ERFDS. The word Projectile was added to the index and title reference to event 5.3 "Aircraft/Projectile Crash", to make it consistent with the EALs within it's classification.
10	3/15/99	All	The following non-intent enhancement were made: Software revised to Microsoft Word which re-formatted pages along with other enhancements; minor typographical errors corrected; two references revised - one added; SOS/ASOS replaced with SM/US; index page, effluent added to gaseous; vital area definition enhanced; spent fuel pit revised to pool on Table 7-2; SP revised to EAB in Event 7.1; TVA Load Dispatcher/Water Resources revised to River Systems Operations and revised ERFDS/P-2500 to ICS.
11	4/15/99	2, 34	Non intent change. Typo corrected. Changed >1.0 to >0.1.
11A	7/1/99	3,26	Corrected typo on phone number The remaining pages of this procedure are Rev 11 only page 3, and the fold out page for 26 have been changed.
12	9/30/99	All	Non intent change. Minor editorial/format changes made. Typographical errors corrected. Seismic windows revised to reflect DCN-50007 per ERPI Report 6695. (LTL) Lower toxicity limit replaced with (PEL) Permissible Exposure Limit. This revision is also part of the resolution to PER 99-009326-000.
13	12/08/99	All	Non-intent change. Revised page 33 for resolution of PER 99-015478-000. Minor editorial change to Event 5.1 step 1 of the Alert classification.
14	04/10/00	All (Pg.4 & 45)	Non-intend change. Revised page 45 for DCN 50484, stage 1 which moved 0-RE-90-101B, & -132B from ICS Screen 4RM2 to 4RM1. DCN also moved 1-RE-90-421B thru -424B and 0-RE-90-120 & -121 from ICS Screen 4RM1 to 4RM2. This revision allows all liquid radiation monitors to be observable on one ICS screen and all gaseous radiation monitors to be observed on a separate ICS screen.
15	08/17/00	All (Pg. 4, 11A & B)	Intent change. Revised CNTMT Rad Monitors (1-RE-90-271, 272, 273, & 274) readings to correspond with the new TI-RPS-162, "Response of the Primary Containment High Range Monitors" readings (Reference EDC-50600). This analysis resulted in a revision to the EALs 1.1.5 on the Barrier matrix page, 11b. This revision resolves action items from CORP PER 99-000038-000. This revision was also determined not to reduce the level of effectiveness of the procedure or REP.

REVISION LOG (Continued)

Revision Number	Implementation Date	Pages Affected	Description of Revision
16	3/30/01	All (Pg. 11 & 14)	Plan effectiveness determinations reviews indicate the following revisions do not reduce the level of effectiveness of the procedure or REP: Intent change. Revised CNTMT Rad Monitors readings in the Barrier Matrix (1.3) to support new dose assessment methodology. Non intent change. Revised reference from annunciator alarm printer to annunciator monitor per DCN D-50301.
17	09/25/01	All Page 6, 11B	Plan effectiveness determinations reviews indicate the following revisions do not reduce the level of effectiveness of the procedure or REP: Intent change. Procedure revised to Non-Quality related per requirements of NQAP & pending revision to SPP-2.2. The coversheet and records section of the procedure was revised to reflect this change. Non-Intent change. Corrected typo on Barrier Matrix.
18	02/15/02	All 2, 11B, 44	Plan effectiveness determinations reviews indicate the following revisions do not reduce the level of effectiveness of the procedure or REP: Non-Intent change. Changes to the EALs in this revision consist of changing β - γ to gamma in Section 7.0 to ensure consistency with NUMARC/NESP-007, Reg Guide 1.101, and NEI 99-01 rev 4. Clarification to EAL 1.3.3 (containment isolation status also made per this reference.) This standardizes these issues with the other TVAN sites. These changes were approved by the State of Tennessee.
19	06/05/02	All 4, 7 & 30	Plan effectiveness determinations on these change(s) indicate the following revisions do not reduce the level of effectiveness of the procedure or REP. Intent change(s): A revision to the Security Event (4.6) was made to incorporate change(s) resulting from the NEI to NRC (Mr. Bruce Boger) letter dated 12/18/01 requesting conformation for an EAL basis change to include response to a Credible Site Specific Threat. Table 4-3 was revised to incorporate this additional EAL. This meets the compliance of the NRC's 10/6/01 Safeguards Advisory on this matter. This represents an additional EAL and does not change existing criteria in the Security Event Basis. Revised 5.1 Interfacing documents by noting the termination of EPIP 9 with reference to EPIP 16.
20	07/09/02	ALL, pg. 2, 10, 13, 15, 20, 24, 30, 32, 39, 43	Plan effectiveness determinations on these change(s) indicate the following revisions do not reduce the level of effectiveness of the procedure or REP. Intent change(s): Reference to T/S 3.4.16 in Event 2.4 EAL 1(a) revised to correspond to levels in AOI-28. Credible Site-Specific was added to the definition pages. Removed reference to the definition in Table 4-3 SECURITY EVENTS to standardize with other TVAN sites.
21	03/03/2003	2, 15	Plan effectiveness determinations on these change(s) indicate the following revisions do not reduce the level of effectiveness of the procedure or REP. Non-intent change: Deleted reference to table which was deleted from AOI-28, Ref. WBPER 03-004004-000.

WBN	EMERGENCY PLAN CLASSIFICATION FLOWCHART	EP-1 Revision 21 Page 5 of 49
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1.0 PURPOSE⁴

This Procedure provides guidance in determining the classification and declaration of an emergency based on plant conditions.

2.0 RESPONSIBILITY^{2,4}

The responsibility of declaring an Emergency based on the guidance within this procedure belongs to the Shift Manager/Site Emergency Director (SM/SED) or designated Unit Supervisor (US) when acting as the SM or the TSC Site Emergency Director (SED). These duties CAN NOT be delegated.

3.0 INSTRUCTIONS⁴

3.1 The criteria in WBN EP-1 are given for GUIDANCE ONLY: knowledge of actual plant conditions or the extent of the emergency may require that additional steps be taken. In all cases, this logic procedure should be combined with the sound judgment of the SM/SED and/or the TSC SED to arrive at a classification for a particular set of circumstances.

3.2 The Nuclear Power (NP) Radiological Emergency Plan (REP) will be activated when any one of the conditions listed in this logic is detected.

3.3 Classification Determination

3.3.1 To determine the classification of the emergency, review the Initiating Conditions of the Events described in this procedure with the known or suspected conditions and CARRY OUT the notifications and actions referenced.

3.3.2 If a Critical Safety Function (CSF) is listed as an Initiating Condition: the respective status tree criteria will be monitored and used to determine the Event classification for the modes listed on the classification flowchart.

3.3.3 The highest classification for which an Emergency Action level (EAL) currently exists shall be declared.

WBN	EMERGENCY PLAN CLASSIFICATION FLOWCHART	EPIP-1 Revision 21 Page 6 of 49
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3.0 INSTRUCTIONS (continued)

- 3.3.4 After an Event classification, if the following investigation shows that Initiating Conditions were met that dictate a higher Event classification, the new event classification shall be declared at the clock time of the determination.
- 3.3.5 IF an EAL for a higher classification was exceeded but the present situation indicates a lower classification, the fact that the higher classification occurred **SHALL** be reported to the NRC and Central Emergency Control Center (CECC), but should not be declared.
- 3.3.6 IF the Parameter is indeterminate due to instrument malfunction and the existence of the condition **CAN NOT** be reasonably discounted (i.e., spurious or false alarm that can be substantiated within 15 minutes) the condition is considered **MET** and the SM/SED **SHALL** follow the indications provided until such time as the alarm is verified to be false.
- 3.3.7 IF an EAL was exceeded, but the emergency has been totally resolved (prior to declaration), the emergency condition that was appropriate shall not be declared but reported to the NRC and Operations Duty Specialist (ODS) at the same clock time.
- 3.3.8 The **ACCEPTABLE** time frame for notification to the Operation Duty Specialist (ODS) is considered to be five (5) minutes. This is the time period between declaration of the emergency and notifying the ODS.

4.0 RECORDS

4.1 Non-QA Records

None

WBN	EMERGENCY PLAN CLASSIFICATION FLOWCHART	EPIP-1 Revision 21 Page 7 of 49
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5.0 REFERENCES

5.1 Interfacing References

BP-236, *Event Critique and Root Cause Analysis*

SPP 3.5, *Regulatory Reporting Requirements*

WBN-EPIP-2, *Unusual Event*

WBN-EPIP-3, *Alert*

WBN-EPIP-4, *Site Area Emergency*

WBN-EPIP-5, *General Emergency*

WBN-EPIP-9, *Loss of Meteorological Data (Canceled see EPIP-16)*

WBN-EPIP-13, *Termination of the Emergency and Recovery*

WBN-EPIP-14, *Radiological Control Response*

WBN-EPIP-16, *Initial Dose Assessment For Radiological Emergencies*

CECC-EPIP-9, *Emergency Environmental Radiological Monitoring Procedures*

SI-4.04, *Measurement of Identified and Unidentified Leakage of the Reactor Coolant System*

5.2 Other Documents

10 CFR 50, *Domestic Licensing of Production and Utilization Facilities*

10 CFR 20, *Standards for Protection From Radiation*

REG GUIDE-1.101, *Emergency Planning and Preparedness For Nuclear Power Reactors endorsing NUMARC NESP-007 Methodology for Development of Emergency Action Levels.*

Site Technical Specifications (Tech Specs), Abnormal Operating Instructions (AOIs), Emergency Operating Procedures (EOPs), Set Point Verification documents, Chemistry Technical documents (CTDs), and the Final Safety Analysis Report (FSAR) are also referenced in Appendix C of the Radiological Emergency Plan.

ICS Operator's Manual

EPPOS #2, "NRC EP Position on Timeliness of Classification of Emergency Conditions

EPRI Report 6695 Guidelines for Nuclear Power Plant Response to Earthquakes.

EMERGENCY

PLAN

CLASSIFICATION

FLOWCHART ^{1,3,4,5}

FISSION PRODUCT BARRIER MATRIX (Modes 1-4) 1

- 1.1 Fuel Clad
- 1.2 RCS
- 1.3 Containment

SYSTEM DEGRADATION 2

- 2.1 Loss of Instrumentation
- 2.2 Loss of Function/Communication
- 2.3 Failure of Reactor Protection
- 2.4 Fuel Clad Degradation
- 2.5 RCS Unidentified Leakage
- 2.6 RCS Identified Leakage
- 2.7 Uncontrolled Cool Down
- 2.8 Turbine Failure
- 2.9 Technical Specification
- 2.10 Safety Limit

LOSS OF POWER 3

- 3.1 Loss of AC (Power Ops)
- 3.2 Loss of AC (Shutdown)
- 3.3 Loss of DC

HAZARDS and SED JUDGMENT 4

- 4.1 Fire
 - 4.2 Explosion
 - 4.3 Flammable Gas
 - 4.4 Toxic Gas
 - 4.5 Control Room Evacuation
 - 4.6 Security
 - 4.7 SED Judgment
- Table 4-1 Table 4-2 Table 4-3
Figure 4-A Figure 4-B

DESTRUCTIVE PHENOMENON 5

- 5.1 Earthquake
 - 5.2 Tornado
 - 5.3 Aircraft/Projectile Crash
 - 5.4 River Level High
 - 5.5 River Level Low
 - 5.6 Watercraft Crash
- Table 5-1 Figure 5-A

SHUTDOWN SYSTEM DEGRADATION 6

- 6.1 Loss of Shutdown Systems
- 6.2 Loss of AC (Shutdown)
- 6.3 Loss of DC (Shutdown)
- 6.4 Fuel Handling

RADIOLOGICAL 7

- 7.1 Gaseous Effluent
 - 7.2 Liquid Effluent
 - 7.3 Radiation Levels
 - 7.4 Fuel Handling
- Table 7-1 Table 7-2
Figure 7-A
-

DEFINITIONS/ACRONYMS

UNUSUAL EVENT, ALERT, SITE AREA EMERGENCY and GENERAL EMERGENCY: (see SED Judgment 4.7)

BOMB: An explosive device (See EXPLOSION)

CIVIL DISTURBANCE: A group of twenty (20) or more persons violently protesting station operations or activities at the site

CREDIBLE SITE-SPECIFIC -The determination is made by WBN senior plant management through use of information found in the Safeguards Contingency Plan

CRITICAL-SAFETY FUNCTION (CSFs): A plant safety function required to prevent significant release of core radioactivity to the environment. There are six CSFs Sub-criticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS)

EVENT: Assessment of an EVENT commences when recognition is made that one or more of the conditions associated with the event exist. Implicit in this definition is the need for timely assessment, i.e. within 15 minutes

EXCLUSION AREA BOUNDARY (EAB): The demarcation of the area surrounding the WBN units in which postulated FSAR accidents will not result in population doses exceeding the criteria of 10 CFR Part 100 Refer to Figure 7-A.

EXPLOSION: A rapid, violent, unconfined combustion, or a catastrophic failure of pressurized equipment that imparts energy of sufficient force to potentially damage permanent structures required for safe operation

EXTORTION: An attempt to cause an action at the station by threat of force

FAULTED: (Steam Generator) Existence of secondary side leakage (i.e., steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

FIRE: Combustion characterized by heat and light. Source of smoke such as slipping drive belts or overheated electrical components do not constitute fires. Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed.

FLAMMABLE GAS: Combustible gases maintained at concentrations less than the LOWER EXPLOSIVE LIMIT (LEL) will not explode due to ignition.

