Facility:	Perry	**Task No:**	299-910-03-01
Task Title:	Determine Dress Out Requirements for a Contaminated Area	**JPM No:**	2003 NRC RO/SRO A.3
K/A Reference:	2.3.4		
Examinee:		**NRC Examiner:**	
Facility Evaluator:	N/A	**Date:**	

Method of testing

Simulated Performance **Actual Performance**

Classroom **Simulator** Plant

Task Standard: Candidate determines that he must wear Full Protective Clothing and a Discrete Particle Suit, as a minimum, in order to enter the RWCU Heat Exchanger Room.

Required Materials: Radiation Work Permit (marked up copy)
Survey Map (marked up copy)

General References: HPI-C0005, Rev. 7
PAP-0114, Rev. 4

Time Critical Task: NO

Validation Time: 15 minutes

READ TO THE EXAMINEE

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this Job Performance Measure will be satisfied.

Initial Conditions: The plant is operating at 100% power. There has been an increasing temperature trend in the Reactor Water Cleanup (RWCU) System Heat Exchanger Room over the past 12 hours. Operations Management has determined that entry into the room is required in order to confirm if any leakage exists, and to quantify the amount.

As a member of the Relief Crew, you have been assigned to enter the room to perform this activity.

Initiating Cue: Determine the minimum protective clothing requirements for you to enter the RWCU Heat Exchanger Room.

(Denote Critical Steps with an asterisk)

* **Performance Step: 1** Locate the correct Radiation Work Permit (RWP) in order to determine the Radiation Protection Program requirements for the assigned task.

Standard: Locates the Specific RWP that allows entry into the RWCU Heat Exchanger Room for Leakage Inspection.

Comment: **Cue: Provide Candidate with marked up RWP <u>after</u> he either shows or states that the RWPs are kept at the RRA entrance.**

* **Performance Step: 2** Locate the correct Survey Map in order to determine the contamination levels inside the RWCU Heat Exchanger Room.

Standard: Locates the Survey Map for the RWCU Heat Exchanger Room.

Comment: **Cue: Provide Candidate with marked up Survey Map <u>after</u> he either shows or states that the Survey Maps are kept at the RRA entrance.**

* **Performance Step: 3** Determine the minimum protective clothing requirements for the assigned job task.

Standard: Evaluates RWP and associated Survey Map, and determines that he must wear Full Protective Clothing and a Discrete Particle Suit, as a minimum, in order to enter the RWCU Heat Exchanger Room.

Comment:

Terminating Cue:

When the Candidate has determined that he must wear Full Protective Clothing and a Discrete Particle Suit, as a minimum, the evaluation for this JPM is complete.

Job Performance Measure No. 2003 NRC RO/SRO A.3

Examinee's Name:

Examiner's Name:

Date Performed:

Facility Evaluator: N/A

Number of Attempts:

Time to complete:

Question Documentation:

Question:

Response:

Result: SAT OR UNSAT

Examiner's Signature and Date: _____ _____

INITIAL CONDITIONS: The plant is operating at 100% power. There has been an increasing temperature trend in the Reactor Water Cleanup (RWCU) System Heat Exchanger Room over the past 12 hours. Operations Management has determined that entry into the room is required in order to confirm if any leakage exists, and to quantify the amount.

As a member of the Relief Crew, you have been assigned to enter the room to perform this activity.

INITIATING CUE: Determine the minimum protective clothing requirements for you to enter the RWCU Heat Exchanger Room.

Perry Nuclear Power Plant
RADIATION WORK PERMIT (RWP) **For Training Purposes Only**

Location: Containment 652'	RWP Number: 03XXXX

Description: Leakage Inspection in RWCU Heat Exchanger Room

This SRWP AUTHORIZES entry into the following areas:
LHRA - Locked High Radiation Areas
HRA - High Radiation Areas
RA - Radiation Areas
HCA - Highly Contaminated Areas
CA - Contamination Areas
DPZ - Discrete Particle Zones
ARA - Airborne Radioactivity Areas

MWO Number: N/A	Contact: O. P. Raitor	Contact Phone Number: 5932	Work Code: OP
Completion Date: 01-JAN-2004	Person-mrem Estimate: 0.5	Person-Hrs Estimate: 4	

WORK AREA SURVEY

Dose Rates: 3 -- 30,000 mrem/hr.
Contamination Levels: <1000 – 150,000 dpm/100 cm^2

DOSIMETRY REQUIREMENTS

External Alarming Device M-G DRD Telemetry
TLD

PROTECTIVE CLOTHING

CA	Full Protective Clothing	DPZ	Discrete Particle Suit
HCA	Fram Tech 2 or Equivalent	WET	Plastic Suit

RESPIRATORY PROTECTION EQUIPMENT

Respirator Type: None

RWP TASKS

1 Ops Support
2 Maintenance Support
3 Decon

WORKER INSTRUCTIONS

1. A documented ALARA brief is required when entering the RWCU Heat Exchanger Room.
2. Locked High Radiation Area Door Guards are required for entries into Locked High Radiation Areas.
3. A Radiation protection Brief is required prior to entering a High Radiation Area, a Contamination Area or an Airborne Radioactivity Area.
4. A Radiation Protection Brief is required prior to moving Radioactive Material > 5 mrem/hr at 30 cm, Working > 6 feet above floor level or when breaching radioactive systems.
5. Discrete Particle Controls are to be implemented as directed by RP Supervision IAW HPI-L009.

ALARA NOTES

1. External Alarming Devices and Telemetry shall be used per direction of RP Supervision.
2. MG Setpoints should be evaluated for each entry and changes documented in the Narrative Log.
3. MG Setpoints for dose rate shall not be set above 5000 mRem/hr.
4. No entries permitted into VHRA's on this RWP.

GENERAL RP NOTES

A B-DAC calculation is required for ARA >1.0 DAC
CONTINUOUS Radiation Protection Coverage is required for entries into LHRA's and DPZ's

PREPARED BY: T. Nightman Date: mm/dd/yy RP PLANNING Al Arnold Date: mm/dd/yy
RP SUPERVISOR: A Penniman Date: mm/dd/yy PHP APPROVAL: N/A Date: N/A
WORK SUPERVISOR: OP Raitor Date: mm/dd/yy TERMINATED BY: _____ Date: ___

RADIOLOGICAL SURVEY REPORT

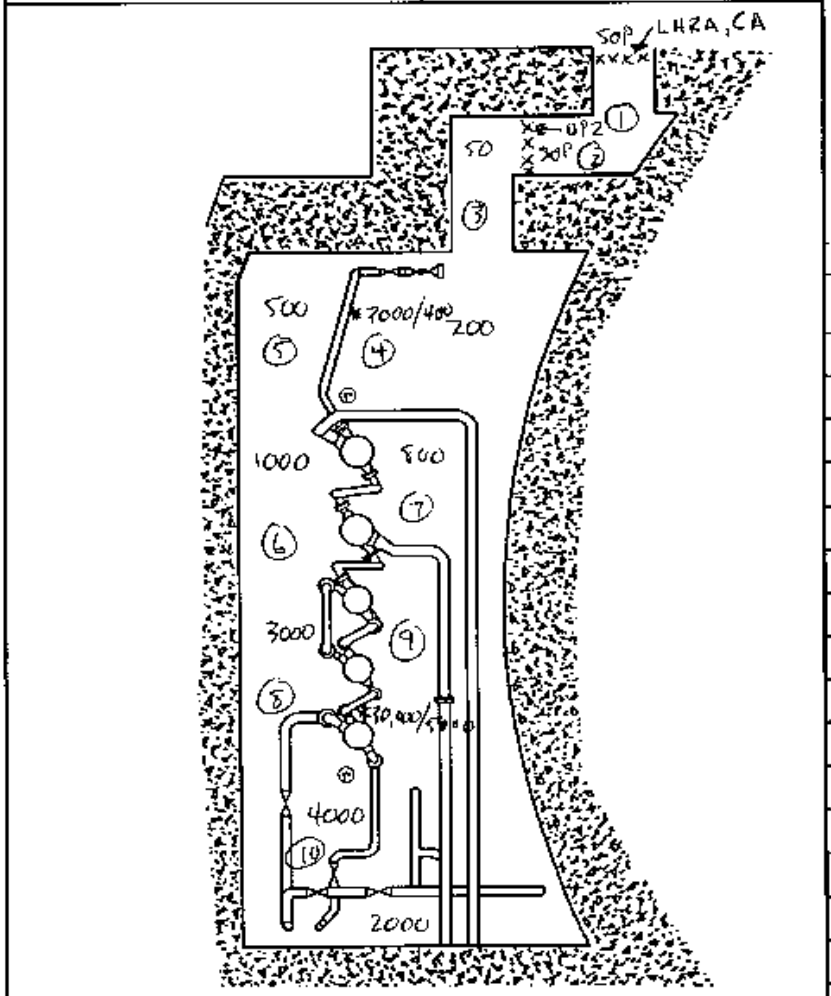
PNPP No. 7247 Rev. 9/17/02

RWP NUMBER	SURVEY NUMBER
07 - XXXX	03 - XXXX

BUILDING	ELEVATION	AREA / ROOM / SYSTEM	DATE	TIME
CTMT	652'	Containment RWCU Heat Exchanger Room	MM/DD/YY	10:30

PURPOSE	% POWER
LEAK INSPECTION	100

AIR SAMPLE #1 ☐N/A	AIR SAMPLE #2 ☐N/A
P N/A uCi/cc C N/A uCi/cc	P N/A uCi/cc C N/A uCi/cc



all dose rates are in mrem/hr @ waist level unless otherwise noted.
All smears are < 1000 dpm/100 cm² β,γ unless otherwise noted.

Legend:
☐ - Air Sample, ⊙-Smear
XXXXX – Boundary, ### - Direct Frisk
*/ - Contact/30cm
Bkgd – Background
MDA- Minimum Detectable Activity
NDB- No Detectable Beta
SOP – Step Off Pad
CA – Contamination Area
HCA – Highly Contaminated Area
DPZ – Discrete Particle Zone
ARA – Airborne Radioactivity Area
RA – Radiation Area
NRA – Neutron Radiation Area
HRA – High Radiation Area
LHRA – Locked High Radiation Area
VHRA – Very High Radiation Area
N - Neutron, - Alpha, B- Beta, - Gamma

Smear #	Smear Location	dpm/100cm² β,γ	dpm/100cm² α
2	SOP	5K	N/A
3	FLOOR	8K	
4	FLOOR	20K	
5	FLOOR	10K	
6	FLOOR	40K	
7	FLOOR	60K	
8	FLOOR	30K	
9	FLOOR	90K	
10	FLOOR	50K	

INSTRUMENTS USED

MODEL	INSTRUMENT NUMBER	CAL DUE DATE
TELETECTOR	LT XXXXX	MM/DD/YY
RO-2A	LT XXXXX	MM/DD/YY
RM-14	LT XXXXX	MM/DD/YY
N/A	N/A	N/A

SURVEY BY

NAME (Print)	SIGNATURE	DATE
ROGER A. MEADER	R.A. Meader	MM/DD/YY

REVIEWED BY

NAME (Print)	SIGNATURE	DATE
SUPERVISOR	Supervisor	MM/DD/YY

Facility:	Perry	**Task No:**	299-893-03-01
Task Title:	Evaluate Proposed Work Schedule Against Established Overtime Guidelines	**JPM No:**	2003 NRC RO A.1.a
K/A Reference:	2.1.2		
Examinee:		**NRC Examiner:**	
Facility Evaluator:	N/A	**Date:**	

Method of testing

Simulated Performance **Actual Performance**

Classroom **Simulator** Plant

Task Standard: Determines that he cannot work the proposed overtime without additional authorization (a Technical Specification Overtime Deviation Request is requested by his Supervisor and authorized by the Plant Manager or his designees), for either one of the following reasons;

- He will not receive a break of at least 8 hours from Sat. to Sun., or,
- He will exceed 24 hours in the 48 hour period from 1200 on Fri. through 1200 on Sun.

