

March 20, 2003

Mr. David A. Christian  
Senior Vice President and  
Chief Nuclear Officer  
Virginia Electric and Power Company  
5000 Dominion Blvd.  
Glen Allen, Virginia 23060

SUBJECT: ISSUANCE OF RENEWED FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7 FOR NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2, AND RENEWED FACILITY OPERATING LICENSE NOS. DPR-32 AND DPR-37 FOR SURRY POWER STATION, UNIT NOS. 1 AND 2

Dear Mr. Christian:

The U.S. Nuclear Regulatory Commission (NRC) has issued Renewed Facility Operating License Nos. NPF-4 (Enclosure 1) and NPF-7 (Enclosure 2) together with the Technical Specifications, Appendix A (Enclosure 3), and the Environmental Protection Plan, Appendix B, (Enclosure 4), for the North Anna Power Station, Unit Nos. 1 and 2 (North Anna, Units 1 and 2). The NRC has also issued Renewed Facility Operating License Nos. DPR-32 (Enclosure 5) and DPR-37 (Enclosure 6) together with the Technical Specifications, Appendix A (Enclosure 7), for the Surry Power Station, Unit Nos. 1 and 2 (Surry, Units 1 and 2). These renewed facility operating licenses have been issued on the basis of our review of your applications dated May 29, 2001, as supplemented by letters dated November 30, 2001, January 4 (two letters), January 16, January 17, February 1 (two letters), February 5, May 22 (two letters), July 25, August 23, October 1, October 15, November 4, and December 2, 2002. The Technical Specifications for North Anna, Units 1 and 2, and Surry, Units 1 and 2, were not amended as a result of our review.

Renewed Facility Operating License No. NPF-4 (the renewed license for North Anna, Unit 1) expires at midnight, April 1, 2038. Renewed Facility Operating License No. NPF-7 (the renewed license for North Anna, Unit 2) expires at midnight, August 21, 2040.

Renewed Facility Operating License No. DPR-32 (the renewed license for Surry, Unit 1) expires at midnight, May 25, 2032. Renewed Facility Operating License No. DPR-37 (the renewed license for Surry, Unit 2), expires at midnight, January 29, 2033.

The technical basis for issuing the renewed licenses is set forth in NUREG-1766, "Safety Evaluation Report Related to the License Renewal of North Anna Power Station, Units 1 and 2, and Surry Power Station, Units 1 and 2," dated December 2002. The results of the environmental reviews related to the issuance of the renewed licenses are contained in NUREG-1437, Supplement 6, "Generic Environmental Impact Statement for License Renewal of Nuclear Plants, Supplement 6, Regarding Surry Power Station, Units 1 and 2," dated November 30, 2002, and in NUREG-1437, Supplement 7, "Generic Environmental Impact

D. Christian

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Statement for License Renewal of Nuclear Plants, Supplement 7, Regarding North Anna Power Station, Units 1 and 2," dated November 30, 2002.

Enclosure 8 is a copy of the related *Federal Register* notice of issuance of the renewed licenses. The original has been sent to the Office of the Federal Register for publication.

Sincerely,

**/RA/**

Omid Tabatabai, Project Manager  
License Renewal Section  
License Renewal and Environmental Impacts Program  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Docket Nos.: 50-338, 50-339, 50-280, and 50-281

Enclosures: 1. Renewed Facility Operating License No. NPF-4  
2. Renewed Facility Operating License No. NPF-7  
3. Technical Specifications, Appendix A, for North Anna, Units 1 and 2  
4. Environmental Protection Plan, Appendix B, for North Anna, Units 1 and 2  
5. Renewed Facility Operating License No. DPR-32  
6. Renewed Facility Operating License No. DPR-37  
7. Technical Specifications, Appendix A, for Surry, Units 1 and 2  
8. *Federal Register* Notice

cc w/encls 1, 2, 5, 6, and 8: See next page

D. Christian

- 2 -

License Renewal of Nuclear Plants, Supplement 7, Regarding North Anna Power Station, Units 1 and 2," dated November 30, 2002.

Enclosure 8 is a copy of the related *Federal Register* notice of issuance of the renewed licenses. The original has been sent to the Office of the Federal Register for publication.