HOSTAGE: A person(s) held as leverage against the station to ensure that demands will be met by the station

INEFFECTIVE: The specified restoration action(s) does not result in a reduction in the level of severity of the RED PATH condition within 15 minutes from identification of the Core Cooling CSF Status Tree RED PATH. A reduction in the level of severity is an improvement in the applicable parameters, e.g., Increasing Trend in Reactor Vessel Water Level (Full RVLIS) and/or Decreasing Trend on Core Thermocouple Temperatures.

INITIATING CONDITIONS: Plant Parameters, radiation monitor readings or personnel observations that identify an Event for purposes of Emergency Plan Classification.

INTRUSION/INTRUDER: Suspected hostile individual present in a protected area without authorization

ODCM: Offsite Dose Calculation Manual

ORANGE PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge

PROJECTILE: An object ejected, thrown, or launched towards a plant structure. The source of the projectile may be onsite or offsite. Damage is sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein.

PROTECTED AREA: Encompasses all owner controlled areas within the security protected area fence as shown on Figure 4-A.

RED PATH: Monitoring of one or more CSFs by the FR-0 which indicates that the CSF(s) is under extreme challenge, prompt operator action is required.

RUPTURED: (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than charging pump capacity.

SABOTAGE: Deliberate damage, misalignment, or mis-operation of plant equipment with the intent to render the equipment inoperable

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following (1) An automatic turbine runback > 15% thermal reactor power; (2) Electrical load rejection > 25% full electrical load, (3) Reactor Trip or (4) Safety Injection System Activation

SITE PERIMETER (SP): Encompasses all owner controlled areas in the immediate site environs as shown on Figures 4-A and 7-A.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine)

UNPLANNED: An event or action that is not the expected result of normal operations, testing, or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED

UNPLANNED: (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

VALID: An indication or report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment, i.e., within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analyses. Damage is sufficient enough to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes deformation due to heat or impact, denting, penetration, rupture, cracking, and/or paint blistering. Surface blemishes (e.g., paint chipping, scratches) should NOT be included

VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation

1.1 Fuel Clad Barrier	
1. Critical Safety Function Status	
LOSS	Potential LOSS
Core Cooling Red	Core Cooling Orange OR Heat Sink Red (RHR Not in Service)
-OR-	
2. Primary Coolant Activity Level	
LOSS	Potential LOSS
RCS sample activity is Greater Than 300 μ Ci/gm dose equivalent iodine-131	Not applicable
-OR-	
3. Incore TCs Hi Quad Average	
LOSS	Potential LOSS
Greater Than 1200°F	Greater Than 727°F
-OR-	
4. Reactor Vessel Water Level	
LOSS	Potential LOSS
Not Applicable	VALID RVLIS level <33% (No RCP running)
-OR-	
5. Containment Radiation Monitors	
LOSS	Potential LOSS
VALID reading increase of Greater Than: 74 R/hr On 1-RE-90-271 and 272 OR 59 R/hr On 1-RE-90-273 and 274	Not Applicable
-OR-	
6. Site Emergency Director Judgment	
Any condition that, in the Judgment of the SM/SED, Indicates Loss or Potential Loss of the Fuel Clad Barrier Comparable to the Conditions Listed Above.	

1.2 RCS Barrier	
1. Critical Safety Function Status	
LOSS	Potential LOSS
Not Applicable	Pressurized Thermal Shock Red OR Heat Sink Red (RHR Not in Service)
-OR-	
2. RCS Leakage/LOCA	
LOSS	Potential LOSS
RCS Leak results in Loss of subcooling (<65°F Indicated), [85°F ADV]	Non Isolatable RCS Leak Exceeding The Capacity of <u>One</u> Charging Pump (CCP) In the Normal Charging Alignment. OR RCS Leakage Results In Entry Into E-1
-OR-	
3. Steam Generator Tube Rupture	
LOSS	Potential LOSS
SGTR that results in a safety injection actuation OR Entry into E-3	Not Applicable
-OR-	
4. Reactor Vessel Water Level	
LOSS	Potential LOSS
VALID RVLIS level <33% (No RCP Running)	Not Applicable
-OR-	
5. Site Emergency Director Judgment	
Any condition that, in the Judgment of the SM/SED, Indicates Loss or Potential Loss of the RCS Barrier Comparable to the Conditions Listed Above.	

Modes: 1, 2, 3, 4

INSTRUCTIONS

NOTE. A condition is considered to be MET if, in the judgment of the Site Emergency Director, the condition will be MET imminently (i.e., within 1 to 2 hours, in the absence of a viable success path) The classification shall be made as soon as this determination is made

1. In the matrix to the left, review the **INITIATING CONDITIONS** in all columns and identify which, if any, **INITIATING CONDITIONS** are MET. Circle these **CONDITIONS**.
2. For each of the three barriers, identify if any **LOSS** or **Potential LOSS INITIATING CONDITIONS** have been MET.
3. If a CSF is listed as an **INITIATING CONDITION**; the respective status tree criteria will be monitored and used to determine the **EVENT** classification for the Modes listed on the classification flowchart.
4. Compare the barrier losses and potential losses to the **EVENTS** below and make the appropriate declaration.

F I S S I O N P R O D U C T B A R R I E R M A T R I X U 1

1.3 CNTMT Barrier	
1. Critical Safety Function Status	
LOSS	Potential LOSS
Not Applicable	Containment (FR-Z.1) Red OR Actions of FR-C.1 (Red Path) are INEFFECTIVE
-OR-	
2. Containment Pressure/Hydrogen	
LOSS	Potential LOSS
Rapid unexplained decrease following initial increase OR Containment pressure or Sump level <u>Not</u> increasing (with LOCA in progress)	Containment Hydrogen Increases to >4% by volume OR Pressure >2.8 PSIG (Phase B) with < One full train of Containment spray
-OR-	
3. Containment Isolation Status	
LOSS	Potential LOSS
Containment Isolation is Incomplete (when required) AND a Release Path to the Environment Exists	Not Applicable
-OR-	
4. Containment Bypass	
LOSS	Potential LOSS
RUPTURED S/G is also FAULTED outside CNTMT OR Prolonged (>4 Hours) Secondary Side release outside CNTMT from a S/G with a SGTL > T/S Limits	Unexplained VALID increase in area or ventilation RAD monitors in areas adjacent to CNTMT (with LOCA in progress)
-OR-	
5. Significant Radioactivity in Containment	
LOSS	Potential LOSS
Not Applicable	VALID Reading increase of Greater Than: 108 R/hr on 1-RE-90-271 and 1-RE-90-272 OR 86 R/hr on 1-RE-90-273 and 1-RE-90-274
-OR-	
6. Site Emergency Director Judgment	
condition that, in the Judgment of the SM/SED, Indicates or Potential Loss of the CNTMT Barrier Comparable to the Conditions Listed Above	

EVENTS

<u>UNUSUAL EVENT</u>	<u>ALERT</u>
Loss <u>or</u> Potential LOSS of Containment Barrier	Any LOSS <u>or</u> Potential LOSS of Fuel Clad barrier OR Any LOSS <u>or</u> Potential LOSS of RCS barrier
<u>SITE AREA EMERGENCY</u>	<u>GENERAL EMERGENCY</u>
LOSS <u>or</u> Potential LOSS of any two barriers	LOSS of any two barriers and Potential LOSS of third barrier

FISSION PRODUCT BARRIER MATRIX (Modes 1-4)

- 1.1 Fuel Clad
- 1.2 RCS
- 1.3 Containment

1

SYSTEM DEGRADATION

- | | |
|------------------------------------|-----------------------------|
| 2.1 Loss of Instrumentation | 2.6 RCS Identified Leakage |
| 2.2 Loss of Function/Communication | 2.7 Uncontrolled Cool Down |
| 2.3 Failure of Reactor Protection | 2.8 Turbine Failure |
| 2.4 Fuel Clad Degradation | 2.9 Technical Specification |
| 2.5 RCS Unidentified Leakage | 2.10 Safety Limit |

2

LOSS OF POWER

- 3.1 Loss of AC (Power Ops)
- 3.2 Loss of AC (Shutdown)
- 3.3 Loss of DC

3

HAZARDS and SED JUDGMENT

- | | | |
|---------------|-------------------|-----------------------------|
| 4.1 Fire | 4.3 Flammable Gas | 4.5 Control Room Evacuation |
| 4.2 Explosion | 4.4 Toxic Gas | 4.6 Security |
| Table 4-1 | Table 4-2 | 4.7 SED Judgment |
| Figure 4-A | Figure 4-B | Table 4-3 |

4

DESTRUCTIVE PHENOMENON

- | | |
|----------------------------------|----------------------|
| 5.1 Earthquake | 5.4 River Level High |
| 5.2 Tornado | 5.5 River Level Low |
| 5.3 Aircraft/Projectile
Crash | 5.6 Watercraft Crash |
| Table 5-1 | Figure 5-A |

5

SHUTDOWN SYSTEM DEGRADATION

- 6.1 Loss of Shutdown Systems
- 6.2 Loss of AC (Shutdown)
- 6.3 Loss of DC (Shutdown)
- 6.4 Fuel Handling

6

RADIOLOGICAL

- | | |
|----------------------|----------------------|
| 7.1 Gaseous Effluent | 7.3 Radiation Levels |
| 7.2 Liquid Effluent | 7.4 Fuel Handling |
| Table 7-1 | Table 7-2 |
| Figure 7-A | |

7

DEFINITIONS/ACRONYMS

UNUSUAL EVENT, ALERT, SITE AREA EMERGENCY and GENERAL EMERGENCY: (see SED Judgment 4 7)

BOMB: An explosive device (See EXPLOSION)

CIVIL DISTURBANCE: A group of twenty (20) or more persons violently protesting station operations or activities at the site

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EXPLOSION: A rapid, violent, unconfined combustion, or a catastrophic failure of pressurized equipment that imparts energy of sufficient force to potentially damage permanent structures required for safe operation.

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FAULTED: (Steam Generator) Existence of secondary side leakage (i e , steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

FIRE: Combustion characterized by heat and light. Source of smoke such as slipping drive belts or overheated electrical components do not constitute fires Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed

FLAMMABLE GAS: Combustible gases maintained at concentrations less than the LOWER EXPLOSIVE LIMIT (LEL) will not explode due to ignition

HOSTAGE: A person(s) held as leverage against the station to ensure that demands will be met by the station

INEFFECTIVE: The specified restoration action(s) does not result in a reduction in the level of severity of the RED PATH condition within 15 minutes from identification of the Core Cooling CSF Status Tree RED PATH A reduction in the level of severity is an improvement in the applicable parameters, e g , Increasing Trend in Reactor Vessel Water Level (Full RVLIS) and/or Decreasing Trend on Core Thermocouple Temperatures.

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RED PATH: Monitoring of one or more CSFs by the FR-0 which indicates that the CSF(s) is under extreme challenge; prompt operator action is required.

RUPTURED: (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than charging pump capacity

SABOTAGE: Deliberate damage, misalignment, or mis-operation of plant equipment with the intent to render the equipment inoperable.

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following: (1) An automatic turbine runback > 15% thermal reactor power; (2) Electrical load rejection > 25% full electrical load, (3) Reactor Trip or (4) Safety Injection System Activation

SITE PERIMETER (SP): Encompasses all owner controlled areas in the immediate site environs as shown on Figures 4-A and 7-A.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e g., chlorine)

UNPLANNED: An event or action that is not the expected result of normal operations, testing, or maintenance Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED.

UNPLANNED: (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP) Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, e g , alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

VALID: An indication or report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel Implicit in this definition is the need for timely assessment, i e , within 15 minutes.

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analyses Damage is sufficient enough to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes: deformation due to heat or impact, denting, penetration, rupture, cracking, and/or paint blistering Surface blemishes (e g , paint chipping, scratches) should NOT be included

VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation.

		2.1 Loss of Instrumentation		2.2 Loss of Function	
		Mode	Initiating/Condition	Mode	Initiating/Condition
GENERAL SITE ALERT UNUSUAL EVENT			Refer to "Fission Product Barner Matrix" and "Radiological Effluents" (Section 7)		Refer to "Fission Product Barner Matrix"
	1,2 3,4		<p>Inability to monitor a SIGNIFICANT TRANSIENT in progress (1 and 2 and 3 and 4)</p> <ol style="list-style-type: none"> 1. Loss of most (>75%) of MCR annunciators (and Annunciator Monitor) or indications 2. SIGNIFICANT TRANSIENT in progress 3. Loss of ICS Computer and SPDS 4. Inability to directly monitor any of the following CSFs: <ul style="list-style-type: none"> Sub-criticality PTS Core Cooling Containment Heat Sink Inventory 	1,2 3,4	<p>Complete loss of function needed to achieve or maintain Hot Shutdown (1 or 2)</p> <ol style="list-style-type: none"> 1. CSF status tree indicates Core Cooling Red 2 CSF status tree indicates Heat Sink Red (RHR not in service) <p>Note: Also Refer to "Failure of Rx Protection" (2.3) and "Fission Product Barner Matrix"</p>
	1,2 3,4		<p>UNPLANNED loss of most (>75%) MCR annunciators (and Annunciator Printer) or indications for >15 minutes with either a SIGNIFICANT TRANSIENT in progress or ICS computer and SPDS Unavailable (1 and 2 and 3)</p> <ol style="list-style-type: none"> 1. UNPLANNED loss of most (>75%) MCR annunciators (and Annunciator Monitor) or indications for >15 minutes. 2. SM/SED Judgment that increased surveillance is required to Safely operate the unit (beyond Shift compliment) 3. (a or b) <ul style="list-style-type: none"> a SIGNIFICANT TRANSIENT in Progress b. Loss of ICS Computer and SPDS 	4	<p>Complete loss of function needed to achieve Cold Shutdown when Shutdown required by Tech Specs (1 and 2 and 3)</p> <ol style="list-style-type: none"> 1. Shutdown is required 2. Loss of RHR capability 3 Loss of secondary heat sink and condenser
	1,2 3,4		<p>UNPLANNED loss of most or All Safety System annunciators or indications in the Control Room for >15 Minutes (1 and 2 and 3)</p> <ol style="list-style-type: none"> 1. UNPLANNED loss of most (>75%) MCR annunciators (and Annunciator Monitor) or indications for >15 minutes 2 SM/SED Judgment that increased surveillance is required to Safely operate the unit (beyond Shift compliment) 3 ICS Computer or SPDS is in service and capable of displaying data requested 	ALL	<p>A Unplanned loss of all In-Plant Communication capability (1 and 2 and 3)</p> <ol style="list-style-type: none"> 1 UNPLANNED loss of EPABX (PAX) phones UNPLANNED loss of all sound powered phones UNPLANNED loss of all radios <p style="text-align: center;">or</p> <p>B UNPLANNED loss of all Offsite Communication capability (1 and 2 and 3 and 4 and 5)</p> <ol style="list-style-type: none"> 1. UNPLANNED loss of all EPABX (PAX) phones UNPLANNED loss of all Radio frequencies UNPLANNED loss of all OPX (Microwave) system UNPLANNED loss of all 1 FB-Bell lines UNPLANNED loss of all FTS 2000 (NRC) system

