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NLS2003041

March 18, 2003

U.S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, D.C. 20555-0001

Subject:

Emergency Plan Implementing Procedure

Cooper Nuclear Station, NRC Docket 50-298, DPR-46

Pursuant to the requirements of 10 CFR 50, Appendix E, Section V, "Implementing Procedures," Nebraska Public Power District is transmitting the following Emergency Plan Implementing Procedure (EPIP):

EPIP 5.7.17

Revision 30

"Dose Assessment"

Should you have any questions concerning this matter, please contact me at 402-825-5233.

Sincerely,

Ø. A. HuttonPlant Manager

/nr

Enclosure

cc: Regional Administrator w/enclosure (2)

USNRC - Region IV

NPG Distribution w/o enclosure

Senior Resident Inspector w/enclosure

USNRC

Records w/o enclosure

ATTACHARENT 3 LIST OF DECLIFATORY COMMARTMENTS	<u> </u>		
ATTACHMENTS EIST OF REGULATOR I COMMITMENTS	ATTACHMENT 3	LIST OF REGULATORY COMMITMENTS	

Correspondence Number: NLS2003041

The following table identifies those actions committed to by Nebraska Public Power District (NPPD) in this document. Any other actions discussed in the submittal represent intended or planned actions by NPPD. They are described for information only and are not regulatory commitments. Please notify the NL&S Manager at Cooper Nuclear Station of any questions regarding this document or any associated regulatory commitments.

COMMITMENT	COMMITTED DATE OR OUTAGE
COMMINICIAL	OR OUTAGE
None	

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CNS OPERATIONS MANUAL EPIP PROCEDURE 5.7.17

DOSE ASSESSMENT

OPERATOR A

USE: REFERENCE EFFECTIVE: 3/7/03 3

APPROVAL: SORC/IQA OWNER: J. A. BEDNAR DEPARTMENT: EP

1.	PUR	$ exttt{POSE} \dots$			
2.			ID LIMITATIONS 2		
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4.	COM	PUTER DOSE	PROJECTION (CNS-DOSE)		
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	ATT	ACHMENT 1	HAND-CALCULATED DOSE PROJECTION (NON-		
			CENTERLINE)		
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			FOR CNS-DOSE		
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	ATTA	ACHMENT 8	INFORMATION SHEET30		
		~ ~			
_		D0 GD			
1.	PUR.	POSE			
[]	1.1	This procedur	re provides instructions for performing a dose projection using the		
ГЛ	1.1	-	Computer Program.		
		ONE BOOK C	ompator rogram.		
[]	1.2	This procedur	e provides a manual backup method for performing dose		
. ,		assessment.	F		
[]	1.3	This procedur	re provides instructions for making a rapid gross estimation of		
- -		core damage l	based on in-containment high range radiation monitor readings		

for primary containment LOCA events.

[]	1.4	This procedure provides instructions for obtaining meteorological data from alternate sources if the primary sources are not available. The general order of preference will be PMIS, National Weather Service, and then the use of historically determined default values.					
2.	PREC	CAUTION	NS AND LII	MITATIONS			
[]	2.1	Actual d	lose rates w	rill vary as a function of:			
	[]	2.1.1	The total o	curies released.			
	[]	2.1.2	Release ra	te.			
	[]	2.1.3	The durati	ion of the release.			
	[]	2.1.4	The isotop	ic mixture of the release.			
	[]	2.1.5	Meteorolog	gical conditions.			
[]	2.2	-	and refine ove paramete	dose calculations upon significant changes in one or more of ers.			
[]	2.3	Should a release occur which necessitates rapid decision making concerning the recommendation of protective actions, the guidance contained in Procedure 5.7.20 should be followed.					
[]	2.4	Attachment 7 should be used to estimate core damage only in cases where the high range in-containment radiation monitors are exposed to coolant or steam (i.e., only for primary containment LOCA situations). For other accident sequences, a Reactor Coolant System (RCS) sample and Core Damage Assessment Program (CORDAM) must be used. The Post-Accident Sampling System (PASS) may be used, as required, to obtain the RCS sample.					
[]	2.5			AN monitor(s) is (are) inoperable, Release Rate all be performed using Procedure 5.7.16.			
3.	REQU	JIREME	NTS				
[]	3.1	Ensure	following ed	quipment and materials are available, as needed:			
	[]	3.1.1	COMPUT	ERIZED DOSE PROJECTION (CNS-DOSE)			
		[]	3.1.1.1	Computer terminals.			
		[]	3.1.1.2	Computer printers.			

		3.1.2	MANUAL	LY CALCULATED DOSE PROJECTION
		[]	3.1.2.1	Environs map.
		[]	3.1.2.2	χ/Q isopleths (off-centerline only).
		[]	3.1.2.3	Scientific calculator.
[]	3.2	A releas	se of airbori	ne radioactive material has or may occur.
[]	asses	sment is	unavailable	logical or Radiological data needed to perform dose e or "unhealthy", refer to Attachment 5 for alternate source: codes" are defined in Attachment 6.
[]				ot familiar with the use of PMIS, Attachment 6 provides an s on access and selected use of PMIS.
4.	COM	PUTER I	DOSE PRO	JECTION (CNS-DOSE)
[]	4.1		_	rojection program on a PMIS terminal, enter the turn-on terminal logged into either the Primary or Backup System.
[]	4.2	However compute non-PM privileg	er, this is re er and fami IIS termina es to run Pl	n program can also be run on a non-PMIS terminal. served for personnel having access to an account on the cliar with its use. To start the dose projection program on a l, on either PMIS computer, login to an account that has MIS software and run program [NPDOSEZ.
[]	4.3	data wi the orig Health meteoro	ll be loaded in of the re "quality coo ological and	gram is started or the "New Sample" option is selected, new into the program. Verify that Field 1 correctly indicates lease and the data displayed is "healthy" and correct. des" are defined in Attachment 6. Alternate sources of radiological data needed to run CNS-DOSE or perform a re found in Attachment 5.
[]	4.4		gical, Opera	is in the effluent stream and if it is functional. Consult with ations, and Engineering personnel for this determination, if
[]	4.5	for this	time estima	tion of release (consult with Operations and/or Engineering ate) in hours. If the estimated duration of release cannot be a 4 hour default value.

l J	degra calcul condi condi	ded ques lations. ' tions are tion thre een Radio	tion and has a significant impact on the resultant dose projection. The core is considered to be degraded if any of the following listed met <u>OR</u> if they were met and have subsequently dropped below the shold. The answer to the core degraded question is coordinated blogical Protection, Chemistry, Operations, and Engineering, if		
[]	4.6		ine if the core is degraded (fuel cladding loss) as indicated by any of the g conditions:		
	[]	4.6.1	15,000 mrem/hr on SJAE monitor.		
	[]	4.6.2	Reactor Coolant Sample > 300 μ Ci/gm Dose Equivalent I-131.		
	[]	4.6.3	LOCA with DW Rad Monitor reading > 2500 REM/hr.		
	[]	4.6.4	Non-LOCA with DW Rad Monitor reading > 115 REM/hr.		
	[]	4.6.5	Main Steam Line Radiation Monitor Readings ≥ 1200 mrem/hr.		
	[]	4.6.6	Reactor water level below 0" FZ (Fuel Zone).		
[]	4.7	DETER	MINE IF RELEASE PATHWAY IS THROUGH REACTOR BUILDING		
	[]	4.7.1	If release bypasses Reactor Building (i.e., direct venting of drywell or a release from the Turbine Building), then enter N.		
	[]	4.7.2	If release is through Reactor Building, then enter Y.		
[]	4.8	Make co	orrections or changes, as necessary.		
.[]	4.9	Use the	ENTER key to accept data and move to the next field.		
[]	4.10	Press th	ne RESULTS option to display the dose projections.		
[]	4.11	Select e results.	ither the PRINT or HARD COPY option to make a hard copy of the		
[]	4.12	Select the "New Sample" or "Edit" option to return to the previous display and obtain new data or make additional changes.			
[]	4.13	Exit the termina	e program by entering "Q" or pressing the "CANC" key on PMIS als.		
[]	4.14	Select t	he "Help" option for additional program operational information.		