Sincerely,

**/RA/**

Omid Tabatabai, Project Manager  
License Renewal Section  
License Renewal and Environmental Impacts Program  
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Office of Nuclear Reactor Regulation

Docket Nos.: 50-338, 50-339, 50-280, and 50-281

- Enclosures:
1. Renewed Facility Operating License No. NPF-4
  2. Renewed Facility Operating License No. NPF-7
  3. Technical Specifications, Appendix A, for North Anna, Units 1 and 2
  4. Environmental Protection Plan, Appendix B, for North Anna, Units 1 and 2
  5. Renewed Facility Operating License No. DPR-32
  6. Renewed Facility Operating License No. DPR-37
  7. Technical Specifications, Appendix A, for Surry, Units 1 and 2
  8. *Federal Register* Notice

cc w/encls 1, 2, 5, 6, and 8: See next page

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O. Tabatabai

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S. Collins

W. Borchardt

B. Sheron

C. Grimes

G. Hill (2 copies/docket no.)

ACRS

J. Munday, RII

Because the Technical Specifications for the North Anna Units 1 and 2, and Surry Units 1 and 2 are simply being reissued with the renewed licenses, only Virginia Power and Electric Company will receive the package with the Technical Specifications attached. If a hard copy of the Technical Specifications is needed, please contact the project manager, Omid Tabatabai, at 301 415-3738.

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

RENEWED FACILITY OPERATING LICENSE

Renewed License No. NPF-4

1. The U.S. Nuclear Regulatory Commission (the Commission) having previously made the findings set forth in License No. NPF-4 issued April 1, 1978, has now found that:
  - A. The application to renew License No. NPF-4 filed by the Virginia Electric and Power Company (VEPCO or the licensee) and the Old Dominion Electric Cooperative (ODEC), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;
  - B. Actions have been identified and have been or will be taken with respect to (1) managing the effects of aging during the period of extended operation on the functionality of structures and components that have been identified to require review under 10 CFR 54.21(a)(1), and (2) time-limited aging analyses that have been identified to require review under 10 CFR 54.21(c), such that there is reasonable assurance that the activities authorized by this renewed license will continue to be conducted in accordance with the current licensing basis, as defined in 10 CFR 54.3, for North Anna Power Station, Unit No. 1, and that any changes made to the plant's current licensing basis in order to comply with 10 CFR 54.29(a) are in accord with the Act and the Commission's regulations;
  - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - D. There is reasonable assurance: (i) that the activities authorized by this renewed operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;

- E. VEPCO is technically and financially qualified to engage in the activities authorized by this renewed operating license in accordance with the rules and regulations of the Commission;
  - F. VEPCO and the ODEC have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
  - G. The issuance of this renewed operating license will not be inimical to the common defense and security or to the health and safety of the public;
  - H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of Renewed Facility Operating License No. NPF-4, subject to the conditions for protection of the environment set forth herein, is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied;
  - I. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this renewed license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40, and 70, including 10 CFR Sections 30.33, 40.32, 70.23, and 70.31; and
  - J. ODEC is a partial financial owner of the facility and will not operate the facility.
2. On the basis of the foregoing findings regarding this facility, Facility Operating License No. NPF-4, issued April 1, 1978, is superseded by Renewed Facility Operating License No. NPF-4, which is hereby issued to VEPCO and ODEC to read as follows:
- A. This renewed license applies to the North Anna Power Station, Unit No. 1, a pressurized water reactor and associated equipment (the facility), owned by VEPCO and ODEC. The facility is located near Mineral, in Louisa County, Virginia, and is described in the "Updated Final Safety Analysis Report" and the Environmental Report as supplemented and amended.
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:
    - (1) Pursuant to Section 103 of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," VEPCO and ODEC to possess and VEPCO to use and operate the facility at the designated location in Louisa County, Virginia, in accordance with the procedures and limitations set forth in this renewed license;

- (2) Pursuant to the Act and 10 CFR Part 70, VEPCO to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report;
- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or component; and
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, VEPCO to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

VEPCO is authorized to operate the North Anna Power Station, Unit No. 1, at reactor core power levels not in excess of 2893 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 235, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following the issuance of the condition or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license, supported by a favorable evaluation by the Commission:

- a. VEPCO may use up to four (4) fuel assemblies containing advanced zirconium based alloys as described in the licensee's submittal dated September 4, 1996, as supplemented February 3, 1997.
- b. If VEPCO plans to remove or to make significant changes in the normal operation of equipment that controls the amount of radioactivity in effluents from the North Anna Power Station, the Commission shall be notified in writing regardless of whether the change affects the amount of radioactivity in the effluents.
- c. VEPCO shall implement a procedure that will prohibit entry into an extended Emergency Diesel Generator Outage Time (14 days), for scheduled maintenance purposes, if severe weather conditions are expected, as described in the licensee's application dated June 25, 1998, and evaluated in the staff's Safety Evaluation dated August 26, 1998.
- d. The licensee is authorized to relocate certain Technical Specification requirements previously included in Appendix A to licensee-controlled documents, as described in Table R, Relocated Specifications and Removed Details, attached to the NRC staff's Safety Evaluation enclosed with Amendment No. 231. These requirements shall be relocated to the appropriate documents no later than September 2, 2002.
- e. The schedule for performing surveillance requirements (SRs) that are new or revised in Amendment No. 231 shall be as follows:

For SRs that are new in this amendment, the first performance is due at the end of the first surveillance interval that begins on the date of implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being reduced, the first reduced surveillance interval begins upon completion of the first surveillance performed after implementation of this amendment.

For SRs that existed prior to this amendment that have modified acceptance criteria, the first performance subject to the modified acceptance criteria is due at the end of the first surveillance interval that began on the date the surveillance was last performed prior to the implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being extended, the first extended surveillance interval begins upon completion of the last surveillance performed prior to implementation of this amendment.

This license condition is effective as of its date of issuance.

- (4) The licensee is authorized to receive from the Surry Power Station, Unit Nos. 1 and 2, possess, and store irradiated Surry Power Station fuel assemblies containing special nuclear material, enriched to not more than 4.1 percent by weight U-235, subject to the following conditions:
- a. Surry Power Station fuel assemblies may not be placed in North Anna Power Station, Unit Nos. 1 and 2, reactors.
  - b. Irradiated fuel shipped to North Anna Power Station shall have been removed from the Surry Power Station reactors no less than 730 days prior to shipment.
  - c. No more than 500 Surry Power Station irradiated fuel assemblies shall be received for storage at the North Anna Power Station, Unit Nos. 1 and 2, spent fuel pool.

(5) Environmental Protection Plan

The Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 197, is hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Environmental Protection Plan.

D. Fire Protection

VEPCO shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility and as approved in the SER dated February 1979 subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not

adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

E. Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "North Anna Power Station Security Plan," with revisions submitted through February 24, 1988; "North Anna Power Station Guard Training and Qualification Plan," with revisions submitted through May 14, 1987; and "North Anna Power Station Safeguard Contingency Plan," with revisions submitted through January 9, 1987. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

F. Updated Final Safety Analysis Report

- a. The licensee's Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on July 25, October 1, November 4, and December 2, 2002, describes certain future inspection activities to be completed before the period of extended operation. The licensee shall complete these activities no later than April 1, 2018, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.
- b. The Updated Final Safety Analysis Report supplement as revised on July 25, October 1, November 4, and December 2, 2002, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following the issuance of this renewed license. Until that update is complete, the licensee may make changes to the programs described in such supplement without prior Commission approval, provided that the licensee evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

- G. This renewed license is effective as of the date of issuance and shall expire at midnight on April 1, 2038.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

Samuel J. Collins, Director  
Office of Nuclear Reactor Regulation

Attachments:

1. Appendix A, Technical Specifications
2. Appendix B, Environmental Protection Plan

Date of Issuance: March 20, 2003

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

RENEWED FACILITY OPERATING LICENSE

Renewed License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) having previously made the findings set forth in License NPF-7 issued on August 21, 1980, has now found that:
  - A. The application to renew License No. NPF-7 filed by Virginia Electric and Power Company (VEPCO or the licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations as set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
  - B. Actions have been identified and have been or will be taken with respect to (1) managing the effects of aging during the period of extended operation on the functionality of structures and components that have been identified to require review under 10 CFR 54.21(a)(1), and (2) time-limited aging analyses that have been identified to require review under 10 CFR 54.21(c), such that there is reasonable assurance that the activities authorized by this renewed license will continue to be conducted in accordance with the current licensing basis, as defined in 10 CFR 54.3, for North Anna Power Station, Unit No. 2, and that any changes made to the plant's current licensing basis in order to comply with 10 CFR 54.29(a) are in accord with the Act and the Commission's regulations;
  - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - D. There is reasonable assurance: (i) that the activities authorized by this renewed operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - E. VEPCO is technically and financially qualified to engage in the activities authorized by this renewed operating license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;

- F. VEPCO and the Old Dominion Electric Cooperative (ODEC) have satisfied the applicable provisions of 10 CFR Part 140 of the Commission's regulations;
  - G. The issuance of this renewed operating license will not be inimical to the common defense and security or to the health and safety of the public;
  - H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of Renewed Facility Operating License No. NPF-7, subject to the conditions for protection of the environment set forth herein, is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied;
  - I. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this renewed license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40, and 70; and
  - J. ODEC is a partial financial owner of the facility and will not operate the facility.
2. On the basis of the foregoing findings regarding this facility, Facility Operating License No. NPF-7, issued on August 21, 1980, is superceded by Renewed Facility Operating License No. NPF-7, which is hereby issued to VEPCO and the ODEC to read as follows:
- A. This renewed license applies to the North Anna Power Station, Unit No. 2, a pressurized water nuclear reactor and associated equipment (the facility), owned by VEPCO and ODEC. The facility is located near Mineral, in Louisa County, Virginia, and is described in VEPCO's Updated Final Safety Analysis Report and Environmental Report as supplemented and amended.
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:
    - (1) Pursuant to Section 103 of the Act and 10 CFR Part 50, VEPCO and ODEC to possess and VEPCO to use and operate the facility at the designated location in Louisa County, Virginia, in accordance with the procedures and limitations set forth in this renewed license;
    - (2) Pursuant to the Act and 10 CFR Part 70, VEPCO to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in VEPCO's Updated Final Safety Analysis Report;

- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
  - (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations as set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level  
  
VEPCO is authorized to operate the facility at steady state reactor core power levels not in excess of 2893 megawatts (thermal).
  - (2) Technical Specifications  
  
The Technical Specifications contained in Appendix A, as revised through Amendment No. 216, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.
  - (3) Additional Conditions  
  
The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following the issuance of the condition or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license supported by a favorable evaluation by the Commission:
    - a. If VEPCO plans to remove or to make significant changes in the normal operation of equipment that controls the amount of radioactivity in effluents from the North Anna Power Station, the

NRC shall be notified in writing regardless of whether the change affects the amount of radioactivity in the effluents.

VEPCO shall report any violations of this requirement within 24 hours by telephone and confirmed by telegram, mailgram, or facsimile transmission to the Director of the Regional Office, or his designate, no later than the first working day following the violation, with a written follow-up report within 14 days.

- b. The licensee shall implement a procedure that will prohibit entry into an extended Emergency Diesel Generator Outage Time (14 days), for scheduled maintenance purposes, if severe weather conditions are expected, as described in the licensee's application dated June 25, 1998, and evaluated in the staff's Safety Evaluation dated August 26, 1998.
- c. The licensee is authorized to relocate certain Technical Specification requirements previously included in Appendix A to licensee-controlled documents, as described in Table R, Relocated Specifications and Removed Details, attached to the NRC staff's Safety Evaluation enclosed with Amendment No. 212. These requirements shall be relocated to the appropriate documents no later than September 2, 2002.
- d. The schedule for performing surveillance requirements (SRs) that are new or revised in Amendment No. 212 shall be as follows:

For SRs that are new in this amendment, the first performance is due at the end of the first surveillance interval that begins on the date of implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being reduced, the first reduced surveillance interval begins upon completion of the first surveillance performed after implementation of this amendment.

For SRs that existed prior to this amendment that have modified acceptance criteria, the first performance subject to the modified acceptance criteria is due at the end of the first surveillance interval that began on the date the surveillance was last performed prior to the implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being extended, the first extended surveillance

interval begins upon completion of the last surveillance performed prior to implementation of this amendment.