2.3 Failure of Rx Protection	
Mode	Initiating/Condition
1,2	<p>Loss of Core cooling capability and VALID Trip Signals did <u>not</u> result in a reduction of Rx power to <5% and decreasing (1 and 2)</p> <ol style="list-style-type: none"> (a or b) <ol style="list-style-type: none"> CSF status tree indicates Core Cooling Red CSF status tree indicates Heat Sink Red FR-S.1 entered and subsequent actions <u>Did Not</u> result in a Rx Power of <5% and decreasing
1,2	<p>Rx power <u>Not</u> <5% and decreasing after VALID Auto and Manual trip signals (1 and 2 and 3)</p> <ol style="list-style-type: none"> VALID Rx Auto Trip signal received or required Manual Rx Trip from the MCR was <u>Not</u> successful FR-S 1 has been entered
1,2	<p>Automatic Rx trip did not occur after VALID Trip signal and manual trip from MCR was successful (1 and 2)</p> <ol style="list-style-type: none"> VALID Rx Auto Trip signal received or required Manual Rx Trip from the MCR <u>was</u> successful and power is <5% and decreasing
	Not Applicable

GENERAL SITE ALERT UNUSUAL EVENT

2.4 Fuel Clad Degradation	
Mode	Initiating/Condition
	Refer to "Fission Product Barner Matrix"
	Refer to "Fission Product Barner Matrix"
	Refer to "Fission Product Barner Matrix"
1,2, 3,4, 5	<p>Reactor Coolant System specific activity exceeds LCO (Refer to WBN Tech. Spec. 3.4.16)</p> <ol style="list-style-type: none"> Radiochemistry analysis indicates (a or b) <ol style="list-style-type: none"> Dose equivalent Iodine (I-131) >0.265 μCi/gm for >48 Hours or >21 μCi/gm Specific activity >100/E μCi/gm

SYSTEM DEGRADATION U1

		2.5 RCS Unidentified Leakage		2.6 RCS Identified Leakage	
		Mode	Initiating/Condition	Mode	Initiating/Condition
GENERAL			Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"
	SITE		Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"
			Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"
ALERT			Refer to "Fission Product Barrier Matrix"		Refer to "Fission Product Barrier Matrix"
	UNUSUAL	1,2 3,4, *5	<p>Unidentified or pressure boundary RCS leakage >10 GPM</p> <p>1. Unidentified or pressure boundary leakage (as defined by Tech. Spec.) >10 GPM as indicated below (a or b)</p> <p>a 1-SI-68-32 results</p> <p>b With RCS Temperature and PZR Level Stable, VCT Level Dropping at a Rate >10 GPM</p> <p><i>*Note: Applies to Mode 5 if RCS Pressurized</i></p>	1,2, 3,4, *5	<p>Identified RCS leakage >25 GPM</p> <p>1. Identified RCS leakage (as defined by Tech. Spec) >25 GPM (a or b)</p> <p>a. 1-SI-68-32 results</p> <p>b Level rise in excess of 25 GPM total into PRT, RCDT or CVCS Holdup Tank</p> <p><i>*Note Applies to Mode 5 if RCS Pressurized</i></p>

2.7 Uncontrolled Cooldown	
Mode	Initiating/Condition
GENERAL SITE ALERT	Refer to "Fission Product Barrier Matrix"
	Refer to "Fission Product Barrier Matrix"
	Refer to "Fission Product Barrier Matrix"
1,2,3	<p>UNPLANNED rapid depressurization of the Main Steam System resulting in a rapid RCS cooldown and Safety Injection Initiation (1 and 2)</p> <ol style="list-style-type: none"> Rapid depressurization of Main Steam System (<675 psig) Safety Injection has initiated or is required

2.8 Turbine Failure								
Mode	Initiating/Condition							
SYSTEM DEGRADATION U1	Refer to "Fission Product Barrier Matrix"							
	Refer to "Fission Product Barrier Matrix"							
	<p>Turbine Failure has generated PROJECTILES that cause VISIBLE DAMAGE to any area containing Safety Related equipment</p> <p>1. Turbine PROJECTILES has resulted in VISIBLE DAMAGE in any of the following areas</p> <table border="0"> <tr> <td>Control Building</td> <td>Diesel Generator Bldg</td> </tr> <tr> <td>Auxiliary Building</td> <td>RWST</td> </tr> <tr> <td>Unit #1 Containment</td> <td>Intake Pumping Station</td> </tr> <tr> <td></td> <td>CST</td> </tr> </table>	Control Building	Diesel Generator Bldg	Auxiliary Building	RWST	Unit #1 Containment	Intake Pumping Station	
Control Building	Diesel Generator Bldg							
Auxiliary Building	RWST							
Unit #1 Containment	Intake Pumping Station							
	CST							
1,2,3	<p>Turbine Failure results in Casing penetration</p> <p>1. Turbine Failure which results in penetration of the Turbine Casing or Damage to Main Generator Seals</p>							

2.9 Technical Specification	
Mode	Initiating/Condition
GENERAL SITE ALERT UNUSUAL EVENT	<i>Not Applicable</i>
	<i>Not Applicable</i>
	<i>Not Applicable</i>
	Inability to reach required Shutdown within Tech. Spec. limits (1 and 2) 1,2 3,4 1. Any Tech Spec. LCO Statement, requiring a Mode reduction, has been entered 2. The Unit has not been placed in the required Mode within the time prescribed by the LCO Action Statement

2.10 Safety Limit	
Mode	Initiating/Condition
	<i>Not Applicable</i>
	<i>Not Applicable</i>
	<i>Not Applicable</i>
1,2, 3,4, 5	Safety Limits have been Exceeded (1 or 2) 1. The combination of thermal power, RCS temperature, and RCS pressure > safety limits as indicated by WBN Tech. Spec Figure 2 1 1-1 "Reactor Core Safety Limits" 2. RCS/Pressurizer pressure exceeds safety limit (>2735 psig)

FISSION PRODUCT BARRIER MATRIX (Modes 1-4)

- 1.1 Fuel Clad
- 1.2 RCS
- 1.3 Containment

1

SYSTEM DEGRADATION

- 2.1 Loss of Instrumentation
- 2.2 Loss of Function/Communication
- 2.3 Failure of Reactor Protection
- 2.4 Fuel Clad Degradation
- 2.5 RCS Unidentified Leakage
- 2.6 RCS Identified Leakage
- 2.7 Uncontrolled Cool Down
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- 2.9 Technical Specification
- 2.10 Safety Limit

2

LOSS OF POWER

- 3.1 Loss of AC (Power Ops)
- 3.2 Loss of AC (Shutdown)
- 3.3 Loss of DC

3

HAZARDS and SED JUDGMENT

- 4.1 Fire
 - 4.2 Explosion
 - 4.3 Flammable Gas
 - 4.4 Toxic Gas
 - 4.5 Control Room Evacuation
 - 4.6 Security
 - 4.7 SED Judgment
- Table 4-1
Figure 4-A
- Table 4-2
Figure 4-B
- Table 4-3

4

DESTRUCTIVE PHENOMENON

- 5.1 Earthquake
 - 5.2 Tornado
 - 5.3 Aircraft/Projectile Crash
 - 5.4 River Level High
 - 5.5 River Level Low
 - 5.6 Watercraft Crash
- Table 5-1
- Figure 5-A

5

SHUTDOWN SYSTEM DEGRADATION

- 6.1 Loss of Shutdown Systems
- 6.2 Loss of AC (Shutdown)
- 6.3 Loss of DC (Shutdown)
- 6.4 Fuel Handling

6

RADIOLOGICAL

- 7.1 Gaseous Effluent
 - 7.2 Liquid Effluent
 - 7.3 Radiation Levels
 - 7.4 Fuel Handling
- Table 7-1
Figure 7-A
- Table 7-2

7

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VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation

3.1 Loss of AC (Power Ops)		
	Mode	Initiating/Condition
GENERAL	1,2,3,4	Prolonged loss of Offsite and Onsite AC power (1 and 2) 1. 1A <u>and</u> 1B 6 9KV Shutdown Bds de-energized for >15 minutes 2. (a or b) a Core Cooling Red <u>or</u> Orange b Restoration of Either 1A <u>or</u> 1B 6 9KV Shutdown Bds is not likely within 4 hours of loss.
	1,2,3,4	Loss of Offsite <u>and</u> Onsite AC Power > 15 minutes 1. 1A and 1B 6 9KV Shutdown Bds de-energized for >15 minutes
SITE	1,2,3,4	Loss of Offsite Power for >15 minutes (1 and 2) 1. C <u>and</u> D CSSTs not available for >15 minutes 2. 1A <u>or</u> 1B Diesel Generator not available
ALERT	1,2,3,4	Loss of Offsite Power for >15 minutes (1 and 2) 1. C <u>and</u> D CSSTs not available for >15 minutes 2. Each Diesel Generator is supplying power to its respective Shutdown Board
UNUSUAL	1,2,3,4	Loss of Offsite Power for >15 minutes (1 and 2) 1. C <u>and</u> D CSSTs not available for >15 minutes 2. Each Diesel Generator is supplying power to its respective Shutdown Board
EVENT	1,2,3,4	Loss of Offsite Power for >15 minutes (1 and 2) 1. C <u>and</u> D CSSTs not available for >15 minutes 2. Each Diesel Generator is supplying power to its respective Shutdown Board

3.2 Loss of AC (Shutdown)	
Mode	Initiating/Condition
	<i>Not Applicable</i>
	<i>Not Applicable</i>
5,6, or De-fuel	UNPLANNED loss of Offsite <u>and</u> Onsite AC power for >15 minutes 1. 1A and 1B 6 9KV Shutdown Bds de-energized for >15 minutes <i>Also Refer to "Loss of Shutdown Systems" (6.1)</i>
5,6, or De-fuel	UNPLANNED loss of Offsite Power for >15 minutes (1 and 2) 1. C <u>and</u> D CSSTs not available for >15 minutes 2. Either Diesel Generator is supplying power to its respective Shutdown Board

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3.3 Loss of DC Power

Mode	Initiating/Condition
GENERAL SITE	<p>Refer to "Fission Product Barrier Matrix" and "Loss of Function" (2.2)</p>
	<p>Loss of All Vital DC Power for >15 minutes</p> <p>1. Voltage <105V DC on 125V DC Vital Battery Buses 1-I and 1-II and 1-III and 1-IV for >15 minutes</p> <p>Also Refer to "Fission Product Barrier Matrix", "Loss of Function" (2.2), and "Loss of Instrumentation" (2 1)</p>
ALERT UNUSUAL EVENT	<p>Also Refer to "Fission Product Barrier Matrix", "Loss of Function" (2 2), and "Loss of Instrumentation" (2 1)</p>
	<p>UNPLANNED Loss of the Required Train of DC power for >15 minutes (1 or 2)</p> <p>1. Voltage <105V DC on 125V DC Vital Battery Buses 1-I and 1-III for >15 minutes</p> <p>2. Voltage <105V DC on 125V DC Vital Battery Buses 1-II and 1-IV for >15 minutes</p>

FISSION PRODUCT BARRIER MATRIX (Modes 1-4)

- 1.1 Fuel Clad
- 1.2 RCS
- 1.3 Containment

1

SYSTEM DEGRADATION

- | | |
|------------------------------------|-----------------------------|
| 2.1 Loss of Instrumentation | 2.6 RCS Identified Leakage |
| 2.2 Loss of Function/Communication | 2.7 Uncontrolled Cool Down |
| 2.3 Failure of Reactor Protection | 2.8 Turbine Failure |
| 2.4 Fuel Clad Degradation | 2.9 Technical Specification |
| 2.5 RCS Unidentified Leakage | 2.10 Safety Limit |

2

LOSS OF POWER

- 3.1 Loss of AC (Power Ops)
- 3.2 Loss of AC (Shutdown)
- 3.3 Loss of DC

3

HAZARDS and SED JUDGMENT

- | | | |
|---------------|-------------------|-----------------------------|
| 4.1 Fire | 4.3 Flammable Gas | 4.5 Control Room Evacuation |
| 4.2 Explosion | 4.4 Toxic Gas | 4.6 Security |
| Table 4-1 | Table 4-2 | 4.7 SED Judgment |
| Figure 4-A | Figure 4-B | Table 4-3 |

4

DESTRUCTIVE PHENOMENON

- | | |
|----------------------------------|----------------------|
| 5.1 Earthquake | 5.4 River Level High |
| 5.2 Tornado | 5.5 River Level Low |
| 5.3 Aircraft/Projectile
Crash | 5.6 Watercraft Crash |
| Table 5-1 | Figure 5-A |

5

SHUTDOWN SYSTEM DEGRADATION

- 6.1 Loss of Shutdown Systems
- 6.2 Loss of AC (Shutdown)
- 6.3 Loss of DC (Shutdown)
- 6.4 Fuel Handling

6

RADIOLOGICAL

- | | |
|----------------------|----------------------|
| 7.1 Gaseous Effluent | 7.3 Radiation Levels |
| 7.2 Liquid Effluent | 7.4 Fuel Handling |
| Table 7-1 | Table 7-2 |
| Figure 7-A | |

7

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ODCM: Offsite Dose Calculation Manual

ORANGE PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge

PROJECTILE: An object ejected, thrown, or launched towards a plant structure. The source of the projectile may be onsite or offsite. Damage is sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein

PROTECTED AREA: Encompasses all owner controlled areas within the security protected area fence as shown on Figure 4-A.