Required Materials: NOP-LP-1002, Rev 0

General References: NOP-LP-1002, Rev 0

Time Critical Task: NO

Validation Time: 10 minutes

READ TO THE EXAMINEE

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this Job Performance Measure will be satisfied.

Initial Conditions: The plant is operating at 100% power. Operations Section shift manning is being severely affected due to a flu epidemic. All healthy licensed operators are being scheduled to work extra hours in order to cover shift vacancies. Listed below is your proposed Work Schedule for the next 10 days.

Sun	Mon	Tue	Wed	Thu	Fri	Sat	Sun	Mon	Tue
Off	Off	12-24	12-24	12-24	12-24	16-24	06-14	Off	08-16

Note: All work hours shown exclude turnover time.

Initiating Cue: As a licensed operator, determine if you can work the proposed overtime without any additional authorization.

(Denote Critical Steps with an asterisk)

Note: Ensure a copy of NOP-LP-1002, Fitness For Duty Program, is available for use by the Candidate.

* **Performance Step: 4.14.3**

The following guidelines are applicable to personnel performing safety-related functions (e.g., licensed Senior Reactor Operators, Reactor Operators, Health Physicists, Auxiliary Operators, Perry Plant Operators, Perry Plant Attendants and Key Maintenance Personnel).

Overtime Deviation shall be controlled as specified in each plant's licensing documents, and may be supplemented as directed by site management.

- An individual should not work more than 16 hours straight (excluding shift turnover time).

- An individual should not work more than 16 hours in any 24 hour period, 24 hours in any 48 hour period, or 72 hours in any 7 day period (all excluding shift turnover time).

- A break of at least 8 hours should be allowed between work periods. (Shift turnover time should not be part of the 8 hour break period.)

- Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift or crew.

- The crew performing fuel loading operations should not normally be expected to work more than 12 hours out of each 24 hour period when scheduled for consecutive daily duty.

Standard: Determines that he cannot work the proposed overtime without additional authorization (a Technical Specification Overtime Deviation Request is requested by his Supervisor and authorized by the Plant Manager or his designees), for either one of the following reasons;

- He will not receive a break of at least 8 hours from Sat. to Sun., or,
- He will exceed 24 hours in the 48 hour period from 1200 on Fri. through 1200 on Sun.

Comment:

Terminating Cue:

When the Candidate has determined that he cannot work the proposed overtime without additional authorization, the evaluation for this JPM is complete.

Job Performance Measure No. 2003 NRC RO A.1.a

Examinee's Name:

Examiner's Name:

Date Performed:

Facility Evaluator: N/A

Number of Attempts:

Time to complete:

Question Documentation:

Question:

Response:

Result: SAT OR UNSAT

Examiner's Signature and Date: _____ _____

INITIAL CONDITIONS: The plant is operating at 100% power. Operations Section shift manning is being severely affected due to a flu epidemic. All healthy licensed operators are being scheduled to work extra hours in order to cover shift vacancies. Listed below is your proposed Work Schedule for the next 10 days.

Sun	Mon	Tue	Wed	Thu	Fri	Sat	Sun	Mon	Tue
Off	Off	12-24	12-24	12-24	12-24	16-24	06-14	Off	08-16

Note: All work hours shown exclude turnover time.

INITIATING CUE: As a licensed operator, determine if you can work the proposed overtime without any additional authorization.

Facility:	Perry	**Task No:**	N/A
Task Title:	Determine the Effect(s) of Relay 1E51A-K51 Removal	**JPM No:**	2003 NRC RO A.1.b
K/A Reference:	2.1.24		
Examinee:		**NRC Examiner:**	
Facility Evaluator:	N/A	**Date:**	

Method of testing

Simulated Performance **Actual Performance**

Classroom **Simulator** **Plant**

Task Standard: Candidate determines that the removal of Relay 1E51A-K51 will defeat the RCIC automatic suction transfer (i.e., prevent the automatic opening of RCIC Pump Suppression Pool Suction Isolation Valve, 1E51-F031, on low CST level or high SP level).

Required Materials: Electrical Drawings – 208-075 Series

General References: Electrical Drawings – 208-075 Series

Time Critical Task: NO

Validation Time: 20 minutes

READ TO THE EXAMINEE

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this Job Performance Measure will be satisfied.

Initial Conditions:	The plant is operating at 100% power. The Reactor Core Isolation Cooling System (RCIC) is in the normal Standby Readiness lineup. Agastat Relays has issued an industry-wide notice that has identified a specific batch of relays that were not built to design specifications. Agastat Relays has recommended that these relays be replaced as soon as practicable. Perry has identified that currently installed Relay 1E51A-K51 was part of this batch.
Initiating Cue:	The Unit Supervisor directs you to determine the potential effect(s) on the RCIC System if Relay 1E51A-K51 were to be removed with the plant in its current operating condition.

(Denote Critical Steps with an asterisk)

Note: The Candidate will require access to the Plant Electrical (208) Drawings located either in the Simulator, TEC Reference Library, or the Unit 1 Control Room.

* **Performance Step: 1** Using the Plant Electrical (208) Drawings, determine the potential effect(s) on the RCIC System if Relay 1E51A-K51 were removed with the plant in its current operating condition.

Standard: Determines that the removal of Relay 1E51A-K51 will defeat the RCIC automatic suction transfer (i.e., prevent the automatic opening of RCIC Pump Suppression Pool Suction Isolation Valve, 1E51-F031, on low CST level or high SP level).

Comment: Detailed discussion of Relay 1E51A-K51 removal.

1. B208-075 Sh 02:

   - Locates Relay 1E51A-K51 (in far-left column).
     - Determines relay is shown on Sh 04.
     - Determines relay contact T1-M1 is shown on Sh 21 (for valve E51-F031).

2. B208-075 Sh 04 Line 25:

   - Identifies Relay 1E51A-K51.
     - Determines relay is normally de-energized.
     - Determines relay would not energize on either a low CST level (K78) or high SP level (K79) (B208-075 Sh 10 Lines 05 & 17).
       Note: Relay K50 is normally de-energized, therefore, relay contact M1-R1 is normally closed.

3. B208-075 Sh 21 Line 09:

   - Identifies Relay 1E51A-K51 contact M1-T1.
     - Determines contact would not close.
     - Determines RCIC Pump Suppression Pool Suction Isolation Valve, 1E51-F031, would not automatically open if either a low CST level or high SP level condition were to occur.

Note: It is not critical for the Candidate to determine the Operability of the RCIC System in accordance with Technical Specifications.

Terminating Cue:

When the Candidate has determined that the RCIC automatic suction transfer has been defeated, the evaluation for this JPM is complete

Job Performance Measure No. 2003 NRC RO A.1.b

Examinee's Name:

Examiner's Name:

Date Performed:

Facility Evaluator: N/A

Number of Attempts:

Time to complete:

Question Documentation:

Question:

Response:

Result: SAT OR UNSAT

Examiner's Signature and Date: _____ _____

INITIAL CONDITIONS: The plant is operating at 100% power. The Reactor Core Isolation Cooling System (RCIC) is in the normal Standby Readiness lineup. Agastat Relays has issued an industry-wide notice that has identified a specific batch of relays that were not built to design specifications. Agastat Relays has recommended that these relays be replaced as soon as practicable. Perry has identified that currently installed Relay 1E51A-K51 was part of this batch.

INITIATING CUE: The Unit Supervisor directs you to determine the potential effect(s) on the RCIC System if Relay 1E51A-K51 were to be removed with the plant in its current operating condition.

Facility:	Perry	**Task No:**	299-655-03-01 299-574-03-01
Task Title:	Perform Independent Verification of Tag Placement	**JPM No:**	2003 NRC RO A.2
K/A Reference:	2.2.13		
Examinee:		**NRC Examiner:**	
Facility Evaluator:	N/A	**Date:**	

Method of testing

Simulated Performance **Actual Performance**

Classroom **Simulator** Plant

Task Standard: Candidate determines that valve P42-F295B was incorrectly closed and tagged. Notifies the Clearance Authority of the tagging discrepancy.

Required Materials: NOP-OP-1001, Rev 1
Clearance PY1-P47-0040 (Copy for training only)

General References: NOP-OP-1001, Rev 1

Time Critical Task: NO

Validation Time: 10 minutes

READ TO THE EXAMINEE

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this Job Performance Measure will be satisfied.

Initial Conditions: The plant is operating in MODE 1. The chilled water coils for Controlled Access and Misc. Equipment Areas HVAC System (CA&MEA) Supply Plenum M21-B001B has a 2 gpm leak. The chilled water coils are being isolated and tagged per Clearance PY1-P47-0040. The initial placement of the Control Room tags has been completed.

Initiating Cue:	The Unit Supervisor directs you, as a Reactor Operator, to perform the independent verification of the Control Room tag placement for Clearance PY1-P47-0040 in accordance with NOP-OP-1001.

(Denote Critical Steps with an asterisk)

Note: The Evaluator will role-play as the SRO / Clearance Authority for this Admin JPM.

Note: Hand the Candidate the Clearance for his review and independent verification of tag placement.

Note: The NOMS Clearance System will <u>not</u> be required / used for this JPM.

Note: Each Control Room clearance tag independent verification can be evaluated in any order and will be documented in separate Performance Steps.

Note: The following Performance Step is for Valve P47-F290B.

Performance Step: 4.10.2 Verify tags hung as follows:

1. For each tag, perform the following:

    a. Confirm that the proper component was selected by comparing the information placed on the tag, the clearance and in-plant label. Abbreviated terms need not match exactly.
    b. Confirm the component in the specified position.
    c. Confirm the tag is securely fastened to the component in a visible location.
    d. Check Placement 2nd Verification for that tag on the clearance.

