

Nuclear Power Business Unit
TEMPORARY CHANGE REVIEW AND APPROVAL

Change # Z

Note: Refer to NP 1.2.3, Temporary Procedure Changes, for requirements:

11-4-2002
 31 pages
 # 233

I - INITIATION

Doc Number AOP 10 Current Rev 0

Document Title Control Room Inaccessibility

Existing Effective Temporary Changes 2002-0764

Brief Description Add caution to Attachments A and B to address AI
(Identify specific changes on Form PBF-0026c, Document Review and Approval Continuation)

Initiate PBF-0026h and include with the change.

Other documents required to be effective concurrently with the temporary changes pre-screened according to NP 5.1.8? NO YES (Provide)

Screening completed according to NP 5.1.8? NA YES (Attach)

Safety Evaluation Required? NO YES (If Yes, a revision may be processed or final)

Determine if the change constitutes a Change Of Intent to the procedure by (If any answers are YES, a revision may be processed or final reviews and approvals shall be required):

Will the proposed change:

- Require a change to, affect or invalidate a requirement, commitment description in the Current or ISFSI Licensing Basis (as defined in NP 1.2.3)? YES NO
- Cause an increase in magnitude, significance or impact such that it should be processed as a revision? YES NO
- Delete or modify a prerequisite, initial condition, precaution, limitation or other steps that could have safety significance or affect the procedure's margin of safety? YES NO
- Delete QC hold points, Independent Verification or Concurrent Check steps without the related step(s) that require the performance also being deleted? YES NO
- Change Tech Spec or other regulatory acceptance criteria other than for re-baselining purposes? YES NO
- Require a change to the procedure Purpose or change the procedure classification? YES NO

Initiated By (print/sign) Ross Groehler Date 11/04/2002

II - INITIAL APPROVAL

This change is correct and complete, can be performed as written, and does not adversely affect personnel or nuclear safety, or Plant operating conditions.

Group Supervisor (print/sign) TEVANDRA KOSCIW Date 11/4/02
(Cannot be the Initiator)

This change does not adversely affect Plant operating conditions. (Safety Related procedures only)

Senior Reactor Operator (print/sign) R. J. W. Date 11/4/02
(Cannot be the Initiator or Group Supervisor)

III - PROCEDURE OWNER REVIEW

Permanent One-time Use Expiration Date, Event or Condition: _____

Hold change until procedure completed (final review and approval still required within 14 days of initial approval)

QR/MSS Review NOT Required (Admin/NNSR only) QR Review Required MSS Review Required (Reference NP 1.6.5)

Procedure Owner (print/sign) TEVANDRA KOSCIW Date 11/4/02
This Change and supporting requirements correctly completed and processed

IV - FINAL REVIEW AND APPROVAL

(Must be completed within 14 days of initial approval) (The Initiator, OR and Approval Authority shall be independent from each other)

QR/MSS (print/sign) R. J. W. Date 11/4/02
Indicates 50.59/72.48 applicability assessed, any necessary screenings/evaluations performed, determination made as to whether additional cross-disciplinary review required, and if required, performed.

MSS Meeting No. _____

Approval Authority (print/sign) DAVID HIZAK Date 11/4/02

V - REVISION INFORMATION FOR PERMANENT CHANGES

Post Typing Review (print/sign) _____ Date 11/3/02
Indicates temporary change(s) incorporated exactly as approved and no other changes made to document.

Incorporated into Revision Number _____ Effective Date 11/3/02

TEMPORARY CHANGE AFFECTED MANUAL LOCATION

Procedure Number AOP 10 Revision 0 Unit PB0
 Title Control Room Inaccessability
 Temporary Change Number 2002-0865

I - IMMEDIATELY AFTER INITIAL APPROVAL ON PBF-0026e (Non-Intent changes)
 (after Final Approval if change of intent involved)

This procedure change has been processed as follows: (Manual/Location)	Date Performed
<input type="checkbox"/> Copy included in work package for field implementation. (WO No. _____)	
<input checked="" type="checkbox"/> Copy filed in Control Room temp change binder (Operations only).	11-5-02
<input checked="" type="checkbox"/> Original change package provided to <u>IGV</u> to obtain Procedure Owner Review (e.g., Owner review may be coordinated by In-Group OA II, Procedure Writer, Procedure Supervisor, etc.).	11-4-02
<input type="checkbox"/>	

Performed By (print and sign) Carol Schroeder *Carol Schroeder* Date 11/04/2002

II - PROCEDURE OWNER REVIEW ON PBF-0026e
 (may be performed by OA II, Procedure Writer, etc.)

This procedure change has been processed as follows: (Manual/Location)	Date Performed
<input checked="" type="checkbox"/> Copy sent to Document Control Distribution Lead for Master File. (Not required for one-time use change)	11-5-02
<input type="checkbox"/> Copy filed in Group satellite file. (Not required for one-time use changes.)	
<input type="checkbox"/> Copy filed in Group one-time use file.	
<input checked="" type="checkbox"/> Original Temp Change provided to <u>DGD</u> to obtain Final Approvals (e.g., final approval may be coordinated by In-Group OA II, Procedure Writer, Procedure Supervisor, etc.)	11-4-02
<input checked="" type="checkbox"/> U1/U2	11-5-02
<input checked="" type="checkbox"/> PAB	↓
<input checked="" type="checkbox"/> OPS OFFICE	
<input checked="" type="checkbox"/> OPS SHOP	
<input checked="" type="checkbox"/> Simulator (OAI)	
<input checked="" type="checkbox"/> 6 Copies (WCC)	

Performed By (print and sign) Carol Schroeder *Carol Schroeder* Date 11/04/2002

Nuclear Power Business Unit
TEMPORARY CHANGE REVIEW AND APPROVAL

Ops Office
 Change #1
 Page 1 of _____

Note: Refer to NP 1.2.3, Temporary Procedure Changes, for requirements.

I - INITIATION

Doc Number AOP 10 Current Rev 0 Unit PB0 Temp Change No. 2002-0764
 Document Title Control Room Inaccessibility
 Existing Effective Temporary Changes N/A
 Brief Description Add caution to Attachments A and B to address AFW Minimum Flow requirements
 (Identify specific changes on Form PBF-0026c, Document Review and Approval Continuation, and include with the package)

Initiate PBF-0026h and include with the change.
 Other documents required to be effective concurrently with the temporary change: N/A
 Changes pre-screened according to NP 5.1.8? NO YES (Provide documentation according to NP 5.1.8)
 Screening completed according to NP 5.1.8? NA YES (Attach copy)
 Safety Evaluation Required? NO YES (If Yes, a revision may be processed or final reviews and approvals shall be obtained before implementing)

Determine if the change constitutes a Change Of Intent to the procedure by evaluating the following questions.
 (If any answers are YES, a revision may be processed or final reviews and approvals shall be obtained before implementing)

Will the proposed change:

	YES	NO
1. Require a change to, affect or invalidate a requirement, commitment, evaluation or description in the Current or ISFSI Licensing Basis (as defined in NP 5.1.8 and NP 5.1.7)?	<input type="checkbox"/>	<input checked="" type="checkbox"/>
2. Cause an increase in magnitude, significance or impact such that it should be processed as a revision?	<input type="checkbox"/>	<input checked="" type="checkbox"/>
3. Delete or modify a prerequisite, initial condition, precaution, limitation or other steps that could have safety significance or affect the procedure's margin of safety?	<input checked="" type="checkbox"/>	<input type="checkbox"/>
4. Delete QC hold points, Independent Verification or Concurrent Check steps without the related step(s) that require the performance also being deleted?	<input type="checkbox"/>	<input checked="" type="checkbox"/>
5. Change Tech Spec or other regulatory acceptance criteria other than for re-baselining purposes?	<input type="checkbox"/>	<input checked="" type="checkbox"/>
6. Require a change to the procedure Purpose or change the procedure classification?	<input type="checkbox"/>	<input checked="" type="checkbox"/>

Initiated By (print/sign) Ross Groehler [Signature] Date 10/29/2002

II - INITIAL APPROVAL

This change is correct and complete, can be performed as written, and does not adversely affect personnel or nuclear safety, or Plant operating conditions.

Group Supervisor (print/sign) [Signature] Date 10/29/02
 (Cannot be the Initiator)

This change does not adversely affect Plant operating conditions. (Safety Related procedures only)

Senior Reactor Operator (print/sign) [Signature] Date 10/29/02
 (Cannot be the Initiator or Group Supervisor)

III - PROCEDURE OWNER REVIEW

Permanent One-time Use Expiration Date, Event or Condition: _____
 Hold change until procedure completed (final review and approval still required within 14 days of initial approval)
 QR/MSS Review NOT Required (Admin/NNSR only) QR Review Required MSS Review Required (Reference NP 1.6.5)
 Procedure Owner (print/sign) [Signature] Date 10/29/02
 This Change and supporting requirements correctly completed and processed.

IV - FINAL REVIEW AND APPROVAL

(Must be completed within 14 days of initial approval) (The Initiator, OR and Approval Authority shall be independent from each other)

QR/MSS (print/sign) [Signature] Date 10/29/02
 Indicates 50.59/72.48 applicability assessed, any necessary screenings/evaluations performed, determination made as to whether additional cross-disciplinary review required, and if required, performed.

MSS Meeting No. _____
 Approval Authority (print/sign) [Signature] Date 10-29-02

V - REVISION INFORMATION FOR PERMANENT CHANGES

Post Typing Review (print/sign) / Date _____
 Indicates temporary change(s) incorporated exactly as approved and no other changes made to document.

Incorporated into Revision Number _____ Effective Date _____

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Point Beach Nuclear Plant
DOCUMENT REVIEW AND APPROVAL CONTINUATION

Page ___ of ___

Doc Number AOP 10 Revision 0 Unit PB0
 Title Control room inaccessibility
 Temporary Change Number 2002-0764

Description of Changes:

Step * Change/Reason

Attachment A
pg 5 of 18,
Attachment B
pg 7 of 18

Add caution associated with AFW minimum flow requirement: To prevent AFW pump damage monitor and maintain minimum AFW discharge flow or stop the affected AFW pump as necessary to control S/G levels./ See SCR 2002-0458, CAP029908, P-38A MD AFWP has inadequate Recirc Flow during IT-10.

See also CAP 029952

Other Comments

* Note: Recording of Step Number(s) is not required for multiple occurrences of identical information or when not beneficial to reviewers.

TEMPORARY CHANGE AFFECTED MANUAL LOCATION

Procedure Number AOP-10 Revision 0 Unit PB0
 Title CONTROL ROOM INACCESSIBILITY
 Temporary Change Number 2002-0764

I - IMMEDIATELY AFTER INITIAL APPROVAL ON PBF-0026e (Non-Intent changes)
 (after Final Approval if change of intent involved)

This procedure change has been processed as follows: (Manual/Location)	Date Performed
<input type="checkbox"/> Copy included in work package for field implementation. (WO No. _____)	
<input checked="" type="checkbox"/> Copy filed in Control Room temp change binder (Operations only).	10-30-02
<input checked="" type="checkbox"/> Original change package provided to <u>TGV</u> to obtain Procedure Owner Review (e.g., Owner review may be coordinated by In-Group OA II, Procedure Writer, Procedure Supervisor, etc.).	10-29-02
<input type="checkbox"/>	

Performed By (print and sign) Carol Schroeder Carol Schroeder Date 10-30-02

II - PROCEDURE OWNER REVIEW ON PBF-0026e
 (may be performed by OA II, Procedure Writer, etc.)

This procedure change has been processed as follows: (Manual/Location)	Date Performed
<input checked="" type="checkbox"/> Copy sent to Document Control Distribution Lead for Master File. (Not required for one-time use change)	10-30-02
<input type="checkbox"/> Copy filed in Group satellite file. (Not required for one-time use changes.)	
<input type="checkbox"/> Copy filed in Group one-time use file.	
<input checked="" type="checkbox"/> Original Temp Change provided to <u>DDS</u> to obtain Final Approvals (e.g., final approval may be coordinated by In-Group OA II, Procedure Writer, Procedure Supervisor, etc.)	10-29-02
<input checked="" type="checkbox"/> <u>U1/U2</u>	10-30-02
<input checked="" type="checkbox"/> <u>PAB</u>	↓
<input checked="" type="checkbox"/> <u>Ops Shop</u>	
<input checked="" type="checkbox"/> <u>Ops Office</u>	
<input checked="" type="checkbox"/> <u>Simulator (TNG OAT)</u>	
<input checked="" type="checkbox"/> <u>Multiple Copies (WCC) 6</u>	

Performed By (print and sign) Carol Schroeder Carol Schroeder Date 10-30-02

Point Beach Nuclear Plant
10 CFR 50.59/72.48 SCREENING (NEW RULE)

SCR 2002-0458
Verify SCR number on all pages
Page 1

Title of Proposed Activity: AFW minimum flow requirement change to AOP, EOP, CSP, ECA, SEP, OI-62 A/B procedures

Associated Reference(s) #: Removal of internals from AF-117 and upgrade open function of AFW pumps minirecirc vlaves to safety -related (MR 02-029); SCR 2002-005-01 EOP/ARP actions for AFW mini-recirc requirement ; 2002-0055, P-38A/B mini recirc flow orifice replacment (MR 99-029 *A, *B) ; Flowsolve Corporation Pump Division letter dated March 2, 20012; CAP 29908; CAP 29952

Prepared by: Eric A. Schmidt / John P. Schroeder
Name (Print)

Eric A. Schmidt / John P. Schroeder Date: 10/24/02
Signature

Reviewed by: K. S. Sigala
Name (Print)

K. S. Sigala Date: 10/29/02
Signature

PART I (50.59/72.48) - DESCRIBE THE PROPOSED ACTIVITY AND SEARCH THE PLANT AND ISFSI LICENSING BASIS (Resource Manual 5.3.1)

NOTE: The "NMC 10 CFR 50.59 Resource Manual" (Resource Manual) and NEI 96-07, Appendix B, Guidelines for 10 CFR 72.48 Implementation should be used for guidance to determine the proper responses for 10 CFR 50.59 and 10 CFR 72.48 screenings.