RED PATH: Monitoring of one or more CSFs by the FR-0 which indicates that the CSF(s) is under extreme challenge, prompt operator action is required.

RUPTURED: (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than charging pump capacity

SABOTAGE: Deliberate damage, misalignment, or mis-operation of plant equipment with the intent to render the equipment inoperable

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following: (1) An automatic turbine runback > 15% thermal reactor power; (2) Electrical load rejection > 25% full electrical load; (3) Reactor Trip or (4) Safety Injection System Activation.

SITE PERIMETER (SP): Encompasses all owner controlled areas in the immediate site environs as shown on Figures 4-A and 7-A.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e g , chlorine)

UNPLANNED: An event or action that is not the expected result of normal operations, testing, or maintenance. Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED

UNPLANNED. (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or setpoints that exceed a condition specified on the DP, e g , alarm thresholds, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

VALID: An indication or report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment, i e , within 15 minutes

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analyses. Damage is sufficient enough to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes deformation due to heat or impact, denting, penetration, rupture, cracking, and/or paint blistering. Surface blemishes (e g , paint chipping, scratches) should NOT be included

VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation.

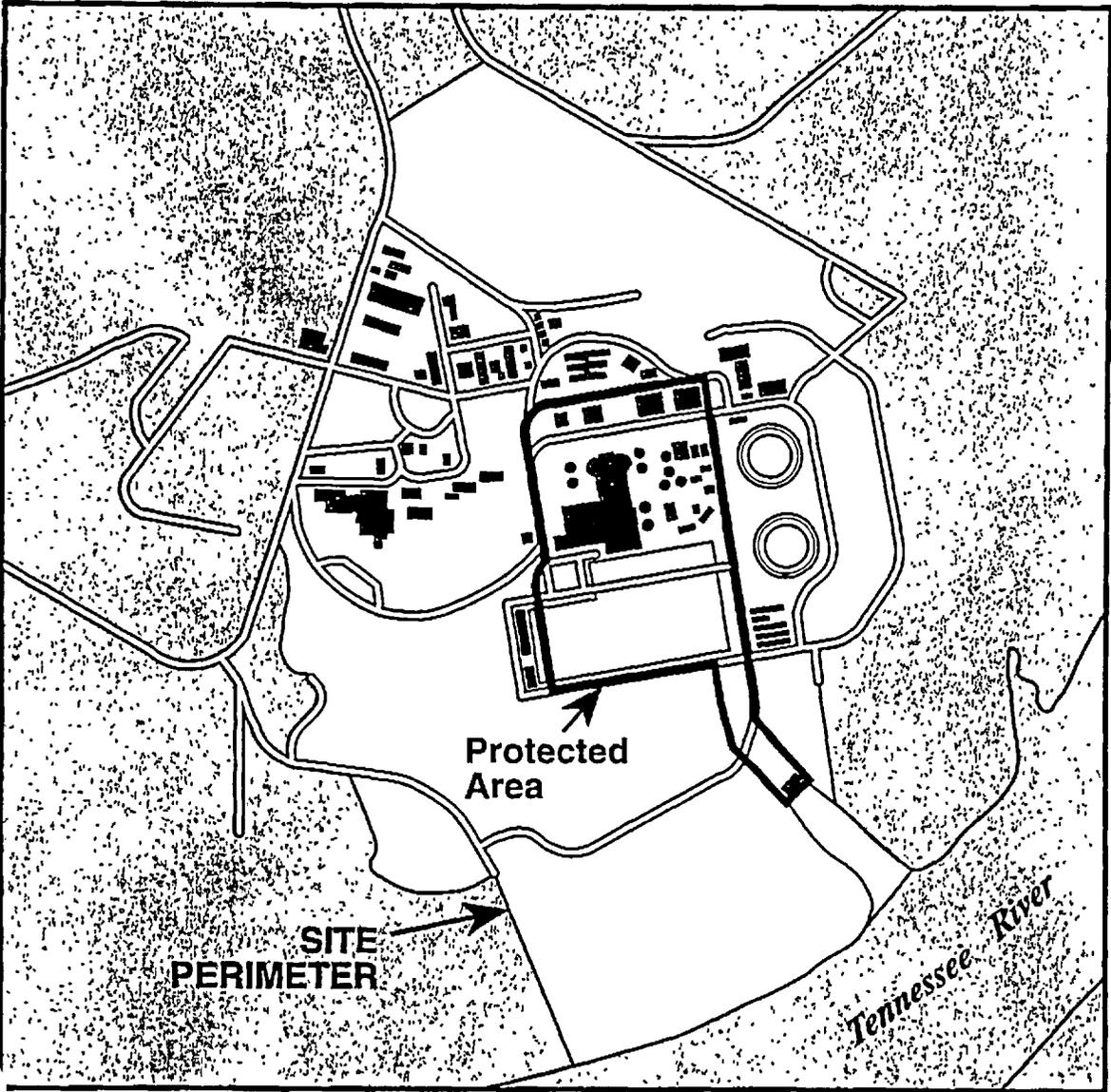
4.1 FIRE	
Mode	Initiating/Condition
GENERAL SITE ALERT UNUSUAL EVENT	Refer to "Fission Product Barrier Matrix"
	Refer to "Control Room Evacuation," (4.5) or Fission Product Barrier Matrix"
	<p>FIRE in any of the areas listed in Table 4-1 that is affecting Safety Related equipment (1 and 2)</p> <p>1 FIRE in any of the areas listed in Table 4-1</p> <p>2. (a or b)</p> <p>a VISIBLE DAMAGE to permanent structure <u>or</u> Safety Related equipment in the specified area is observed due to the FIRE</p> <p>b Control Room indication of degraded Safety System <u>or</u> component response due to the FIRE</p>
All	<p>FIRE in the PROTECTED AREA threatening any of the areas listed in Table 4-1 that is <u>Not</u> extinguished within 15 minutes from the Time of Control Room notification <u>or</u> verification of Control Room Alarm (Figure 4-A)</p>

4.2 Explosions	
Mode	Initiating/Condition
GENERAL SITE ALERT UNUSUAL EVENT	Refer to "Fission Product Barrier Matrix"
	Refer to "Fission Product Barrier Matrix"
	<p>EXPLOSION in any of the areas listed in Table 4-1 that is affecting Safety Related equipment (1 and 2)</p> <p>1. EXPLOSION in any of the areas listed in Table 4-1</p> <p>2 (a or b)</p> <p>a An EXPLOSION has caused VISIBLE DAMAGE to Safety Related equipment</p> <p>b Control Room indication of degraded Safety System <u>or</u> component response due to the EXPLOSION</p> <p>Refer to "Security" (4.6)</p>
All	<p>UNPLANNED EXPLOSION within the PROTECTED AREA resulting in VISIBLE DAMAGE to any permanent structure <u>or</u> equipment (Figure 4-A)</p> <p>Refer to "Security" (4.6)</p>

**TABLE 4-1
PLANT AREAS ASSOCIATED WITH FIRE AND EXPLOSION EALS**

Unit #1 Reactor Building	Additional Diesel Generator Building
Auxiliary Building	Intake Pumping Station
Control Building	Additional Equipment Buildings (Unit 1&2)
Diesel Generator Building	RWST
CST	

**Figure 4-A
PROTECTED AREA/SITE PERIMETER**



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4.3 Flammable Gas	
Mode	Initiating/Condition
GENERAL	Refer to "Fission Product Barrier Matrix"
	Refer to "Fission Product Barrier Matrix"
SITE	Refer to "Fission Product Barrier Matrix"
	Refer to "Fission Product Barrier Matrix"
ALERT	UNPLANNED release of Flammable Gas within a facility structure containing Safety Related equipment <u>or</u> associated with Power production
	<p>All</p> <p>1. Plant personnel report the average of three readings taken in a ~10ft triangular Area is >25% (LEL) Lower Explosive Limit, as indicated on the monitoring instrument within any building listed in Table 4-2</p>
UNUSUAL	A. UNPLANNED release of Flammable Gas within the SITE PERIMETER
	<p>All</p> <p>1. Plant personnel report the average of three readings taken in a ~10ft Triangular Area is >25% (LEL) Lower Explosive Limit, as indicated on the monitoring instrument within the SITE PERIMETER (Refer to Figure 4-B)</p> <p style="text-align: center;"><u>OR</u></p> <p>B. Confirmed report by Local, County, <u>or</u> State Officials that a Large Offsite Flammable Gas release has occurred within One Mile of the Site with potential to enter the SITE PERIMETER in concentrations >25% of LEL Lower Explosive Limit (Refer to Figure 4-B)</p>
EVENT	

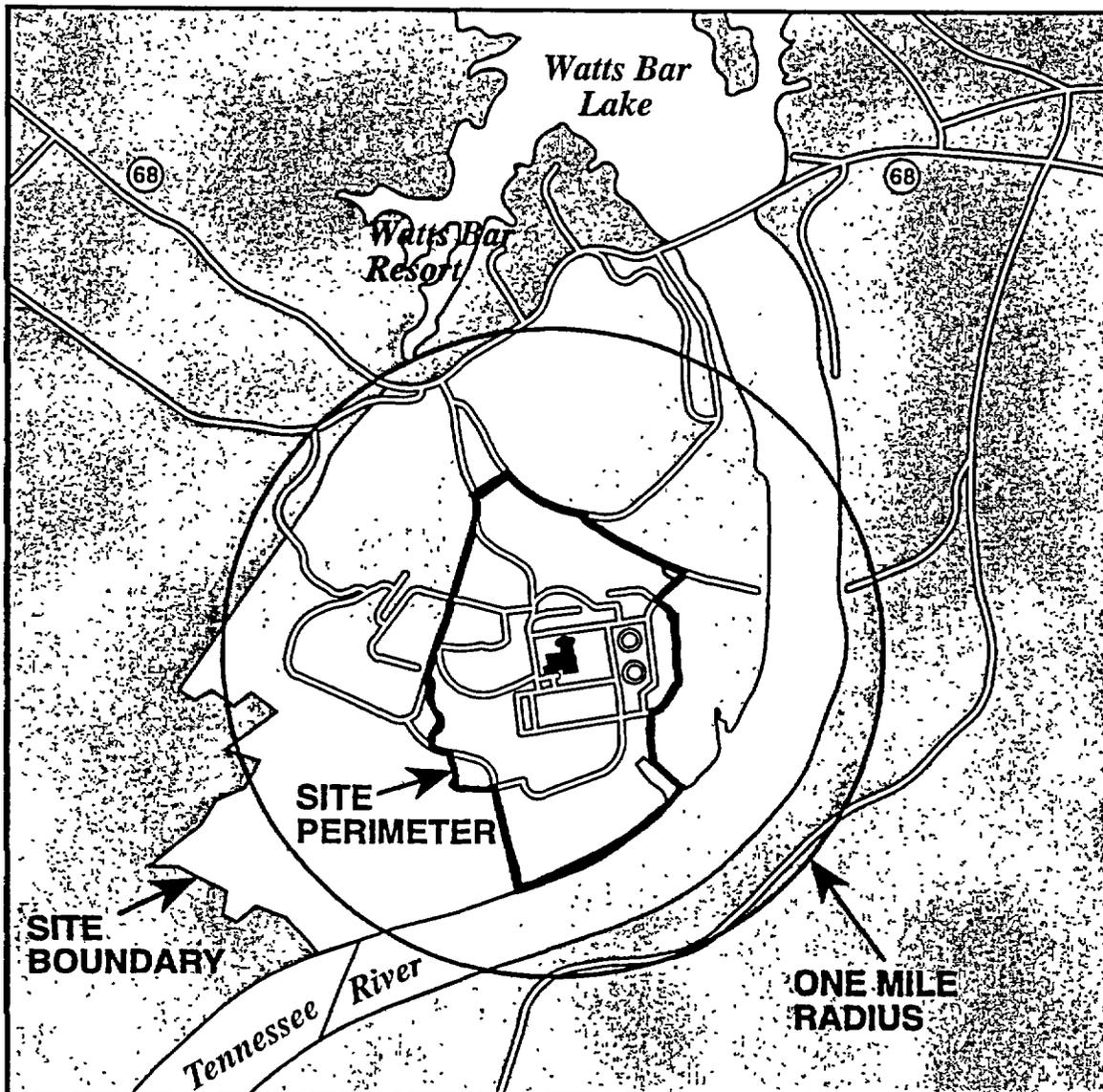
4.4 Toxic Gas	
Mode	Initiating/Condition
GENERAL	Refer to "Fission Product Barrier Matrix"
	Refer to "Fission Product Barrier Matrix"
SITE	Refer to "Fission Product Barrier Matrix"
	Refer to "Fission Product Barrier Matrix"
ALERT	Release of TOXIC GAS within a facility structure which Prohibits Safe Operation of systems required to establish <u>or</u> maintain Cold S/D (1 and 2 and 3)
	<p>All</p> <p>1. Plant personnel report TOXIC GAS within any building listed in Table 4-2</p> <p>2. (a or b)</p> <p>a Plant personnel report Severe Adverse Health Reactions due to TOXIC GAS (i.e , burning eyes, nose, throat, dizziness)</p> <p>b. Sampling indications > (PEL) Permissible Exposure Limit</p> <p>3 Plant personnel would be unable to perform actions necessary to establish and maintain Cold Shutdown while utilizing appropriate personnel protection equipment</p>
UNUSUAL	A. Normal Operations impeded due to access restrictions caused by TOXIC GAS concentrations within a Facility Structure listed in Table 4-2
	<p>All</p> <p style="text-align: center;"><u>OR</u></p> <p>B. Confirmed report by Local, County, <u>or</u> State Officials that a Large Offsite TOXIC GAS release has occurred within One Mile of the Site with potential to enter the Site Perimeter in concentrations >than the (PEL) Permissible Exposure Limit thus causing an Evacuation (Figure 4-B)</p>
EVENT	