Standard: **For valve P47-F290B:**

- Determines that the component was properly selected by comparing the information placed on the tag, the clearance and in-plant label.
- Confirms the component is in the specified position (closed).
- Confirms the red tag is securely fastened to the valve control switch.
- Checks Placement 2nd Verification for that tag on the clearance.

Comment: **Note: It is <u>not</u> required for the Candidate to enter NOMS and electronically sign for Placement 2nd Verification for that tag on the clearance.**

Cue: Inform the Candidate that he will <u>not</u> be required to update NOMS for the Placement 2nd Verification of tags.

Note: The following Performance Step is for Valve P47-F295B.

* **Performance Step: 4.10.2** Verify tags hung as follows:

1. For each tag, perform the following:

a. Confirm that the proper component was selected by comparing the information placed on the tag, the clearance and in-plant label. Abbreviated terms need not match exactly.
b. Confirm the component in the specified position.
c. Confirm the tag is securely fastened to the component in a visible location.
d. Check Placement 2nd Verification for that tag on the clearance.

Standard: **For valve P47-F295B:**

- Determines that the component was not properly selected by comparing the information placed on the tag, the clearance and in-plant label.
  - ♦ **Determines red tag was placed on the control switch for valve P42-F295B instead of the control switch for valve P47-F295B.**
- Confirms the component is not in the specified position (closed).
  - ♦ **Determines valve P42-F295B was closed instead of valve P47-F295B.**
- Confirms the red tag is not securely fastened to the valve control switch.
  - ♦ **Determines red tag was secured on the control switch for valve P42-F295B instead of the control switch for valve P47-F295B.**
- Does not check Placement 2nd Verification for that tag on the clearance.

Comment:

*	**Performance Step: 4.9.2**	Correcting discrepancies: If conflicts between clearances or technical problems arise, Then inform the Clearance Authority.
	Standard:	The Clearance Authority (SRO) is informed that the tag for valve P47-F295B was initially placed on the wrong valve (P42-F295B). Does not remove the incorrectly placed tag on valve P42-F295B, reposition valve P42-F295B to open, close correct valve P47-F295B, and tag valve P47-F295B control switch.
	Comment:	**Note: The Candidate is not allowed to correct the clearance discrepancy (as discussed in NOP-OP-1001). The clearance will have to be revised.**

Terminating Cue:

When the Candidate completes the independent verification for the red tag for valve P47-F290B control switch and determines that the red tag for valve P47-F295B control switch was improperly placed and informs the Clearance Authority, the evaluation for this JPM is complete.

Job Performance Measure No. 2003 NRC RO A.2

Examinee's Name:

Examiner's Name:

Date Performed:

Facility Evaluator: N/A

Number of Attempts:

Time to complete:

Question Documentation:

Question:

Response:

Result: SAT OR UNSAT

Examiner's Signature and Date: _____ _____

Clearance Coversheet
Tagout: PY-CYC-009
Clearance: PY1-P47-0040 *01/24/2003 13:10*

Component to be Worked:

0M21B0001B

CONTR. ACCESS & EQUIP. AREA HVAC SYS. SUPPLY PLENUM

0-CC-679-A/04

Description /Reason

Isolate Cooling Coils for Flushing and Leakage Repairs

Placement Notes

Results in Loss of P47 Chilled Water flow to CAA Cooling Coil 'B' and Computer Room Cooling Coil "B"

Cautions

Isolating Flow per Request - Suggest Shutdown of M21 B and M27 B HVAC Trains due to loss of Chilled Water Flow

Completion Notes

Clearance Attributes:

Attribute Description	Attribute Value
Electrical Hot Work	
Fire Impairment (PAP-1910)_____	
LCO Tracking (OAI-1701)	
LCO No. _____	
Independent Verification (PAP-0205)	
OEO _____	
Temporary Shielding (RPI-0122)	
Shielding Req. No. _____	
Temp. Mod or INFO Tags (PAP-1402/1404)	
Tag No. _____	
Other _____	

Work Order List:

Number		Equipment ID	Description
03-000222-000	PRINTED	0M21B0001B	CLEAN AND INSPECT COOLING COILS. REPAIR COIL LEAKAGE

Clearance Verification:

Status	Description	User	Verification Date
Prepared	Prepared By	Apley, Marc	01/14/2003 12:38
Reviewed	Reviewed By	Apley, Marc	01/24/2003 13:10
Second Review	Second Review By	N/A	01/24/2003 13:10
Approved	Approved By	TESTER Baker	01/24/2003 13:10
Issued for Work	Issued for Work By		00/00/0000 00:00
Restoration Review	Restoration Review By		00/00/0000 00:00
Removal Authorized	Removal Authorized By		00/00/0000 00:00
Clearance Closed	Clearance Closed By		00/00/0000 00:00

Grounds Tracking List:

INITIAL CONDITIONS: The plant is operating in MODE 1. The chilled water coils for Controlled Access and Misc. Equipment Areas HVAC System (CA&MEA) Supply Plenum M21-B001B has a 2 gpm leak. The chilled water coils are being isolated and tagged per Clearance PY1-P47-0040. The initial placement of the Control Room tags has been completed.

INITIATING CUE: The Unit Supervisor directs you, as a Reactor Operator, to perform the independent verification of the Control Room tag placement for Clearance PY1-P47-0040 in accordance with NOP-OP-1001.

Clearance Tag Hang List *01/24/2003 13:10*

Tagout: PY-CYC-009

Clearance: PY1-P47-0040

Tag Serial No.	Tag Type	Equipment Equipment Description Equipment Location	Ver. Req.	Place. Seq.	Placement Configuration	Placement 1st Verif Date/Time	Placement 2nd Verif Date/Time	Tag Placement Notes
12188	Red	* 0P47F0290B C/S * CCCW OUT FROM CAA CLG COIL B C/S * 0H13P0904		1	AUTO/CLOSED	1/24/03 1400		
12189	Red	* 0P47F0290B DISC * CCCW OUT FROM CAA CLG COIL B DISC * 0-CC-620-A/03 EF1C09 - X		2	OFF			
12190	Red	* 0P47F0290B * CCCW OUT FROM CAA CLG COIL B * 0-CC-679-B/03		3	TAGGED			
12191	Red	* 0P47F0295B C/S * CCCW IN TO CAA CLG COIL B C/S * 0H13P0904		1	AUTO/CLOSED	1/24/03 1400		
12192	Red	* 0P47F0295B DISC * CCCW IN TO CAA CLG COIL B DISC * 0-CC-620-A/03 EF1C09 - Y		2	OFF			
12193	Red	* 0P47F0295B * CCCW IN TO CAA CLG COIL B * 0-CC-679-B/03		3	TAGGED			

Facility:	Perry	**Task No:**	N/A
Task Title:	Perform Site Accountability Actions for Control Room Staff	**JPM No:**	2003 NRC RO A.4
K/A Reference:	2.4.39		
Examinee:		**NRC Examiner:**	
Facility Evaluator:	N/A	**Date:**	

Method of testing

Simulated Performance Actual Performance

Classroom Simulator **Plant**

Task Standard: Candidate obtains temporary relief as the Licensed Operator 'At the Controls' and then locates the Emergency Personnel Accountability card reader located in either the Unit 1 or 2 Control Room and simulates inserting his keycard into the card reader.

Required Materials: EPI-B0005, Rev 6, PIC 7
PAP-0126, Rev 2, PIC 2

General References: EPI-B0005, Rev 6, PIC 7
PAP-0126, Rev 2, PIC 2

Time Critical Task: NO

Validation Time: 5 minutes

READ TO THE EXAMINEE

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this Job Performance Measure will be satisfied.

Initial Conditions: The plant is in MODE 3. You are the Licensed Operator 'At the Controls' when a Site Area Emergency is announced. In addition, the Emergency Coordinator has announced that site accountability is now in effect.

Initiating Cue: The Shift Manager directs you, as a member of the Control Room staff, to perform your action(s) for site accountability.

(Denote Critical Steps with an asterisk)

Note: The Evaluator will role-play as another licensed Reactor Operator for the Candidate.

Note: The Candidate may explain that he can give his keycard to another member of his crew who will then insert his keycard into the designated accountability card reader for him. If this occurs, then cue the Candidate that he is required to demonstrate the action(s) himself.

Performance Step: 6.8.3.1	The Licensed Operator 'At the Controls' may be temporarily relieved for personal reasons by another licensed operator as long as a verbal turnover of applicable items below has occurred: a. The status of work in progress, with emphasis on unit/personnel safety items and safety related systems. b. Any changes in the Unit status since shift turnover. c. The status of testing and evolutions that has changed since shift turnover. d. Any alarms or equipment out-of-service that has changed since shift turnover.
Standard:	Obtains temporary relief from another licensed Reactor Operator in order to leave the Horseshoe Area.
Comment:	**Cue: As another Reactor Operator on his crew, inform the Candidate that he is temporarily relieved.** Note: The Unit 1 Control Room Emergency Personnel Accountability card reader is located outside of the Horseshoe Area near the main entrance to the Unit 1 Control Room. Note: Per PAP-0126, Step 6.3.1, the Licensed Operator 'At the Controls' shall not leave the Horseshoe Area until relieved by another licensed operator or senior reactor operator, except as specified in 6.3.2. Note: PAP-0126, Step 6.3.2 does not allow the Licensed Operator 'At the Controls' to momentarily leave the Horseshoe Area without relief for the sole purpose of performing site accountability actions. **Note: This Step is not critical. If the Candidate does not obtain temporary relief, this would only be a violation of plant procedure (PAP-0126).**

*	**Performance Step: 5.6.1.1**	Utilize Card Readers in either Unit 1 or 2 Control Rooms for accountability purposes.
	Standard:	Locates the Emergency Personnel Accountability card reader in either the Unit 1 or 2 Control Room. Inserts (simulates) and then withdraws keycard into the designated accountability card reader.
	Comment:	**Cue: The red light has blinked on the Accountability card reader.** Note: Control Room staff shall respond as outlined in Control Room Staff and General Site Accountability Actions (EPI-B0005, Attachment 3).

Terminating Cue:

When the Candidate completes his actions to demonstrate Site Accountability, the evaluation for this JPM is complete.

Job Performance Measure No. 2003 NRC RO A.4

Examinee's Name:

Examiner's Name:

Date Performed:

Facility Evaluator: N/A

Number of Attempts:

Time to complete:

Question Documentation:

Question:

Response:

Result: SAT OR UNSAT

Examiner's Signature and Date: _____ _____

INITIAL CONDITIONS:	The plant is in MODE 3. You are the Licensed Operator 'At the Controls' when a Site Area Emergency is announced. In addition, the Emergency Coordinator has announced that site accountability is now in effect.
INITIATING CUE:	The Shift Manager directs you, as a member of the Control Room staff, to perform your action(s) for site accountability.

Facility:	Perry	**Task No:**	343-710-03-02 299-892-03-01
Task Title:	Evaluate Proposed Work Schedule Against Established Overtime Guidelines	**JPM No:**	2003 NRC SRO A.1.a
K/A Reference:	2.1.2		
Examinee:		**NRC Examiner:**	
Facility Evaluator:	N/A	**Date:**	

Method of testing

Simulated Performance **Actual Performance**

Classroom **Simulator** Plant

Task Standard: Candidate determines that Fred can work the proposed overtime without additional authorization and either Bill or John cannot work the proposed overtime without additional authorization, for the following reasons;

- Bill will exceed 72 hours in the 7 day period from Mon. through Sun., or,
- John will exceed 24 hours in the 48 hour period from 0800 on Sat. through 0800 on Mon.

Candidate completes a Technical Specification Overtime Deviation Request for either Bill or John.

Required Materials: NOP-LP-1002, Rev 0
Access to Computer Workstation with Lotus Notes software (or hardcopy of Technical Specification Overtime Deviation Request Form 7699)

General References: NOP-LP-1002, Rev 0

Time Critical Task: NO

Validation Time: 15 minutes

READ TO THE EXAMINEE

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this Job Performance Measure will be satisfied.

Initial Conditions: The plant is operating at 100% power. You are the Unit Supervisor for your crew. Two (2) Reactor Operators from your crew are needed to work a 12-hour overtime assignment on Sunday (2/10) from 0800 to 2000.

It is not expected that any of the Reactor Operators who work on Sunday will work the following Monday.

Listed below is the current work week schedule for the three (3) Reactor Operators on your crew.

	Sun 2/3	Mon 2/4	Tue 2/5	Wed 2/6	Thu 2/7	Fri 2/8	Sat 2/9
Bill	Off	08-20	08-20	08-16	08-16	08-20	08-20
Fred	Off	08-16	08-20	08-20	08-16	08-16	08-16
John	Off	08-20	08-16	08-20	08-20	off	08-24

Note: All work hours shown exclude turnover time.

Initiating Cue: Determine which Reactor Operator(s) can work the proposed overtime without any additional authorization and complete any form(s) required in order to fill the two (2) overtime assignments.

Answer Key – Hard Copy

TECHNICAL SPECIFICATION OVERTIME DEVIATION REQUEST PNPP No. 7699 Rev. 9/27/00	DEPT/SECTION PNPPD/OPS

DEVIATION CODES:

1 - MORE THAN 16 HRS STRAIGHT
2 - MORE THAN 16 HRS IN 24 HRS
3 - MORE THAN 24 HRS IN 48 HRS
4 - MORE THAN 72 HRS IN 7 DAYS
5 - LESS THAN 8 HR BREAK BETWEEN ASSIGNMENTS (CODES 01 -04 EXCLUDE SHIFT TURNOVER TIME)

NAME	SOCIAL SECURITY No.	CODE	START DATE	END DATE	REASON Reason shall include as a minimum: 1) the specific work to be performed, 2) why other individuals cannot not be used and 3) why the work cannot be rescheduled. General statements such as "outage support" and "to meet the schedule" are not acceptable.
Bill	**xxx-xx-xxxx**	**4**	**2/10/2003**	**2/10/2003**	**Work additional hours due to flu epidemic.**
John	**yyy-yy-yyyy**	**3**	**2/10/2003**	**2/10/2003**	**Work additional hours due to flu epidemic.**

Supervisors requesting an overtime deviation are signifying that they are familiar with the condition of the individual involved and that the individual's condition is such that the additional overtime hours can be safely worked.

REQUESTED BY	***GENERAL MANAGER/DIRECTOR APPROVAL***
_____ / _____ / _____ *Signature* *Date* *Section*	_____ / _____ *Signature* *Date*

(Denote Critical Steps with an asterisk)

Note: This JPM is intended to be performed using the electronic database version of the Technical Specification Overtime Deviation Request (PNPP No. 7699) located in Lotus Notes. However, if the Candidate desires, he can complete a hardcopy of the form.

Note: Ensure a copy of NOP-LP-1002, Fitness For Duty Program, is available for use by the Candidate.

Note: Each Reactor Operator (Bill, Fred, and John) can be evaluated in any order and will be documented in separate Performance Steps.

Note: The following Performance Steps are for Bill.

* **Performance Step: 4.14.3**

The following guidelines are applicable to personnel performing safety-related functions (e.g., licensed Senior Reactor Operators, Reactor Operators, Health Physicists, Auxiliary Operators, Perry Plant Operators, Perry Plant Attendants and Key Maintenance Personnel).

Overtime Deviation shall be controlled as specified in each plant's licensing documents, and may be supplemented as directed by site management.

- An individual should not work more than 16 hours straight (excluding shift turnover time).

- An individual should not work more than 16 hours in any 24 hour period, 24 hours in any 48 hour period, or 72 hours in any 7 day period (all excluding shift turnover time).

- A break of at least 8 hours should be allowed between work periods. (Shift turnover time should not be part of the 8 hour break period.)

- Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift or crew.

- The crew performing fuel loading operations should not normally be expected to work more than 12 hours out of each 24 hour period when scheduled for consecutive daily duty.

Standard:

Determines that **Bill** will exceed 72 hours in the 7 day period from Mon. through Sun.

Determines that **Bill** cannot work the proposed overtime without additional authorization (a Technical Specification Overtime Deviation Request is requested by his Supervisor and authorized by the Plant Manager or his designees).

Comment:

Note: The following Performance Step is only critical if the Candidate determines that Bill (not John) will be designated as the second Reactor Operator to work the proposed overtime on Sunday.

Note: Provide the Candidate with the following directions before he prepares the electronic version of the Technical Specification Overtime Deviation Request:

1. Use a fictitious Social Security Number for Bill.

2. Do not submit the completed Technical Specification Overtime Deviation Request for approval.

3. Print a copy of the completed Technical Specification Overtime Deviation Request.

4. Closeout the completed Technical Specification Overtime Deviation Request without saving the request.

5. Exit the Lotus Notes electronic database.

*	**Performance Step: 4.14.5**	Deviations from the overtime guidelines are normally requested by the immediate supervisor and authorized by the Plant Manager or his designees using the Technical Specification Overtime Deviation Request (PNPP No. 7699).
	Standard:	Using Lotus Notes, accesses the Overtime Deviation electronic database from the Perry Main menu on PY-NOTES01. Properly completes the Technical Specification Overtime Deviation Request. • **The following items are critical: Name (Bill), Code (4).** **See attached Technical Specification Overtime Deviation Request Answer Key.**
	Comment:	**Note: The 'Start Date' and 'End Date' blocks are not critical items because they are used for administrative purposes only. As a minimum, the dates should both be 2/10/03, but they could include the entire time interval of the hours worked or limit exceeded.** Note: If the electronic database version of the form is not available, then provide the Candidate with a hardcopy of PNPP No. 7699.

Note: The following Performance Step is for Fred.

Performance Step: 4.14.3	The following guidelines are applicable to personnel performing safety-related functions (e.g., licensed Senior Reactor Operators, Reactor Operators, Health Physicists, Auxiliary Operators, Perry Plant Operators, Perry Plant Attendants and Key Maintenance Personnel). Overtime Deviation shall be controlled as specified in each plant's licensing documents, and may be supplemented as directed by site management. - An individual should not work more than 16 hours straight (excluding shift turnover time). - An individual should not work more than 16 hours in any 24 hour period, 24 hours in any 48 hour period, or 72 hours in any 7 day period (all excluding shift turnover time). - A break of at least 8 hours should be allowed between work periods. (Shift turnover time should not be part of the 8 hour break period.) - Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift or crew. - The crew performing fuel loading operations should not normally be expected to work more than 12 hours out of each 24 hour period when scheduled for consecutive daily duty.
Standard:	Determines that Fred <u>can</u> work the proposed overtime without additional authorization.
Comment:	Note: Fred should be designated for 1 of the 2 proposed overtime assignments since he does not require additional authorization.

Note: The following Performance Steps are for John.

*	**Performance Step: 4.14.3**	The following guidelines are applicable to personnel performing safety-related functions (e.g., licensed Senior Reactor Operators, Reactor Operators, Health Physicists, Auxiliary Operators, Perry Plant Operators, Perry Plant Attendants and Key Maintenance Personnel). Overtime Deviation shall be controlled as specified in each plant's licensing documents, and may be supplemented as directed by site management. - An individual should not work more than 16 hours straight (excluding shift turnover time). - An individual should not work more than 16 hours in any 24 hour period, 24 hours in any 48 hour period, or 72 hours in any 7 day period (all excluding shift turnover time). - A break of at least 8 hours should be allowed between work periods. (Shift turnover time should not be part of the 8 hour break period.) - Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift or crew. - The crew performing fuel loading operations should not normally be expected to work more than 12 hours out of each 24 hour period when scheduled for consecutive daily duty.
	Standard:	Determines that **John** will exceed 24 hours in the 48 hour period from 0800 on Sat. through 0800 on Mon. Determines that **John** cannot work the proposed overtime without additional authorization (a Technical Specification Overtime Deviation Request is requested by his Supervisor and authorized by the Plant Manager or his designees).
	Comment:	

Note: The following Performance Step is only critical if the Candidate determines that John (not Bill) will be designated as the second Reactor Operator to work the proposed overtime on Sunday.

Note: Provide the Candidate with the following directions before he prepares the electronic version of the Technical Specification Overtime Deviation Request:

1. Use a fictitious Social Security Number for John.

2. Do not submit the completed Technical Specification Overtime Deviation Request for approval.

3. Print a copy of the completed Technical Specification Overtime Deviation Request.

4. Closeout the completed Technical Specification Overtime Deviation Request without saving the request.

5. Exit the Lotus Notes electronic database.

*	**Performance Step: 4.14.5**	Deviations from the overtime guidelines are normally requested by the immediate supervisor and authorized by the Plant Manager or his designees using the Technical Specification Overtime Deviation Request (PNPP No. 7699).
	Standard:	Using Lotus Notes, accesses the Overtime Deviation electronic database from the Perry Main menu on PY-NOTES01. Properly completes the Technical Specification Overtime Deviation Request. • **The following items are critical: Name (John), Code (3).** **See attached Technical Specification Overtime Deviation Request Answer Key.**
	Comment:	**Note: The 'Start Date' and 'End Date' blocks are not critical items because they are used for administrative purposes only. As a minimum, the dates should both be 2/10/03, but they could include the entire time interval of the hours worked or limit exceeded.** Note: If the electronic database version of the form is not available, then provide the Candidate with a hardcopy of PNPP No. 7699.

Terminating Cue:

When the Candidate has determined that Fred can work the proposed overtime without additional authorization and either Bill or John cannot work the proposed overtime without additional authorization (and completes a Technical Specification Overtime Deviation Request for either Bill or John), the evaluation for this JPM is complete.

Job Performance Measure No. 2003 NRC SRO A.1.a

Examinee's Name:

Examiner's Name:

Date Performed:

Facility Evaluator: N/A

Number of Attempts:

Time to complete:

Question Documentation:

Question:

Response:

Result: SAT OR UNSAT

Examiner's Signature and Date: _____ _____

INITIAL CONDITIONS:

The plant is operating at 100% power. You are the Unit Supervisor for your crew. Two (2) Reactor Operators from your crew are needed to work a 12-hour overtime assignment on Sunday (2/10) from 0800 to 2000.

It is not expected that any of the Reactor Operators who work on Sunday will work the following Monday.

Listed below is the current work week schedule for the three (3) Reactor Operators on your crew.

	Sun 2/3	Mon 2/4	Tue 2/5	Wed 2/6	Thu 2/7	Fri 2/8	Sat 2/9
Bill	Off	08-20	08-20	08-16	08-16	08-20	08-20
Fred	Off	08-16	08-20	08-20	08-16	08-16	08-16
John	Off	08-20	08-16	08-20	08-20	off	08-24

Note: All work hours shown exclude turnover time.

INITIATING CUE:

Determine which Reactor Operator(s) can work the proposed overtime without any additional authorization and complete any form(s) required in order to fill the two (2) overtime assignments.

TECHNICAL SPECIFICATION OVERTIME DEVIATION REQUEST

PNPP No. 7699 Rev. 9/27/00

DEVIATION CODES:

1 - MORE THAN 16 HRS STRAIGHT
2 - MORE THAN 16 HRS IN 24 HRS
3 - MORE THAN 24 HRS IN 48 HRS
4 - MORE THAN 72 HRS IN 7 DAYS
5 - LESS THAN
(CODES 01 -04

NAME	SOCIAL SECURITY No.	CODE	START DATE	END DATE	
					Reason shall inclu performed, 2) why work cannot be re support" and "to n

Supervisors requesting an overtime deviation are signifying that they are familiar with the condition of the individual involved and that the individual's can be safely worked.

REQUESTED BY

_____ / / _____
Signature *Date* *Section*

GENERAL MANAGER/DIRECTOR

_____
Signature

Facility:	Perry	**Task No:**	341-513-03-02 341-640-03-02 341-641-03-02
Task Title:	Make a Four Hour (10CFR50.72 Non-Emergency) Notification	**JPM No:**	2003 NRC SRO A.1.b
K/A Reference:	2.1.17		
Examinee:		**NRC Examiner:**	
Facility Evaluator:	N/A	**Date:**	

Method of testing

Simulated Performance **Actual Performance**

Classroom **Simulator** Plant

Task Standard: The Candidate determines that a Four Hour Notification is required and determines when the notification is due in accordance with PAP-1604, Reports Management. Form PNPP No. 6912, Event Notification, is correctly prepared.

Required Materials: PAP-1604 Rev 6, PIC 2
Event Notification (Form PNPP No. 6912)

General References: PAP-1604 Rev 6, PIC 2

Time Critical Task: NO

Validation Time: 20 minutes

READ TO THE EXAMINEE

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this Job Performance Measure will be satisfied.

Initial Conditions: You are the on-duty Shift Manager. The plant was conducting a shutdown in preparation for entering a refueling outage. The shutdown schedule called for power to be reduced to 20% and then a manual reactor scram was to be inserted.

Thirty minutes ago while shifting reactor feed pumps, a problem with the Master Level Controller resulted in an excessive feed rate and an increasing RPV level. The Reactor Operator inserted a manual reactor scram just prior to reaching RPV Level 8. Reactor power at the time of the scram was 30%. All equipment operated as designed following the scram and plant conditions are now stable.

Initiating Cue: As the Shift Manager, evaluate this event for reportability per PAP-1604, Reports Management, and perform any required notifications to the NRC Operations Center within the required time frame.

ANSWER KEY

EVENT NOTIFICATION

Perry Nuclear Power Plant

PNPP No. 6912 Rev. 1/16/01 — Unit 1 — PAP-1604

Caller's Name/Signature	Title	Time/Date of Call

Event Time & Zone	Event Date	Power/Mode Before	Power/Mode After
XX:XX Eastern	X/XX/2003	Mode 1	Mode 3

	EVENT CLASSIFICATION	
	General Emergency	GEN/AAEC
	Site Area Emergency	SIT/AAEC
	Alert	ALE/AAEC
	Unusual Event	UNU/AAEC
X	50.72 Non-Emergency (see next columns)	
	Physical Security (73.71)	DDDD
	Material /Exposure	B???
	Fitness for Duty	HFIT
	Other (see last column)	
	Information Only	NNF

	1-Hr Non-Emergency 10CFR50.72 (b)(1)	
	TS Deviation	ADEV
	4-Hr Non-Emergency 10CFR50.72(b)(2)	
	(i) TS Required S/D	ASHU
	(iv)(A) ECSS Discharge to RCS	ACCS
X	(iv)(B) RPS Actuation (scram)	APRS
	(xi) Offsite Notification	APRE
	8-Hr Non-Emergency 10CFR50.72(b)(3)	
	(ii)(A) Degraded Condition	ADEG
	(ii)(B) Unanalyzed Condition	AUNA
	(iv)(A) Specified System Actuation	AESF

	(v)(A) Safe S/D Capability	AINA
	(v)(B) RHR Capability	AINB
	(v)(C) Control of Rad Release	AINC
	(v)(D) Accident Mitigation	AIND
	(xii) Offsite Medical	AMED
	(xiii) Lost Comm/Asmt/Resp	ACOM
	60-Day Optional 10CFR50.73(a)(1)	
	Invalid Specified System Actuation	AINV
	Other Unspecified Requirement (Identify)	
		NONR
		NONR

DESCRIPTION

Include: Systems affected, actuations & their initiating signals, causes, effect of event on plant, actions taken or planned, etc.

CR No.

Manual actuation of the Reactor Protection System (RPS) due to failure of the Feedwater Level Control system.

NOTIFICATIONS	YES	NO	WILL BE
NRC Resident			
State(s)			
Local			
Other Gov Agencies			
Media/Press Release			

Anything unusual or not understood? ☐ Yes *(Explain above)* ☐ No

Did all systems function as required? ☐ Yes ☐ No *(Explain above)*

Mod of operation until corrected:

Estimated Restart Date:

Additional info on back? ☐ Yes ☐ No

Other Information Requested by NRC:

Name of Individual Contacted:	ENF No.

ANSWER KEY

EVENT NOTIFICATION (Cont.)

PNPP No. 6912 Rev. 1/16/01 — ADDITIONAL INFORMATION — PAP-1604

RADIOLOGICAL RELEASES: CHECK OR FILL IN APPLICABLE ITEMS *(specific details/explanations should be covered in event description)*

	Liquid Release		Gaseous Release		Unplanned Release		Planned Release		Ongoing		Terminated
	Monitored		Unmonitored		Offsite Release		T.S. Exceeded		RM Alarms		Areas Evacuated
	Personnel Exposed or Contaminated				Offsite Protective Actions Recommended				*State release path in description		

	Release Rate (Ci/sec)	% T.S. Limit	HOO Guide	Total Activity (Ci)	% T.S. Limit	HOO Guide
Noble Gas			0.1 Ci/sec			1000 Ci
Iodine			10 uCi/sec			0.01 Ci
Particulate			1 uCi/sec			1 mCi
Liquid *(excluding tritium & dissolved noble gases)*			10 uCi/min			0.1 Ci
Liquid (tritium)			0.2 Ci/min			5 Ci
Total Activity						

	Plant Stack	Condenser/Air Ejector	Main Steam Line	Other
RAD Monitor Readings:				
Alarm Setpoints:				
% T.S. Limit *(if applicable)*				

RCS TUBE LEAKS: CHECK OR FILL IN APPLICABLE ITEMS: *(specific details/explanations should be covered in event description)*

Location of the Leak *(e.g., valve, pipe, etc.):*

Leak Rate:	Units: gpm/gpd	T.S. Limits:	Sudden or Long Term Development:
Leak Start Date:	Time:	Coolant Activity & Units:	
List of Safety Related Equipment Not Operational:			

EVENT DESCRIPTION *(Continued from front)*

(Denote Critical Steps with an asterisk)

* **Performance Step: 6.1.1.1** Ensure that each event or condition is evaluated under the notification requirements specified in Attachment 1.

Standard: Evaluates PAP-1604, Attachment 1, and determines that a Four Hour (10CFR50.72 Non-Emergency) Notification is required due to the RPS actuation (when the reactor is critical).

Comment: Note: The Candidate must determine that the manual reactor scram was not part of the pre-planned shutdown sequence.

Note: The Candidate is not required to physically notify the NRC Operations Center in the next Step.

* **Performance Step: 6.1.1.2** Ensure that the required notifications are made within the specified time requirements to the NRC Operations Center via the Emergency Notification System (ENS) unless otherwise specified in Attachment 1.

Standard: Determines that the notification is due **four hours** from the time of the manual reactor scram.

Comment: **Note: The Evaluator may need to prompt the Candidate to verbalize the latest time that the notification is required to be made.**

*	**Performance Step: 6.1.1.3**	Ensure that information provided to the NRC is recorded on an Event Notification (form PNPP No. 6912, Attachment 3) including any additional information requested by the NRC Duty Officer.
	Standard:	Form PNPP No. 6912, Initial Notification, is properly filled out. • **Only the RPS Actuation box is a critical step for the completion of the form.**
	Comment:	**Cue: If asked, inform the Candidate that the Condition Report (CR) has not been written yet.** **Note: See attached copy of completed Form PNPP No. 6912, Event Notification, in order to verify proper completion of the Event Notification Form.** **Note: The Candidate should identify where he can obtain Form PNPP No. 6912. When Candidate identifies the need for Form PNPP No. 6912, hand the Candidate a blank form.** Note: The Candidate is only required to complete Form PNPP No. 6912 through the completion of the Description block.

Terminating Cue:

When Form PNPP No. 6912, Event Notification, is properly filled out through the completion of the Description block, the evaluation for this JPM is complete.

Job Performance Measure No. 2003 NRC SRO A.1.b

Examinee's Name:

Examiner's Name:

Date Performed:

Facility Evaluator: N/A

Number of Attempts:

Time to complete:

Question Documentation:

Question:

Response:

Result: SAT OR UNSAT

Examiner's Signature and Date: _____ _____

INITIAL CONDITIONS:

You are the on-duty Shift Manager. The plant was conducting a shutdown in preparation for entering a refueling outage. The shutdown schedule called for power to be reduced to 20% and then a manual reactor scram was to be inserted.

Thirty minutes ago while shifting reactor feed pumps, a problem with the Master Level Controller resulted in an excessive feed rate and an increasing RPV level. The Reactor Operator inserted a manual reactor scram just prior to reaching RPV Level 8. Reactor power at the time of the scram was 30%. All equipment operated as designed following the scram and plant conditions are now stable.

INITIATING CUE:

As the Shift Manager, evaluate this event for reportability per PAP-1604, Reports Management, and perform any required notifications to the NRC Operations Center within the required time frame.

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EVENT NOTIFICATION

Perry Nuclear Power Plant

PNPP No. 6912 Rev. 1/16/01 — Unit 1 — PAP-1604

Caller's Name/Signature	Title	Time/Date of Call

Event Time & Zone	Event Date	Power/Mode Before	Power/Mode After

EVENT CLASSIFICATION		
	General Emergency	GEN/AAEC
	Site Area Emergency	SIT/AAEC
	Alert	ALE/AAEC
	Unusual Event	UNU/AAEC
	50.72 Non-Emergency (see next columns)	
	Physical Security (73.71)	DDDD
	Material /Exposure	B???
	Fitness for Duty	HFIT
	Other (see last column)	
	Information Only	NNF

1-Hr Non-Emergency 10CFR50.72 (b)(1)			
	TS Deviation		ADEV
4-Hr Non-Emergency 10CFR50.72(b)(2)			
	(i)	TS Required S/D	ASHU
	(iv)(A)	ECSS Discharge to RCS	ACCS
	(iv)(B)	RPS Actuation (scram)	APRS
	(xi)	Offsite Notification	APRE
8-Hr Non-Emergency 10CFR50.72(b)(3)			
	(ii)(A)	Degraded Condition	ADEG
	(ii)(B)	Unanalyzed Condition	AUNA
	(iv)(A)	Specified System Actuation	AESF

	(v)(A) Safe S/D Capability	AINA
	(v)(B) RHR Capability	AINB
	(v)(C) Control of Rad Release	AINC
	(v)(D) Accident Mitigation	AIND
	(xii) Offsite Medical	AMED
	(xiii) Lost Comm/Asmt/Resp	ACOM
60-Day Optional 10CFR50.73(a)(1)		
	Invalid Specified System Actuation	AINV
Other Unspecified Requirement (Identify)		
		NONR
		NONR

DESCRIPTION

Include: Systems affected, actuations & their initiating signals, causes, effect of event on plant, actions taken or planned, etc.

CR No.

NOTIFICATIONS	YES	NO	WILL BE
NRC Resident			
State(s)			
Local			
Other Gov Agencies			
Media/Press Release			

Anything unusual or not understood? ☐ Yes *(Explain above)* ☐ No

Did all systems function as required? ☐ Yes ☐ No *(Explain above)*

Mod of operation until corrected:

Estimated Restart Date:

Additional info on back? ☐ Yes ☐ No

Other Information Requested by NRC:

Name of Individual Contacted:	ENF No.

EVENT NOTIFICATION (Cont.)

PNPP No. 6912 Rev. 1/16/01 — ADDITIONAL INFORMATION — PAP-1604

RADIOLOGICAL RELEASES: CHECK OR FILL IN APPLICABLE ITEMS *(specific details/explanations should be covered in event description)*

	Liquid Release		Gaseous Release		Unplanned Release		Planned Release		Ongoing		Terminated
	Monitored		Unmonitored		Offsite Release		T.S. Exceeded		RM Alarms		Areas Evacuated
	Personnel Exposed or Contaminated				Offsite Protective Actions Recommended				*State release path in description		

	Release Rate (Ci/sec)	% T.S. Limit	HOO Guide	Total Activity (Ci)	% T.S. Limit	HOO Guide
Noble Gas			0.1 Ci/sec			1000 Ci
Iodine			10 uCi/sec			0.01 Ci
Particulate			1 uCi/sec			1 mCi
Liquid *(excluding tritium & dissolved noble gases)*			10 uCi/min			0.1 Ci
Liquid (tritium)			0.2 Ci/min			5 Ci
Total Activity						

	Plant Stack	Condenser/Air Ejector	Main Steam Line	Other
RAD Monitor Readings:				
Alarm Setpoints:				
% T.S. Limit *(if applicable)*				

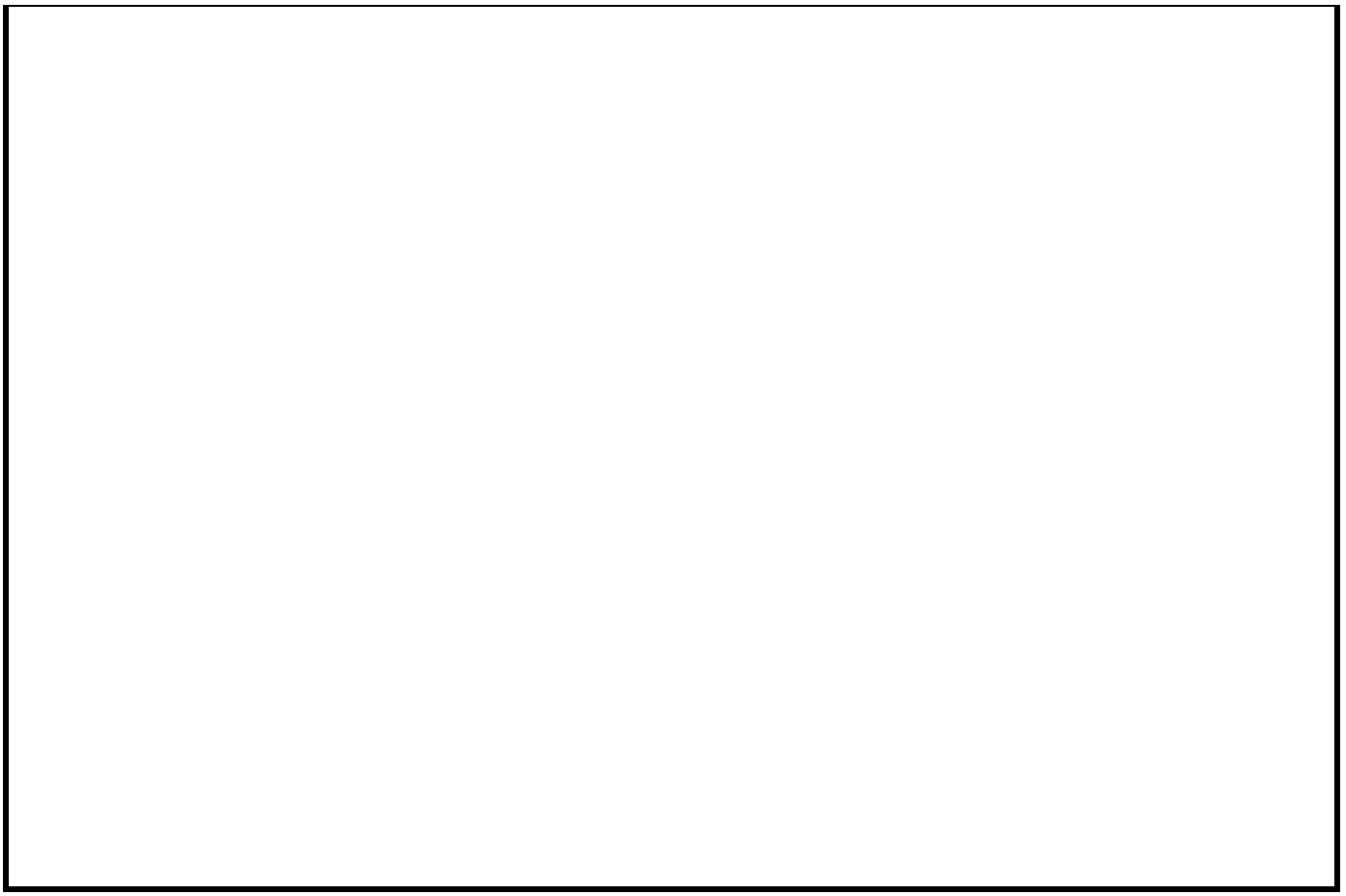
RCS TUBE LEAKS: CHECK OR FILL IN APPLICABLE ITEMS: *(specific details/explanations should be covered in event description)*

Location of the Leak *(e.g., valve, pipe, etc.):*

Leak Rate:	Units: gpm/gpd	T.S. Limits:	Sudden or Long Term Development:
Leak Start Date:	Time:	Coolant Activity & Units:	

List of Safety Related Equipment Not Operational:

EVENT DESCRIPTION *(Continued from front)*



Facility:	Perry	**Task No:**	343-723-03-02 343-692-03-02
Task Title:	Controlling Containment Penetration to Meet the Isolation Requirements of Tech Spec 3.6.1.3	**JPM No:**	2003 NRC SRO A.2
K/A Reference:	2.2.15		
Examinee:		**NRC Examiner:**	
Facility Evaluator:	N/A	**Date:**	

Method of testing

Simulated Performance **Actual Performance**

Classroom **Simulator** **Plant**

Task Standard: The Candidate determines the following administratively-controlled boundaries are required to isolate penetration P113:

Identifies the following automatic valves are to be closed and de-activated (i.e., MCC disconnect opened) with OPS Admin Tags placed on each valve's control switch, MCC disconnect, and local valve handwheel operator;

- Valve 1E12-F028A and its disconnect EF1B07-R
- Valve 1E12-F037A and its disconnect EF1B07-S
- Valve 1E12-F042A and its disconnect EF1B07-X

Required Materials: OAI-1701, Rev 3
TAI-1120-7, Rev 2, PIC 1
PDB-G0001, Rev 1, PIC 5
Technical Specification 3.6.1.3 and Bases
Mechanical and Electrical Drawings – 302 and 208 Series

General References: OAI-1701, Rev 3
TAI-1120-7, Rev 2, PIC 1
PDB-G0001, Rev 1, PIC 5
Technical Specification 3.6.1.3 and Bases
Mechanical and Electrical Drawings – 302 and 208 Series

Time Critical Task: NO

Validation Time: 20 minutes

READ TO THE EXAMINEE

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this Job Performance Measure will be satisfied.

Initial Conditions: The plant is in MODE 1. During the performance of SVI-E12-T2001, RHR A Pump and Valve Operability Test, valve 1E12-F027A (RHR A TO CNTMT SHUTOFF) blew all 3 main line fuses during its valve stroke time test. The valve is currently open and cannot be manually closed because it is mechanically bound. The valve has been declared inoperable and Technical Specification 3.6.1.3 (PCIVs), Condition A has been entered.

Initiating Cue: The Shift Manager, directs you as the Shift Engineer, to determine the component(s) that will require Operations Administrative Tags in accordance with OAI-1701, Attachment 1 in order to comply with Technical Specification 3.6.1.3, Condition A.

(Denote Critical Steps with an asterisk)

Note: The Candidate is <u>not</u> required to prepare the Operations Administrative Control Tagout; only to identify the components to be tagged as discussed in OAI-1701, Attachment 1 (Emergent Items).

Note: The Candidate is <u>not</u> required to generate the LCO Tracking Form or document in the Plant Narrative Log the actions taken to meet Technical Specification 3.6.1.3 Action A as discussed in OAI-1701, Attachment 1 (Emergent Items).

Note: The Candidate may refer to any of the following plant references in order to determine the penetration (P113) associated with Valve 1E12-F027A and the corresponding components that must be administratively controlled:

- **Mechanical and Electrical Drawings – 302 and 208 Series**
- **PDB-G0001, Containment Isolation Valve Table**
- **TAI-1120-7, Containment Penetration Listing**
- **SVI-E12-T9113, Type C LLRT of 1E12 Penetration P113**

*	**Performance Step: OAI-1701, Emergent Items / WCU Clearance Personnel**	Equipment deficiencies resulting in inoperable Primary Containment Isolation Valves should be handled as follows: • The Operations Administrative tags should be prepared for the inoperable equipment and have the same information that would have been provided by the Clearance Unit. When Technical Specifications require a component to be closed and de-activated, the normal practice will be to provide tags for the control switch, disconnect or air supply, and local operator for the component(s) maintaining control of the penetration.
	Standard:	Identifies the following automatic valves are to be closed and de-activated (i.e., MCC disconnect opened) with OPS Admin Tags placed on each valve's control switch, MCC disconnect, and local valve handwheel operator; - Valve 1E12-F028A and its disconnect EF1B07-R - Valve 1E12-F037A and its disconnect EF1B07-S - Valve 1E12-F042A and its disconnect EF1B07-X
	Comment:	Note: The Candidate may identify manual valve 1E12-F604A is to be closed and capped with an OPS Admin Tag placed on its local handwheel. This is <u>not</u> a critical item; it is only a conservative action which is <u>not</u> required to meet Required Action A.1. Note: Since valve 1E12-F027A cannot be closed, this valve <u>cannot</u> be administratively controlled in order to meet Required Action A.1 for Tech Spec 3.6.1.3. Note: Automatic valves 1E12-F028A, 1E12-F037A, and 1E12-F042A are normally closed when RHR A is in standby readiness.

Terminating Cue:

When the Candidate has determined the components that must be administratively controlled in order to isolate penetration P113, the evaluation for this JPM is complete.

Job Performance Measure No. 2003 NRC SRO A.2

Examinee's Name:

Examiner's Name:

Date Performed:

Facility Evaluator: N/A

Number of Attempts:

Time to complete:

Question Documentation:

Question:

Response:

Result: SAT OR UNSAT

Examiner's Signature and Date: _____ _____

INITIAL CONDITIONS:	The plant is in MODE 1. During the performance of SVI-E12-T2001, RHR A Pump and Valve Operability Test, valve 1E12-F027A (RHR A TO CNTMT SHUTOFF) blew all 3 main line fuses during its valve stroke time test. The valve is currently open and cannot be manually closed because it is mechanically bound. The valve has been declared inoperable and Technical Specification 3.6.1.3 (PCIVs), Condition A has been entered.
INITIATING CUE:	The Shift Manager, directs you as the Shift Engineer, to determine the component(s) that will require Operations Administrative Tags in accordance with OAI-1701, Attachment 1 in order to comply with Technical Specification 3.6.1.3, Condition A.

Facility:	Perry	**Task No:**	344-526-05-02 344-532-05-02
Task Title:	Issue a Revised Off-Site Protective Action Recommendation (PAR) Following a Change in Meteorological Conditions	**JPM No:**	2003 NRC SRO A.4
K/A Reference:	2.4.44		
Examinee:		**NRC Examiner:**	
Facility Evaluator:	N/A	**Date:**	

Method of testing

Simulated Performance **Actual Performance**

Classroom **Simulator** Plant

Task Standard: The Candidate revises the default PAR to evacuate Subareas 1, 2 and 3. Within the next 15 minutes, form PNPP No. 7794, Initial Notification, is correctly prepared in accordance with EPI-B1, Emergency Notification System.

Required Materials:
EPI-B1, Rev 11
EPI-B8, Rev 8, PIC 5
Initial Notification (Form PNPP No. 7794)
Marked up copy of Initial Notification (Form PNPP No. 7794)

General References:
EPI-B1, Rev 11
EPI-B8, Rev 8, PIC 5

Time Critical Task: YES 15 minutes for Step 5.1.10

Validation Time: 20 minutes

READ TO THE EXAMINEE

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this Job Performance Measure will be satisfied.

Initial Conditions:

You are the on-duty Shift Manager and Emergency Coordinator. The Plant is in a General Emergency due to a Loss Of Coolant Accident with an unisolable primary containment penetration discharging outside containment.

The General Emergency default Protective Action Recommendation (PAR) was issued to evacuate Subarea 1 and the Lake. (See attached Initial Notification Form) This was based on the meteorological conditions at the time. No dose projection data was initially available.

You have just been provided with the following updated radiological and meteorological conditions due to a change in the wind direction.

Release In Progress

Projected Dose from 5 – 10 miles:
TEDE: < 500 mr
CDEct: < 2 Rem

Wind Speed: 10 MPH
Wind Direction: 350°

Initiating Cue:

As the Emergency Coordinator, evaluate the updated radiological and meteorological conditions with respect to the current default PAR per EPI-B8, Protective Actions and Guides, and perform any required action(s).

ANSWER KEY

INITIAL NOTIFICATION

PNPP No. 7794 Rev. 9/17/01 Page 1 of 2 EPI-B1

1. This is the Perry Nuclear Power Plant:
☒ **Control Room** ☐ **Technical Support Center (TSC)** ☐ **Emergency Operations Facility (EOF)**
☐ **Backup EOF**

(*Communicator*: State your NAME and ERO POSITION TITLE.)

2. This is a(n): ☐ **Actual Emergency** ☒ **Drill**

For step 3 below: Use only step 'a' when classifying or reclassifying an event. Use both steps 'a' & 'b' when simultaneously classifying and terminating from an Unusual Event or Alert. Use step 'c' when classifying after a transitory event. Use step 'd' when revising a protective action recommendation.

3. a. A (n) ☐ **UNUSUAL EVENT** ☐ **ALERT** ☐ **SITE AREA EMERGENCY** ☐ **GENERAL EMERGENCY has been declared at _____ hours on ___/___/__ based on EAL(s): _____**

b. The emergency situation has been terminated at _____ hours on __/__/____.
(Time) (Date)

c. A transitory event has occurred which would have required the declaration of a(n):
☐ **ALERT** ☐ **SITE AREA EMERGENCY** ☐ **GENERAL EMERGENCY**
but was mitigated prior to classification. Current event status is at a(n):
☐ **UNUSUAL EVENT** ☐ **ALERT** ☐ **SITE AREA EMERGENCY**
declared at _____ hours on __/__/_____ based on EAL(s): _____.
(Time) (Date)

d. General Emergency protective actions are being changed at XX:XX hours on X/XX/2003.
(Time) (Date)

4. Brief non-technical description of event: **A Loss of Coolant Accident (LOCA) has occurred with leakage outside containment.**

5. ☐ **a.** NO unplanned radioactive release has occurred.
☒ **b.** An unplanned radioactive release is in progress.

6. Utility recommended protective actions:
☐ **a. None.**
☒ **b. Evacuation of people in Subareas: (1) (2) (3) 4 5 6 7 Lake Erie (circle)**

7. I repeat, this is a(n): ☐ **Actual Emergency** ☒ **Drill**

COMMENTS:

Notification is due at: XX:XX (Time) hours on **X / XX / 2003** (Date); _____ EMERGENCY COORDINATOR APPROVAL (signature)

ANSWER KEY

COMMUNICATOR INSTRUCTIONS:

A. Ensure Items 1-7 (page 1) are completed, and Emergency Coordinator has approved release of information.

B. Pickup the "5-Way" Ringdown. As parties answer, perform a roll call to verify that the State and county agencies listed below are on-line; record time contacted below. If party does NOT answer, initiate a separate call per EPI-B1.

	TIME CONTACTED	5-WAY USED? YES	NO		TIME CONTACTED	5-WAY USED? YES	NO
ASHTABULA COUNTY	_____	☐	☐	LAKE COUNTY	_____	☐	☐
GEAUGA COUNTY	_____	☐	☐	STATE OF OHIO	_____	☐	☐

Read the following: **"Please obtain an Initial Notification form to copy this transmission. Communication on the "5-Way" Circuit is being recorded."** (Pause 5-10 seconds to allow agencies to obtain form before continuing.) Read the following: **"The current date and time is: date / _____ / , _____ time."**

C. Transmit data on page 1. When completed, record the name of contact below; request a call back if the 5-Way was NOT used.

NOTE: The following step can be done in parallel with step C. above if additional communicators are available.

D. Once State and county agencies have been contacted, initiate call on NRC ENS Circuit. Read the following: **"The following is a communication from the Perry Nuclear Power Plant. Communication on the ENS Circuit is being recorded."**

E. Transmit data on page 1. When completed, record the name of contact below; request a call back if the ENS was NOT used.

TIME NRC CONTACTED:_____

If the "5-Way" Ringdown or ENS Circuit was **NOT** used, a verification call back is required.

ORGANIZATION	PERSON CONTACTED	JOB TITLE	TIME OF CALLBACK (If Applicable)
Ashtabula County			☐ N/A
Geauga County			☐ N/A
Lake County			☐ N/A
State of Ohio			☐ N/A
Nuclear Regulatory Commission			☐ N/A

COMMON OFFSITE ACRONYMS:

SD	Sheriff's Department	EOC	Emergency Operations Center
HP	Highway Patrol	EMA	Emergency Management Agency
OSHP	Ohio State Highway Patrol	OEMA	Ohio Emergency Management Agency

Communicator(s) Name: (1) _____ (2) _____

[TSC & EOF ONLY] Forward a copy of completed form to the Information Liaison and Regulatory Affairs Coordinator.

(Denote Critical Steps with an asterisk)

*	**Performance Step: 5.1.3.3**	Ensure that PAR is periodically evaluated based on degrading plant conditions or changes in wind direction or other meteorological conditions using Attachments 2 and 3, as appropriate.
	Standard:	Evaluates EPI-B8, Attachments 2 and 3, and determines PAR must be revised to evacuate **Subareas 1, 2 and 3**, based on the new wind direction.
	Comment:	Note: This Step is critical but not time critical.

Note: Candidate will refer to EPI-B1, Emergency Notification System, to perform the following step.

*	**Performance Step: 5.1.10**	Upon the decision to reclassify the event or issue a revised offsite PAR, perform an initial notification to the State of Ohio, local counties, and NRC per section 5.1.3.
	Standard:	Form PNPP No. 7794, Initial Notification, is properly filled out **within 15 minutes** from the decision to issue the revised PAR. Items 3.d, 6.b, and Emergency Coordinator Approval line are critical steps for completion of the form.
	Comment:	**Note: See attached copy of completed Form PNPP No. 7794, Initial Notification, in order to verify proper completion of the Initial Notification Form.** **Note: The Candidate should identify where he can obtain Form PNPP No. 7794. When Candidate identifies the need for Form PNPP No. 7794, hand the Candidate a blank form.** Note: The Candidate is <u>not</u> required to complete any other E-Plan forms such as PNPP No. 9100, Pager Messages. Note: The Evaluator will role-play as the Control Room Communicator by accepting the completed Form PNPP No. 7794 when the Candidate is ready for the Control Room Communicator to make the initial notifications to the counties, state, and NRC.

Terminating Cue:

When Form PNPP No. 7794, Initial Notification, is properly filled out, the evaluation for this JPM is complete.

Job Performance Measure No. 2003 NRC SRO A.4

Examinee's Name:

Examiner's Name:

Date Performed:

Facility Evaluator: N/A

Number of Attempts:

Time to complete:

Question Documentation:

Question:

Response:

Result: SAT OR UNSAT

Examiner's Signature and Date: _____ _____

INITIAL CONDITIONS:

You are the on-duty Shift Manager and Emergency Coordinator. The Plant is in a General Emergency due to a Loss Of Coolant Accident with an unisolable primary containment penetration discharging outside containment.

The General Emergency default Protective Action Recommendation (PAR) was issued to evacuate Subarea 1 and the Lake. (See attached Initial Notification Form) This was based on the meteorological conditions at the time. No dose projection data was initially available.

You have just been provided with the following updated radiological and meteorological conditions due to a change in the wind direction.

Release In Progress

Projected Dose from 5 – 10 miles:
TEDE: < 500 mr
CDEct: < 2 Rem

Wind Speed: 10 MPH
Wind Direction: 350°

INITIATING CUE:

As the Emergency Coordinator, evaluate the updated radiological and meteorological conditions with respect to the current default PAR per EPI-B8, Protective Actions and Guides, and perform any required action(s).

INITIAL NOTIFICATION

PNPP No. 7794 Rev. 9/17/01 Page 1 of 2 EPI-B1

1. This is the Perry Nuclear Power Plant:
   ☒ Control Room ☐ Technical Support Center (TSC) ☐ Emergency Operations Facility (EOF)
   ☐ Backup EOF

   (*Communicator*: State your NAME and ERO POSITION TITLE.)

2. This is a(n): ☐ Actual Emergency ☒ Drill

For step 3 below: Use only step 'a' when classifying or reclassifying an event. Use both steps 'a' & 'b' when simultaneously classifying and terminating from an Unusual Event or Alert. Use step 'c' when classifying after a transitory event. Use step 'd' when revising a protective action recommendation.

3. a. A (n) ☐ UNUSUAL EVENT ☐ ALERT ☐ SITE AREA EMERGENCY ☒ GENERAL EMERGENCY has been declared at XXXX hours on 214103 based on EAL(s): A61

   b. The emergency situation has been terminated at _____ (Time) hours on __/__/__ (Date).

   c. A transitory event has occurred which would have required the declaration of a(n):
   ☐ ALERT ☐ SITE AREA EMERGENCY ☐ GENERAL EMERGENCY
   but was mitigated prior to classification. Current event status is at a(n):
   ☐ UNUSUAL EVENT ☐ ALERT ☐ SITE AREA EMERGENCY
   declared at _____ (Time) hours on __/__/__ (Date) based on EAL(s): _____.

   d. General Emergency protective actions are being changed at _____ (Time) hours on __/__/__ (Date)

4. Brief non-technical description of event: A Loss of coolant accident has occured with leakage outside containment.

5. ☐ a. NO unplanned radioactive release has occurred.
   ☒ b. An unplanned radioactive release is in progress.

6. Utility recommended protective actions:
   ☐ a. None.
   ☒ b. Evacuation of people in Subareas: (1) 2 3 4 5 6 7 (Lake Erie) (circle)

7. I repeat, this is a(n): ☐ Actual Emergency ☒ Drill

COMMENTS:

Notification is due at: XXXX (Time) hours on 214103 (Date) Robert Manger EMERGENCY COORDINATOR APPROVAL (signature)

INITIAL NOTIFICATION

COMMUNICATOR INSTRUCTIONS:

A. Ensure Items 1-7 (page 1) are completed, and Emergency Coordinator has approved release of information.

B. Pickup the "5-Way" Ringdown. As parties answer, perform a roll call to verify that the State and county agencies listed below are on-line; record time contacted below. If party does NOT answer, initiate a separate call per EPI-B1.

	TIME CONTACTED	5-WAY USED? YES	NO		TIME CONTACTED	5-WAY USED? YES	NO
ASHTABULA COUNTY	XXXX	☒	☐	LAKE COUNTY	XXXX	☒	☐
GEAUGA COUNTY	XXXX	☒	☐	STATE OF OHIO	XXXX	☒	☐

Read the following: **"Please obtain an Initial Notification form to copy this transmission. Communication on the "5-Way" Circuit is being recorded."** (Pause 5-10 seconds to allow agencies to obtain form before continuing.) Read the following: **"The current date and time is: date** 04 04 03, XXXX **time."**

C. Transmit data on page 1. When completed, record the name of contact below; request a call back if the 5-Way was NOT used.

NOTE: The following step can be done in parallel with step C. above if additional communicators are available.

D. Once State and county agencies have been contacted, initiate call on NRC ENS Circuit. Read the following: **"The following is a communication from the Perry Nuclear Power Plant. Communication on the ENS Circuit is being recorded."**

E. Transmit data on page 1. When completed, record the name of contact below; request a call back if the ENS was NOT used.

TIME NRC CONTACTED: XXXX

If the "5-Way" Ringdown or ENS Circuit was **NOT** used, a verification call back is required.

ORGANIZATION	PERSON CONTACTED	JOB TITLE	TIME OF CALLBACK (If Applicable)
Ashtabula County	Joseph Smith	Commissioner	☒ N/A
Geauga County	Robert Simmons	Sheriff	☒ N/A
Lake County	Robert Ausuldish	Commissioner	☒ N/A
State of Ohio	Tom Montgomery	EOC Director	☒ N/A
Nuclear Regulatory Commission	Bill Johnson	Ops Center Supervisor	☒ N/A

COMMON OFFSITE ACRONYMS:

SD	Sheriff's Department	EOC	Emergency Operations Center
HP	Highway Patrol	EMA	Emergency Management Agency
OSHP	Ohio State Highway Patrol	OEMA	Ohio Emergency Management Agency

Communicator(s) Name: (1) Jeffery Sell (2) _____

[TSC & EOF ONLY] Forward a copy of completed form to the Information Liaison and Regulatory Affairs Coordinator.

INITIAL NOTIFICATION

1. This is the Perry Nuclear Power Plant:

☐ **Control Room** ☐ **Technical Support Center (TSC)** ☐ **Emergency Operations Facility (EOF)**
☐ **Backup EOF**

(*Communicator*: State your NAME and ERO POSITION TITLE.)

2. This is a(n): ☐ **Actual Emergency** ☐ **Drill**

For step 3 below: Use only step 'a' when classifying or reclassifying an event. Use both steps 'a' & 'b' when simultaneously classifying and terminating from an Unusual Event or Alert. Use step 'c' when classifying after a transitory event. Use step 'd' when revising a protective action recommendation.

3. a. A (n) ☐ **UNUSUAL EVENT** ☐ **ALERT** ☐ **SITE AREA EMERGENCY** ☐ **GENERAL EMERGENCY has been declared at** _____ **hours on** ____/__/____ **based on EAL(s):** _____.

b. The emergency situation has been terminated at _____ **hours on** __/__/____.
(Time) (Date)

c. A transitory event has occurred which would have required the declaration of a(n):

☐ **ALERT** ☐ **SITE AREA EMERGENCY** ☐ **GENERAL EMERGENCY**

but was mitigated prior to classification. Current event status is at a(n):

☐ **UNUSUAL EVENT** ☐ **ALERT** ☐ **SITE AREA EMERGENCY**
declared at _____ **hours on** __/__/_____ **based on EAL(s):** _____.
(Time) (Date)

d. General Emergency protective actions are being changed at _____ **hours on** __/__/____.
(Time) (Date)

4. Brief non-technical description of event: _____

_____

_____

_____

5. ☐ **a.** NO unplanned radioactive release has occurred.
☐ **b.** An unplanned radioactive release is in progress.

6. Utility recommended protective actions:

☐ **a. None.**
☐ **b. Evacuation of people in Subareas: 1 2 3 4 5 6 7 Lake Erie (circle)**

7. I repeat, this is a(n): ☐ **Actual Emergency** ☐ **Drill**

COMMENTS:

Notification is due at: _____ hours on ____/__/____; _____
(Time) (Date) EMERGENCY COORDINATOR APPROVAL (signature)

INITIAL NOTIFICATION

PNPP No. 7794 Rev. 9/17/01 Page 2 of 2 EPI-B1

COMMUNICATOR INSTRUCTIONS:

A. Ensure Items 1-7 (page 1) are completed, and Emergency Coordinator has approved release of information.

B. Pickup the "5-Way" Ringdown. As parties answer, perform a roll call to verify that the State and county agencies listed below are on-line; record time contacted below. If party does NOT answer, initiate a separate call per EPI-B1.

	TIME CONTACTED	5-WAY USED? YES	NO		TIME CONTACTED	5-WAY USED? YES	NO
ASHTABULA COUNTY	_____	☐	☐	LAKE COUNTY	_____	☐	☐
GEAUGA COUNTY	_____	☐	☐	STATE OF OHIO	_____	☐	☐

Read the following: **"Please obtain an Initial Notification form to copy this transmission. Communication on the "5-Way" Circuit is being recorded."** (Pause 5-10 seconds to allow agencies to obtain form before continuing.) Read the following: **"The current date and time is: date / / , time."**

C. Transmit data on page 1. When completed, record the name of contact below; request a call back if the 5-Way was NOT used.

NOTE: The following step can be done in parallel with step C. above if additional communicators are available.

D. Once State and county agencies have been contacted, initiate call on NRC ENS Circuit. Read the following: **"The following is a communication from the Perry Nuclear Power Plant. Communication on the ENS Circuit is being recorded."**

E. Transmit data on page 1. When completed, record the name of contact below; request a call back if the ENS was NOT used.

TIME NRC CONTACTED:_____

If the "5-Way" Ringdown or ENS Circuit was **NOT** used, a verification call back is required.

ORGANIZATION	PERSON CONTACTED	JOB TITLE	TIME OF CALLBACK (If Applicable)
Ashtabula County			☐ N/A
Geauga County			☐ N/A
Lake County			☐ N/A
State of Ohio			☐ N/A
Nuclear Regulatory Commission			☐ N/A

COMMON OFFSITE ACRONYMS:

SD	Sheriff's Department	EOC	Emergency Operations Center
HP	Highway Patrol	EMA	Emergency Management Agency
OSHP	Ohio State Highway Patrol	OEMA	Ohio Emergency Management Agency

Communicator(s) Name: (1) _____ (2) _____

[TSC & EOF ONLY] Forward a copy of completed form to the Information Liaison and Regulatory Affairs Coordinator.