This license condition is effective as of its date of issuance.

- e. VEPCO may operate one lead test assembly containing advanced zirconium-based alloys for one cycle, to a lead rod burnup not exceeding 75,000 MWD/MTU, as described in the licensee's submittal dated February 11, 2002.
- (4) The licensee is authorized to receive from the Surry Nuclear Power Station, Unit Nos. 1 and 2, possess, and store irradiated Surry Power Station fuel assemblies containing special nuclear material, enriched to not more than 4.1 percent by weight U-235, subject to the following conditions:
- a. Surry Power Station fuel assemblies may not be placed in North Anna Power Station, Unit Nos. 1 and 2, reactors.
  - b. Irradiated fuel shipped to North Anna Power Station shall have been removed from the Surry Power Station reactors no less than 730 days prior to shipment.
  - c. No more than 500 Surry Power Station irradiated fuel assemblies shall be received for storage at the North Anna Power Station, Unit Nos. 1 and 2, spent fuel pool.

(5) Environmental Protection Plan

The Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 178, is hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Environmental Protection Plan.

D. Fire Protection

VEPCO shall implement and maintain in effect all provisions of the approved fire protection program as described in the licensee's Updated Final Safety Analysis Report for the facility and as approved in the SER dated February 1979 subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

E. Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "North Anna Power Station Physical Security Plan," with revisions submitted through February 24, 1988; "North Anna Power Station Guard Training and Qualification Plan," with revisions submitted through May 14, 1987; and "North Anna Power Station Safeguards Contingency Plan," with revisions submitted through January 9, 1987. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

F. Updated Final Safety Analysis Report

- (1) The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on July 25, October 1, November 4, and December 2, 2002, describes certain future inspection activities to be completed before the period of extended operation. The licensee shall complete these activities no later than August 21, 2020, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.
- (2) The Updated Final Safety Analysis Report supplement as revised on July 25, October 1, November 4, and December 2, 2002, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following the issuance of this renewed license. Until that update is complete, the licensee may make changes to the programs described in such supplement without prior Commission approval, provided that the licensee evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

- G. This renewed license is effective as of the date of issuance and shall expire at midnight on August 21, 2040.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

Samuel J. Collins, Director  
Office of Nuclear Reactor Regulation

Attachments:

1. Appendix A, Technical Specifications
2. Appendix B, Environmental Protection Plan

Date of Issuance: March 20, 2003

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

RENEWED FACILITY OPERATING LICENSE

Renewed License No. DPR-32

The Nuclear Regulatory Commission (the Commission) having previously made the findings set forth in License DPR-32 issued May 25, 1972, has now found that:

- a. The application to renew License No. DPR-32 filed by the Virginia Electric and Power Company (VEPCO or the licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the rules and regulations of the Commission set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
- b. Actions have been identified and have been or will be taken with respect to (1) managing the effects of aging during the period of extended operation on the functionality of structures and components that have been identified to require review under 10 CFR 54.21(a)(1), and (2) time-limited aging analyses that have been identified to require review under 10 CFR 54.21(c), such that there is reasonable assurance that the activities authorized by this renewed license will continue to be conducted in accordance with the current licensing basis, as defined in 10 CFR 54.3, for Surry Power Station, Unit No. 1, and that any changes made to the plant's current licensing basis in order to comply with 10 CFR 54.29(a) are in accord with the Act and the Commission's regulations;
- c. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
- d. There is reasonable assurance: (i) that the activities authorized by this renewed operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
- e. VEPCO is technically and financially qualified to engage in the activities authorized by this renewed operating license in accordance with the rules and regulations of the Commission;
- f. The applicable provisions of 10 CFR Part 140 have been satisfied;

- g. The issuance of this renewed license will not be inimical to the common defense and security or to the health and safety of the public.