TABLE 4-2
Plant Structures Associated With TOXIC or Flammable Gas EALs

Unit #1 & 2 Reactor Buildings
Auxiliary Building
Control Building
Diesel Generator Building

Additional Diesel Generator Building
Intake Pumping Station
Additional Equipment Bldgs (Unit 1&2)
CDWE Building
Turbine Building

Figure 4-B
ONE MILE RADIUS/SITE PERIMETER



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GENERAL
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PERT
UNUSUAL
EVENTS

4.5 Control Room Evacuation

Mode	Initiating/Condition
	<i>Refer to "Fission Product Barrier Matrix"</i>
All	<p>Evacuation of the Control Room has been initiated <u>and</u> Control of all necessary equipment <u>Has Not</u> been established within 15 minutes of manning the Auxiliary Control Room (1 and 2 and 3)</p> <ol style="list-style-type: none"> 1. (a or b) <ol style="list-style-type: none"> a. AOI-30 2 "Fire Safety Shutdown" entered b. AOI-27 "Main Control Room Inaccessibility" entered 2. SM/SED Orders Control Room evacuation 3. Control has <u>Not</u> been established at the Remote Shutdown Panel within 15 minutes of manning the Auxiliary Control Room and transfer of switches on Panels L11A and L11B
All	<p>Evacuation of the Control Room is Required (1 and 2)</p> <ol style="list-style-type: none"> 1. (a or b) <ol style="list-style-type: none"> a. AOI-30 2 "Fire Safe Shutdown" entered b. AOI-27 "Main Control Room Inaccessibility" entered 2. SM/SED Orders Control Room evacuation
	<i>Not Applicable</i>

4.6 Security

Mode	Initiating/Condition
All	<p>Security Event resulting in loss of Control of the Plant</p> <ol style="list-style-type: none"> 1. Hostile Armed Force has taken Control of the Plant, Control Room, <u>or</u> Remote shutdown capability
All	<p>Security Event has <u>or</u> is occurring which results in Actual <u>or</u> Likely Failures of Plant Functions needed to Protect the Public</p> <ol style="list-style-type: none"> 1. VITAL AREA, other than the Control Room, has been penetrated by a Hostile Armed Force
All	<p>Confirmed Security Event which indicates an Actual <u>or</u> Potential Substantial Degradation in the level of Safety of the Plant (1 or 2 or 3)</p> <ol style="list-style-type: none"> 1. BOMB discovered within a VITAL AREA 2. CIVIL DISTURBANCE ongoing within the PROTECTED AREA 3. PROTECTED AREA has been penetrated by a Hostile Armed Force <p><i>Refer to Figure 4-A For a Drawing of Protected Area and Site Perimeter</i></p>
All	<p>Confirmed Security Event which indicates a Potential Degradation in the level of Safety of the Plant (1 or 2)</p> <ol style="list-style-type: none"> 1. BOMB discovered within the PROTECTED AREA 2. Security Shift Supervisor reports one <u>or</u> more of the events listed in Table 4-3

4.7 Emergency Director Judgment

Mode	Initiating/Condition
All	Events are in progress <u>or</u> have occurred which involve Actual <u>or</u> Imminent Substantial Core Degradation <u>or</u> Melting With Potential for Loss of Containment Integrity. Releases can be reasonable expected to exceed EPA Plume Protective Action Guidelines Exposure Levels outside the EXCLUSION AREA BOUNDARY, Refer to Figure 7-A.
All	Events are in progress <u>or</u> have occurred which involve Actual <u>or</u> Likely Major Failures of Plant Functions needed for the Protection of the Public. Any releases are not expected to result in Exposure Levels which Exceed EPA Plume Protective Action Guidelines Exposure Levels outside the EXCLUSION AREA BOUNDARY, Refer to Figure 7-A.
All	Events are in progress <u>or</u> have occurred which involve Actual <u>or</u> Potential Substantial Degradation of the Level of Safety of the Plant. Any releases are expected to be limited to small fractions of the EPA Plume Protective Action Guidelines Exposure Levels.
All	Unusual Events are in Progress <u>or</u> have occurred which indicate a Potential Degradation of the Level of Safety of the Plant. No releases of Radioactive Material requiring Offsite Response <u>or</u> Monitoring are expected unless further degradation of Safety Systems occurs.

GENERAL

SITE

ALERT

UNUSUAL

EVENT

HAZARDS / SEVERE JUDGMENT U1

**Table 4-3
SECURITY EVENTS**

- a SABOTAGE/INTRUSION has occurred or is occurring within the PROTECTED AREA
- b HOSTAGE/EXTORTION Situation that Threatens to interrupt Plant Operations
- c CIVIL DISTURBANCE ongoing between the SITE PERIMETER and PROTECTED AREA
- d Hostile STRIKE ACTION within the PROTECTED AREA which threatens to interrupt Normal Plant Operations (Judgment Based on behavior of Strikers and/or Intelligence received)
- e. A CREDIBLE SITE-SPECIFIC security threat notification.

FISSION PRODUCT BARRIER MATRIX (Modes 1-4)

- 1.1 Fuel Clad
- 1.2 RCS
- 1.3 Containment

1

SYSTEM DEGRADATION

- | | |
|------------------------------------|-----------------------------|
| 2.1 Loss of Instrumentation | 2.6 RCS Identified Leakage |
| 2.2 Loss of Function/Communication | 2.7 Uncontrolled Cool Down |
| 2.3 Failure of Reactor Protection | 2.8 Turbine Failure |
| 2.4 Fuel Clad Degradation | 2.9 Technical Specification |
| 2.5 RCS Unidentified Leakage | 2.10 Safety Limit |

2

LOSS OF POWER

- 3.1 Loss of AC (Power Ops)
- 3.2 Loss of AC (Shutdown)
- 3.3 Loss of DC

3

HAZARDS and SED JUDGMENT

- | | | |
|---------------|-------------------|-----------------------------|
| 4.1 Fire | 4.3 Flammable Gas | 4.5 Control Room Evacuation |
| 4.2 Explosion | 4.4 Toxic Gas | 4.6 Security |
| Table 4-1 | Table 4-2 | 4.7 SED Judgment |
| Figure 4-A | Figure 4-B | Table 4-3 |

4

DESTRUCTIVE PHENOMENON

- | | |
|---|------------------------------------|
| 5.1 Earthquake | 5.4 River Level High |
| 5.2 Tornado | 5.5 River Level Low |
| 5.3 Aircraft/Projectile
Crash
Table 5-1 | 5.6 Watercraft Crash
Figure 5-A |

5

SHUTDOWN SYSTEM DEGRADATION

- 6.1 Loss of Shutdown Systems
- 6.2 Loss of AC (Shutdown)
- 6.3 Loss of DC (Shutdown)
- 6.4 Fuel Handling

6

RADIOLOGICAL

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| 7.1 Gaseous Effluent | 7.3 Radiation Levels |
| 7.2 Liquid Effluent
Table 7-1
Figure 7-A | 7.4 Fuel Handling
Table 7-2 |

7

DEFINITIONS/ACRONYMS

UNUSUAL EVENT, ALERT, SITE AREA EMERGENCY and GENERAL EMERGENCY: (see SED Judgment 4 7)

BOMB: An explosive device (See EXPLOSION)

CIVIL DISTURBANCE: A group of twenty (20) or more persons violently protesting station operations or activities at the site

CREDIBLE SITE-SPECIFIC -The determination is made by WBN senior plant management through use of information found in the Safeguards Contingency Plan

CRITICAL-SAFETY FUNCTION (CSFs): A plant safety function required to prevent significant release of core radioactivity to the environment There are six CSFs: Sub-criticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS)

EVENT: Assessment of an EVENT commences when recognition is made that one or more of the conditions associated with the event exist. Implicit in this definition is the need for timely assessment, i.e. within 15 minutes

EXCLUSION AREA BOUNDARY (EAB): The demarcation of the area surrounding the WBN units in which postulated FSAR accidents will not result in population doses exceeding the criteria of 10 CFR Part 100 Refer to Figure 7-A.

EXPLOSION: A rapid, violent, unconfined combustion, or a catastrophic failure of pressurized equipment that imparts energy of sufficient force to potentially damage permanent structures required for safe operation

EXTORTION: An attempt to cause an action at the station by threat of force

FAULTED: (Steam Generator) Existence of secondary side leakage (i.e., steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized.

FIRE: Combustion characterized by heat and light Source of smoke such as slipping drive belts or overheated electrical components do not constitute fires Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed

FLAMMABLE GAS: Combustible gases maintained at concentrations less than the LOWER EXPLOSIVE LIMIT (LEL) will not explode due to ignition

HOSTAGE: A person(s) held as leverage against the station to ensure that demands will be met by the station

INEFFECTIVE: The specified restoration action(s) does not result in a reduction in the level of severity of the RED PATH condition within 15 minutes from identification of the Core Cooling CSF Status Tree RED PATH A reduction in the level of severity is an improvement in the applicable parameters, e.g., Increasing Trend in Reactor Vessel Water Level (Full RVLIS) and/or Decreasing Trend on Core Thermocouple Temperatures.

INITIATING CONDITIONS: Plant Parameters, radiation monitor readings or personnel observations that identify an Event for purposes of Emergency Plan Classification

INTRUSION/INTRUDER: Suspected hostile individual present in a protected area without authorization

ODCM: Offsite Dose Calculation Manual

ORANGE PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge

PROJECTILE: An object ejected, thrown, or launched towards a plant structure The source of the projectile may be onsite or offsite Damage is sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein

PROTECTED AREA: Encompasses all owner controlled areas within the security protected area fence as shown on Figure 4-A.

RED PATH: Monitoring of one or more CSFs by the FR-0 which indicates that the CSF(s) is under extreme challenge; prompt operator action is required.

RUPTURED: (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than charging pump capacity

SABOTAGE: Deliberate damage, misalignment, or mis-operation of plant equipment with the intent to render the equipment inoperable

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following (1) An automatic turbine runback > 15% thermal reactor power; (2) Electrical load rejection > 25% full electrical load, (3) Reactor Trip or (4) Safety Injection System Activation

SITE PERIMETER (SP): Encompasses all owner controlled areas in the immediate site environs as shown on Figures 4-A and 7-A.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e.g., chlorine)

UNPLANNED: An event or action that is not the expected result of normal operations, testing, or maintenance Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED

UNPLANNED: (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP) Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

VALID: An indication or report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel Implicit in this definition is the need for timely assessment, i.e., within 15 minutes

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analyses Damage is sufficient enough to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes: deformation due to heat or impact, denting, penetration, rupture, cracking, and/or paint blistering Surface blemishes (e.g., paint chipping, scratches) should NOT be included.

VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation

5.1 Earthquake

5.2 Tornado

	Mode	Initiating/Condition
GENERAL		Refer to "Fission Product Barrier Matrix"
		Refer to "Fission Product Barrier Matrix"
SITE		Refer to "Fission Product Barrier Matrix"
		Refer to "Fission Product Barrier Matrix"
ALERT	All	<p>Earthquake detected by site seismic instrumentation (1 and 2)</p> <ol style="list-style-type: none"> (a and b) <ol style="list-style-type: none"> Ann 166 D indicates "OBE Spectra Exceeded" Ann.166 E indicates "Seismic Recording Initiated" (a or b) <ol style="list-style-type: none"> Ground motion sensed by Plant personnel National Earthquake Information Center at 1-(303) 273-8500 can confirm the event
	All	<p>Earthquake detected by site seismic instrumentation (1 and 2)</p> <ol style="list-style-type: none"> Ann 166 E indicator "Seismic Recording Initiated" (a or b) <ol style="list-style-type: none"> Ground motion sensed by Plant personnel National Earthquake Information Center at 1-(303) 273-8500 can confirm the event
UNUSUAL EVENT		

	Mode	Initiating/Condition
GENERAL		Refer to "Fission Product Barrier Matrix"
		Refer to "Fission Product Barrier Matrix"
SITE		Refer to "Fission Product Barrier Matrix"
		Refer to "Fission Product Barrier Matrix"
ALERT	All	<p>Tornado <u>or</u> High Winds strikes any structure listed in Table 5-1 and results in VISIBLE DAMAGE (1 and 2)</p> <ol style="list-style-type: none"> Tornado or High Winds (Sustained >80 mph > one minute) strikes any structure listed in Table 5-1 (a or b) <ol style="list-style-type: none"> Confirmed report of any VISIBLE DAMAGE Control Room indications of degraded Safety System <u>or</u> component response due to event <p><i>Note: Site Met Data Instrumentation fails to 0 at >100 mph National Weather Service Mornstown 1-(423) 586-8400 can provide additional information if needed</i></p>
	All	<p>Tornado within the SITE PERIMETER</p> <ol style="list-style-type: none"> Plant personnel report a Tornado has been sighted within the SITE PERIMETER (Refer to Figure 5-A)
UNUSUAL EVENT		

5.3 Aircraft/Projectile Crash

Mode	Initiating/Condition
GENERAL	Refer to "Fission Product Barrier Matrix"
SITE	Refer to "Fission Product Barrier Matrix"
ALERT	<p>All Aircraft <u>or</u> PROJECTILE impacts (Strikes) any Plant structure listed in Table 5-1 resulting in VISIBLE DAMAGE (1 and 2)</p> <ol style="list-style-type: none"> Plant personnel report aircraft <u>or</u> PROJECTILE has impacted any structure listed in Table 5-1 (a or b) <ol style="list-style-type: none"> Confirmed report of any VISIBLE DAMAGE Control Room indications of degraded Safety System <u>or</u> component response due to the event within the specified areas
UNUSUAL EVENT	<p>All Aircraft crash <u>or</u> PROJECTILE impact within the SITE PERIMETER</p> <ol style="list-style-type: none"> Plant personnel report a Aircraft Crash <u>or</u> PROJECTILE impact within the SITE PERIMETER (Refer to Figure 5-A)

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**Table 5-1
Plant Structures Associated With
Tornado/Hi Wind and Aircraft EALs**

- Unit #1 and 2 Reactor Buildings
- Auxiliary Building
- Control Building
- Diesel Generator Building
- Additional Diesel Generator Building
- Intake Pumping Station
- Additional Equipment Buildings (Units 1 & 2)
- CDWE Building
- Turbine Building
- RWST
- CST

5.4 River Level HIGH		
	Mode	Initiating/Condition
GENERAL		Refer to "Fission Product Barrier Matrix"
		Refer to "Fission Product Barrier Matrix"
SITE		Refer to "Fission Product Barrier Matrix"
		Refer to "Fission Product Barrier Matrix"
ALERT	All	<p>River Reservoir level is at Stage II Flood Warning (1 or 2)</p> <ol style="list-style-type: none"> River Reservoir level >727 Ft Stage II Flood Warning (AOI-7) has been issued by River Systems Operations
	All	<p>River Reservoir level is at Stage I Flood Warning (1 or 2 or 3)</p> <ol style="list-style-type: none"> River Reservoir level >726.5 Ft from April 16 thru September 30 River Reservoir level >714.5 Ft from October 1 thru April 15 Stage I Flood Warning (AOI-7) has been issued by River Systems Operations
UNUSUAL EVENT		Refer to "Fission Product Barrier Matrix"
		Refer to "Fission Product Barrier Matrix"

5.5 River Level LOW		
	Mode	Initiating/Condition
GENERAL		Refer to "Fission Product Barrier Matrix"
		Refer to "Fission Product Barrier Matrix"
SITE		Refer to "Fission Product Barrier Matrix"
		Refer to "Fission Product Barrier Matrix"
ALERT	All	<p>River Reservoir level is <668 Ft (AOI-22) as reported by River Systems Operations</p>
	All	<p>River Reservoir level is ≤673 Ft (AOI-22) as reported by River Systems Operations</p>
UNUSUAL EVENT		Refer to "Fission Product Barrier Matrix"
		Refer to "Fission Product Barrier Matrix"

5.6 Watercraft Crash

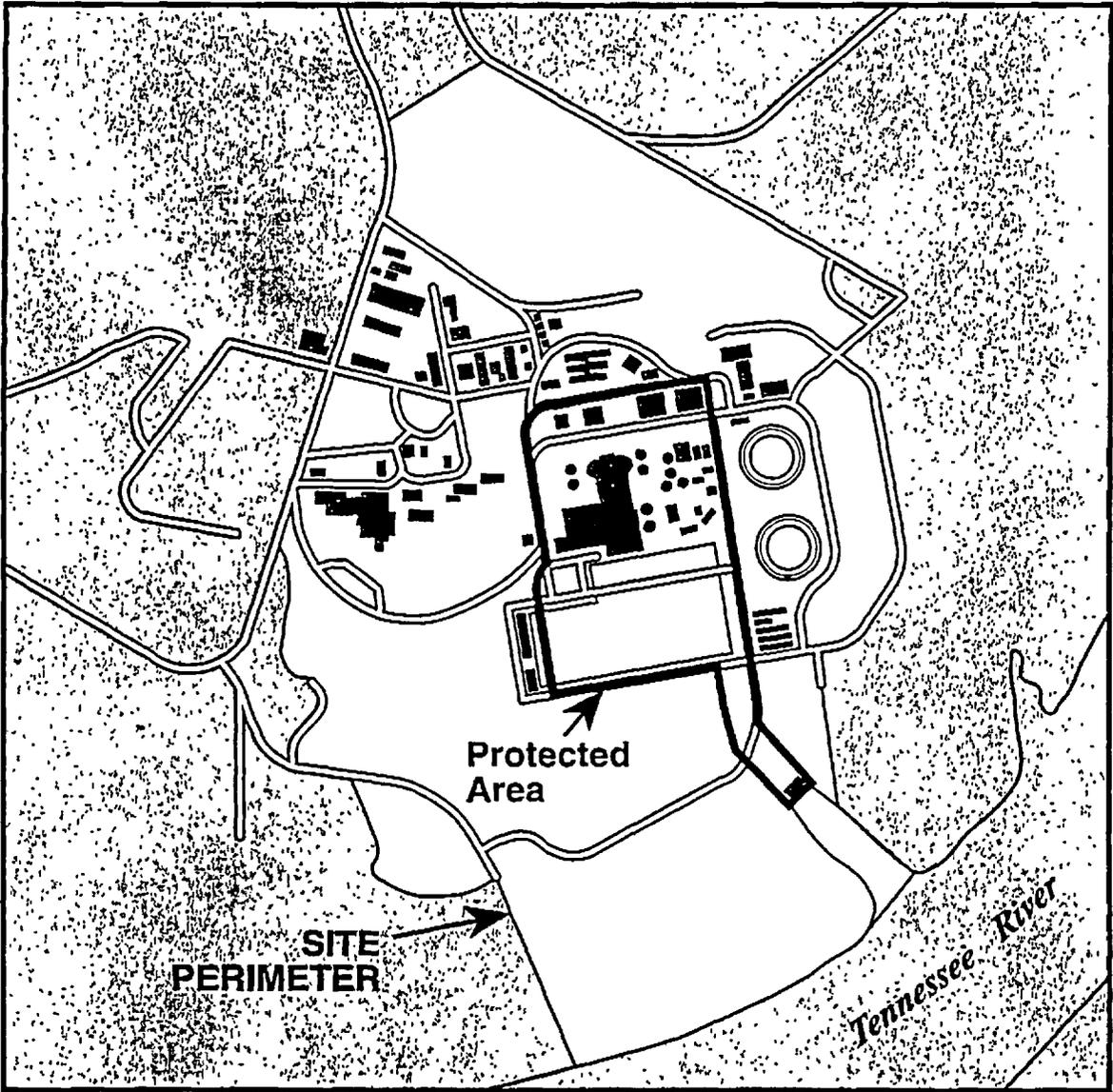
Mode	Initiating/Condition	
GENERAL SITE	Refer to "Fission Product Barrier Matrix"	
		Refer to "Fission Product Barrier Matrix"
		Refer to "Fission Product Barrier Matrix"
UNUSUAL EVENT	All Watercraft Strikes the Intake Pumping Station resulting in a reduction of Essential Raw Cooling Water (ERCW) or Raw Cooling Water (RCW) (1 and 2) 1. Plant personnel report a Watercraft has struck the Intake Pumping Station 2 (a or b or c) a ERCW Supply Header Pressure Train A O-PI-67-18A is <15 psig b ERCW Supply Header Pressure Train B O-PI-67-17A is <15 psig c. RCW Supply Header Pressure O-PI-24-22 is <15 psig	

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Figure 5-A
PROTECTED AREA/SITE PERIMETER



FISSION PRODUCT BARRIER MATRIX (Modes 1-4)

- 1.1 Fuel Clad
- 1.2 RCS
- 1.3 Containment

1

SYSTEM DEGRADATION

- | | |
|------------------------------------|-----------------------------|
| 2.1 Loss of Instrumentation | 2.6 RCS Identified Leakage |
| 2.2 Loss of Function/Communication | 2.7 Uncontrolled Cool Down |
| 2.3 Failure of Reactor Protection | 2.8 Turbine Failure |
| 2.4 Fuel Clad Degradation | 2.9 Technical Specification |
| 2.5 RCS Unidentified Leakage | 2.10 Safety Limit |

2

LOSS OF POWER

- 3.1 Loss of AC (Power Ops)
- 3.2 Loss of AC (Shutdown)
- 3.3 Loss of DC

3

HAZARDS and SED JUDGMENT

- | | | |
|---------------|-------------------|-----------------------------|
| 4.1 Fire | 4.3 Flammable Gas | 4.5 Control Room Evacuation |
| 4.2 Explosion | 4.4 Toxic Gas | 4.6 Security |
| Table 4-1 | Table 4-2 | 4.7 SED Judgment |
| Figure 4-A | Figure 4-B | Table 4-3 |

4

DESTRUCTIVE PHENOMENON

- | | |
|----------------------------------|----------------------|
| 5.1 Earthquake | 5.4 River Level High |
| 5.2 Tornado | 5.5 River Level Low |
| 5.3 Aircraft/Projectile
Crash | 5.6 Watercraft Crash |
| Table 5-1 | Figure 5-A |

5

SHUTDOWN SYSTEM DEGRADATION

- 6.1 Loss of Shutdown Systems
- 6.2 Loss of AC (Shutdown)
- 6.3 Loss of DC (Shutdown)
- 6.4 Fuel Handling

6

RADIOLOGICAL

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| 7.1 Gaseous Effluent | 7.3 Radiation Levels |
| 7.2 Liquid Effluent | 7.4 Fuel Handling |
| Table 7-1 | Table 7-2 |
| Figure 7-A | |

7

DEFINITIONS/ACRONYMS

UNUSUAL EVENT, ALERT, SITE AREA EMERGENCY and GENERAL EMERGENCY: (see SED Judgment 4 7)

BOMB: An explosive device (See EXPLOSION)

CIVIL DISTURBANCE: A group of twenty (20) or more persons violently protesting station operations or activities at the site

CREDIBLE SITE-SPECIFIC -The determination is made by WBN senior plant management through use of information found in the Safeguards Contingency Plan

CRITICAL-SAFETY FUNCTION (CSFs): A plant safety function required to prevent significant release of core radioactivity to the environment. There are six CSFs: Sub-criticality, Core Cooling, Heat Sink, Pressurized Thermal Shock, Integrity (Containment) and Inventory (RCS)

EVENT: Assessment of an EVENT commences when recognition is made that one or more of the conditions associated with the event exist. Implicit in this definition is the need for timely assessment, i e within 15 minutes

EXCLUSION AREA BOUNDARY (EAB): The demarcation of the area surrounding the WBN units in which postulated FSAR accidents will not result in population doses exceeding the criteria of 10 CFR Part 100 Refer to Figure 7-A.

EXPLOSION: A rapid, violent, unconfined combustion, or a catastrophic failure of pressurized equipment that imparts energy of sufficient force to potentially damage permanent structures required for safe operation

EXTORTION: An attempt to cause an action at the station by threat of force

FAULTED: (Steam Generator) Existence of secondary side leakage (i e , steam or feed line break) that results in an uncontrolled decrease in steam generator pressure or the steam generator being completely depressurized

FIRE: Combustion characterized by heat and light Source of smoke such as slipping drive belts or overheated electrical components do not constitute fires Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed

FLAMMABLE GAS: Combustible gases maintained at concentrations less than the LOWER EXPLOSIVE LIMIT (LEL) will not explode due to ignition

HOSTAGE: A person(s) held as leverage against the station to ensure that demands will be met by the station.

INEFFECTIVE: The specified restoration action(s) does not result in a reduction in the level of severity of the RED PATH condition within 15 minutes from identification of the Core Cooling CSF Status Tree RED PATH. A reduction in the level of severity is an improvement in the applicable parameters, e g , Increasing Trend in Reactor Vessel Water Level (Full RVLIS) and/or Decreasing Trend on Core Thermocouple Temperatures

INITIATING CONDITIONS: Plant Parameters, radiation monitor readings or personnel observations that identify an Event for purposes of Emergency Plan Classification.

INTRUSION/INTRUDER: Suspected hostile individual present in a protected area without authorization

ODCM: Offsite Dose Calculation Manual

ORANGE PATH: Monitoring of one or more CSFs by FR-0 which indicates that the CSF(s) is under severe challenge

PROJECTILE: An object ejected, thrown, or launched towards a plant structure The source of the projectile may be onsite or offsite Damage is sufficient to cause concern regarding the integrity of the affected structure or the operability or reliability of safety equipment contained therein

PROTECTED AREA: Encompasses all owner controlled areas within the security protected area fence as shown on Figure 4-A.

RED PATH: Monitoring of one or more CSFs by the FR-0 which indicates that the CSF(s) is under extreme challenge, prompt operator action is required.

RUPTURED: (Steam Generator) Existence of primary to secondary leakage of a magnitude greater than charging pump capacity.

SABOTAGE: Deliberate damage, misalignment, or mis-operation of plant equipment with the intent to render the equipment inoperable

SIGNIFICANT TRANSIENT: An UNPLANNED event involving one or more of the following (1) An automatic turbine runback > 15% thermal reactor power; (2) Electrical load rejection > 25% full electrical load; (3) Reactor Trip or (4) Safety Injection System Activation

SITE PERIMETER (SP): Encompasses all owner controlled areas in the immediate site environs as shown on Figures 4-A and 7-A.

