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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

May 15, 1979

Docket Nos. 50-266  
and 50-301

LICENSEE: Wisconsin Electric Power Company

FACILITY: Point Beach Nuclear Plant, Unit Nos. 1 and 2

*plant 3.3.9  
(12-1.1)*

SUBJECT: SUMMARY OF MEETING HELD ON MAY 11, 1979 TO DISCUSS THE  
AUXILIARY FEEDWATER SYSTEM AT POINT BEACH NUCLEAR PLANT

On May 11, 1979, the NRC staff met with representatives of Wisconsin Electric Power Company for a detailed discussion of the auxiliary feedwater system at Point Beach Nuclear Plant.

Attendees are listed in Attachment 1.

The meeting was a part of a larger scope task to consider what, if any, changes are required for non-B&W plants in light of the Three Mile Island Unit 2 experience. The material discussed is listed in Attachment 2.

The information gathered at this meeting will be included in an NRC staff report to the Commission. It will include information generic to W plants and specific information on Point Beach. The report will include any recommended changes the staff believes are needed and present conclusions as to whether or not continued operation is justified (all CE and W plants).

At the conclusion of the meeting, the information written by the staff with respect to this system was reviewed with the licensee.

*[Handwritten signature: Charles M. Trammell]*

Charles M. Trammell, Project Manager  
Operating Reactors Branch #1  
Division of Operating Reactors

Attachments:

- 1. List of Attendees
- 2. Material Discussed

cc: w/attachments  
See next page

*HA/25*

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POINT BEACH

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MAY 22 1979

Meeting Summary for  
Wisconsin Electric Power Company - 2 -

Docket Files

NRC PDR

Local PDR

ORBI Reading

NRR Reading

H. Denton

E. Case

Y. Stello

D. Eisenhut

B. Grimes

R. Vollmer

A. Schwencer

D. Ziemann

P. Check

G. Lainas

D. Davis

B. Grimes

T. Ippolito

R. Reid

Y. Noonan

G. Knighton

D. Brinkman

Project Manager

OELD

OI&E (3)

C. Parrish

ACRS (16)

NRC Participants

J. Buchanan

TERA

Licensee

Mr. Bruce Churchill, Esquire

Shaw, Pittman, Potts and Trowbridge

1800 M Street, N.W.

Washington, D. C. 20036

Document Department

University of Wisconsin

Stevens Point Library

Stevens Point, Wisconsin 54481

Mr. Glenn A. Read, Manager

Point Beach Nuclear Plant

Route 3, Box 48

Two Rivers, Wisconsin 54244

LIST OF ATTENDEES  
POINT BEACH NUCLEAR PLANT  
May 11, 1979 MEETING

WISCONSIN ELECTRIC POWER COMPANY

R. Newton  
F. Rhodes

NRC

W. LeFave  
S. Asselin (Sandia)  
C. Trammell

## Attachment 2

As part of its on-going review of the Three Mile Island Unit-2 accident, the staff finds that it needs additional information regarding the auxiliary feedwater systems (AFWS). This information as outlined below, is required to evaluate AFWS reliability for Combustion Engineering (CE) and Westinghouse (W) designed pressurized water reactors. The requested information is in addition to that requested in the IE Bulletins, and should be brought to the meeting scheduled with the staff on May 8 thru May 12, 1979.

Written system description (as built) including:

- List of Support Systems for Auxiliary Feed System Operation (Both Electric and Steam)
- Water Supplies for AFWS (primary and backup)

Current operating procedures and test and maintenance requirements including:

- All LCO's for AFWS, main FW system and related support systems.
- Listing of operator actions (local and/or control room) and timing requirements for such actions.
- Procedures for reinitiating main feedwater flow.

As Built P&IDs with symbol keys including condensate and steam side

Ledgible Equipment layouts drawings including:

- Isometrics, if available
- Identification of inhibits preventing accessibility to AFWS components and related electrical equipment

Relevant control systems description including:

- Schematic or logic control diagrams
- Listing of actuation signals/logic and control
  - MSIS logic for isolating AFWS, if installed
  - electric power dependences
  - All "readouts" available in control room for AFWS operation

AC & DC Power

- One line diagrams (normal and emergency power supplies)
- Divisional designation e.g., Train A, Train B, requirements on all AFWS components and support systems
- List of normal valve states and loss-of-actuation power failure position

Operating Experience, including

- Number of main feedwater interruptions per year experienced to date for each unit
- Number of demands on AFWS per year to date (test and actual) for each unit
- Summary of AFWS malfunctions, problems, failures

Provide Available reliability analyses

Steam Generator dry-out times (assuming loss of all feedwater flow, with 100% initial power, with Reactor trip, no line breaks)

System design bases including:

- Seismic and environmental qualification
- Code and Quality, QA

Provide written responses to the following set of questions by 11/15

Describe backup systems available (to auxiliary feedwater) for providing feedwater to steam generators. Discuss actions and time required to make these systems available. Are procedures available? If so, provide:

Provide the following procedures:

- loss of offsite power
- loss of feedwater
- LOCA (small and large)
- Steam-Line Break

Provide following information for PORV's:

- Number
- capacity
- setpoints (open and close)
- manufacturer and model
- indications of position
- record of periods isolated (isolation valve shut)
- challenges during life of plant (from plant records)
  - including performance of valve, cause of challenge.
- experience of two-phase or subcooled discharge of PORVs and safety valves with description of valve performance

Provide indications of PORV isolation valve in the control room.

Provide the following information on ECCS:

- initiation setpoints
- system description
- pump performance characteristics. (head curves)

Provide reactor protection system trip setpoints.

Provide information on charging pumps, how they relate to ECCS including:

- number
- flow vs. pressure
- power sources and backup
- water sources
- seismic qualification

List all challenges (and cause) to ECCS as indicated on plant records.

List and discuss all instances during which your plant has undergone natural circulation.

Describe all automatic and manual features which can stop the reactor coolant pumps.