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February 28, 2003

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
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Washington, D.C. 20555-0001

Gentlemen:

In the Matter of) Docket No. 50-259
Tennessee Valley Authority)

BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 1 - REGULATORY FRAMEWORK FOR THE RESTART OF UNIT 1

Reference: TVA letter dated December 13, 2002, Regulatory Framework for the Restart of Unit 1.

Table 1 of the referenced letter listed NRC generic communications for which TVA has outstanding actions. In the generation of that table, TVA inadvertently omitted Bulletin 79-12, Short Period Scrams At BWR Facilities. This discrepancy was discovered during a review of outstanding commitments on Unit 1. Actions for Bulletin 79-12 have been completed on Units 2 and 3, and TVA's commitment management program still contains the commitment to implement those actions on Unit 1. The commitment was appropriately included in the Unit 1 restart scope. TVA has reviewed Table 1 in the December 13, 2002, letter, and no other omissions were identified. TVA has also reviewed all NRC generic letters and bulletins issued since the original Unit 1 framework letter was issued on July 10, 1991. No additional NRC generic letters or bulletins were identified for which TVA has open commitments.

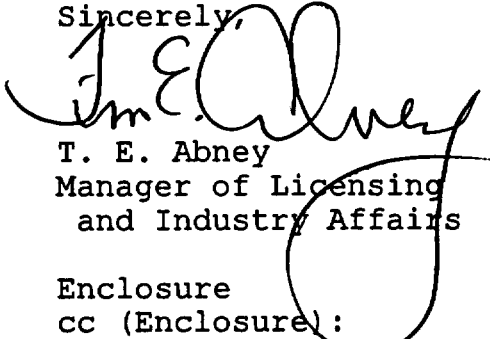
Enclosed is a revision to Table 1 of the referenced letter which includes Bulletin 79-12 as a generic communication action requiring implementation. TVA regrets any confusion this oversight may have caused.

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If you have questions regarding this issue, please call me
at (729) 256-2636.

Sincerely,



T. E. Abney
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Enclosure

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ENCLOSURE

Table 1: Regulatory Issues

Open Commitments to Implement Actions

Bulletins:

79-02	Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts
79-12	Short Period Scrams At BWR Facilities
79-14	Seismic Analysis for As-Built Safety Related Piping Systems
80-06	Engineered Safety Feature Reset Controls
84-02	Failure of GE Type HFA Relays in Use in Class 1E Safety Systems
86-02	Static "O" Ring Differential Pressure Switches
88-07	Power Oscillation in Boiling Water Reactors
88-10	Nonconforming Molded-Case Circuit Breakers
93-03	Resolution of Issues Related to Reactor Vessel Water Level Instrumentation in BWRs
96-03	Potential Plugging of Emergency Core Cooling Suction Strainers by Debris in Boiling Water Reactors

Generic Letters:

82-33	Instrumentation to Follow the Course of an Accident - R.G. 1.97
83-08	Modification of Vacuum Breakers on Mark 1 Containments
83-28	Salem ATWS
88-11	Radiation Embrittlement of Reactor Vessel Materials and its Impact on Plant Operations
88-14	Instrument Air Supply System Problems Affecting Safety Related Equipment
89-06	Safety Parameter Display System
89-08	Erosion/Corrosion-Induced Pipe Wall Thinning
89-10	Safety-Related Motor-Operated Valve Testing and Surveillance
89-16	Installation of a Hardened Wetwell Vent
96-01	Testing of Safety-Related Logic Circuits
96-05	Periodic Verification of Design Basis Capability of Motor Operated Valves
98-01	Readiness of Computer Systems at Nuclear Power Plants

NUREG-0737 (TMI Action Plan) Action Items

I.D.1	Control Room Design Review
I.D.2	Safety Parameter Display Console
II.B.3	Post-Accident Sampling System
II.E.4.2.1 -4	Containment Isolation Dependability - Implement Diverse Isolation
II.F.1.2.C	Accident - Monitoring - Containment High Range Radiation
II.F.1.2.D	Accident - Monitoring - Containment Pressure
II.F.1.2.E	Accident - Monitoring - Containment Water Level
II.F.2.4	(Generic Letter 84-23) - Instrumentation for Detection of Inadequate Core Cooling
II.K.3.13	HPCI/RCIC Initiation Levels
II.K.3.18	ADS Actuation Modifications
II.K.3.28	Qualification of ADS Accumulators

Commitment to Provide Submittal

Bulletins:

90-01 Loss of Fill Oil in Rosemount Transmitters
93-02 Debris Plugging of Emergency Core Cooling Suction Strainers

Generic Letters:

87-02 Verification of Seismic Adequacy of Mechanical and Electrical
 Equipment In Operating Reactors
88-20, S 4 Individual Plant Examination of External Events for Severe Accident
 Vulnerabilities
92-04 Resolution of the Issues Related to Reactor Vessel Water Level
 Instrumentation in BWRs
95-07 Pressure Locking and Thermal Binding of Safety-Related and Power-
 Operated Gate Valves
96-06 Assurance of Equipment Operability and Containment Integrity During
 Design Basis Accident Conditions
97-04 Assurance of Sufficient Net Positive Suction for Emergency Core
 Cooling and Containment Heat Removal Pumps
98-04 Potential for Degradation of the Emergency Core Cooling System and the
 Containment Spray System After a Loss of Coolant Accident Because of
 Construction and Protective Coating Deficiencies and Foreign Material
 in Containment

Requirement for Submittal Within a Specified Period After Actions are Complete

Bulletins:

88-03 Inadequate Latch Engagement in HFA Type Relays Manufactured by GE
88-04 Potential Safety Related Pump Loss
95-02 Unexpected Clogging of a RHR Pump Strainer While Operating in
 Suppression Pool Cooling Mode

Generic Letters:

88-01 NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping
89-13 Service Water System Problems Affecting Safety Related Equipment
92-01 Reactor Vessel Structural Integrity
94-02 Long-Term Solutions and Upgrade of Interim Operating Recommendations
 for Thermal-Hydraulic Instabilities in BWRs
94-03 Intergranular Stress Corrosion Cracking of Core Shrouds in BWRs