On the basis of the foregoing findings regarding this facility, Facility Operating License No. DPR-32, issued May 25, 1972, is superceded by Renewed Facility Operating License No. DPR-32, which is hereby issued to the VEPCO to read as follows:

1. This renewed license applies to the Surry Power Station, Unit No. 1, a pressurized, light water moderated and cooled reactor, and associated steam generators and electric generating equipment (the facility). The facility is located on the licensee's 840-acre site on a point of land called Gravel Neck on the James River, approximately 14 miles northwest of Newport News and 25 miles northwest of Norfolk, Virginia, and is described in the Updated Final Safety Analysis Report.
2. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses the licensee:
  - A. Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility at the designated location in Surry County, Virginia, in accordance with the procedures and limitations set forth in this renewed license;
  - B. Pursuant to the Act and 10 CFR Parts 40 and 70, to receive, possess, and use at any time, source and special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report;
  - C. Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use at any time, any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  - D. Pursuant to the Act and 10 CFR Parts, 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
  - E. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2546 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 234, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. Reports

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.

E. Deleted by Amendment 65

F. Deleted by Amendment 71

G. Deleted by Amendment 227

H. Deleted by Amendment 227

I. Fire Protection

The licensee shall implement and maintain in effect the provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report and as approved in the SER dated September 19, 1979, (and Supplements dated May 29, 1980, October 9, 1980, December 18, 1980, February 13, 1981, December 4, 1981, April 27, 1982, November 18, 1982, January 17, 1984, February 25, 1988, and

July 23, 1992), and the Safety Evaluation issued December 16, 1998, for Technical Specification Amendment No. 217 subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

J. Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Surry Power Station Physical Security Plan," with revisions submitted through February 24, 1988; "Surry Power Station Guard Training and Qualification Plan," with revisions submitted through May 29, 1987; and "Surry Power Station Safeguards Contingency Plan," with revisions submitted through January 9, 1987. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

K. Deleted by Amendment 227

L. Deleted by Amendment 227

M. Deleted by Amendment 227

N. Deleted by Amendment No. 203

O. Deleted by Amendment 227

P. Updated Final Safety Analysis Report

(1) The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on July 25, October 1, November 4, and December 2, 2002, describes certain future inspection activities to be completed before the period of extended operation. The licensee shall complete these activities no later than May 25, 2012, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

- (2) The Updated Final Safety Analysis Report supplement as revised on July 25, October 1, November 4, and December 2, 2002, shall be included in the next scheduled update to the licensee's Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following the issuance of this renewed license. Until that update is complete, the licensee may make changes to the programs described in such supplement without prior Commission approval, provided that the licensee evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59, and otherwise complies with the requirements in that section.
4. This renewed license is effective as of the date of issuance, and shall expire at midnight on May 25, 2032.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

Samuel J. Collins, Director  
Office of Nuclear Reactor Regulation

Attachment: Appendix A, Technical Specifications

Date of Issuance: March 20, 2003

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

RENEWED FACILITY OPERATING LICENSE

Renewed License No. DPR-37

The Nuclear Regulatory Commission (the Commission) having previously made the findings set forth in License No. DPR-37 issued January 29, 1973, has now found that:

- a. The application to renew License No. DPR-37 filed by Virginia Electric and Power Company (the licensee), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the rules and regulations of the Commission as set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
- b. Actions have been identified and have been or will be taken with respect to (1) managing the effects of aging during the period of extended operation on the functionality of structures and components that have been identified to require review under 10 CFR 54.21(a)(1), and (2) time-limited aging analyses that have been identified to require review under 10 CFR 54.21(c), such that there is reasonable assurance that the activities authorized by this renewed license will continue to be conducted in accordance with the current licensing basis, as defined in 10 CFR 54.3, for Surry Power Station, Unit No. 2, and that any changes made to the plant's current licensing basis in order to comply with 10 CFR 54.29(a) are in accord with the Act and the Commission's regulations;
- c. The facility will operate in conformity with the application as amended, the provisions of the Act, and the rules and regulations of the Commission;
- d. There is reasonable assurance: (i) that the activities authorized by the renewed operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
- e. The Virginia Electric and Power Company (the licensee) is technically and financially qualified to engage in the activities authorized by the renewed operating license in accordance with the rules and regulations of the Commission;

- f. The applicable provisions of 10 CFR Part 140 have been satisfied; and
- g. The issuance of this renewed license will not be inimical to the common defense and security or to the health and safety of the public.