[]	<u>NOTE</u> - This method reflects the methodology used in the CNS-DOSE Program. It gives only downwind dose values for plume centerline at distances of 1, 2, 5, and 10 miles from the site. For calculating doses at specific receptor locations, the method in Section 6 is used.							
[]	5.1 Obtain release rate from effluent KAMAN monitor digital readout in μ Ci/sec and record value in Block 1 on Attachment 3. If KAMAN is inoperable, complete the appropriate attachment of Procedure 5.7.16 and record the noble gas release rate value (μ Ci/sec) in Block 1 on Attachment 3.							
[]	<u>NOTE</u> - The answer to the question concerning the status of the Standby Gas Treatment System has a significant impact on the resultant dose projection calculation. The answer to this question is coordinated with Radiological, Operations, and Engineering personnel, if available.©							
[]	5.2	Determ	ine if SGT is in the effluent stream.					
	[]	5.2.1	If SGT \underline{is} in the effluent stream, enter 0.01 in Block 2 of Attachment 3.					
	[]	5.2.2	If SGT <u>is not</u> in the effluent stream, enter 1 in Block 2 of Attachment 3.					
[]	degra calcu condi condi	ided questlations. tions are tion threen Radi	dodine to Noble Gas ratio is very dependent on the answer to the core stion and has a significant impact on the resultant dose projection. The core is considered to be degraded if any of the following listed a met <u>OR</u> if they were met and have subsequently dropped below the shold. The answer to the core degraded question is coordinated cological Protection, Chemistry, Operations, and Engineering, if					
[]	5.3		ine if the core is degraded (fuel cladding loss) as indicated by any of the ag conditions:					
	[]	5.3.1	15,000 mrem/hr on SJAE monitor.					
	[]	5.3.2	Reactor Coolant Sample > 300 μCi/gm Dose Equivalent I-131.					
	[]	5.3.3	LOCA with DW Rad Monitor reading > 2500 REM/hr.					
	[]	5.3.4	Non-LOCA with DW Rad Monitor reading > 115 REM/hr.					
	[]	5.3.5	Main Steam Line Radiation Monitor Readings ≥ 1200 mrem/hr.					
	[]	5.3.6	Reactor water level below 0" FZ (Fuel Zone).					
	P.	ROCEDI	URE 5.7.17 REVISION 30 PAGE 5 OF 32					

HAND-CALCULATED DOSE PROJECTION (CENTERLINE)

- [] 5.3.7 If core <u>is</u> degraded, obtain the Iodine to Noble Gas ratio from Table 1 of Attachment 3 and enter that value in Block 3 of Attachment 3.
- [] 5.3.8 If core is not degraded, enter 1.86E-7 in Block 3 of Attachment 3.
- [] 5.4 Obtain the Noble Gas energy factor (MeV/dis) based on time since reactor shutdown in hours from Table 2 on Attachment 3 and enter this value in Block 4 on Attachment 3.
- [] 5.5 Obtain the wind speed in miles per hour (mph) from PMIS and record the value in Block 5 of Attachment 3. If wind speed is not available from PMIS, call the National Weather Service (NWS) in Valley, NE and request an estimate of wind speed at CNS for the appropriate elevation. The telephone number for the NWS may be found in the Emergency Telephone Directory Federal TAB.
 - [] 5.5.1 If the release is from the ERP, use wind speed at the 100 meter level. If 100 meter data is unavailable, use the 60 meter data. If wind speed is unavailable from PMIS, and the NWS cannot be contacted, then use the historical default wind speed value of 13 mph.
 - [] 5.5.2 If the release is from any other source, use the wind speed at the 10 meter level. Either MET tower 10 meter level is acceptable. If 10 meter data is unavailable, use the 60 meter data. If wind speed is unavailable from PMIS, and the NWS cannot be contacted, then use the historical default wind speed value of 8 mph.
- Determine the atmospheric stability class ("A" through "G") from PMIS and record in Block 6 on Attachment 3. If the stability class cannot be obtained from PMIS and the National Weather Service cannot be contacted, use "D" as the default stability class.
 - [] 5.6.1 If using temperatures from the NWS to develop delta-T-based stability class, request the temperatures (10 meter (M) and 100 M) in degrees Centigrade. Determine degrees Centigrade (C) delta-T and the appropriate stability class using the following formula and table:

 $100 \text{ M} ^{\circ}\text{C} - 10 \text{ M} ^{\circ}\text{C} = \text{delta-T} ^{\circ}\text{C}$

delta-T °C	< -1.7	-1.7 to -1.5	-1.5 to -1.3	-1.3 to -0.45	-0.45 to 1.3	1.3 to 3.6	> 3.6
Stability Class	A	В	С	D	E	F	G

	[]	5.7	DETERI	MINE IF RELEASE PATHWAY IS THROUGH REACTOR BUILDING
		[]	5.7.1	If release bypasses Reactor Building (for example, direct venting of drywell or a release from the Turbine Building), then enter 1 in Block 7 on Attachment 3.
]		[]	5.7.2	If release is through the Reactor Building, then enter 0.5 in Block 7 on Attachment 3.
	[]	5.8		TEDE Noble Gas Dose Conversion Factor from Table 3 of Attachment 3 ord in Block 8 on Attachment 3.
	[]	5.9		TEDE Iodine Dose Conversion Factor from Table 3 of Attachment 3 ord in Block 9 on Attachment 3.
	[]	5.10		CDE Iodine Dose Conversion Factor from Table 3 of Attachment 3 and Block 10 on Attachment 3.
	[]	5.11	Compute	e TEDE "sub-calculation" value and record in Block 11 of ent 3.
			[(Block 1	(Block 4)(Block 8)] + [(Block 1)(Block 2)(Block 3)(Block 7)(Block 9)] (Block 5)
	[]	5.12	(Block 6)	ne appropriate release point (ERP or other) and stability class), obtain the mixing factors (χ/Qs) for distances 1, 2, 5, and 10 miles to ble 4 on Attachment 3 and record in Block 12 of Attachment 3.
	[]	5.13	-	e the TEDE dose rate for each distance and record values in Block 13 hment 3.
			(Block 1	1) x (Block 12)
	[]	5.14	Enginee: Attachm	e the duration of the release (consult with Operations and/or ring for this time estimate) in hours and record value in Block 14 on tent 3. If the estimated duration of release cannot be determined, use as a default value.
	[]	5.15	-	e integrated TEDE doses for each distance and record values in 5 on Attachment 3.
			(Block 13	3) x (Block 14)