STRIKE ACTION: A work stoppage within the PROTECTED AREA by a body of workers to enforce compliance with demands made on TVA. The STRIKE ACTION must threaten to interrupt normal plant operations.

TOXIC GAS: A gas that is dangerous to life or limb by reason of inhalation or skin contact (e g , chlorine)

UNPLANNED: An event or action that is not the expected result of normal operations, testing, or maintenance Events that result in corrective or mitigative actions being taken in accordance with abnormal or emergency procedures are UNPLANNED

UNPLANNED: (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP) Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, e g , alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

VALID: An indication or report or condition is considered to be VALID when it is conclusively verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel. Implicit in this definition is the need for timely assessment, i e , within 15 minutes

VISIBLE DAMAGE: Damage to equipment that is readily observable without measurements, testing, or analyses Damage is sufficient enough to cause concern regarding the continued operability or reliability of affected safety structure, system, or component. Example damage includes deformation due to heat or impact, denting, penetration, rupture, cracking, and/or paint blistering. Surface blemishes (e g , paint chipping, scratches) should NOT be included

VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation

6.1 Loss of Shutdown Systems	
Mode	Initiating/Condition
GENERAL SITE ALERT UNUSUAL EVENT	5,6 Note. Additional information will be provided later pending NRC Guidance on Shutdown EALs <i>Refer to "Gaseous Effluents" (7.1)</i>
	5,6 Loss of water level in the Rx vessel that has or will uncover fuel in the Rx vessel with CNTMT closure established (1 and 2 and 3 and 4 and 5) 1. Loss of RHR capability 2 Rx vessel water level < el 718' 3 Incore TCs (if available) indicate RCS temp >200° F 4 RCS is vented/open to CNTMT 5 CNTMT closure is established <i>Note: If CNTMT open, refer to "Gaseous Effluents" (7.1)</i>
	5,6 Inability to maintain Unit in Cold Shutdown (1 and 2 and 3) 1. RHR capability is <u>not</u> available for RCS Cooling 2 Incore TCs (if available) indicate RCS temp. >200° F 3 CNTMT closure is established
	5,6 Note Additional information will be provided later pending NRC Guidance on Shutdown EALs ○

6.2 Loss of AC (Shutdown)	
Mode	Initiating/Condition
	<i>Not Applicable</i>
	<i>Not Applicable</i>
5,6 or De-Fuel	UNPLANNED loss of Offsite <u>and</u> Onsite AC Power for >15 minutes 1. 1A <u>and</u> 1B 6 9 KV Shutdown Bds de-energized for >15 minutes
5,6 or De-Fuel	UNPLANNED loss of All Offsite Power for >15 minutes (1 and 2) 1. C <u>and</u> D CSSTS not available For >15 minutes 2 Either Diesel Generator is supplying power to its respective Shutdown Board

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6.3 Loss of DC (Shutdown)	
Mode	Initiating/Condition
G E N E R A L S I T E A L E R T U N S U A L E V E N T	Not Applicable
	Not Applicable
	Not Applicable
5,6 or De-fuel	UNPLANNED loss of the required Train of DC Power for >15 minutes (1 or 2) 1 Voltage <105V DC on 125V DC Vital Battery Buses 1-I <u>and</u> 1-III for >15 minutes 2. Voltage <105V DC on 125V DC Vital Battery Buses 1-II <u>and</u> 1-IV for >15 minutes

6.4 Fuel Handling	
Mode	Initiating/Condition
	Refer to "Gaseous Effluents" (7.1)
	Refer to "Gaseous Effluents" (7.1)
All	Major damage to Irradiated Fuel, <u>or</u> Loss of water level that has <u>or</u> will uncover Irradiated Fuel outside the Reactor Vessel (1 and 2) 1. VALID alarm on O-RE-90-101 <u>or</u> O-RE-90-102 <u>or</u> O-RE-90-103 <u>or</u> 1-RE-90-130/131 <u>or</u> 1-RE-90-112 <u>or</u> 1-RE-90-400 <u>or</u> 2-RE-90-400 2. (a or b) a Plant personnel report damage of Irradiated Fuel sufficient to rupture Fuel Rods b Plant personnel report water level drop has <u>or</u> will exceed makeup capability such that Irradiated Fuel will be uncovered
All	UNPLANNED loss of water level in Spent Fuel Pool <u>or</u> Reactor Cavity <u>or</u> Transfer Canal with fuel remaining covered (1 and 2 and 3) 1. Plant personnel report water level drop in Spent Fuel Pool <u>or</u> Reactor Cavity, <u>or</u> Transfer Canal 2 VALID alarm on O-RE-90-102 <u>or</u> O-RE-90-103 <u>or</u> 1-RE-90-59 <u>or</u> 1-RE-90-60 3 Fuel remains covered with water

FISSION PRODUCT BARRIER MATRIX (Modes 1-4)

- 1.1 Fuel Clad
- 1.2 RCS
- 1.3 Containment

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SYSTEM DEGRADATION

- 2.1 Loss of Instrumentation
- 2.2 Loss of Function/Communication
- 2.3 Failure of Reactor Protection
- 2.4 Fuel Clad Degradation
- 2.5 RCS Unidentified Leakage
- 2.6 RCS Identified Leakage
- 2.7 Uncontrolled Cool Down
- 2.8 Turbine Failure
- 2.9 Technical Specification
- 2.10 Safety Limit

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LOSS OF POWER

- 3.1 Loss of AC (Power Ops)
- 3.2 Loss of AC (Shutdown)
- 3.3 Loss of DC

3

HAZARDS and SED JUDGMENT

- 4.1 Fire
- 4.2 Explosion
- 4.3 Flammable Gas
- 4.4 Toxic Gas
- 4.5 Control Room Evacuation
- 4.6 Security
- 4.7 SED Judgment
- Table 4-1
- Figure 4-A
- Table 4-2
- Figure 4-B
- Table 4-3

4

DESTRUCTIVE PHENOMENON

- 5.1 Earthquake
- 5.2 Tornado
- 5.3 Aircraft/Projectile Crash
- 5.4 River Level High
- 5.5 River Level Low
- 5.6 Watercraft Crash
- Table 5-1
- Figure 5-A

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SHUTDOWN SYSTEM DEGRADATION

- 6.1 Loss of Shutdown Systems
- 6.2 Loss of AC (Shutdown)
- 6.3 Loss of DC (Shutdown)
- 6.4 Fuel Handling

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RADIOLOGICAL

- 7.1 Gaseous Effluent
- 7.2 Liquid Effluent
- 7.3 Radiation Levels
- 7.4 Fuel Handling
- Table 7-1
- Figure 7-A
- Table 7-2

7

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FIRE: Combustion characterized by heat and light. Source of smoke such as slipping drive belts or overheated electrical components do not constitute fires. Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed.

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UNPLANNED: (With specific regard to radioactivity releases) A release of radioactivity is UNPLANNED if the release has not been authorized by a Discharge Permit (DP). Implicit in this definition are unintentional releases, unmonitored releases, or planned releases that exceed a condition specified on the DP, e.g., alarm setpoints, minimum dilution flow, minimum release times, maximum release rates, and/or discharge of incorrect tank.

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VITAL AREA: Is any area within the PROTECTED AREA which contains equipment, systems, devices, or material, the failure, destruction, or release of which could directly or indirectly endanger the public health and safety by exposure to radiation

7.1 Gaseous Effluents		
	Mode	Initiating/Condition
GENERAL	All	<p>EAB dose resulting from an actual or imminent release of Gaseous Radioactivity that exceeds 1000 mrem TEDE or 5000 mrem Thyroid CDE for the actual or projected duration of the release (1 or 2 or 3)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under General in Table 7-1 for >15 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded. 2. Field survey results indicate >1000 mrem/hr gamma or an I-131 concentration of 3.9E-6 μ Ci/cc at SP 3. EP dose assessment results indicate EAB dose >1000 mrem TEDE or >5000 mrem Thyroid CDE for the actual or projected duration of the release (Figure 7-A)
	All	<p>EAB dose resulting from an actual or imminent release of Gaseous Radioactivity that exceeds 100 mrem TEDE or 500 mrem Thyroid CDE for the actual or projected duration of the release (1 or 2 or 3)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under Site in Table 7-1 for >15 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Field survey results indicate >100 mrem/hr gamma or an I-131 concentration of 3.9E-7 μ Ci/cc at SP 3. EP dose assessment results indicate EAB dose >100 mrem TEDE or >500 mrem Thyroid CDE for the actual or projected duration of the release (Figure 7-A)
SITE	All	<p>Any UNPLANNED release of Gaseous Radioactivity that exceeds 200 times the ODCM Limit for >15 minutes (1 or 2 or 3)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under Alert in Table 7-1 for >15 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Field survey results indicate >10 mrem/hr gamma at SP >15 minutes 3. EP dose assessment results indicate EAB dose >10 mrem TEDE for the duration of the release (Figure 7-A)
	All	<p>Any UNPLANNED release of Gaseous Radioactivity that exceeds 2 times the ODCM Limit for >60 minutes (1 or 2 or 3)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under UE in Table 7-1 for >60 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Field survey results indicate >0.1 mrem/hr gamma at SP for >60 minutes 3. EP dose assessment results indicate EAB dose >0.1 mrem TEDE for the duration of the release (Figure 7-A)
ALERT	All	<p>Any UNPLANNED release of Gaseous Radioactivity that exceeds 200 times the ODCM Limit for >15 minutes (1 or 2 or 3)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under Alert in Table 7-1 for >15 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Field survey results indicate >10 mrem/hr gamma at SP >15 minutes 3. EP dose assessment results indicate EAB dose >10 mrem TEDE for the duration of the release (Figure 7-A)
	All	<p>Any UNPLANNED release of Gaseous Radioactivity that exceeds 2 times the ODCM Limit for >60 minutes (1 or 2 or 3)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under UE in Table 7-1 for >60 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Field survey results indicate >0.1 mrem/hr gamma at SP for >60 minutes 3. EP dose assessment results indicate EAB dose >0.1 mrem TEDE for the duration of the release (Figure 7-A)
UNUSUAL EVENT	All	<p>Any UNPLANNED release of Gaseous Radioactivity that exceeds 200 times the ODCM Limit for >15 minutes (1 or 2 or 3)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under Alert in Table 7-1 for >15 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Field survey results indicate >10 mrem/hr gamma at SP >15 minutes 3. EP dose assessment results indicate EAB dose >10 mrem TEDE for the duration of the release (Figure 7-A)
	All	<p>Any UNPLANNED release of Gaseous Radioactivity that exceeds 2 times the ODCM Limit for >60 minutes (1 or 2 or 3)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under UE in Table 7-1 for >60 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Field survey results indicate >0.1 mrem/hr gamma at SP for >60 minutes 3. EP dose assessment results indicate EAB dose >0.1 mrem TEDE for the duration of the release (Figure 7-A)

7.2 Liquid Effluents		
	Mode	Initiating/Condition
GENERAL	All	Not Applicable
	All	Not Applicable
SITE	All	Not Applicable
	All	<p>Any UNPLANNED release of Liquid Radioactivity that exceeds 200 times the ODCM Limit for >15 minutes (1 or 2)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under Alert in Table 7-1 for >15 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Sample results exceed 200 times the ODCM limit value for an unmonitored release of liquid radioactivity >15 minutes in duration
ALERT	All	<p>Any UNPLANNED release of Liquid Radioactivity that exceeds 200 times the ODCM Limit for >15 minutes (1 or 2)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under Alert in Table 7-1 for >15 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Sample results exceed 200 times the ODCM limit value for an unmonitored release of liquid radioactivity >15 minutes in duration
	All	<p>Any UNPLANNED release of Liquid Radioactivity to the Environment that exceeds 2 times the ODCM Limit for >60 minutes (1 or 2)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under UE in Table 7-1 for >60 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Sample results exceed 2 times the ODCM limit value for an unmonitored release of liquid radioactivity >60 minutes in duration
UNUSUAL EVENT	All	<p>Any UNPLANNED release of Liquid Radioactivity that exceeds 200 times the ODCM Limit for >15 minutes (1 or 2)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under Alert in Table 7-1 for >15 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Sample results exceed 200 times the ODCM limit value for an unmonitored release of liquid radioactivity >15 minutes in duration
	All	<p>Any UNPLANNED release of Liquid Radioactivity to the Environment that exceeds 2 times the ODCM Limit for >60 minutes (1 or 2)</p> <ol style="list-style-type: none"> 1. A VALID rad monitor reading exceeds the values under UE in Table 7-1 for >60 minutes, unless assessment within this time period confirms that the Criterion is Not exceeded 2. Sample results exceed 2 times the ODCM limit value for an unmonitored release of liquid radioactivity >60 minutes in duration

**TABLE 7-1
EFFLUENT RADIATION MONITOR EALS⁽¹⁾**

NOTE: The values below, if exceeded, indicate the need to perform the specified assessment. If the assessment can not be completed within 15 minutes (60 minutes for UE), the declaration shall be made based on the **VALID** reading. As used here, the radiation monitor indications as displayed on ICS are the primary indicators. If ICS is unavailable, utilize the radiation monitor readings in the control room or local indication as necessary.