- 1.1 Describe the proposed activity and the scope of the activity being covered by this screening. (The 10 CFR 50.59 / 72.48 review of other portions of the proposed activity may be documented via the applicability and pre-screening process requirements in NP 5.1.8.) Appropriate descriptive material may be attached.

This screening supports procedural upgrades to address the Auxiliary Feedwater (AFW) System issue as identified in CAP 29908 and CAP 29952. Procedural guidance for operation of AFW System will be changed such that the operator must ensure that discharge flow for P-38 A/B must be greater than 50 gpm and 1/2 P-29 discharge flow must be greater than 75 gpm. If pump flow cannot be maintained within these requirements, the pump must be secured.

- 1.2 Search the PBNP Current Licensing Basis (CLB) as follows: Final Safety Analysis Report (FSAR), FSAR Change Requests (FCRs) with assigned numbers, the Fire Protection Evaluation Report (FPER), the CLB (Regulatory) Commitment Database, the Technical Specifications, the Technical Specifications Bases, and the Technical Requirements Manual. Search the ISFSI licensing basis as follows: VSC-24 Safety Analysis Report, the VSC-24 Certificate of Compliance, the CLB (Regulatory) Commitment Database, and the VSC-24 10 CFR 72.212 Site Evaluation Report. Describe the pertinent design function(s), performance requirements, and methods of evaluation for both the plant and for the cask/ISFSI as appropriate. Identify where the pertinent information is described in the above documents (by document section number and title). (Resource Manual 5.3.1 and NEI 96-07, App. B, B.2)

FSAR 10.2 Auxiliary Feedwater System (AF) - The AFW system shall automatically start and deliver adequate AFW flow to maintain adequate steam generator levels during accidents which may result in main steam safety valve opening, such as: Loss of normal feedwater (LONF) and Loss of all AC power to the station auxiliaries (LOAC). AFW system shall also deliver sufficient flow to the steam generators supporting rapid cooldown during such accidents as: steam generator tube rupture (SGTR) and main steam line break (MSLB).

Each pump has an AOV controlled recirculation line back to the condensate storage tanks to ensure minimum flow to prevent hydraulic instabilities and dissipate pump heat.

TS 3.7.5 Auxiliary Feedwater (AFW) System

TS Bases B 3.7.5 Auxiliary Feedwater (AFW) System

FSAR 7.3.3.4 Manual AFW Flow Control During Plant Shutdown Manual control of steam generator water level using the AF pumps to remove reactor decay and sensible heat.

FPER 6.6.4 Auxiliary Feedwater System The Auxiliary Feedwater Pumps are provided with a mini-recirc line to ensure a minimum amount of flow is established to keep the pumps from dead heading.

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Point Beach Nuclear Plant
10 CFR 50.59/72.48 SCREENING (NEW RULE)

SCR 2002-0458
Verify SCR number on all pages
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FSAR 10.2 Auxiliary Feedwater System (AF)
TS 3.7.5 Auxiliary Feedwater (AFW) System
TS Bases B 3.7.5 Auxiliary Feedwater (AFW) System
FSAR 7.3.3.4 Manual AFW Flow Control During Plant Shutdown
FPER 6.6.4 Auxiliary Feedwater System

I.3 Does the proposed activity involve a change to any Technical Specification? Changes to Technical Specifications require a License Amendment Request (Resource Manual Section 5.3.1.2).

Technical Specification Change : Yes No

If a Technical Specification change is required, explain what the change should be and why it is required.

I.4 Does the proposed activity involve a change to the terms, conditions or specifications incorporated in any VSC-24 cask Certificate of Compliance (CoC)? Changes to a VSC-24 cask Certificate of Compliance require a CoC amendment request.

Yes No

If a storage cask Certificate of Compliance change is required, explain what the change should be and why it is required.

10 CFR 50.59 SCREENING

PART II (50.59) - DETERMINE IF THE CHANGE INVOLVES A DESIGN FUNCTION (Resource Manual 5.3.2)

Compare the proposed activity to the relevant CLB descriptions, and answer the following questions:

YES	NO	QUESTION
<input checked="" type="checkbox"/>	<input type="checkbox"/>	Does the proposed activity involve Safety Analyses or structures, systems and components (SSCs) credited in the Safety Analyses?
<input type="checkbox"/>	<input checked="" type="checkbox"/>	Does the proposed activity involve SSCs that support SSC(s) credited in the Safety Analyses?
<input checked="" type="checkbox"/>	<input type="checkbox"/>	Does the proposed activity involve SSCs whose failure could initiate a transient (e.g., reactor trip, loss of feedwater, etc.) or accident, <u>OR</u> whose failure could impact SSC(s) credited in the Safety Analyses?
<input checked="" type="checkbox"/>	<input type="checkbox"/>	Does the proposed activity involve CLB-described SSCs or procedural controls that perform functions that are required by, or otherwise necessary to comply with, regulations, license conditions, orders or technical specifications?
<input type="checkbox"/>	<input checked="" type="checkbox"/>	Does the activity involve a <i>method of evaluation</i> described in the FSAR?
<input type="checkbox"/>	<input checked="" type="checkbox"/>	Is the activity a <i>test or experiment</i> ? (i.e., a non-passive activity which gathers data)
<input type="checkbox"/>	<input checked="" type="checkbox"/>	Does the activity exceed or potentially affect a <i>design basis limit for a fission product barrier (DBLFPB)</i> ? (NOTE: If <u>THIS</u> questions is answered <u>YES</u> , a 10 CFR 50.59 Evaluation is required.)

If the answers to ALL of these questions are NO, mark Part III as not applicable, document the 10 CFR 50.59 screening in the conclusion section (Part IV), then proceed directly to Part V - 10 CFR 72.48 Pre-screening Questions.

If any of the above questions are marked YES, identify below the specific design function(s), method of evaluation(s) or DBLFPB(s) involved.

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MR-02-029 upgraded the open function of the AFW pumps mini-recirc AOV to safety-related. The safety-related boundary includes the recirc orifice and all associated upstream components and piping. It is postulated that a failure of the piping downstream of the recirc orifice will not have any adverse effects on the AFW system. The availability of the recirculation flowpath provides an additional flowpath to support minimum flow requirements. This procedure change will improve the reliability of the AFW pumps by not relying upon the recirc flow path for operability as it has been concluded that the restrictions in the recirc orifice may not be adequate for use. Whereas current guidance mandates that the operator verify the position of the recirc AOV and the status of the Instrument Air system, these procedural changes will only require the operator to monitor pump discharge flow.

PART III (50.59) - DETERMINE WHETHER THE ACTIVITY INVOLVES ADVERSE EFFECTS (Resource Manual 5.3.3)

If ALL the questions in Part II are answered NO, then Part III is NOT APPLICABLE.

Answer the following questions to determine if the activity has an *adverse effect* on a design function. Any YES answer means that a 10 CFR 50.59 Evaluation is required; EXCEPT where noted in Part III.3.

III.1 CHANGES TO THE FACILITY OR PROCEDURES

YES NO QUESTION

Does the activity adversely affect the *design function* of an SSC credited in safety analyses?

Does the activity adversely affect the method of performing or controlling the *design function* of an SSC credited in the safety analyses?

If any answer is YES, a 10 CFR 50.59 Evaluation is required. If both answers are NO, describe the basis for the conclusion (attach additional discussion as necessary):

Minimum flow requirements will be maintained within recommendations from the vendor by monitoring pump discharge flow and securing the pump as required. Starting and stopping of the AFW pumps has been previously evaluated in 50.59 Evaluation 2002-005, which addressed procedural changes to reduce the potential of pump damage as a result of the loss of the recirculation flow path.

III.2 CHANGES TO A METHOD OF EVALUATION

(If the activity does not involve a method of evaluation, these questions are NOT APPLICABLE.)

YES NO QUESTION

Does the activity use a revised or different method of evaluation for performing safety analyses than that described in the CLB?

Does the activity use a revised or different method of evaluation for evaluating SSCs credited in safety analyses than that described in the CLB?

If any answer is YES, a 10 CFR 50.59 Evaluation is required. If both answers are NO, describe the basis for the conclusion (attach additional discussion, as necessary).

III.3 TESTS OR EXPERIMENTS

If the activity is not a test or experiment, the questions in III.3.a and III.3.b are NOT APPLICABLE.

a. Answer these two questions first:

YES NO QUESTION

Is the proposed test or experiment bounded by other tests or experiments that are described in the CLB?

Are the SSCs affected by the proposed test or experiment isolated from the facility?

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If the answer to **BOTH** questions in V.3.a is **NO**, continue to III.3.b. If the answer to **EITHER** question is **YES**, then describe the basis.

b. Answer these additional questions **ONLY** for tests or experiments which do **NOT** meet the criteria given in III.3.a above. If the answer to either question in III.3.a is **YES**, then these three questions are **NOT APPLICABLE**.

- | YES | NO | QUESTION |
|--------------------------|--------------------------|---|
| <input type="checkbox"/> | <input type="checkbox"/> | Does the activity utilize or control an SSC in a manner that is outside the reference bounds of the design bases as described in the CLB? |
| <input type="checkbox"/> | <input type="checkbox"/> | Does the activity utilize or control an SSC in a manner that is inconsistent with the analyses or descriptions in the CLB? |
| <input type="checkbox"/> | <input type="checkbox"/> | Does the activity place the facility in a condition not previously evaluated or that could affect the capability of an SSC to perform its intended functions? |

If any answer in III.3.b is **YES**, a 10 CFR 50.59 Evaluation is required. If the answers in III.3.b are **ALL NO**, describe the basis for the conclusion (attach additional discussion as necessary):

Part IV - 10 CFR 50.59 SCREENING CONCLUSION (Resource Manual 5.3.4).

Check all that apply:

A 10 CFR 50.59 Evaluation is required or **NOT** required.

A Point Beach FSAR change is required or **NOT** required. If an FSAR change is required, then initiate an FSAR Change Request (FCR) per NP 5.2.6.

A Regulatory Commitment (CLB Commitment Database) change is required or **NOT** required. If a Regulatory Commitment Change is required, initiate a commitment change per NP 5.1.7.

A Technical Specification Bases change is required or **NOT** required. If a change to the Technical Specification Bases is required, then initiate a Technical Specification Bases change per NP 5.2.15.

A Technical Requirements Manual change is required or **NOT** required. If a change to the Technical Requirements Manual is required, then initiate a Technical Requirements Manual change per NP 5.2.15.

10 CFR 72.48 SCREENING

NOTE: NEI 96-07, Appendix B, Guidelines for 10 CFR 72.48 Implementation should be used for guidance to determine the proper responses for 72.48 screenings.

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PART V (72.48) - 10 CFR 72.48 INITIAL SCREENING QUESTIONS

Part V determines if a full 10 CFR 72.48 screening is required to be completed (Parts VI and VII) for the proposed activity.

- | YES | NO | QUESTION |
|--------------------------|-------------------------------------|--|
| <input type="checkbox"/> | <input checked="" type="checkbox"/> | Does the proposed activity involve IN ANY MANNER the dry fuel storage cask(s), the cask transfer/transport equipment, any ISFSI facility SSC(s), or any ISFSI facility monitoring as follows: Multi-Assembly Sealed Basket (MSB), MSB Transfer Cask (MTC), MTC Lifting Yoke, Ventilated Concrete Cask (VCC), Ventilated Storage Cask (VSC), VSC Transporter (VCST), ISFSI Storage Pad Facility, ISFSI Storage Pad Data/Communication Links, or PPCS/ISFSI Continuous Temperature Monitoring System? |

- Does the proposed activity involve IN ANY MANNER SSC(s) installed in the plant specifically added to support cask loading/unloading activities, as follows: Cask Dewatering System (CDW), Cask Reflood System (CRF), or Hydrogen Monitoring System?
- Does the proposed activity involve IN ANY MANNER SSC(s) needed for plant operation which are also used to support cask loading/unloading activities, as follows: Spent Fuel Pool (SFP), SFP Cooling and Filtration (SF), Primary Auxiliary Building Ventilation System (VNPAB), Drumming Area Ventilation System (VNDRM), RE-105 (SFP Low Range Monitor), RE-135 (SFP High Range Monitor), RE-221 (Drumming Area Vent Gas Monitor), RE-325 (Drumming Area Exhaust Low-Range Gas Monitor), PAB Crane, SFP Platform Bridge, Truck Access Area, or Decon Area?
- Does the proposed activity involve a change to Point Beach CLB design criteria for external events such as earthquakes, tornadoes, high winds, flooding, etc.?
- Does the activity involve plant heavy load requirements or procedures for areas of the plant used to support cask loading/unloading activities?
- Does the activity involve any potential for fire or explosion where casks are loaded, unloaded, transported or stored?

If ANY of the Part V questions are answered YES, then a full 10 CFR 72.48 screening is required and answers to the questions in Part VI and Part VII are to be provided. If ALL the questions in Part V are answered NO, then check Parts VI and VII as not applicable. Complete Part VIII to document the conclusion that no 10 CFR 72.48 evaluation is required.

PART VI (72.48) - DETERMINE IF THE CHANGE INVOLVES A ISFSI LICENSING BASIS DESIGN FUNCTION

(If ALL the questions in Part V are NO, then Part VI is NOT APPLICABLE.)