[]	5.16	Compute CDE "sub-calculation" value and record in Block 16 of Attachment 3.					
		(Block 1)(Block 2)(Block 3)(Block 7)(Block 10) (Block 5)					
[]	5.17	Compute the CDE dose rate for each distance and record values in Block 17 on Attachment 3 .					
		(Block 16) x (Block 12)					
[]	5.18	Compute the CDE dose for each distance and record values in Block 18 on Attachment 3.					
		(Block 17) x (Block 14)					
[]	5.19	Refer to Procedure 5.7.1 to determine if an emergency should be declared due to radiological effluent (dose rate or integrated dose to a member of the public) calculated at or beyond 1 mile.					
[]	5.20	Refer to Procedure 5.7.20 to determine if any protective action recommendations should be made to off-site authorities.					
[]	5.21	Recalculate dose projections whenever conditions change significantly.					
[]	5.22	Record name, time, and date at the bottom of Attachment 3.					
6.	HAN	O-CALCULATED DOSE PROJECTION (NON-CENTERLINE)					
[]	6.1	Obtain release rate from effluent KAMAN monitor digital readout in μ Ci/sec and record value in Block 1 on Attachment 1. If KAMAN is inoperable, complete appropriate attachment of Procedure 5.7.16 and record the noble gas release rate value (μ Ci/sec) in Block 1 on Attachment 1.					
[]	Treat calcul	E - The answer to the question concerning the status of the Standby Gas ment System has a significant impact on the resultant dose projection ation. The answer to this question is coordinated with Radiological, ations, and Engineering personnel, if available.					
[]	6.2	Determine if SGT is in the effluent path.					
	[]	6.2.1 If SGT is in effluent path, enter 0.01 in Block 2 on Attachment 1.					
	[]	6.2.2 If SGT is not in effluent path, enter 1 in Block 2 on Attachment 1.					

[]	[] <u>NOTE</u> - The Iodine to Noble Gas ratio is very dependent on the answer to the cord degraded question and has a significant impact on the resultant dose projection calculations. The core is considered to be degraded if any of the following listed conditions are met <u>OR</u> if they were met and have subsequently dropped below the condition threshold. The answer to the core degraded question is coordinated between Radiological Protection, Chemistry, Operations, and Engineering, if available.						
[]	6.3		Determine if the core is degraded (fuel cladding loss) as indicated by any of the following conditions:				
	[]	6.3.1	15,000 mrem/hr on SJAE monitor.				
	[]	6.3.2	Reactor Coolant Sample > 300 μ Ci/gm Dose Equivalent I-131.				
	[]	6.3.3	LOCA with DW Rad Monitor reading > 2500 REM/hr.				
	[]	6.3.4	Non-LOCA with DW Rad Monitor reading > 115 REM/hr.				
	[]	6.3.5	Main Steam Line Radiation Monitor Readings ≥ 1200 mrem/hr.				
	[]	6.3.6	Reactor water level below 0" FZ (Fuel Zone).				
	[]	6.3.7	If core <u>is</u> degraded, obtain the Iodine to Noble Gas ratio from Table 1 of Attachment 1 and enter that value in Block 3 on Attachment 1.				
	[]	6.3.8	If core <u>is not</u> degraded, enter 1.86E-07 in Block 3 on Attachment 1.				
[]	6.4		ine the energy factor (MeV/dis) based on time since reactor shutdown in nd Table 2 on Attachment 1, and enter value in Block 4 on nent 1.				
[]	6.5	value in call the estimate	the wind speed in miles per hour (mph) from PMIS and record the Block 5 on Attachment 1. If wind speed is not available from PMIS, National Weather Service (NWS) in Valley, NE and request an e of wind speed at CNS for the appropriate elevation. The telephone for the NWS may be found in the Emergency Telephone Directory - TAB.				
	[]	6.5.1	If the release is from the ERP, use wind speed at the 100 meter level. If 100 meter data is unavailable, use the 60 meter data. If wind speed is unavailable from PMIS, and the NWS cannot be contacted, then use the historical default wind speed value of 13 mph.				
			_				

- [] 6.5.2 If the release is from any other source, use the wind speed at the 10 meter level. Either MET tower 10 meter level is acceptable. If 10 meter data is unavailable, use the 60 meter data. If wind speed is unavailable from PMIS, and the NWS cannot be contacted, then use the historical default wind speed value of 8 mph.
- [] 6.6 Determine the wind direction (from) in degrees from PMIS and record in Block 6 on Attachment 1. If wind direction is not available from PMIS, call the National Weather Service (NWS) in Valley, NE and request an estimate of wind direction at CNS for the appropriate elevation. The telephone number for the NWS may be found in the Emergency Telephone Directory Federal TAB.
- [] 6.7 Determine the atmospheric stability class ("A" through "G") from PMIS and record in Block 7 on Attachment 1. If the stability class cannot be obtained from the PMIS and the NWS cannot be contacted, use "D" as the default stability class.
 - [] 6.7.1 If using temperatures from the NWS to develop delta-T-based stability class, request the temperatures (10 meter (M) and 100 M) in degrees Centigrade. Determine degrees Centigrade (C) delta-T and the appropriate stability class using the following formula and table:

 $100 \text{ M} ^{\circ}\text{C} - 10 \text{ M} ^{\circ}\text{C} = \text{delta-T} ^{\circ}\text{C}$

delta-T°C	<-1.7	-1.7 to -1.5	-1.5 to -1.3	-1.3 to -0.45	-0.45 to 1.3	1.3 to 3.6	> 3.6
Stability Class	A	В	C	D	E	F	G

- [] 6.8 DETERMINE IF RELEASE PATHWAY IS THROUGH REACTOR BUILDING
- [] 6.8.1 If the release bypasses Reactor Building (for example direct venting of the drywell or a release from the Turbine Building), then enter 1 in Block 8 on Attachment 1.
- [] 6.8.2 If the release is through Reactor Building, then enter 0.5 in Block 8 on Attachment 1.
 - [] 6.9 Obtain TEDE Noble Gas Dose Conversion Factor from Table 3 of Attachment 1 and record in Block 9 on Attachment 1.
 - [] 6.10 Obtain TEDE Iodine Dose Conversion Factor from Table 3 of Attachment 1 and record in Block 10 on Attachment 1.
 - [] 6.11 Obtain CDE Iodine Dose Conversion Factor from Table 3 of Attachment 1 and record in Block 11 on Attachment 1.

