J. Barnie Beasley, Jr., P.E. Vice President

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March 21, 2003

Docket Nos.: 50-348

50-364

NL-03-0372

Energy to Serve Your World sm

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D. C. 20555-0001

Joseph M. Farley Nuclear Plant, Unit 1 and Unit 2
Technical Specifications Revision Request
Removal of Reference to Plant Operations Review Committee

### Ladies and Gentlemen:

In accordance with 10 CFR 50.90, Southern Nuclear Operating Company (SNC) is proposing a change to the Joseph M. Farley Nuclear Plant (FNP) Unit 1 and Unit 2 Technical Specifications (TS). The proposed change will revise TS section 5.5.1 "Offsite Dose Calculation Manual (ODCM)" to remove reference to the Plant Operations Review Committee (PORC) review and acceptance of licensee initiated changes to the ODCM.

During conversion of FNP TS to the Improved Standard Technical Specifications (ISTS), this item was overlooked. PORC, now known as the Plant Review Board (PRB), is defined in the Farley Updated Final Safety Analysis Report (UFSAR) in chapter 17 as part of the FNP Quality Assurance (QA) Program. The PORC/PRB is responsible for reviewing all changes to the ODCM. Changes to the UFSAR chapter 17 are governed by 10 CFR 50.59 and 10 CFR 50.54(a). This change only deletes a reference to the PORC from the TS. There is no change to the function, makeup or scope of responsibility for the PORC introduced by this change. This reference to PORC/PRB in the FNP TS was overlooked during conversion to the ISTS and is being removed to enhance consistency with the generic ISTS and SNC sister plants, Hatch and Vogtle.

Enclosure 1 gives a description of the proposed change. Enclosure 2 provides information supporting a finding of no significant hazards consideration. Enclosure 3 includes the marked-up TS page for the proposed TS change. Enclosure 4 includes the TS page with the proposed change incorporated.

With respect to this proposed change, there is no significant change in the types or significant increase in the amounts of any effluents that may be released offsite and there is no significant increase in individual or cumulative occupational radiation exposure. Consequently, the proposed amendment satisfies the criteria of 10 CFR 51.22 for categorical exclusion from the requirements for an environmental assessment and the human environment is not affected by this amendment.

A009

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A copy of the proposed change has been sent to Dr. D. E. Williamson, the Alabama State Designee, in accordance with 10 CFR 50.91(b)(1).

Mr. J. B. Beasley, Jr. states he is a Vice President of Southern Nuclear Operating Company, is authorized to execute this oath on behalf of Southern Nuclear Operating Company and to the best of his knowledge and belief, the facts set forth in this letter are true.

Respectfully submitted,

SOUTHERN NUCLEAR OPERATING COMPANY

Sworn to and subscribed before me this 21 day of Mach, 2003.

My commission expires: 06/67/65

JBB/JLS/CHM/daj

**Enclosures:** 

Enclosure 1: Description of the Proposed Change Enclosure 2: No Significant Hazards Consideration Enclosure 3: Marked-Up Technical Specification Page Enclosure 4: Clean Typed Technical Specification Page U. S. Nuclear Regulatory Commission NL-03-0372 Page 3

cc: Southern Nuclear Operating Company

Mr. J. D. Woodard, Executive Vice President

Mr. D. E. Grissette, General Manager - Plant Farley

Document Services RTYPE: CFA04.054

U. S. Nuclear Regulatory Commission

Mr. L. A. Reyes, Regional Administrator

Mr. F. Rinaldi, NRR Project Manager - Farley

Mr. T. P. Johnson, Senior Resident Inspector - Farley

Alabama Department of Public Health

Dr. D. E. Williamson, State Health Officer

### **Enclosure 1**

Description of the Proposed Change

### Enclosure 1

### Description of the Proposed Change

### A. Summary of Proposed Change

In accordance with 10 CFR 50.90, Southern Nuclear Operating Company (SNC) is proposing a change to the Joseph M. Farley Nuclear Plant (FNP) Unit 1 and Unit 2 Technical Specifications (TS). The proposed change will revise TS section 5.5.1 "Offsite Dose Calculation Manual (ODCM)," to remove reference to the Plant Operations Review Committee (PORC) review and acceptance of licensee initiated changes to the ODCM.

### B. Description of Proposed Change

The proposed change revises TS 5.5.1.b as follows.

- TS 5.5.1.b currently states, "Shall become effective after review and acceptance by the PORC and the approval of the General Manager - Nuclear Plant; and"
- TS 5.5.1.b will be revised to state, "Shall become effective after the approval of the General Manager Nuclear Plant; and"

### C. Bases for Proposed Change

During conversion of FNP TS to the Improved Standard Technical Specifications (ISTS), this item was overlooked. PORC, also known as the Plant Review Board (PRB), is defined in the Farley Updated Final Safety Analysis Report (UFSAR) in chapter 17 as part of the FNP Quality Assurance (QA) Program. The PORC/PRB is responsible for reviewing all changes to the ODCM. Changes to the UFSAR chapter 17 are governed by 10 CFR 50.59 and 10 CFR 50.54(a). This change only deletes a reference to the PORC from the TS. There is no change to the function, makeup or scope of responsibility for the PORC introduced by this change. This reference to PORC/PRB in the FNP TS was overlooked during conversion to the ISTS and is being removed to enhance consistency with the generic ISTS and SNC sister plants, Hatch and Vogtle.