Monitor	ICS Screen	Units	UE	Alert	Site	General
Total Site	EFF1	μCi/s ⁽²⁾	1.5E+05	1.5E+07	2.5E+08	2.5E+09
U1 Shield Building 1-RE-90-400	EFF1	μCi/s	6.7E+04	6.7E+06	1.0E+08	1.0E+09
U2 Shield Building 2-RE-90-400	EFF1	μCi/s	1.5E+04	1.5E+06	2.5E+07	2.6E+08
Auxiliary Building 0-RE-90-101B	4RM1	cpm	1.2E+04	1.2E+06	***** ⁽¹⁾	***** ⁽¹⁾
Service Building 0-RE-90-132B	4RM1	cpm	4.3E+03	4.3E+05	9.8E+06	***** ⁽¹⁾
U1 Condenser Vacuum Exhaust 1-RE-90-404A	3PAM	μCi/cc ⁽³⁾	5.5E-02	5.5E+00	8.83E+01	8.83E+02
1-RE-90-404B	3PAM	μCi/cc	5.5E-02	5.5E+00	8.83E+01	8.83E+02
S/G Discharge Monitors 1-RE-90-421 thru 424 (B)	4RM2	mR/hr ⁽⁴⁾	NA	3.5E+02	3.5E+03	3.5E+04
Liquid Monitors 0-RE-90-122	n/a 4RM2	μCi/ml ⁽²⁾ cpm	1.8E-05 1.1E+06	1.8E-03 ***** ⁽¹⁾	N/A N/A	N/A N/A
1-RE-90-120,121	4RM2	cpm	1.0E+06	***** ⁽¹⁾	N/A	N/A
0-RE-90-225	4RM2	cpm	9.2E+05	***** ⁽¹⁾	N/A	N/A
0-RE-90-212	4RM2	cpm	1.5E+04	1.5E+06	N/A	N/A
RELEASE DURATION	minutes		60	15	15	15
ASSESSMENT METHOD: ICS or radiation monitor (RM) readings in the MCR or local indication as necessary						

Note: (1) Table values are calculated values. The ***** indicates the monitor is off scale.

- (2) These releases rate values in μCi/s and μCi/ml are provided on the gaseous and liquid release points for Information Only. Actual monitor readings are given in the table corresponding to the monitor for the four emergency classifications
- (3) This eberline channel reads out in cpm in the MCR. Indications of a radioactivity release via this pathway would be S/G blowdown monitors or other indications of primary-to-secondary leakage such as S/G level increase or pressurizer level decrease. ICS calculates μCi/cc and has a visual indication of an alarm condition when the indications exceeds 5.5E-02μCi/cc. This channel was included in the table to provide a means to further assess a release detected by other indications and to provide a path for possible escalation.
- (4) These unit values are based on flow rates through one [1] PORV of 970,000 lb/hr at 1,185 psig, 600°F. Before using these values, ensure a release to the environment is ongoing (e.g. PORV).

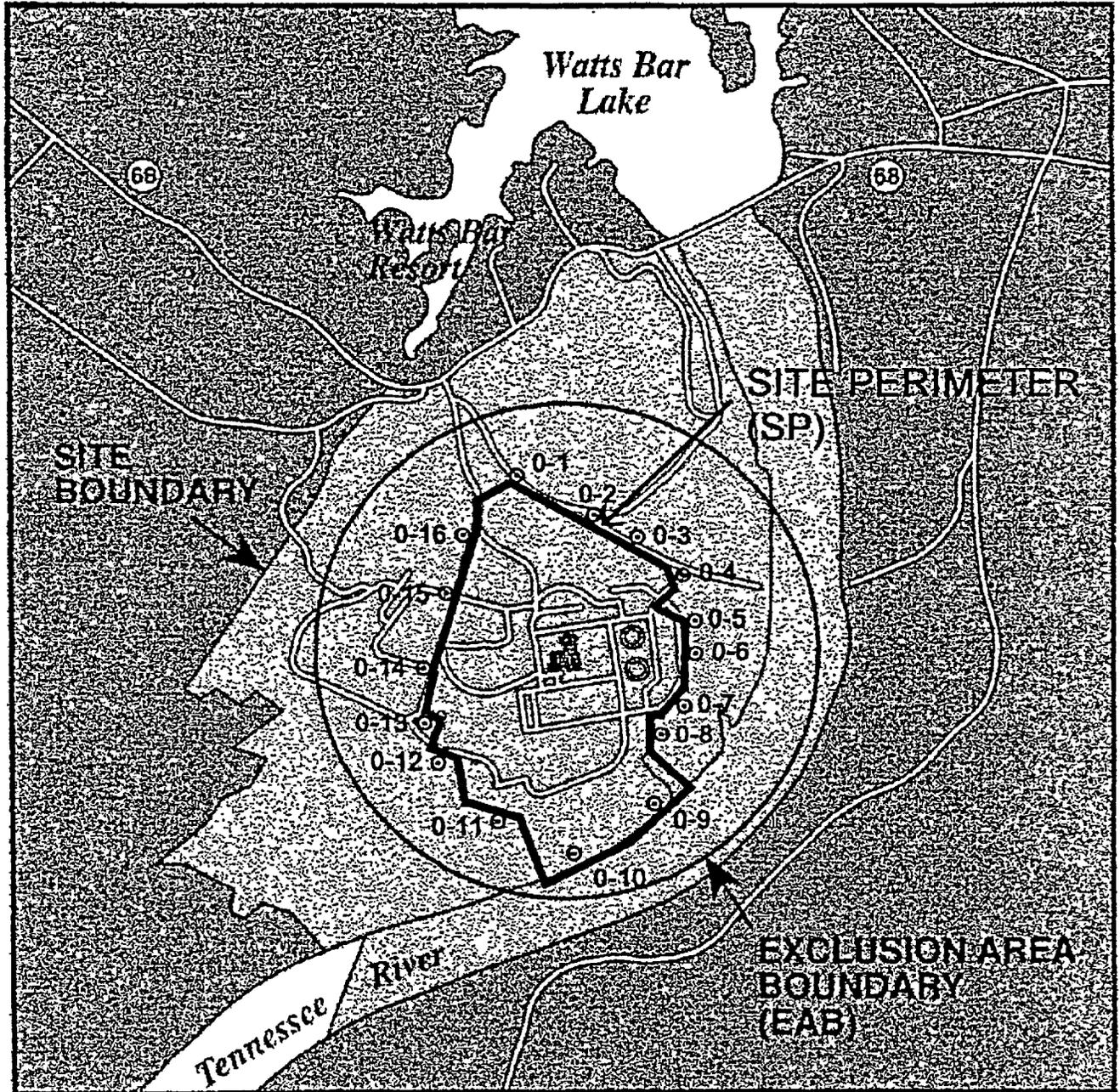
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Figure 7-A
EXCLUSION AREA, SITE BOUNDARY and SITE PERIMETER

NOTE: The Site Boundary used here is consistent with the definition in the Offsite Dose Calculation Manual. Do Not confuse this boundary with the SITE PERIMETER defined in these EALs, or with other definitions of "Site Boundary."



Note: Numbered points are [SP] radiological survey point for all sectors.

7.3 Radiation Levels	
Mode	Initiating/Condition
GENERAL	Refer to "Fission Product Barrier Matrix" or "Gaseous Effluents" (7.1)
	Refer to "Fission Product Barrier Matrix" or "Gaseous Effluents" (7.1)
SITE	Refer to "Fission Product Barrier Matrix" or "Gaseous Effluents" (7.1)
	Refer to "Fission Product Barrier Matrix" or "Gaseous Effluents" (7.1)
ALERT	<p>UNPLANNED increases in Radiation levels within the Facility that impedes Safe Operations or establishment or maintenance of Cold Shutdown (1 or 2)</p> <p>All</p> <ol style="list-style-type: none"> VALID area Radiation Monitor readings or survey results exceed 15 mrem/hr in the Control Room or CAS (a and b) <ol style="list-style-type: none"> VALID area radiation monitor readings exceed values listed in Table 7-2 Access restrictions impede operation of systems necessary for Safe Operation or the ability to establish Cold Shutdown <p>See UNUSUAL EVENT Note Below</p>
	<p>UNPLANNED increase in Radiation levels within the Facility</p> <p>All</p> <ol style="list-style-type: none"> VALID area Radiation Monitor readings increase by a factor 1000 over normal levels <p>Note: In Either the UE or ALERT EAL, the SED must determine the cause of Increase in Radiation Levels and Review Other INITIATING/CONDITIONS for Applicability (e.g., a dose rate of 15 mrem/hr in the Control Room could be caused by a release associated with a DBA).</p>
UNUSUAL EVENT	UNPLANNED increase in Radiation levels within the Facility
	UNPLANNED increase in Radiation levels within the Facility

7.4 Fuel Handling	
Mode	Initiating/Condition
GENERAL	Refer to "Gaseous Effluents" (7.1)
	Refer to "Gaseous Effluents" (7.1)
SITE	Refer to "Gaseous Effluents" (7.1)
	Refer to "Gaseous Effluents" (7.1)
ALERT	<p>Major damage to Irradiated Fuel, or Loss of water level that has or will uncover Irradiated Fuel outside the Reactor Vessel (1 and 2)</p> <p>All</p> <ol style="list-style-type: none"> VALID alarm on 0-RE-90-101 or 0-RE-90-102 or 0-RE-90-103 or 1-RE-90-130/131 or 1-RE-90-112 or 1-RE-90-400 or 2-RE-90-400 (a or b) <ol style="list-style-type: none"> Plant personnel report damage of Irradiated Fuel sufficient to rupture Fuel Rods Plant personnel report water level drop has or will exceed makeup capacity such that Irradiated Fuel will be uncovered
	<p>UNPLANNED loss of water level in Spent Fuel Pool or Reactor Cavity or Transfer Canal with fuel remaining covered (1 and 2 and 3)</p> <p>All</p> <ol style="list-style-type: none"> Plant personnel report water level drop in Spent Fuel Pool, or Reactor Cavity, or Transfer Canal VALID alarm on 0-RE-90-102 or 0-RE-90-103 or 1-RE-90-59 or 1-RE-90-60 Fuel remains covered with water.
UNUSUAL EVENT	UNPLANNED loss of water level in Spent Fuel Pool or Reactor Cavity or Transfer Canal with fuel remaining covered (1 and 2 and 3)
	UNPLANNED loss of water level in Spent Fuel Pool or Reactor Cavity or Transfer Canal with fuel remaining covered (1 and 2 and 3)

Table 7-2

ALERT - RADIATION LEVELS

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Monitor No.	Location Building and Elevation	Monitor Reading *
1&2 RE-90-1	Auxiliary El. 757.0 (spent fuel pool)	2.5 x 10 ³ mR/hr
1-RE-90-2	Auxiliary El. 757.0 (personnel air lock)	2.5 x 10 ⁰ R/hr
0-RE-90-3	Auxiliary El. 729.0 (waste pac. area)	2.5 x 10 ³ mR/hr
0-RE-90-4	Auxiliary El. 713.0 (decon room)	1.5 x 10 ³ mR/hr
0-RE-90-5	Auxiliary El. 737.0 (spt. fuel pool pmp. ar.)	1.5 x 10 ³ mR/hr
1&2-RE-90-6	Auxiliary El. 737.0 (comp. cl. wtr. ht. ex. ar.)	1.5 x 10 ³ mR/hr
1&2-RE-90-7	Auxiliary El. 713.0 (sample room)	2 x 10 ³ mR/hr
1&2-RE-90-8	Auxiliary El. 713.0 (aux feed pump area)	1.5 x 10 ³ mR/hr
0-RE-90-9	Auxiliary El. 692.0 (wst. cond. evap tk. ar.)	1.5 x 10 ³ mR/hr
1&2-RE-90-10	Auxiliary El. 692.0 (cvcs area)	1.5 x 10 ³ mR/hr
0-RE-90-11	Auxiliary El. 676.0 (ctmt. spry. & rhr pmp ar.)	1.5 x 10 ³ mR/hr
1-RE-90-61	Auxiliary El. 736.0 (RB low. cmpt. inst. rm.)	2.5 x 10 ³ mR/hr
0-RE-90-230	Turbine El. 685.0 (conden. demin.)	1.5 x 10 ³ mR/hr
0-RE-90-231	Turbine El. 685.0 (conden demin.)	1.5 x 10 ³ mR/hr

Note: *These monitors read out in mR/hr. It is assumed that this is equivalent to mrem/hr.

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SOURCE NOTES

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|---|--|
| <p>1. NIR-0551, DV-847100 F00012, and MC-850321 809004, MSC-00956, NCO 920030366.</p> | <p>Monitor readings and challenges to barriers are provided in EPIP-1, Section 1 in (1.1 Fuel Clad 1.1.5 and 1.3 CNTMT Barrier 1.3.5), Section 7 (7.1 Gaseous Effluents, 7.2 Liquid Effluents, Table 7-1, 7.3 Radiation Levels, 7.4 Fuel Handling and Table 7-2). Barriers are covered in Section 1, Fission Product Barrier Matrix. Monitor readings are also provided in EPIP-5, App. B, Note 3.</p> |
| <p>2. MC-84 0827 005 035A, MCS-2400</p> | <p>SED duties that can not be delegated. Section 2.0 Responsibility.</p> |
| <p>3. MC-8407 1900 3003, MSC-00701, NCO-920030222 CNTMT</p> | <p>Rad Monitors used in conjunction with a plant parameter to determine emergency classifications. Monitor readings are included with plant parameters for the purposes of emergency classifications. Section 1, Fission Product Barrier Matrix (1.1 Fuel Clad, 1.2 RCS, 1.3 Containment), Section 7 (7.1 Gaseous Effluent, 7.2 Liquid Effluent and 7.3 Radiation Levels and 7.4 Fuel Handling).</p> |
| <p>4. ANSI Standard N.18.7-1976 Subsection 5.3.9.3: 01 POI</p> | <p>EIPs will contain the following elements.</p> |
| <p>5. MSC-02401, NCO-920030998</p> | <p>Chemistry detection of failed fuel.</p> |
| <p>6. EPPOS #2</p> | <p>Emergency Preparedness Position (EPPOS) on timeliness of classification of emergency conditions.</p> |