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[]	6.12	Obtain t	the mixing factor (χ/Q) for the receptor point or location.
	[]	6.12.1	Record location or receptor point ID at the top of Attachment 1.
	[]	6.12.2	Obtain the proper χ/Q isopleth overlay based on stability class and release point.
		[]	6.12.2.1 Overlays are available in the TSC or EOF for both elevated and ground level releases for each stability class. Use ground level isopleths for all releases which are not from the ERP.
	[]	6.12.3	Place the isopleth overlay on an Emergency Planning Zone map scaled to 1" per mile. The preferred map is the "Cooper Nuclear Station 20 Mile Plume Exposure" map with sectors, radii, and wind direction labeled. One is posted in the TSC and EOF.
	[]	6.12.4	Orient the isopleth overlay so the centerline of the isopleth is over the wind direction radius, the open end of the isopleth is downwind, and the asterisk is over CNS.
	[]	6.12.5	Lightly mark the desired receptor location on the isopleth with a pencil.
	[]	NOTE -	- All χ/Qs have negative exponents.
	[]	6.12.6	Using the legend in the lower right hand corner of the isopleth overlay, linearly interpolating as necessary, determine a χ/Q value for the receptor site.
	[]	6.12.7	Record the χ/Q value in Block 12 on Attachment 1.
[]	6.13	Comput	e TEDE dose rate (REM/hr) and record in Block 13 on Attachment 1.
		[(Block 1)	(Block 4)(Block 9)]+[(Block 1)(Block 2)(Block 3)(Block 8)(Block 10)] x (Block 12) (Block 5)
[]	6.14	Enginee on Attac	the duration of the release (consult with Operations and/or ering for this time estimate) in hours and record the value in Block 14 chment 1. If the estimated duration of release cannot be determined, ours as a default value.
[]	6.15	Comput Attachm	e the integrated TEDE dose (REM) and record in Block 15 on nent 1.
		(Block 1	3) x (Block 14)

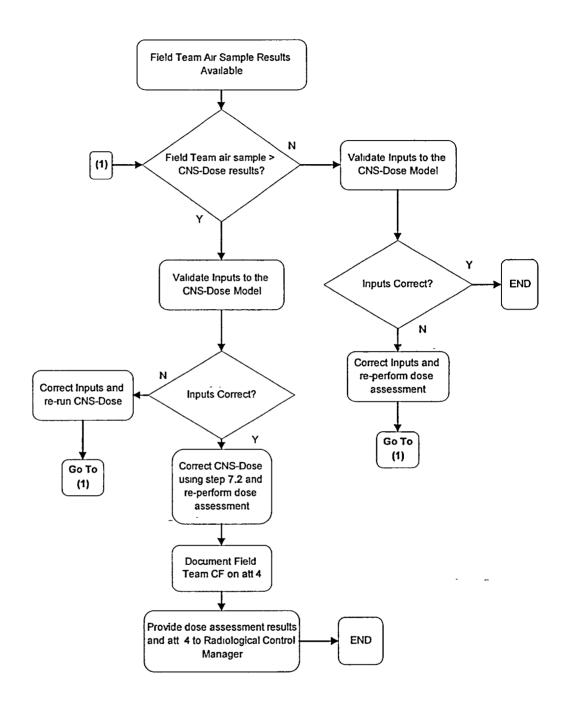
[

[

[

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[]	6.16	Comput	e CDE dose	rate (R	EM/hr) and record in B	lock 16	on Attachment 1.	
		(Block 1	<u>)(Block 2)(I</u> (I	Block 3)(Block 5)	(Block	8)(Block 11) x (B	lock 12	3)	
[]	6.17	Comput	e CDE dose	(REM)	REM) and record in Block 17 on Attachment 1.				
		(Block 1	4) x (Block	16)					
[]	6.18	Record	name, time,	and da	te at tl	ne bottom of Atta	chmen	t 1.	
7.	CORRELATING OFF-SITE SAMPLE RESULTS WITH DOSE PROJECTIONS©								
[]	<u>NOTE</u> 1 - This section describes the methodology to be used to correlate CNS-DOSE results (estimated gross iodine concentrations) with gross iodine concentrations sampled in the field.								
[]	NOTE 2 - This section is to be used by dose assessment personnel in the EOF once field teams have been dispatched and sample results become available.								
[]	assur	ned radio ured. Of	onuclide con f-site sampl	centrati e result	ions un s are u	til actual concen	tration a dose	ed) are based upon as have been correction factor	
[]	7.1	FIELD	TEAM SAM	IPLE TO	O CNS	DOSE COMPAR	RISON		
	[]	7.1.1	Radiologic	al Asses	ssment	Supervisor shall	l :		
		[]	that the ti	me of th	ne field	aring field team a team air sample sment are compa	and tl	nple results, ensure ne time of	
		[]	other than class/relea	n 1, 2, 5, ise point sample	or 10 t t isople results	miles, use the app th to determine v would be at that	propria what C	ed from a distance ate stability NS-DOSE predicted ace prior to performi	
		[]	7.1.1.1	with tl	he pred	field team iodine licted CNS-Dose : as using the decis	iodine	-	
		[]	7.1.1.2	correct	ted dos e in PA	_	ults ar	review the field team ad communicate any the Emergency	
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[] 7.2 APPLYING FIELD TEAM CORRECTION TO CNS-DOSE

- [] 7.2.1 Apply the correction to CNS-Dose using the "Field Adjust" OPTION of CNS-DOSE.
 - [] 7.2.1.1 At the MAIN CNS-DOSE screen, select option "Field Adjust".

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		[]	7.2.1.2	Enter the radius distance from CNS in miles at the prompt (1, 2, 5, and 10 are the only options).		
		[]	7.2.1.3	Enter the gross iodine concentration (in μ Ci/cc) obtained from the field at the prompt.		
		[]	7.2.1.4	After obtaining new Results from CNS-DOSE, compare new PARs to any PARs previously transmitted to off-site authorities.		
8.		E DAMAG ITORS	GE ESTIMA	ATE USING IN-CONTAINMENT HI-RANGE RADIATION		
[]	in-cor prima Coola must	ntainmen ary conta ant Systen be used.	t radiation inment LO(m (RCS) sai	s only used for core damage estimates where the monitors are exposed to coolant or steam (i.e., only for CA situations). For other accident sequences, a Reactor mple and Core Damage Assessment Program (CORDAM) Accident Sampling System (PASS) may be used, as required,		
[]	NOTE 2 - The release from the core may bypass the containment, be retained in the primary system, or not be uniformly mixed. Therefore, a low containment radiation reading does not guarantee a lack of core damage. The levels of damage indicated by the value in Attachment 7 are considered minimum levels unless there are inconsistent monitor readings.					
[]	<u>NOTE</u> 3 - Inconsistent monitor readings may be due to the uneven mixing in containment (e.g., steam rising to the top of the dome). It may take hours for uniform mixing.					
[]	8.1	determi	ne an estim	dinator or designee shall perform following steps to ate of core damage, if decisions must be made which are tions and PASS results are not available.		
	[]	8.1.1	from RMA	ghest in-containment hi-range radiation monitor reading A-RM-40A(B), DRYWELL RAD MONITOR, and record in Attachment 7.		
	[]	8.1.2	Complete	the calculations on Attachment 7.		
	[]	8.1.3	Report res	sults to the TSC Director.		

ATTACHMENT 1 HAND-CALCULATED DOSE PROJECTION (NON-CENTERLINE)

(1) Noble Gas Release Rate from KAMAN or 5.7.16 (μCi/Sec)	(2) Release Path through SBGT? Yes = 0 01; No = 1	(3) Iodine/Noble Gas Ratio (from Table 1)	(4) Energy Factor	(5) Wind Speed (mph) ERP = 13; Other = 8	(6) Wind Direction (° from)	(7) Stability Class Default = D	(8) Release throug Reactor Building? No = 1; Yes = 0.5
	1es - 0 01; No - 1	(Irom Table 1)	(Irom Table 2)	ERF = 15; Other = 8	(* from)	Default = D	
EDE Iodine (s (from Table 3) (10)	Mixing Factor (from Isopleths)					
	TEDE Dose Rate	e (13):(REM/hr)		Duration (Ho	•	TEDE Dose (REM) Block 13) x (Block 14
(Pleak 1)(Pleak 4)(Plea	k 9)]+[(Block 1)(Block 2)(E (Block 5)	Block 3)(Block 8)(Block 10)]	x (Block 12)		(14)	(15)	
(Block 1)(Block 4)(Bloc							
(Block T)(Block 4)(Bloc				2)(Block 3)(Block 8)(Block 11)			CDE Dose (REM) Block 14) x (Block 16)

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ATTACHMENT 1 HAND-CALCULATED DOSE PROJECTION (NON-CENTERLINE)

TABLE 1 - IODINE TO NOBLE GAS RATIO VS. TIME SINCE SHUTDOWN

TABLE 2 - ENERGY FACTORS

TIME SINCE	IODINE/NOBLE GAS RATIO			
SHUTDOWN (hrs)	NON-DEGRADED CORE	DEGRADED CORE		
t < 1	1.86 E-7	2.71 E-1		
1 ≤ t < 2	1.86 E-7	3.57 E-1		
2 ≤ t < 4	1.86 E-7	3.41 E-1		
4 ≤ t < 10	1.86 E-7	2.81 E-1		
10 ≤ t < 30	1.86 E-7	2.30 E-1		
30 ≤ t < 100	1.86 E-7	1.65 E-1		
100 ≤ t	1.86 E-7	1.40 E-1		

TIME SINCE SHUTDOWN (hrs)	ENERGY FACTOR (MeV/dis)
t < 1	0.75
1 ≤ t < 2	0.60
2 ≤ t < 4	0.40
4 ≤ t < 10	0.25
10 ≤ t < 30	0.15
30 ≤ t < 100	0.09
100 ≤ t	0.07

TABLE 3 - DOSE CONVERSION FACTORS

	NON-DEGRADED CORE	DEGRADED CORE
TEDE Noble Gas	1.48 E-3	9.19 E-4
TEDE Iodine	8.77 E-2	2.98 E-2
CDE Iodine	2.04 E 0	4.96 E-1

ATTACHMENT 2 TRANSIT TIMES AND EFFECTIVE AGES OF NOBLE GASES AT RECEPTOR SITES

		GASES	S AT RECEPTOR SITES				
1.	<u>Effective Age</u> is defined as time elapsed (hrs) since shutdown. For off-site locations, the <u>effective age</u> of the isotopic mixture may be obtained through summarizing following components:						
[]	1.1	The effective age at the	e time of release onset.				
[]	1.2 The transit time from the release point to the receptor site (refer to Section 2 below).						
2.	CALCULATION OF TRANSIT TIME FROM THE RELEASE POINT TO THE RECEPTOR LOCATION						
[]	2.1 Estimate the downwind distance (miles) to the receptor location.						
[]] 2.2 Divide the distance in miles by the 100m meter level wind speed (mph) to determine the plume transit time						
	,) RECEPTOR SITE WNWIND DISTANCE (miles)	(2) 100 METER LEVEL WIND SPEED (mph)	(3) PLUME TRANSIT TIME (hrs) (1) ÷ (2)			
3.	DETI	ERMINATION OF EFF	ECTIVE AGES AT RECEPTO	OR SITES			
	N	EFFECTIVE AGE OF MIXTURE AT TIME F RELEASE ONSET (hrs)	(2) TRANSIT TIME FROM RELEASE POINT TO RECEPTOR LOCATION (hrs)	(3) EFFECTIVE AGE OF ISOTOPIC MIXTURE AT RECEPTOR LOCATION (hrs) (1) + (2)			
Nar	ne/Tim	ue/Date⁺					

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ATTACHMENT 3 HAND-CALCULATED DOSE PROJECTION (CENTERLINE)

(1) Noble Gas Release Rate from KAMAN or 5.7.16 (μCi/Sec)	(2) Release Path through SBGT?	(3) Iodine/Noble Gas Ratio	(4) Energy Factor (MeV/dis)	(5) Wind Speed (mph)	(6) Stability Class	(7) Release through Reactor Building? No = 1: Yes = 0.5
	Yes = 0.01; No = 1	(from Table 1)	(from Table 2)	Defaults ERP = 13; Other = 8	Default = D	

For Columns 5 and 6, use PMIS, NWS, or Defaults.

Conversion Factors (from Table 3)		
TEDE Noble Gas	(8)	
TEDE Iodine	(9)	
CDE Iodine	(10)	

TEDE Sub-Calculation (11): _______

[(Block 1)(Block 4)(Block 8)]+[(Block 1)(Block 2)(Block 3)(Block 7)(Block 9)]

(Block 5)

Mixing Factors (from Table 4)		
1 Mile	(12)	
2 Mile	(12)	
5 Mıle	(12)	
10 Mile	(12)	

TEDE RATE (REM/hr) (Block 11 x Block 12)		
1 Mile	(13)	
2 Mile	(13)	
5 Mıle	(13)	
10 Mile'	(13)	

Duration (hours) Default = 4 hrs	_
(14)	
L	_

TEDE Dose (REM) (Block 13 x Block 14)		
1 Mile	(15)	
2 Mile	(15)	
5 Mıle	(15)	
10 Mile	(15)	

CDE Sub-Calculation (16):

[(Block 1)(Block 2)(Block 3)(Block 7)(Block 10)]

(Block 5)

CDE Rate (REM/hr) (Block 16 x Block 12)		
1 Mile	(17)	
2 Mile	(17)	
5 Mıle	(17)	
10 Mile	(17)	

CDE Dose (REM) (Block 14 x Block 17)		
1 Mıle	(18)	
2 Mile	(18)	
5 Mile (18)		
10 Mıle	(18)	

Name/Time/Date: _____/___/

|--|

<u>TABLE 1</u> - IODINE TO NOBLE GAS RATIO VS. TIME SINCE SHUTDOWN

NON-DEGRADED	DEGRADED
CORE	CORE
1.86 E-7	2.71 E-1
1.86 E-7	3.57 E-1
1.86 E-7	3.41 E-1
1.86 E-7	2.81 E-1
1.86 E-7	2.30 E-1
	1.86 E-7 1.86 E-7 1.86 E-7

1.86 E-7

1.86 E-7

 $30 \le t < 100$

 $100 \le t$

TABLE 2 - ENERGY FACTORS

TIME SINCE SHUTDOWN (hrs)	ENERGY FACTOR (MeV/dis)
t < 1	0.75
$1 \le t \le 2$	0.60
$2 \le t \le 4$	0.40
$4 \le t < 10$	0.25
$10 \le t \le 30$	0.15
$30 \le t \le 100$	0.09
100 ≤ t	0.07

TABLE 3 - DOSE CONVERSION FACTORS

1.65 E-1

1.40 E-1

	NON-DEGRADED CORE	DEGRADED CORE
TEDE Noble Gas	1.48 E-3	9.19 E-4
TEDE Iodine	8.77 E-2	2.98 E-2
CDE Iodine	2.04 E 0	4.96 E-1

TABLE 4 - PLUME CENTERLINE X/Q'S (MIXING FACTORS)

RELEASE	STABILITY				articular.	_		
POINT	CLASS	A	- В	C	- <u>D</u>	E	F	G
	1 MILE	2.87E-6	6.04E-6	1.17E-5	8.35E-6	1.03E-6	2.35E-11	1.31E-23
ERP	2 MILE	7.94E-7	1.78E-6	4.55E-6	8.21E-6	4.98E-6	8.12E-8	5.62E-13
(ELEVATED)	5 MILE	1.50E-7	3.42E-7	1.18E-6	3.77E-6	4.66E-6	1.09E-6	5.67E-9
	10 MILE	4.51E-8	1.03E-7	4.58E-7	1.82E-6	3.13E-6	1.44E-6	4.00E-8
OTHER	1 MILE	3.01E-6	6.90E-6	1.73E-5	5.10E-5	1.09E-4	3.07E-4	7.67E-4
THAN ERP (GROUND LEVEL)	2 MILE	8.03E-7	1.84E-6	5.15E-6	1.78E-5	3.86E-5	1.09E-4	2.71E-4
	5 MILE	1.50E-7	3.44E-7	1.21E-6	4.98E-6	1.25E-5	3.52E-5	8.81E-5
	10 MILE	4.51E-8	1.03E-7	4.63E-7	2.07E-6	6.43E-6	1.81E-5	4.52E-5

ATTACHMENT 4	CORRELATING OFF-SITE SAMPLE RESULTS WITH DOSE
	PROJECTIONS

1. CORRECTION FACTOR DETERMINATIONS USING OFF-SITE SAMPLING DATA

(1) SAMPLE	ž.		IODINE CONCENTRATION	
LOCATION	TIME	(μCi/cc)	(μCi/cc)	$(3) \div (4)$
			٠,	

Name/Time/Date:	/	
Name/Ilme/Date.		

3. Route completed form to Emergency Preparedness Department.

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ATTACHMENT 5 METEOROLOGICAL AND RADIOLOGICAL DATA SOURCES FOR CNS-DOSE

NOTE 1 - When the normal source of meteorological data (PMIS MET screen) is not available or is "unhealthy", attempt to obtain the data by PMIS point ID. If PMIS is not available, call the National Weather Service (NWS) in Valley, NE to obtain the data. The telephone number is contained in the Emergency Telephone Directory - Federal TAB. If the NWS cannot be contacted, use default values.

<u>NOTE</u> 2 - If the user is not familiar with the use of PMIS, Attachment 6 provides an overview and instructions on access and selected use of PMIS.

NOTE 3 - The Turn-On-Code "VALUE" is used to display single point values and qualities.

NOTE 4 - The Turn-On-Code "MET" is used to display most meteorological point values and stability classes.

PMIS POINT ID	DESCRIPTION
MET001	100M LVL SIGMA THETA (15 MIN AVE)
MET004	100M LVL TEMPERATURE
MET005	DELTA TEMPERATURE (100M-10M)
MET006	100M LVL WIND DIR. (15 MIN AVE)
MET007	100M LVL WIND SPEED (15 MIN AVE)
MET009	60M LVL SIGMA THETA (15 MIN AVE)
MET012	60M LVL TEMPERATURE
MET013	DELTA TEMPERATURE (100M-60M)
MET014	60M LVL WIND DIR. (15 MIN AVE)
MET015	60M LVL WIND SPEED (15 MIN AVE)
MET017	10M LVL SIGMA THETA (15 MIN AVE)
MET020	10M LVL TEMPERATURE
MET021	DELTA TEMPERATURE (60M-10M)
MET023	10M LVL WIND DIR. (15 MIN AVE)
MET024	10M LVL WIND SPEED (15 MIN AVE)
MET027	PRECIPITATION (15 MIN PERIOD)
MET028	10M TWR SIGMA THETA (15 MIN AVE)
MET029	10M TWR TEMPERATURE
MET030	10M TWR WIND DIR. (15 MIN AVE)
MET031	10M TWR WIND SPEED (15 MIN AVE)
N8000	RX BLDG EFFLUENT FLOW AVE
N8001	TURB BLDG EFF HI RAD MON AVE
N8002	TURB BLDG EFF NORM RAD MON AVE
N8003	TURB BLDG FLOW AVE
N8004	AOG & RW EFF HI RAD MON AVE
N8005	AOG & RW EFF NORM RAD MON AVE
N8006	RX BLDG EFF RAD MON AVE
N8007	AOG & RW BLDG EFF FLOW AVE
N8010	ERP HI RAD MON AVE
N8011	ERP NORMAL RAD MON AVE
N8012	ERP FLOW AVE
N8013	SGT FLOW TO ERP AVE

ATTACHMENT 6 PMIS SYSTEM ACCESS AND USE

1. PLANT MANAGEMENT INFORMATION SYSTEM (PMIS)

1.1 The PMIS System (PMIS) is a set of programs and hardware provided by NPPD that make use of VMS functions and additional peripherals (Data Concentrators) which provides access to plant parameters.

2. PMIS COMPUTERS

2.1 PMIS computers share a common set of peripherals (disk drives, tape drives, terminals, etc.) and software.

3. VMS OPERATING SYSTEM

3.1 The VMS Operating System (VMS) is the host operating system for the PMIS computers. It is a set of programs that interface with the computer hardware and peripherals, and allows the computers to recognize and process commands.

4. PMIS MODES

- 4.1 PMIS has three operational modes, Primary, Primary/Backup, and Backup, and will operate on either computer in one of the three modes. A computer with PMIS operating in either the Primary or Primary/Backup Mode is referred to as the Primary System and the one with PMIS operating in the Backup Mode is referred to as the Backup System.
- 4.2 The Primary and Primary/Backup Modes provide full PMIS capabilities, consisting (in part) of data acquisition and conversion, data display, data archiving, alarm processing, self monitoring, and many other functions that perform specialized calculations and displays.
- 4.3 The Backup Mode monitors the Primary System, transfers information necessary to keep the Backup System files and tables up-to-date, and automatically changes to the Primary Mode when a loss of the Primary System is detected (referred to as a FAILOVER). Although many functions are available on the Backup System, their use is discouraged because the lack of real-time data results in the display of inaccurate information (CNS-DOSE is an exception).

5. PMIS ACCESS

5.1 Access to PMIS is gained through various video display terminals, printer/plotters, and printers, including color graphic Information Display Terminals (IDTs) dedicated exclusively for PMIS access in the Control Room, TSC, and EOF.

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ATTACHMENT 6 PMIS SYSTEM ACCESS AND USE

5.2 The IDTs and printers are selectively connected to either computer through a switching device controlled by PMIS. At system start or during a FAILOVER, all terminals and printers are switched to the Primary System. However, the SWITCH position may be changed at any time after that.

6. SCREEN FORMAT

- 6.1 When a terminal is under control of PMIS (instead of VMS), the screen display will be in a standard format consisting of four areas, OCA, GGDA, SSA, and FKA.
- 6.2 The OCA (Operator Communication Area) consists of the top two (one and two lines on the screen. This area is generally used to prompt-for and receive user inputs and display advisory and warning messages. In addition, some displays that require only one or two lines of screen use the OCA for display. Also (though technically not part of the OCA), the current date and time (updated once a second) is displayed at the right side of the screen on lines 1 and 2.
- 6.3 The GGDA (General and Graphic Display Area) consists of lines 4 through 47 and is used for most displays. In addition, some displays (chiefly functions requiring significant editing) also prompt-for and receive user inputs in the GGDA.
- 6.4 The SSA (SPDS Status Area) consists of lines 45 through 48 and contain four boxes that represent (by color code) the status of the SPDS (Safety Parameter Display System), which is a software system that monitors selected plant parameters and determines overall plant safety status.
- 6.5 The FKA (Function Key Area) consists of the bottom two (50 and 51) lines of the screen. The FKA is used to indicate which of the definable function keys are enabled. It also indicates which mode PMIS is in, the Plant Mode, and whether or not a PMIS "event" has occurred.

7. SCREEN-COPY FUNCTION

7.1 The screen-copy function, which is activated by pressing the HARD COPY key, provides full screen reproduction in color on a printer located in the same general area as the terminal.

8. PRINTER

8.1 The printers are connected to a specific computer and are generally accessed when a "...PRINT..." option is selected and a "logical name" is entered.

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9. LOGICAL NAME

9.1 Printers and terminals are usually referenced by "logical names", in the format of TT00, TT01, etc. (IDTs), and LA00, LA01, etc. (printers). The "logical name" for a device can usually be found on a tag on the device.

10. RESET FUNCTION

10.1 This function, which is activated by pressing the RESET key (PC keyboard) or CONTROL-RESET keys (IDT keyboard), clears the screen, sounds the bell, and resets internal parameters to the default settings, producing the same effect as a re-boot or turning power off and on.

11. IDE FIELD

11.1 User input to PMIS Programs is through an open IDE (Interactive Data Entry) field on the terminal. An open IDE field is denoted by a yellow box that appears in the OCA or GGDA area. Anything typed on the keyboard will be echoed in the box. Erasing or back-spacing is accomplished with the DEL key. All entries into an IDE field must be terminated by pressing the ENTER key unless the field is overfilled or a function key is pressed (the terminal automatically adds a carriage return character in those cases).

12. TURN-ON-CODE

12.1 The Turn-On-Code (TOC) is the mechanism by which commands are issued to PMIS. This is a one to eight character code which is interpreted by PMIS and a corresponding command is issued.

13. PMIS DATABASE ---

13.1 All plant parameters (or additional data based on plant or PMIS parameters) that are processed by PMIS SYSTEM are defined in the PMIS DATABASE, which is a file that specifies the origin of the data, the frequency at which it is processed, the type of processing to be performed, etc. Each parameter is referred to as a "point" and is identified by a one to eight character name or POINT-ID (PID).

14. PMIS DATA PROCESSING

14.1 Some PMIS points are processed by scanning plant sensors (through the Data Concentrator) while others are calculated based on the values of previously processed points or PMIS parameters. All points values are then assigned a quality code stored in the Current Value Table (CVT).

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- 14.2 Data in the CVT is considered to be "real-time" and representative of current plant and system conditions.
- 14.3 At regular intervals (and other special circumstances) point values are also stored in an Archive File, which provides ~ 24 hours of on-line historical information.

15. PMIS DATA ACCESS

15.1 All point values in the CVT and Archive File are accessed by the POINT-ID.

16. QUALITY CODES

16.1 The Quality Code, assigned when point values are assigned, represents the general status and "health" of the point, and determines how it is used by PMIS Programs. The following is a list of PMIS quality codes and related information.

CODE	DESCRIPTION	COLOR	HEALTH
UNK	Value unknown - not yet processed	White	Bad
DEL	Processing has been disabled	Magenta	Bad
INVL	Data concentrator error	Magenta	Bad
RDER	Data concentrator error	Magenta	Bad
OIC	Data concentrator error	Magenta	Bad
BAD	Outside instrument range	Magenta	Bad
STAG	Point failed stagnation check	Magenta	- Bad
UDEF	Undefined (spare)	Magenta	Bad
REDU	Fails redundant point check	Magenta	Bad
HALM	Above high alarm limit	Red	Good
LALM	Below low alarm limit	Red	\mathbf{Good}
HWRN	Above high warning limit	Yellow	Good
LWRN	Below low warning limit	Yellow	Good
ALM	State/Change-of-State alarm	Red	Good
SUB	Value has been substituted	Blue	Good
DALM	Alarm checking has been disabled	Green	Good
NCAL	Value cannot be calculated	\mathbf{W} hite	\mathbf{Good}
INHB	Alarm inhibited by cut-out point	Green	\mathbf{Good}
GOOD	Passes all other checks	Green	Good

16.2 Not listed above is quality code OSUB (Operator Substituted), which is treated the same as SUB, and indicates that the value was substituted within that program. OSUB is not used in the CVT.

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17. PMIS LOGIN

- 17.1 If the current date and time is displayed in the OCA and is being updated about once a second:
 - 17.1.1 If "ENTER PASSWORD..." is displayed on line 2, press the ENTER key.
 - 17.1.2 If "SELECT FUNC. KEY OR TURN ON CODE..." and an open IDE field is displayed on line 2, the IDT is logged into PMIS. No further action is necessary.
 - 17.1.3 If a display is operating, press the CANC key.
 - 17.1.4 If terminal does not respond or does not meet any of the above criteria, press the XOFF key once. The terminal should be automatically reset (screen clears and the bell sounds) after about 30 seconds, and either the "ENTER PASSWORD..." or "...TURN-ON-CODE..." prompt should be displayed. Refer to the applicable previous step for more instruction.
- 17.2 If the current date and time is NOT displayed or is displayed but is not being updated:
 - 17.2.1 Press the RESET key (PC keyboard) or CONTROL-RESET keys (IDT keyboard), wait at least 10 seconds, and press the ENTER key. If the date and time appear and began updating, refer to the previous (date and time updating) step.
 - 17.2.2 If a "\$" is displayed at the left of the screen, enter "LO" and press the ENTER key. After the "...LOGGED OFF..." message is displayed, press the ENTER key again.
 - 17.2.3 After "Username:" is displayed, enter "PMIS" and press the ENTER key. A welcome message followed by "PMIS LOGGED OUT..." will be displayed. Do not press any keys for 5 minutes or until the PMIS login display appears. When the "ENTER PASSWORD..." prompt is issued, refer to the previous (date and time updating) step and login to PMIS.
- 17.3 If neither of the above criteria is met or the specified sequence of events does not occur, contact the Nuclear Information Services (NIS) Department for assistance.

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18. ACTIVATING A TURN-ON-CODE

- 18.1 If a display is currently operating in the area of the screen that the desired TOC requires, press the CANC key.
- 18.2 When "SELECT FUNC. KEY OR TURN ON CODE..." is displayed followed by an open IDE field, enter one of following:
 - 18.2.1 A TOC (i.e., "GROUP" -- activates the Group Display Program; the program will then prompt the user to select a menu option).
 - 18.2.2 A TOC followed by a space and optional text (i.e., "PLOT ARM1" -- activates the Real-Time Plot Program and plots the group "ARM1" without further user input; note that optional text is recognized by only selected TOCs).
 - 18.2.3 Press one of the programmable function keys on the right hand key pad or top row of function keys (i.e., blue "GROUP DISP" key -- functions the same as the first example).
- 18.3 Refer to the FKA for the function keys that are enabled and their descriptions. Use other options as provided by each program.
- 18.4 To exit a program, use the specified exit option (if provided) or press the CANC function key.

19. DETERMINING TO WHICH SYSTEM A TERMINAL IS CONNECTED

The PMIS System to which a terminal is connected is indicated by the "CONSOLE =..." on the bottom line of the FKA as follows:

CONSOLE = PRIMARY -- Connected to the Primary System operating in the Primary Mode.

CONSOLE = PRIM/BAC -- Connected to the Primary System operating in the Primary/Backup Mode.

CONSOLE = BACKUP -- Connected to the Backup System.

CONSOLE = UNKNOWN -- PMIS is in a transition or unknown state.

ATTACHMENT 6 PMIS SYSTEM ACCESS AND USE

20. SWITCHING A DEVICE TO THE OTHER SYSTEM

- 20.1 On a terminal located in the same area as the device to be switched and connected to either PMIS System, activate the TOC "SWITCH".
- 20.2 A list of all devices that can be switched from that terminal will be displayed. Included will be their logical names, description, and the CPU to which the device is connected.
- 20.3 To switch a device, press function key F1 and then enter the logical name at the prompt.
- 20.4 If the device is an IDT, it will be logged off PMIS.
- 20.5 If the device being switched is a terminal other than the one running SWITCH, both are connected to the same system and a TOC is currently active, a message will be displayed to that effect, and the user will be asked if it is to be switched anyway. If the answer is not YES, the device is not switched.

ATTACHMENT 7	CORE DAMAGE ESTIMATION

<u>NOTE</u> - This attachment is only used for core damage estimates where the in-containment radiation monitors are exposed to coolant or steam (i.e., only for primary containment LOCA situations). For other accidents sequences, utilize the Post-Accident Sampling System (PASS) and Core Damage Assessment Program (CORDAM).

(1) HIGHEST DRYWELL RAD MONITOR READING (RMA-RM-40A,B)	(2) 100% CORE MELT FACTOR	(3) CORE MELT FRACTION (1) ÷ (2)	(4) PERCENT CORE MELT (3) x 100	(5) PERCENT CLAD FAILURE (4) x 10
	2.44E+6			

Name/Time/Date:	4	<i>!</i>	1

Report the results of the core damage estimate (Blocks 4 and 5) to the TSC Director.

ATTACHMENT 8 INFORMATION SHEET

1. DISCUSSION

- 1.1 This procedure covers dose projection. Dose projection represents calculation of an accumulated dose at some time in the future if current conditions continue.
- 1.2 The CNS-DOSE Computer Program is a software application operated on the PMIS computers. It makes use of current meteorological and radiological data from PMIS and manually entered data to perform dose projection for the area surrounding CNS. CNS-DOSE is the primary method of dose projection.
 - 1.2.1 The PMIS Computer System consists of two computers operating in a Primary and Backup Mode. Historical data may be obtained from either system; however, current data may be obtained only from the Primary System.
 - 1.2.2 Personal unfamiliar with the operation of PMIS should reference procedures governing the operation of PMIS or refer to Attachment 6.
- 1.3 The manual dose projection methods in this procedure are intended to be used when CNS-DOSE is unavailable. Where possible, data used is from the same source as that used by the computer programs. The hand calculations are divided into two sections. Section 5 is intended to be used by the on-shift personnel for centerline dose projections. Section 6 is intended for dose assessment personnel in projecting non-centerline values.
- 1.4 The correlation methodology as described in Section 8 provides EOF dose assessment personnel with a means of correlating field team iodine concentration data with CNS-DOSE projected iodine concentration. Such a correlation is necessary to determine if initial Protective Action Recommendations (PARs) were adequate to protect the health and safety of the public.
- 1.5 Containment radiation level provides a measure of core damage, because it is an indication of the inventory of airborne fission products (i.e., noble gases, a fraction of the halogens, and a much smaller fraction of the particulates) released from the fuel to the containment (refer to NEDO-22215, Pages 1 and 2; NEDC 02-009).

ATTACHMENT 8 INFORMATION SHEET

2. REFERENCES

2.1 CODES AND STANDARDS

- 2.1.1 NRC Regulatory Guide 1.109, Revision 1, October 1977, Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10CFR50, Appendix I, Iodine Inhalation Dose Factors.
- 2.1.2 NRC Regulatory Guide 1.111, July 1977, Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors.
- 2.1.3 NRC Regulatory Guide 1.145, August 1979, Atmospheric Dispersion Models for Potential Accident Consequence Assessments at Nuclear Power Plants.
- 2.1.4 Health Physics Journal, November 1981, Noble Gas Dose Rate Conversion Factors.
- 2.1.5 ICRP 59, Working Breathing Rate.
- 2.1.6 EPA 400-R-92-001, May 1992, Manual of Protective Action Guides and Protective Actions for Nuclear Incidents.

2.2 DRAWINGS (MAPS)

- 2.2.1 NPPD Drawing CNS-MI-102, Atmospheric Dispersion Model (EPM2) Special Receptor Points, 10 Mile Radius.
- 2.2.2 NPPD Drawing CNS-MI-03, Preselected Radiological Sampling and Monitoring Points in the Vicinity of Cooper Nuclear Station, 10 Mile Radius.
- 2.2.3 NPPD Drawing 2.2 (P3-A-45), Revision 1, Cooper Nuclear Station Site and Property Boundary, 1 Mile Radius.
- 2.2.4 Cooper Nuclear Station 50 Mile Emergency Planning Zone, Revision 2, 50 Mile Radius.

ATTACHMENT 8 INFORMATION SHEET

2.3 VENDOR MANUALS

2.3.1 CNS Number 0984, PMIS Operator's Manual - SAIC Document 502-85500107-72.

2.4 PROCEDURES

- 2.4.1 Emergency Plan Implementing Procedure 5.7.1, Emergency Classification.
- 2.4.2 Emergency Plan Implementing Procedure 5.7.16, Release Rate Determination.
- 2.4.3 Emergency Plan Implementing Procedure 5.7.20, Protective Action Recommendations.

2.5 MISCELLANEOUS

- 2.5.8 Engineering Evaluation EE 02-056, Elimination of Meteorological Instrumentation System Strip Chart Recorder References.
- 2.5.1 General Electric Corporation, NEDO-22215, Procedures for the Determination of the Extent of Core Damage Under Accident Conditions.
- 2.5.2 NEDC 02-004, Estimation of the Steam Jet Air Ejector Radiation Monitor, RMP-RM-150A(B), Readings Following a 1% Fuel Clad release (Degraded Core) in the Reactor Coolant System.
- 2.5.3 NEDC 02-009, Estimation of Primary Containment High Range Monitor, RMA-RM-40A(B), Readings Following 1% Clad Failure in the RCS under Non-LOCA Conditions.
- 2.5.4 NEDO-31400, Safety Evaluation for Eliminating the BWR MSIV Closure Function and Scram Function for the MSL Rad Monitors.
- 2.5.5 NRC Inspection Report 89-35.
- 2.5.6 © NRC Inspection Report 91-12, Emergency Preparedness Annual Inspection Report. Affects Section 7 and NOTE prior to Step 5.2.
- 2.5.7 NRC Inspection Report 92-14, Emergency Preparedness Annual Inspection Report.

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