Compare the proposed activity to the relevant portions of the ISFSI licensing basis and answer the following questions:

- | YES | NO | QUESTION |
|--------------------------|-------------------------------------|---|
| <input type="checkbox"/> | <input checked="" type="checkbox"/> | Does the proposed activity involve cask/ISFSI Safety Analyses or plant/cask/ISFSI structures, systems and components (SSCs) credited in the Safety Analyses? |
| <input type="checkbox"/> | <input checked="" type="checkbox"/> | Does the proposed activity involve plant, cask or ISFSI SSCs that support SSC(s) credited in the Safety Analyses? |
| <input type="checkbox"/> | <input checked="" type="checkbox"/> | Does the proposed activity involve plant, cask or ISFSI SSCs whose function is relied upon for prevention of a radioactive release, <u>OR</u> whose failure could impact SSC(s) credited in the Safety Analyses? |
| <input type="checkbox"/> | <input checked="" type="checkbox"/> | Does the proposed activity involve cask/ISFSI described SSCs or procedural controls that perform functions that are required by, or otherwise necessary to comply with, regulations, license conditions, CoC conditions, or orders? |
| <input type="checkbox"/> | <input checked="" type="checkbox"/> | Does the activity involve a <i>method of evaluation</i> described in the ISFSI licensing basis? |
| <input type="checkbox"/> | <input checked="" type="checkbox"/> | Is the activity a <i>test or experiment</i> ? (i.e., a non-passive activity which gathers data) |
| <input type="checkbox"/> | <input checked="" type="checkbox"/> | Does the activity exceed or potentially affect a cask <i>design basis limit for a fission product barrier (DBLFPB)</i> ? (NOTE: If <u>THIS</u> questions is answered <u>YES</u> , a 10 CFR 72.48 Evaluation is required.) |

If the answers to ALL of these questions are NO, mark Parts VII as not applicable, and document the 10 CFR 72.48 screening in the conclusion section (Part VIII).

If any of the above questions are marked YES, identify below the specific design function(s), method of evaluation(s) or DBLFPB(s) involved.

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PART VII (72.48) - DETERMINE WHETHER THE ACTIVITY INVOLVES ADVERSE EFFECTS (NEI 96-07, Appendix B, Section B.4.2.1)

(If ALL the questions in Part V or Part VI are answered NO, then Part VII is NOT APPLICABLE.)

Answer the following questions to determine if the activity has an *adverse effect* on a design function. Any **YES** answer means that a 10 CFR 72.48 Evaluation is required; **EXCEPT** where noted in Part VII.3.

VII.1 Changes to the Facility or Procedures

YES NO QUESTION

Does the activity adversely affect the *design function* of a plant, cask, or ISFSI SSC credited in safety analyses?

Does the activity adversely affect the method of performing or controlling the *design function* of a plant, cask, or ISFSI SSC credited in the safety analyses?

If any answer is **YES**, a 10 CFR 72.48 Evaluation is required. If both answers are **NO**, describe the basis for the conclusion (attach additional discussion, as necessary):

VII.2 Changes to a Method of Evaluation

(If the activity does not involve a method of evaluation, these questions are NOT APPLICABLE.)

YES NO QUESTION

Does the activity use a revised or different method of evaluation for performing safety analyses than that described in a cask SAR?

Does the activity use a revised or different method of evaluation for evaluating SSCs credited in safety analyses than that described in a cask SAR?

If any answer is **YES**, a 10 CFR 72.48 Evaluation is required. If both answers are **NO**, describe the basis for the conclusion (attach additional discussion, as necessary):

VII.3 Tests or Experiments

(If the activity is not a test or experiment, the questions in VII.3.a and VII.3.b are NOT APPLICABLE.)

a. Answer these two questions first:

YES NO QUESTION

Is the proposed test or experiment bounded by other tests or experiments that are described in the cask ISFSI licensing basis?

Are the SSCs affected by the proposed test or experiment isolated from the cask(s) or ISFSI facility?

If the answer to both questions is **NO**, continue to VII.3.b. If the answer to **EITHER** question is **YES**, then briefly describe the basis.

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b. Answer these additional questions **ONLY** for tests or experiments which do not meet the criteria given in VII.3.a above. If the answer to either question in VII.3.a is **YES**, then these three questions are NOT APPLICABLE:

Point Beach Nuclear Plant
10 CFR 50.59/72.48 SCREENING (NEW RULE)

SCR 2002-0458
Verify SCR number on all pages
Page 7

YES	NO	QUESTION
<input type="checkbox"/>	<input type="checkbox"/>	Does the activity utilize or control an SSC in a manner that is outside the reference bounds of the design bases as described in the ISFSI licensing basis?
<input type="checkbox"/>	<input type="checkbox"/>	Does the activity utilize or control a plant, cask or ISFSI facility SSC in a manner that is inconsistent with the analyses or descriptions in the ISFSI licensing basis?
<input type="checkbox"/>	<input type="checkbox"/>	Does the activity place the cask or ISFSI facility in a condition not previously evaluated or that could affect the capability of a plant, cask, or ISFSI SSC to perform its intended functions?

If any answer in VII.3.b is **YES**, a 10 CFR 72.48 Evaluation is required. If the answers are all **NO**, describe the basis for the conclusion (attach additional discussion as necessary):

PART VIII - DOCUMENT THE CONCLUSION OF THE 10 CFR 72.48 SCREENING

Check all that apply:

A 10 CFR 72.48 Evaluation is required or NOT required. Obtain a screening number and provide the original to Records Management regardless of the conclusion of the 50.59 or 72.48 screening.

A VSC-24 cask Safety Analysis Report change is required or NOT required. If a VSC-24 cask SAR change is required, then contact the Point Beach Dry Fuel Storage group supervisor.

A Regulatory Commitment (CLB Commitment Database) change is required or NOT required. If a Regulatory Commitment Change is required, initiate a commitment change per NP 5.1.7.

A change to the VSC-24 10 CFR 72.212 Site Evaluation Report is required or NOT required. If a VSC-24 10 CFR 72.212 Site Evaluation Report change is required, then contact the Point Beach Dry Fuel Storage group supervisor.

COPY

A. PURPOSE

1. To provide the operators with the instructions required to maintain Hot Shutdown in the event conditions in the Control Room require evacuation as deemed necessary by the Control Room staff.

The following assumptions apply to this procedure:

- Offsite power is available.
- All controls are operational and no failures are expected to occur to the control board which precludes the safe operation of equipment from outside the Control Room.
- No other accident condition exist requiring use of the Emergency Operating Procedures or any other Abnormal Operating Procedure.
- Both Units are at power, Mode 1.

B. SYMPTOMS OR ENTRY CONDITIONS

1. The following are entry conditions for this procedure:
 - a. Toxic gas in the Control Room, requiring evacuation.
 - b. Confirmed bomb threat in or adjacent to the Control Room requiring evacuation.
 - c. Other life threatening conditions, as determined by the DSS or his designee, that cause the Control Room to be uninhabitable.

C. REFERENCES

1. General Design Criteria #19
2. Technical Specifications for Point Beach Nuclear Plant
3. FSAR for Point Beach Nuclear Plant
4. DBD-01. Auxiliary Feedwater
5. DBD-04. Chemical And Volume Control System

COPY

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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NOTES

- Steps in this procedure may be performed out of order as deemed necessary by the DSS or his designee.
- Entry into EOP-O, REACTOR TRIP OR SAFETY INJECTION, is not required.

- 1 Initiate Manual Reactor Trip For Both Units
 - Unit 1 Reactor - TRIPPED
 - Unit 2 Reactor - TRIPPED
- 2 Ensure Both Units Turbine - TRIPPED
 - Unit 1 Turbine - TRIPPED
 - Unit 2 Turbine - TRIPPED
- 3 Shut Main Steam Isolation Valves
 - 1MS-2018
 - 1MS-2017
 - 2MS-2018
 - 2MS-2017
- 4 Adjust Atmospheric Steam Dump Controllers To 1005 PSIG
 - 1HC-468
 - 1HC-478
 - 2HC-468
 - 2HC-478
- 5 Align Charging Pump Suctions To RWST:
 - a. Open RWST to charging pump suction MOV's
 - 1CV-112B
 - 2CV-112B
 - b. Shut VCT outlet to charging pump suction MOV's
 - 1CV-112C
 - 2CV-112C

CONTROL ROOM INACCESSIBILITY

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
6	Start Turbine-Driven AFW Pumps <ul style="list-style-type: none">• 1P-29• 2P-29	
7	Place Motor-Driven AFW Discharge Valves In - MANUAL PULLOUT AND CLOSE <ul style="list-style-type: none">• AF-4021 for S/G 1B• AF-4022 for S/G 2A	

CAUTION

Placing Main Feed Pump control switches in pull-out will defeat auto start of the Motor Driven AFW pumps.

- 8 Stop Main Feedwater Pumps And Place Control Switches In - AUTO
 - 1P-28A
 - 1P-28B
 - 2P-28A
 - 2P-28B
- 9 Stop Heater Drain Tank Pumps
 - 1P-27A
 - 1P-27B
 - 1P-27C
 - 2P-27A
 - 2P-27B
 - 2P-27C
- 10 Ensure Only One Condensate Pump Running Per Unit
 - o P-25A
 - OR
 - o P-25B
- 11 Evacuate Control Room And Obtain Copies Of This Procedure From The Work Control Center
- 12 Notify CAS Of Control Room Evacuation

CONTROL ROOM INACCESSIBILITY

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
------	--------------------------	-----------------------

13 Dispatch Four Licensed Operators To Perform Local Actions:

- a. Attachment A. Unit 1 AFW PUMP OPERATOR
- b. Attachment B. Unit 2 AFW PUMP OPERATOR
- c. Attachment C. UNIT 1 CHARGING PUMP OPERATOR
- d. Attachment D. UNIT 2 CHARGING PUMP OPERATOR

14 Dispatch Operators To Perform Local Actions:

- a. Attachment E. TURBINE HALL OPERATOR
- b. Attachment F. PAB OPERATOR

15 Direct STA To Report To TSC And Implement Emergency Plan

* 16 Locally Monitor Operating Equipment *
* Until Control Room Can Be Re-Entered *

* 17. Check Control Room - HABITABLE * WHEN Control Room can be re-entered. *
* * THEN go to EOP-0, REACTOR TRIP OR *
* * SAFETY INJECTION. *

-END-

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
------	--------------------------	-----------------------

ATTACHMENT A
(Page 1 of 2)
Unit 1 AFW PUMP OPERATOR

CAUTION

To prevent AFW pump damage, monitor and maintain minimum AFW discharge flow greater than 75 gpm or stop the affected AFW Pump as necessary to control S/G levels.

- | | |
|---|--|
| <p>A1 Check Turbine-Driven AFW Pump -
RUNNING</p> <ul style="list-style-type: none">• 1P-29 | <p>Direct PAB Operator to locally align steam supply to turbine-driven AFW pump:</p> <ul style="list-style-type: none">o Locally open B S/G steam supply valve.• 1MS-2019o Locally open A S/G steam supply valve.• 1MS-2020 |
|---|--|

* A2 Manually Control A S/G Level: *
* *
* a. Engage clutch and throttle *
* 1AF-4001 to maintain S/G level *
* -BETWEEN 300 INCHES AND *
* 330 INCHES *
* *
* • 1LI-460-AA on 1RK-38 *
* • 1LI-460-BA on 1RK-38 *

* A3 Manually Control B S/G Level: *
* *
* a. Engage clutch and throttle *
* 1AF-4000 to maintain S/G level *
* -BETWEEN 300 INCHES AND *
* 330 INCHES *
* *
* • 1LI-470-AA on 1RK-38 *
* • 1LI-470-BA on 1RK-38 *

7/1/02 - 27/1/02

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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ATTACHMENT A
(Page 2 of 2)
Unit 1 AFW PUMP OPERATOR

CAUTION

To prevent AFW pump damage, monitor and maintain minimum AFW discharge flow or stop the affected AFW pump as necessary to control S/G levels.

- o P-38A minimum flow - GREATER THAN 50 GPM
- o P-29 minimum flow - GREATER THAN 75 GPM

A4 Maintain S/G Levels Between
300 Inches And 330 Inches Using
Turbine-Driven AFW Pump

IF level can NOT be maintained using
turbine-driven AFW pump, THEN
maintain A S/G level using
motor-driven AFW pump as follows:

- a. At N-01, place P-38A in - LOCAL.
- b. At N-01, depress start
pushbutton.
- c. Open AF-4023.
- d. Throttle 1AF-31 to maintain A S/G
level - BETWEEN 300 INCHES AND
330 INCHES.
 - 1LI-460-AA on 1RK-38
 - 1LI-460-BA on 1RK-38
- e. Inform DOS that B S/G atmospheric
steam dump should be isolated.
 - 1MS-2015

A5 Inform DOS That S/G Levels Are -
BETWEEN 300 INCHES AND 330 INCHES

-END-

TCN 2002 - 0865

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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ATTACHMENT B
(Page 1 of 2)
Unit 2 AFW PUMP OPERATOR

CAUTION

To prevent AFW pump damage, monitor and maintain minimum AFW discharge flow greater than 75 gpm or stop the affected AFW Pump as necessary to control S/G levels.

- | | |
|---|--|
| <p>B1 Check Turbine-Driven AFW Pump -
RUNNING</p> <ul style="list-style-type: none">• 2P-29 | <p>Direct PAB Operator to locally align steam supply to turbine-driven AFW pump:</p> <ul style="list-style-type: none">o Locally open B S/G steam supply valve.• 2MS-2019o Locally open A S/G steam supply valve.• 2MS-2020 |
|---|--|

* B2 Manually Control A S/G Level: *

* *

* a. Engage clutch and throttle *

* 2AF-4001 to maintain S/G level *

* -BETWEEN 300 INCHES AND *

* 330 INCHES *

* *

- 2LI-460-AA on 2RK-38 *
- 2LI-460-BA on 2RK-38 *

* B3 Manually Control B S/G Level: *

* *

* a. Engage clutch and throttle *

* 2AF-4000 to maintain S/G level *

* -BETWEEN 300 INCHES AND *

* 330 INCHES *

* *

- 2LI-470-AA on 2RK-38 *
- 2LI-470-BA on 2RK-38 *

7-1-01 2002-07/0

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

ATTACHMENT B
(Page 2 of 2)
Unit 2 AFW PUMP OPERATOR

CAUTION

To prevent AFW pump damage, monitor and maintain minimum AFW discharge flow or stop the affected AFW pump as necessary to control S/G levels.

- o P-38B minimum flow - GREATER THAN 50 GPM
- o P-29 minimum flow - GREATER THAN 75 GPM

B4 Maintain S/G Levels Between
300 Inches And 330 Inches Using
Turbine-Driven AFW Pump

IF level can NOT be maintained using
turbine-driven AFW pump, THEN
maintain B S/G level using
motor-driven AFW pump as follows:

- a. At N-02, place P-38B in - LOCAL.
- b. At N-02, depress start
pushbutton.
- c. Open AF-4020.
- d. Throttle AF-45 to maintain B S/G
level - BETWEEN 300 INCHES AND
330 INCHES.
 - 2LI-470-AA on 2RK-38
 - 2LI-470-BA on 2RK-38
- e. Inform DOS that A S/G atmospheric
steam dump should be isolated.
 - 2MS-2016

B5 Inform DOS That S/G Levels Are -
BETWEEN 300 INCHES AND 330 INCHES

-END-

7/12/2009 - 08:10

CONTROL ROOM INACCESSIBILITY

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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ATTACHMENT C
(Page 1 of 2)
UNIT 1 CHARGING PUMP OPERATOR

- | | |
|---|---|
| <p>C1 Ensure Charging Pump Suction -
ALIGNED TO RWST:</p> <ul style="list-style-type: none">• RWST to charging pump suction MOV
- OPEN• 1CV-112B• VCT outlet to charging pump
suction MOV - SHUT• 1CV-112C | <p>Align charging pump suction to RWST:</p> <ul style="list-style-type: none">a. Open RWST to charging pump
suction.<ul style="list-style-type: none">• 1CV-358b. Shut VCT to charging pump suction
MOV.<ul style="list-style-type: none">• 1CV-112C |
|---|---|

CAUTION

Placing pressurizer heaters in local defeats heater low level cutout.

- *****
- | | |
|--|--|
| <p>* C2 Check PZR Pressure - BETWEEN
* 2200 PSIG AND 2250 PSIG
*
* • PI-449B
*
*
*
*
*****</p> | <p>Locally operate back-up heaters to
maintain PZR pressure - BETWEEN
2200 PSIG AND 2250 PSIG

o Bank C
o Bank D
*****</p> |
|--|--|

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
------	--------------------------	-----------------------

ATTACHMENT C
 (Page 2 of 2)
 UNIT 1 CHARGING PUMP OPERATOR

C3 Check If Letdown Should Be Established:

a. Check PZR level - GREATER THAN 16% AND RISING

- LI-433C

b. Establish letdown:

1) Inside 1B52-426M, locally open letdown line isolation

- 1RC-427

2) Locally open letdown isolations as necessary

- o 1CV-200A
- o 1CV-200B
- o 1CV-200C

a. WHEN PZR level greater than 16%. THEN do Step C3.b.

b. Perform the following:

a) Operate charging pumps as necessary to maintain PZR level - BETWEEN 20% AND 45%

- OI-15, CHARGING PUMP LOCAL CONTROL STATION OPERATION

b) Go to Step C5.

NOTES

- PZR level may require time to stabilize after letdown restoration.
- Charging pumps should remain running in auto/remote if possible.

 * C4 Check PZR Level - BETWEEN 20% AND Operate charging pumps as necessary, *
 * 25% to maintain PZR level- BETWEEN *
 * 20% AND 45% *
 * • LI-433C *
 * • OI 15, CHARGING PUMP LOCAL CONTROL *
 * STATION OPERATION *

C5 Inform DOS Of The Following:

- PZR level
- PZR pressure

CONTROL ROOM INACCESSIBILITY

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
------	--------------------------	-----------------------

ATTACHMENT D
 (Page 2 of 2)
 UNIT 2 CHARGING PUMP OPERATOR

D3 Check If Letdown Should Be Established:

a. Check PZR level - GREATER THAN 16% AND RISING

- LI-433C

b. Establish letdown:

1) Inside 2B52-427J, locally open letdown line isolation

- 2RC-427

2) Locally open letdown isolations as necessary

- o 2CV-200A
- o 2CV-200B
- o 2CV-200C

a. WHEN PZR level greater than 16%, THEN do Step D3.b.

b. Perform the following:

a) Operate charging pumps as necessary to maintain Pzr level - BETWEEN 20% AND 45%

- OI-15, CHARGING PUMP LOCAL CONTROL STATION OPERATION

b) Go to Step D5.

NOTES

- PZR level may require time to stabilize after letdown restoration.
- Charging pumps should remain running in auto if possible.

```

*****
* D4 Check PZR Level - BETWEEN Operate charging pumps as necessary *
* 20% AND 25% to maintain PZR level- BETWEEN *
* • LI-433C 20% AND 45% *
* • OI 15, CHARGING PUMP LOCAL CONTROL *
* STATION OPERATION *
*****
  
```

D5 Inform DOS Of The Following:

- PZR level
- PZR pressure

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
------	--------------------------	-----------------------

ATTACHMENT E
(Page 1 of 3)
TURBINE HALL OPERATOR

NOTE

All or some of the actions performed in this attachment may have been performed in the control room prior to evacuation.

- | | |
|---|--|
| <p>E1 Check Both Reactors Tripped Prior To Evacuating Control Room</p> <ul style="list-style-type: none">• Unit 1 Reactor - TRIPPED• Unit 2 Reactor - TRIPPED | <p>Perform the following:</p> <ul style="list-style-type: none">a. Open reactor trip breakers for BOTH UNITS.<ul style="list-style-type: none">• Unit 1 reactor trip breakers- OPEN• Unit 2 reactor trip breakers- OPENb. Open reactor trip bypass breakers for BOTH UNITS.<ul style="list-style-type: none">• Unit 1 reactor trip bypass breakers - OPEN• Unit 2 reactor trip bypass breakers - OPEN |
| <p>E2 Check Both Turbines Tripped Prior To Evacuating Control Room</p> <ul style="list-style-type: none">• Unit 1 Turbine - TRIPPED• Unit 2 Turbine - TRIPPED | <p>Perform the following:</p> <ul style="list-style-type: none">a. At Unit 1 Front Standard, rotate trip lever to trip position.b. At Unit 2 Front Standard, rotate trip lever to trip position. |
| <p>E3 Inform DOS Both Reactors And Both Turbines Are Tripped</p> | |
| <p>E4 At 1A01, Check The Following Pumps Stopped Prior To Evacuating The Control Room:</p> <ul style="list-style-type: none">• 1P-28A Steam Generator Feed Pump• 1P-27A Heater Drain Tank Pump• 1P-27C Heater Drain Tank Pump | <p>Locally open breakers as necessary per Attachment H. LOCAL BREAKER OPERATION.</p> <ul style="list-style-type: none">• 1A52-05• 1A52-08• 1A52-09 |

CONTROL ROOM INACCESSIBILITY

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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ATTACHMENT E
(Page 2 of 3)
TURBINE HALL OPERATOR

- | | | |
|-----|--|--|
| E5 | At 1A02, Check The Following Pumps Stopped Prior To Evacuating The Control Room:
<ul style="list-style-type: none">• 1P-28B Steam Generator Feed Pump• 1P-27B Heater Drain Tank Pump | Locally open breakers as necessary per Attachment H, LOCAL BREAKER OPERATION.
<ul style="list-style-type: none">• 1A52-13• 1A52-10 |
| E6 | At 2A01, Check The Following Pumps Stopped Prior To Evacuating The Control Room:
<ul style="list-style-type: none">• 2P-28A Steam Generator Feed Pump• 2P-27A Heater Drain Tank Pump• 2P-27C Heater Drain Tank Pump | Locally open breakers as necessary per Attachment H, LOCAL BREAKER OPERATION.
<ul style="list-style-type: none">• 2A52-23• 2A52-20• 2A52-19 |
| E7 | At 2A02, Check The Following Pumps Stopped Prior To Evacuating The Control Room:
<ul style="list-style-type: none">• 2P-28B Steam Generator Feed Pump• 2P-27B Heater Drain Tank Pump | Locally open breakers as necessary per Attachment H, LOCAL BREAKER OPERATION.
<ul style="list-style-type: none">• 2A52-30• 2A52-33 |
| E8 | At 1A01 or 1A02, Check One Unit 1 Condensate Pump Breaker - OPEN
<ul style="list-style-type: none">o 1A52-07 for 1P-25A <p style="text-align: center;"><u>OR</u></p> <ul style="list-style-type: none">o 1A52-11 for 1P-25B | Locally open breaker per Attachment H, LOCAL BREAKER OPERATION. |
| E9 | At 2A01 or 2A02, Check One Unit 2 Condensate Pump Breaker - OPEN
<ul style="list-style-type: none">o 2A52-21 for 2P-25A <p style="text-align: center;"><u>OR</u></p> <ul style="list-style-type: none">o 2A52-32 for 2P-25B | Locally open breaker per Attachment H, LOCAL BREAKER OPERATION. |
| E10 | Inform DOS Of The Following:
<ul style="list-style-type: none">• Main Feedwater pumps - STOPPED• Heater Drain Tank pumps - STOPPED• One Condensate pump - RUNNING | |

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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ATTACHMENT E
(Page 3 of 3)
TURBINE HALL OPERATOR

- | | | |
|-----|---|--|
| E11 | Check CST Level - GREATER THAN 15 FT | Direct Water Treatment Operator to commence filling CST. |
| | <ul style="list-style-type: none">• LI-4025 for T-24A• LI-4031 for T-24B | |

-END-

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
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ATTACHMENT F
(Page 1 of 1)
PAB OPERATOR

- | | | |
|----|--|---|
| F1 | Check Unit 1 MSIVs Were Shut Prior To Control Room Evacuation <ul style="list-style-type: none">• 1MS-2018 for S/G 1A• 1MS-2017 for S/G 1B | Shut Unit 1 MSIVs as follows: <ul style="list-style-type: none">a. At 1RK-33, depress both pushbuttons.<ul style="list-style-type: none">• 1MS PB-2018A, train A• 1MS PB-2018B, train Bb. At 1RK-34, depress both pushbuttons.<ul style="list-style-type: none">• 1MS PB-2017A, train A• 1MS PB-2017B, train B |
| F2 | Check Unit 2 MSIVs Were Shut Prior To Control Room Evacuation <ul style="list-style-type: none">• 2MS-2018 for S/G 2A• 2MS-2017 for S/G 2B | Shut Unit 2 MSIVs as follows: <ul style="list-style-type: none">a. At 2RK-33, depress both pushbuttons.<ul style="list-style-type: none">• 2MS PB-2018A, train A• 2MS PB-2018B, train Bb. At 2RK-34, depress both pushbuttons.<ul style="list-style-type: none">• 2MS PB-2017A, train A• 2MS PB-2017B, train B |
| F3 | De-Energize Unit 1 Motor-Driven AFW Pump Discharge MOV's <ul style="list-style-type: none">• Open Bkr. 1B52-328F for AF-4023• Open Bkr. 1B52-428C for AF-4021 | |
| F4 | De-Energize Unit 2 Motor-Driven AFW Pump Discharge MOV's <ul style="list-style-type: none">• Open Bkr. 2B52-328C for AF-4022• Open Bkr. 2B52-428F for AF-4020 | |
| F5 | Inform DOS Unit 1 And Unit 2 Motor-Driven AFW Pump Discharge MOV's - DE-ENERGIZED | |

-END-

CONTROL ROOM INACCESSIBILITY

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
------	--------------------------	-----------------------

ATTACHMENT G
(Page 1 of 1)
DOS CHECKLIST

G1 Check the following Unit 1 actions completed:

COMPLETED	Unit 1 ACTION PERFORMED
()	Reactor tripped
()	Motor-Driven AFW pump discharge valves breakers open
()	PZR level - 20% to 45%
()	PZR pressure - 2200 PSIG to 2250 PSIG
()	Turbine tripped
()	Both Main Feedwater Pumps stopped
()	All Heater Drain Pumps stopped
()	One Condensate Pump running
()	All S/G levels between 300 inches and 330 inches

G2 Check the following Unit 2 actions completed:

COMPLETED	Unit 2 ACTION PERFORMED
()	Reactor tripped
()	Motor-Driven AFW pump discharge valves breakers open
()	PZR level - 20% to 45%
()	PZR pressure - 2200 PSIG to 2250 PSIG
()	Turbine tripped
()	Both Main Feedwater Pumps stopped
()	All Heater Drain Pumps stopped
()	One Condensate Pump running
()	All S/G levels between 300 inches and 330 inches

-END-

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
------	--------------------------	-----------------------

ATTACHMENT H
(Page 1 of 1)
LOCAL BREAKER OPERATION

- H1 Local Opening Of A 4160 Vac Breaker:
- a. Open breaker cubicle door.
 - b. Check mechanical indicator indicates closed.
 - c. Open breaker by performing one of the following:
 - o Depress trip/open push plate.
 - OR
 - o Depress trip/open button.
 - OR
 - o Lift red tab on open coil.
 - d. Check mechanical indicator indicates open.
 - e. Close breaker cubicle door.
- H2 Local Opening Of A 480 Vac Breaker:
- a. Check mechanical indicator indicates closed.
 - b. Depress square trip push button.
 - c. Check mechanical indicator indicates open.

-END-