On the basis of the foregoing findings regarding this facility, Facility Operating License No. DPR-37, issued January 29, 1973, is superceded by Renewed Facility Operating License No. DPR-37, which is hereby issued to the licensee to read as follows:

- 1. This renewed license applies to the Surry Power Station, Unit No. 2, a pressurized, light water moderated and cooled reactor, and associated steam generators and electric generating equipment (the facility). The facility is located on the licensee's 840-acre site on a point of land called Gravel Neck on the James River, approximately 14 miles northwest of Newport News and 25 miles northwest of Norfolk, Virginia, and is described in the licensee's Updated Final Safety Analysis Report.
- 2. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses the licensee:
  - A. Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility at the designated location in Surry County, Virginia, in accordance with the procedures and limitations set forth in this renewed license;
  - B. Pursuant to the Act and 10 CFR Parts 40 and 70, to receive, possess, and use at any time, source and special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report;
  - C. Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  - D. Pursuant to the Act and 10 CFR Parts, 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

- E. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:
- A. Maximum Power Level  
  
The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2546 megawatts (thermal).
  - B. Technical Specifications  
  
The Technical Specifications contained in Appendix A, as revised through Amendment No. 233, are hereby incorporated in this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.
  - C. Reports  
  
The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.
  - D. Records  
  
The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.
  - E. Deleted by Amendment 54
  - F. Deleted by Amendment 59 and Amendment 65
  - G. Deleted by Amendment 227
  - H. Deleted by Amendment 227

I. Fire Protection

The licensee shall implement and maintain in effect the provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report and as approved in the SER dated September 19, 1979 (and Supplements dated May 29, 1980, October 9, 1980, December 18, 1980, February 13, 1981, December 4, 1981, April 27, 1982, November 18, 1982, January 17, 1984, February 25, 1988, and July 23, 1992), and the Safety Evaluation issued December 16, 1998, for Technical Specification Amendment No. 217 subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

J. Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Surry Power Station Physical Security Plan," with revisions submitted through February 24, 1988; "Surry Power Station Guard Training and Qualification Plan," with revisions submitted through May 29, 1987; and "Surry Power Station Safeguards Contingency Plan," with revisions submitted through January 9, 1987. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

K. Deleted by Amendment 227

L. Deleted by Amendment 227

M. Deleted by Amendment 227

N. Deleted by Amendment 203

O. Deleted by Amendment 227

P. Updated Final Safety Analysis Report

- (1) The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on July 25, 2002, October 1, 2002, November 4, 2002, and December 2, 2002 describes certain future inspection activities to be completed before the period of extended operation. The licensee shall complete these activities no later than January 29, 2013, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.
  - (2) The Updated Final Safety Analysis Report supplement as revised on July 25, 2002, October 1, 2002, November 4, 2002, and December 2, 2002, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following the issuance of this renewed license. Until that update is complete, the licensee may make changes to the programs described in such supplement without prior Commission approval, provided that the licensee evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.
4. This renewed license is effective as of the date of issuance, and shall expire at midnight on January 29, 2033.

FOR THE NUCLEAR REGULATORY COMMISSION

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Samuel J. Collins, Director  
Office of Nuclear Reactor Regulation

Attachment: Appendix A, Technical Specifications

Date of Issuance: March 20, 2003

Renewed License No. DPR-37

Virginia Electric and Power Company

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Because the Technical Specifications for the North Anna Units 1 and 2, and Surry Units 1 and 2 are simply being reissued with the renewed licenses, only Virginia Power and Electric Company will receive the package with the Technical Specifications attached. If a hard copy of the Technical Specifications is needed, please contact the project manager, Omid Tabatabai, at 301 415-3738.