### D. Conclusion

This change to TS does not reduce the review and recommendation responsibilities of the PORC/PRB. All review and recommendation functions, including that for the ODCM, will continue to be addressed as required by the UFSAR. This requested change is concluded to be acceptable.

### **Enclosure 2**

No Significant Hazards Consideration

### **Enclosure 2**

### No Significant Hazards Consideration

According to 10 CFR 50.92(c), "Issuance of amendment," a proposed amendment to an operating license involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

Southern Nuclear Operating Company (SNC) is proposing a change to the Joseph M. Farley Nuclear Plant (FNP) Unit 1 and Unit 2 Technical Specifications (TS). The proposed change will revise TS section 5.5.1 "Offsite Dose Calculation Manual (ODCM)" to remove reference to the Plant Operations Review Committee (PORC) review and acceptance of licensee initiated changes to the ODCM.

During conversion of FNP TS to the Improved Standard Technical Specifications (ISTS), this item was overlooked. PORC, now known as the Plant Review Board (PRB), is defined in the Farley Updated Final Safety Analysis Report (UFSAR) in chapter 17 as part of the FNP Quality Assurance (QA) Program. The PORC/PRB is responsible for reviewing all changes to the ODCM. Changes to the UFSAR chapter 17 are governed by 10 CFR 50.59 and 10 CFR 50.54(a). This change only deletes a reference to the PORC from the TS. There is no change to the function, makeup or scope of responsibility for the PORC introduced by this change. This reference to PORC/PRB in the FNP TS was overlooked during conversion to the ISTS and is being removed to enhance consistency with the generic ISTS and SNC sister plants, Hatch and Vogtle.

Information supporting the determination that the criteria set forth in 10 CFR 50.92 are met for this amendment request is indicated below.

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

The proposed change for TS section 5.5.1.b removes the reference to the Plant Operations Review Committee review and acceptance of licensee initiated changes to the ODCM. This change is an administrative change and does not change plant design or responses.

The proposed change does not involve changing any structure, system, or component, or affect reactor operations. It is not an initiator of an accident and does not change any existing safety analysis previously analyzed in the UFSAR. As such, the proposed change does not involve a significant increase in the probability of an accident previously evaluated. Since the proposed change does not alter the plant design, it does not involve a significant increase in the consequences of an accident previously evaluated.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

### 2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

The proposed change for TS section 5.5.1.b removes the reference to the Plant Operations Review Committee review and acceptance of licensee initiated changes to the ODCM. This change is an administrative change and does not change plant design or responses.

The proposed change will not alter any plant design basis or postulated accident. In addition, the proposed change does not impact any plant systems or components.

Therefore, the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

### 3. Does the proposed change involve a significant reduction in a margin of safety?

The proposed change for TS section 5.5.1.b removes the reference to the Plant Operations Review Committee review and acceptance of licensee initiated changes to the ODCM. This change is an administrative change and does not change plant design or responses. The proposed change does not impact accident offsite dose, containment pressure or temperature, emergency core cooling system setpoints, reactor protection system settings or any other parameter that could affect a margin of safety.

Therefore, the proposed change does not involve a significant reduction in a margin of safety.

### **Enclosure 3**

Marked-Up Technical Specification Page

### 5.0 ADMINISTRATIVE CONTROLS

### 5.5 Programs and Manuals

The following programs shall be established, implemented, and maintained.

### 5.5.1 Offsite Dose Calculation Manual (ODCM)

- a. The ODCM shall contain the methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm and trip setpoints, and in the conduct of the radiological environmental monitoring program; and
- b. The ODCM shall also contain the radioactive effluent controls and radiological environmental monitoring activities, and descriptions of the information that should be included in the Annual Radiological Environmental Operating, and Radioactive Effluent Release Reports required by Specification 5.6.2 and Specification 5.6.3.

Licensee initiated changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained.

  This documentation shall contain:
  - 1. sufficient information to support the change(s) together with the appropriate analyses or evaluations justifying the change(s), and
  - a determination that the change(s) maintain the levels of radioactive effluent control required by 10 CFR 20.1302, 40 CFR 190, 10 CFR 50.36a, and 10 CFR 50, Appendix I, and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations;
- b. Shall become effective after review and acceptance by the PORS and the approval of the General Manager Nuclear Plant; and
- c. Shall be submitted to the NRC in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.

### Enclosure 4

Clean Typed Technical Specification Page

List of Affected Pages

5.5-1

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- b. The ODCM shall also contain the radioactive effluent controls and radiological environmental monitoring activities, and descriptions of the information that should be included in the Annual Radiological Environmental Operating, and Radioactive Effluent Release Reports required by Specification 5.6.2 and Specification 5.6.3.

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- b. Shall become effective after the approval of the General Manager Nuclear Plant; and
- c. Shall be submitted to the NRC in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.