UNITED STATES NUCLEAR REGULATORY COMMISSION  
DOCKET NOS. 50-338, 50-339, 50-280, AND 50-281  
VIRGINIA ELECTRIC AND POWER COMPANY  
NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2, AND  
SURRY POWER STATION, UNIT NOS. 1 AND 2  
NOTICE OF ISSUANCE OF RENEWED FACILITY  
OPERATING LICENSE NOS. NPF-4, NPF-7, DPR-32, AND DPR-37  
FOR AN ADDITIONAL 20-YEAR PERIOD

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Renewed Facility Operating License Nos. NPF-4, NPF-7, DPR-32, and DPR-37 to Virginia Electric and Power Company (the licensee), the operator of the North Anna Power Station, Unit Nos. 1 and 2 (North Anna, Units 1 and 2), and Surry Power Station, Unit Nos. 1 and 2 (Surry, Units 1 and 2). Renewed Facility Operating License No. NPF-4 authorizes operation of North Anna, Unit 1, by the licensee at reactor core power levels not in excess of 2893 megawatts thermal in accordance with the provisions of the North Anna, Unit 1, renewed license and its Technical Specifications. Renewed Facility Operating License No. NPF-7 authorizes operation of North Anna, Unit 2, by the licensee at reactor core power levels not in excess of 2893 megawatts thermal in accordance with the provisions of the North Anna, Unit 2, renewed license and its Technical Specifications. Renewed Facility Operating License No. DPR-32 authorizes operation of Surry, Unit 1, by the licensee at reactor core power levels not in excess of 2546 megawatts thermal in accordance with the provisions of the Surry, Unit 1, renewed license and its Technical Specifications. Renewed Facility Operating License

No. DPR-37 authorizes operation of Surry, Unit 2, by the licensee at reactor core power levels not in excess of 2546 megawatts thermal in accordance with the provisions of the Surry, Unit 2, renewed license and its Technical Specifications.

North Anna, Units 1 and 2, are pressurized water nuclear reactors located in Louisa County, 40 miles northwest of the city of Richmond, Virginia. Surry, Units 1 and 2, are pressurized water nuclear reactors located in Surry County, 14 miles northwest of the city of Newport News, Virginia.

The applications for the renewed licenses complied with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. As required by the Act and the Commission's regulations in 10 CFR Chapter I, the Commission has made appropriate findings, which are set forth in each license. Prior public notice of the action involving the proposed issuance of these renewed licenses and of an opportunity for a hearing regarding the proposed issuance of these renewed licenses was published in the *Federal Register* on July 27, 2001 (66 FR 39213).

For further details with respect to this action, see (1) the Virginia Electric and Power Company's license renewal applications for North Anna, Units 1 and 2, and Surry, Units 1 and 2, dated May 29, 2001, as supplemented by letters dated November 30, 2001, January 4 (two letters), January 16, January 17, February 1 (two letters), February 5, May 22 (two letters), June 13, July 11, July 25, August 23, October 1, October 15, November 4, December 2, and December 11, 2002; (2) the Commission's safety evaluation report, dated November 5, 2002, and December 2002 (NUREG-1766); (3) the licensee's updated final safety analysis report; and (4) the Commission's final environmental impact statements (NUREG-1437, Supplement 6, for Surry, Units 1 and 2, and NUREG-1437, Supplement 7, for North Anna, Units 1 and 2), dated November 2002. These documents are available at the NRC's Public Document Room, One White Flint North, 11555 Rockville Pike, first floor, Rockville, Maryland 20852, and can be

viewed from the NRC Public Electronic Reading Room at  
<http://www.nrc.gov/reading-rm/adams.html>.

Copies of Renewed Facility Operating License Nos. NPF-4, NPF-7, DPR-32, and DPR-37 may be obtained by writing to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555-0001, Attention: Director, Division of Regulatory Improvement Programs. Copies of the safety evaluation report (NUREG-1766), and the final environmental impact statements (NUREG-1437, Supplement 6, for Surry, Units 1 and 2, and NUREG-1437, Supplement 7, for North Anna, Units 1 and 2) may be purchased from the National Technical Information Service, Springfield, Virginia 22161-0002 (<http://www.ntis.gov>), 1-800-553-6847, or the Superintendent of Documents, U.S. Government Printing Office, P.O. Box 371954, Pittsburgh, PA 15250-7954 ([http://www.access.gpo.gov/su\\_docs](http://www.access.gpo.gov/su_docs)), 202-512-1800. All orders should clearly identify the NRC publication number and the requestor's Government Printing Office deposit account number or VISA or MasterCard number and expiration date.

Dated at Rockville, Maryland, this 20<sup>th</sup> day of March 2003.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

Pao-Tsin Kuo, Program Director  
License Renewal and Environmental Impacts  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation