

APPENDIX A

**VALIDATION OF THE RADIATION TRANSPORT MODELS
BASED ON NEUTRON DOSIMETRY MEASUREMENTS**

A.1 Neutron Dosimetry

Comparisons of measured dosimetry results to both the calculated and least squares adjusted values for the surveillance capsules withdrawn from Millstone Unit 2 are described herein. The sensor sets from these capsules have been analyzed in accordance with the current dosimetry evaluation methodology described in Regulatory Guide 1.190, "Calculational and Dosimetry Methods for Determining Pressure Vessel Neutron Fluence."^[A-1] One of the main purposes for presenting this material is to demonstrate that the overall measurements agree with the calculated and least squares adjusted values to within $\pm 20\%$ as specified by Regulatory Guide 1.190, thus serving to validate the calculated neutron exposures previously reported in Section 6.2 of this report. This information may also be useful in the future, in particular, as least squares adjustment techniques become accepted in the regulatory environment.

A.1.1 Sensor Reaction Rate Determinations

In this section, the results of the evaluations of three neutron sensor sets withdrawn as part of the Millstone Unit 2 Reactor Vessel Materials Surveillance Program are presented. The capsule designation, location within the reactor, and time of withdrawal of each of these dosimetry sets were as follows:

<u>Capsule ID</u>	<u>Equivalent Azimuthal Location</u>	<u>Withdrawal Time</u>	<u>Irradiation Time [EFPY]</u>
W-97	7°	End of Cycle 3	3.0
W-97 supplemental ^[1]	7°	End of Cycle 10	5.0
W-104	14°	End of Cycle 10	10.0
W-83	7°	End of Cycle 14	15.3

[1] The W-97 supplemental capsule is a fourth dosimeter set that was first put into service at the beginning of the sixth fuel cycle. Since the W-97 supplemental capsule dosimetry differed substantially from the W-104 dosimetry measurements jointly described in Reference A-3, the W-97 supplemental capsule was not used to validate the Millstone Unit 2 transport calculations

The azimuthal locations included in the above tabulation represent the first octant equivalent azimuthal angle of the geometric center of the respective surveillance capsules.

The passive neutron sensors contained in the W-97, W-104, and W-83 surveillance capsules are summarized as follows:

<u>Sensor Material</u>	<u>Reaction Of Interest</u>	<u>Sensor Sets Evaluated</u>		
		<u>Capsule W-97</u>	<u>Capsule W-104</u>	<u>Capsule W-83</u>
Copper [Cd]	$^{63}\text{Cu}(n,\alpha)^{60}\text{Co}$	---	---	---
Iron	$^{54}\text{Fe}(n,p)^{54}\text{Mn}$	X	X	X
Nickel [Cd]	$^{58}\text{Ni}(n,p)^{58}\text{Co}$	X	X	X
Titanium	$^{46}\text{Ti}(n,p)^{46}\text{Sc}$	X	X	X
Uranium-238	$^{238}\text{U}(n,f)^{137}\text{Cs}$	---	---	---
Uranium-238 [Cd]	$^{238}\text{U}(n,f)^{137}\text{Cs}$	X	X	---
Cobalt-Aluminum	$^{59}\text{Co}(n,\gamma)^{60}\text{Co}$	X	---	X
Cobalt-Aluminum [Cd]	$^{59}\text{Co}(n,\gamma)^{60}\text{Co}$	X	---	X
Sulfur	$^{32}\text{S}(n,p)^{32}\text{P}$	---	---	---

In regards to the neutron sensors listed above, the cadmium-covered ^{63}Cu sensor reaction was not measured for capsule W-83 since the copper wire amalgamated with the cadmium. Similarly, the capsule W-97 ^{63}Cu sensor was also rejected on the basis of sample integrity since melting of the cadmium shield had also occurred. The cadmium-covered ^{63}Cu sensor reaction for capsule W-104 was also rejected based on previous performance issues with cadmium melting in the aforementioned capsules.

The bare uranium sensor measurements for capsules W-97 and W-83 were also excluded from this assessment. The bare $^{238}\text{U}(\text{n},\text{f})$ measurement is dominated by contributions from thermal neutron reactions in ^{235}U impurities. These thermal contributions add significant uncertainty to the determination of the $^{238}\text{U}(\text{n},\text{f})$ reaction rate. The bare $^{238}\text{U}(\text{n},\text{f})$ sensor reactions for capsule W-104 were not reported in Reference A-3.

The cadmium-covered ^{238}U sensor provides greater accuracy in the measurement of this fast neutron reaction and was therefore included in the reevaluation of capsules W-97 and W-104. However, the cadmium-covered ^{238}U sensors for Capsule W-83 were statistically inconsistent (in excess of 3σ) with $^{238}\text{U}(\text{n},\text{f})^{137}\text{Cs}[\text{Cd}]$ measurements from other similar plants; hence, this sensor reaction rate was excluded from this assessment.

The sulfur sensor reaction was not measured for capsule W-83 due to the short half-life of ^{32}P (14.28 days); similarly, these sensors were not reevaluated for capsule W-97. Furthermore, the sulfur sensor reactions for capsule W-104 were not reported in Reference A-3. Therefore, the sulfur reaction was not utilized in the assessment of these capsules.

Pertinent physical and nuclear characteristics of the passive neutron sensors are listed in Table A-1. The use of passive monitors such as those listed above does not yield a direct measure of the energy dependent neutron flux at the point of interest. Rather, the activation or fission process is a measure of the integrated effect that the time and energy dependent neutron flux has on the target material over the course of the irradiation period. An accurate assessment of the average neutron flux level incident on the various monitors may be derived from the activation measurements only if the irradiation parameters are well known. In particular, the following variables are of interest:

- the measured specific activity of each monitor,
- the physical characteristics of each monitor,
- the operating history of the reactor,
- the energy response of each monitor, and
- the neutron energy spectrum at the monitor location.

The radiometric counting of the neutron sensors from capsule W-97 was reported by Combustion Engineering (C-E)^[A-2]. The radiometric counting of the neutron sensors from capsule W-104 was reported by Babcock & Wilcox (B&W)^[A-3]. The radiometric counting of the sensors from capsule W-83 was completed at the Pace Analytical Services Laboratory located at the Westinghouse Waltz Mill Site. The Pace radiometric counting followed established ASTM procedures. Following sample preparation and weighing, the specific activity of each sensor was determined by means of a high-resolution gamma spectrometer. For the iron, nickel, titanium, and cobalt-aluminum sensors, these analyses were

performed by direct counting of each of the individual samples. In the case of the uranium fission sensors, the analyses were carried out by direct counting preceded by dissolution and chemical separation of cesium from the sensor material.

The irradiation history of the reactor over the irradiation periods experienced by capsules W-97, W-104, and W-83 was based on the reported monthly power generation of Millstone Unit 2 from initial reactor criticality through the end of the dosimetry evaluation period. For the sensor sets utilized in the surveillance capsules, the half-lives of the product isotopes are long enough that a monthly histogram describing reactor operation has proven to be an adequate representation for use in radioactive decay corrections for the reactions of interest in the exposure evaluations. The irradiation history applicable to capsules W-97, W-104, and W-83 is given in Table A-2.

Having the measured specific activities, the physical characteristics of the sensors, and the operating history of the reactor, reaction rates referenced to full-power operation were determined from the following equation:

$$R = \frac{A}{N_0 F Y \sum \frac{P_j}{P_{ref}} C_j [1 - e^{-\lambda t_j}] [e^{-\lambda t_d}]}$$

where:

- R = Reaction rate averaged over the irradiation period and referenced to operation at a core power level of P_{ref} (rps/nucleus).
- A = Measured specific activity (dps/gm).
- N_0 = Number of target element atoms per gram of sensor.
- F = Weight fraction of the target isotope in the sensor material.
- Y = Number of product atoms produced per reaction.
- P_j = Average core power level during irradiation period j (MW).
- P_{ref} = Maximum or reference power level of the reactor (MW).
- C_j = Calculated ratio of $\phi(E > 1.0 \text{ MeV})$ during irradiation period j to the time weighted average $\phi(E > 1.0 \text{ MeV})$ over the entire irradiation period.
- λ = Decay constant of the product isotope (1/sec).
- t_j = Length of irradiation period j (sec).
- t_d = Decay time following irradiation period j (sec).

and the summation is carried out over the total number of monthly intervals comprising the irradiation period.

In the equation describing the reaction rate calculation, the ratio $[P_j]/[P_{ref}]$ accounts for month-by-month variation of reactor core power level within any given fuel cycle as well as over multiple fuel cycles. The

ratio C_j , which was calculated for each fuel cycle using the transport methodology discussed in Section 6.2, accounts for the change in sensor reaction rates caused by variations in flux level induced by changes in core spatial power distributions from fuel cycle to fuel cycle. For a single-cycle irradiation, C_j is normally taken to be 1.0. However, for multiple-cycle irradiations, particularly those employing low leakage fuel management, the additional C_j term should be employed. The impact of changing flux levels for constant power operation can be quite significant for sensor sets that have been irradiated for many cycles in a reactor that has transitioned from non-low leakage to low leakage fuel management. The fuel cycle specific neutron flux values along with the computed values for C_j are listed in Table A-3. These flux values represent the cycle dependent results at the radial and azimuthal center of the respective capsules at the axial elevation of the active fuel midplane.

Calculations for the reactions whose products have short half-lives indicated that C_j factors based on cycle average flux values were appropriate for capsule W-97 but not appropriate for capsules W-104 and W-83 due to the change in the spectra over the life of the surveillance capsules resulting from the removal of the thermal shield at the end of the fifth fuel cycle. The effect of this spectral change was accounted for by determining C_j factors based on individual reaction rates. As a result, the C_j factors that were utilized in the final analyses for capsules W-104 and W-83 are based on individual reaction rates determined from the synthesized transport calculations as reported in Table A-3.

Prior to using the measured reaction rates in the least-squares evaluations of the dosimetry sensor sets, corrections were made to the ^{238}U measurements to account for the presence of ^{235}U impurities in the sensors as well as to adjust for the build-in of plutonium isotopes over the course of the irradiation. Corrections were also made to the ^{238}U sensor reaction rates to account for gamma ray induced fission reactions that occurred over the course of the capsule irradiation. The correction factors applied to the Millstone Unit 2 fission sensor reaction rates are summarized as follows:

Correction	Capsule W-97	Capsule W-104	Capsule W-83 ^[1]
^{235}U Impurity/Pu Build-in	0.872	0.848	0.819
$^{238}\text{U}(\gamma, f)$	0.903	0.844	0.846
Net ^{238}U Correction	0.787	0.716	0.693

[1] The cadmium covered U foil from this dosimetry set was not used in the least squares evaluation for the W-83 capsule since it was statistically inconsistent with comparable measurement data obtained from similar plants.

These factors were applied in a multiplicative fashion to the decay corrected uranium fission sensor reaction rates.

Results of the sensor reaction rate determinations for capsules W-97, W-104 and W-83 are given in Table A-4. In Table A-4, the measured specific activities, decay corrected saturated specific activities, and computed reaction rates for each sensor indexed to the radial center of the capsule are listed. The fission sensor reaction rates are listed both with and without the applied corrections for ^{238}U impurities, plutonium build-in, and gamma ray induced fission effects.

A.1.2 Least Squares Evaluation of Sensor Sets

Least squares adjustment methods provide the capability of combining the measurement data with the corresponding neutron transport calculations resulting in a Best Estimate neutron energy spectrum with

associated uncertainties. Best Estimates for key exposure parameters such as $\phi(E > 1.0 \text{ MeV})$ or dpa/s along with their uncertainties are then easily obtained from the adjusted spectrum. In general, the least squares methods, as applied to surveillance capsule dosimetry evaluations, act to reconcile the measured sensor reaction rate data, dosimetry reaction cross-sections, and the calculated neutron energy spectrum within their respective uncertainties. For example,

$$R_i \pm \delta_{R_i} = \sum_g (\sigma_{ig} \pm \delta_{\sigma_{ig}})(\phi_g \pm \delta_{\phi_g})$$

relates a set of measured reaction rates, R_i , to a single neutron spectrum, ϕ_g , through the multigroup dosimeter reaction cross-section, σ_{ig} , each with an uncertainty δ . The primary objective of the least squares evaluation is to produce unbiased estimates of the neutron exposure parameters at the location of the measurement.

For the least squares evaluation of the Millstone Unit 2 surveillance capsule dosimetry, the FERRET code^[A-4] was employed to combine the results of the plant specific neutron transport calculations and sensor set reaction rate measurements to determine best-estimate values of exposure parameters ($\phi(E > 1.0 \text{ MeV})$ and dpa) along with associated uncertainties for the three in-vessel capsules considered herein.

The application of the least squares methodology requires the following input:

- 1 - The calculated neutron energy spectrum and associated uncertainties at the measurement location.
- 2 - The measured reaction rates and associated uncertainty for each sensor contained in the multiple foil set.
- 3 - The energy dependent dosimetry reaction cross-sections and associated uncertainties for each sensor contained in the multiple foil sensor set.

For the Millstone Unit 2 application, the calculated neutron spectrum was obtained from the results of plant specific neutron transport calculations described in Section 6.2 of this report. The sensor reaction rates were derived from the measured specific activities using the procedures described in Section A.1.1. The dosimetry reaction cross-sections and uncertainties were obtained from the Sandia National Laboratory Radiation Metrology Laboratory (SNLRML) dosimeter cross-section library^[A-5]. The SNLRML library is an evaluated dosimetry reaction cross-section compilation recommended for use in LWR evaluations by ASTM Standard E1018, "Application of ASTM Evaluated Cross-Section Data File, Matrix E 706 (IIB)".

The uncertainties associated with the measured reaction rates, dosimetry cross-sections, and calculated neutron spectrum were input to the least squares procedure in the form of variances and covariances. The assignment of the input uncertainties followed the guidance provided in ASTM Standard E 944, "Application of Neutron Spectrum Adjustment Methods in Reactor Surveillance."

The following provides a summary of the uncertainties associated with the least squares evaluation of the Millstone Unit 2 surveillance capsule sensor sets.

Reaction Rate Uncertainties

The overall uncertainty associated with the measured reaction rates includes components due to the basic measurement process, irradiation history corrections, and corrections for competing reactions. A high level of accuracy in the reaction rate determinations is assured by utilizing laboratory procedures that conform to the ASTM National Consensus Standards for reaction rate determinations for each sensor type.

After combining all of these uncertainty components, the sensor reaction rates derived from the counting and data evaluation procedures were assigned the following net uncertainties for input to the least squares evaluation:

Reaction	Uncertainty
$^{63}\text{Cu}(n,\alpha)^{60}\text{Co}$	5%
$^{54}\text{Fe}(n,p)^{54}\text{Mn}$	5%
$^{58}\text{Ni}(n,p)^{58}\text{Co}$	5%
$^{46}\text{Ti}(n,p)^{46}\text{Sc}$	5%
$^{238}\text{U}(n,f)^{137}\text{Cs}$	10%
$^{59}\text{Co}(n,\gamma)^{60}\text{Co}$	5%

These uncertainties are given at the 1σ level.

Dosimetry Cross-Section Uncertainties

The reaction rate cross-sections used in the least squares evaluations were taken from the SNLRML library. This data library provides reaction cross-sections and associated uncertainties, including covariances, for 66 dosimetry sensors in common use. Both cross-sections and uncertainties are provided in a fine multigroup structure for use in least squares adjustment applications. These cross-sections were compiled from the most recent cross-section evaluations and they have been tested with respect to their accuracy and consistency for least squares evaluations. Further, the library has been empirically tested for use in fission spectra determination as well as in the fluence and energy characterization of 14 MeV neutron sources.

For sensors included in the Millstone Unit 2 surveillance program, the following uncertainties in the fission spectrum averaged cross-sections are provided in the SNLRML documentation package.

Reaction	Uncertainty
$^{63}\text{Cu}(n,\alpha)^{60}\text{Co}$	4.08-4.16%
$^{54}\text{Fe}(n,p)^{54}\text{Mn}$	3.05-3.11%
$^{58}\text{Ni}(n,p)^{58}\text{Co}$	4.49-4.56%
$^{46}\text{Ti}(n,p)^{46}\text{Sc}$	4.51-4.87%
$^{238}\text{U}(n,f)^{137}\text{Cs}$	0.54-0.64%
$^{59}\text{Co}(n,\gamma)^{60}\text{Co}$	0.79-3.59%

These tabulated ranges provide an indication of the dosimetry cross-section uncertainties associated with the sensor sets used in LWR irradiations.

Calculated Neutron Spectrum

The neutron spectra input to the least squares adjustment procedure were obtained directly from the results of plant specific transport calculations for each surveillance capsule irradiation period and location. The spectrum for each capsule was input in an absolute sense (rather than as simply a relative spectral shape). Therefore, within the constraints of the assigned uncertainties, the calculated data were treated equally with the measurements.

While the uncertainties associated with the reaction rates were obtained from the measurement procedures and counting benchmarks and the dosimetry cross-section uncertainties were supplied directly with the SNLRML library, the uncertainty matrix for the calculated spectrum was constructed from the following relationship:

$$M_{gg'} = R_n^2 + R_g * R_{g'} * P_{gg'}$$

where R_n specifies an overall fractional normalization uncertainty and the fractional uncertainties R_g and $R_{g'}$ specify additional random groupwise uncertainties that are correlated with a correlation matrix given by:

$$P_{gg'} = [1 - \theta] \delta_{gg'} + \theta e^{-H}$$

where

$$H = \frac{(g - g')^2}{2\gamma^2}$$

The first term in the correlation matrix equation specifies purely random uncertainties, while the second term describes the short-range correlation's over a group range γ (θ specifies the strength of the latter term). The value of δ is 1.0 when $g = g'$, and is 0.0 otherwise.

The set of parameters defining the input covariance matrix for the Millstone Unit 2 calculated spectra was as follows:

Flux Normalization Uncertainty (R_n)	15%
Flux Group Uncertainties ($R_g, R_{g'}$)	
($E > 0.0055$ MeV)	15%
(0.68 eV $< E < 0.0055$ MeV)	29%
($E < 0.68$ eV)	52%
Short Range Correlation (θ)	
($E > 0.0055$ MeV)	0.9
(0.68 eV $< E < 0.0055$ MeV)	0.5
($E < 0.68$ eV)	0.5

Flux Group Correlation Range (γ)	
($E > 0.0055$ MeV)	6
(0.68 eV $< E < 0.0055$ MeV)	3
($E < 0.68$ eV)	2

A.1.3 Comparisons of Measurements and Calculations

Results of the least squares evaluations of the dosimetry from the Millstone Unit 2 surveillance capsules considered herein are provided in Tables A-5 and A-6. In Table A-5, measured, calculated, and best-estimate values for sensor reaction rates are given for each capsule. Also provided in this tabulation are ratios of the measured reaction rates to both the calculated and least squares adjusted reaction rates. These ratios of M/C and M/BE illustrate the consistency of the fit of the calculated neutron energy spectra to the measured reaction rates both before and after adjustment. In Table A-6, comparison of the calculated and best estimate values of neutron flux ($E > 1.0$ MeV) and iron atom displacement rate are tabulated along with the BE/C ratios observed for each of the capsules.

The data comparisons provided in Tables A-5 and A-6 show that the adjustments to the calculated spectra are relatively small and well within the assigned uncertainties for the calculated spectra, measured sensor reaction rates, and dosimetry reaction cross-sections. Further, these results indicate that the use of the least squares evaluation results in a reduction in the uncertainties associated with the exposure of the surveillance capsules. From Section 6.4 of this report, it may be noted that the uncertainty associated with the unadjusted calculation of neutron fluence ($E > 1.0$ MeV) and iron atom displacements at the surveillance capsule locations is specified as 12% at the 1σ level. From Table A-6, it is noted that the corresponding uncertainties associated with the least squares adjusted exposure parameters have been reduced to 6-7% for neutron flux ($E > 1.0$ MeV) and 6% for iron atom displacement rate. Again, the uncertainties from the least-squares evaluation are at the 1σ level.

Further comparisons of the measurement results with calculations are given in Tables A-7 and A-8. These comparisons are given on two levels. In Table A-7, calculations of individual threshold sensor reaction rates are compared directly with the corresponding measurements. These threshold reaction rate comparisons provide a good evaluation of the accuracy of the fast neutron portion of the calculated energy spectra. In Table A-8, calculations of fast neutron exposure rates in terms of $\phi(E > 1.0$ MeV) and dpa/s are compared with the best estimate results obtained from the least squares evaluation of the capsule dosimetry results. These two levels of comparison yield consistent and similar results with all measurement-to-calculation comparisons falling well within the 20% limits specified as the acceptance criteria in Regulatory Guide 1.190.

It should be noted that although comparisons between the measured and calculated values for the ^{46}Ti sensors are included in Table A-7, they were not used in determining the average measurement to calculation (M/C) ratios since a bias exists in the SNLRML cross section for the $^{46}\text{Ti}(n,p)$ reaction. This bias may be observed in the data contained in ASTM Standard Practice E261, "Determining Neutron Fluence, Fluence Rate, and Spectra by Radioactivation Techniques." Specifically, Table 3 of ASTM E261 indicates that the sum in quadrature of the experimental uncertainty and the calculated uncertainty for $^{46}\text{Ti}(n,p)^{46}\text{Sc}$ in the ^{235}U thermal fission field is 6.86%. Also indicated in the same table is the ratio of the calculated cross-section to the experimentally measured cross section (C/E) that is given

as 0.899. Since the difference between the calculated and measured cross-section is greater than the uncertainties involved supports the hypothesis that the calculated cross-section is biased low.

In the case of the direct comparison of measured and calculated sensor reaction rates, the M/C comparisons for fast neutron reactions range from 0.87–1.15 for the 8 samples included in the data set. The overall average M/C ratio for the entire set of Millstone Unit 2 data is 1.00 with an associated standard deviation of 9.6%.

In the comparisons of best estimate and calculated fast neutron exposure parameters, the corresponding BE/C comparisons for the capsule data sets range from 0.94–1.12 for neutron flux ($E > 1.0$ MeV) and from 0.93 to 1.10 for iron atom displacement rate. The overall average BE/C ratios for neutron flux ($E > 1.0$ MeV) and iron atom displacement rate are 1.02 with a standard deviation of 9.1% and 1.01 with a standard deviation of 8.6%, respectively.

Based on these comparisons, it is concluded that the calculated fast neutron exposures provided in Section 6.2 of this report are validated for use in the assessment of the condition of the materials comprising the beltline region of the Millstone Unit 2 reactor pressure vessel.

Table A-1

Nuclear Parameters Used In The Evaluation Of Neutron Sensors

<u>Monitor Material</u>	<u>Reaction of Interest</u>	<u>Target Atom Fraction^[1]</u>	<u>90% Response Range^[2] (MeV)</u>	<u>Product Half-life</u>	<u>Fission Yield (%)</u>
Copper	$^{63}\text{Cu}(n,\alpha)$	0.6917	5.0 – 12.0	5.271y	
Iron	$^{54}\text{Fe}(n,p)$	0.0585	2.3 – 8.8	312.3 d	
Nickel	$^{58}\text{Ni}(n,p)$	0.6808	1.9 – 8.8	70.82 d	
Titanium	$^{46}\text{Ti}(n,p)$	0.0800	4.0 – 10.5	83.81 d	
Uranium-238	$^{238}\text{U}(n,f)$	1.0000	1.4 – 8.0	30.07 y	6.02
Cobalt-Aluminum	$^{59}\text{Co}(n,\gamma)$	0.0015	non-threshold	5.271 y	

[1] The counting results identified by B&W for the capsule W-104 reactions were reported in Reference A-3 based on the weight of the target material in the sample rather than the total weight of the dosimeter material. As a result, the target atom fraction used in the analysis of the capsule W-104 sensors was unity.

[2] The 90% response range is defined such that, in the neutron spectrum characteristic of the Millstone Unit 2 surveillance capsules located at 7° and 14° from the core cardinal axes, approximately 90% of the sensor response is due to neutrons in the energy range specified with approximately 5% of the total response due to neutrons with energies below the lower limit and 5% of the total response due to neutrons with energies above the upper limit.

Table A-2

Monthly Thermal Generation During The First Fourteen Fuel Cycles
of The Millstone Unit 2 Reactor

<u>Year</u>	<u>Month</u>	<u>Thermal Generation (MWt-hr)</u>	<u>Year</u>	<u>Month</u>	<u>Thermal Generation (MWt-hr)</u>	<u>Year</u>	<u>Month</u>	<u>Thermal Generation (MWt-hr)</u>
1975	10	0	1979	1	1662527	1982	1	0
1975	11	169382	1979	2	1692785	1982	2	0
1975	12	470151	1979	3	539470	1982	3	808939.7
1976	1	726120	1979	4	0	1982	4	1570359
1976	2	727079	1979	5	417578	1982	5	1982946
1976	3	993918	1979	6	1230123	1982	6	1935852
1976	4	1275861	1979	7	1989041	1982	7	1020076
1976	5	1161586	1979	8	745654	1982	8	1724588
1976	6	1534194	1979	9	1933841	1982	9	1703478
1976	7	1197834	1979	10	1991854	1982	10	1693915
1976	8	1373055	1979	11	0	1982	11	1644259
1976	9	1727678	1979	12	1613522	1982	12	1938553
1976	10	1787608	1980	1	2001005	1983	1	1801321
1976	11	1740007	1980	2	1683788	1983	2	1634909
1976	12	910583	1980	3	1740751	1983	3	806911
1977	1	1004692	1980	4	1831957	1983	4	1921628
1977	2	1697538	1980	5	499545	1983	5	1712695
1977	3	1849165	1980	6	404890	1983	6	0
1977	4	1348306	1980	7	1900317	1983	7	0
1977	5	368843	1980	8	958023	1983	8	0
1977	6	362122	1980	9	0	1983	9	0
1977	7	1329348	1980	10	659001	1983	10	0
1977	8	1885613	1980	11	1862268	1983	11	0
1977	9	1667984	1980	12	1995148	1983	12	0
1977	10	1516798	1981	1	782184	1984	1	809931
1977	11	1205392	1981	2	1752019	1984	2	1507373
1977	12	0	1981	3	2001106	1984	3	1971567
1978	1	0	1981	4	1934118	1984	4	1935357
1978	2	0	1981	5	987167	1984	5	1998215
1978	3	0	1981	6	1853185	1984	6	1877970
1978	4	107169	1981	7	1963023	1984	7	1903852
1978	5	1596598	1981	8	1904455	1984	8	1995846
1978	6	1752746	1981	9	1923108	1984	9	1910948
1978	7	1790474	1981	10	1869618	1984	10	2001607
1978	8	1806986	1981	11	1942822	1984	11	1524599
1978	9	1704986	1981	12	267821	1984	12	1935523
1978	10	1875411						
1978	11	1839813						
1978	12	1900976						

Table A-2 cont'd

Monthly Thermal Generation During The First Fourteen Fuel Cycles
of The Millstone Unit 2 Reactor

<u>Year</u>	<u>Month</u>	<u>Thermal Generation (MWt-hr)</u>	<u>Year</u>	<u>Month</u>	<u>Thermal Generation (MWt-hr)</u>	<u>Year</u>	<u>Month</u>	<u>Thermal Generation (MWt-hr)</u>
1985	1	1994395	1988	1	0	1991	1	1323684
1985	2	900708	1988	2	439630	1991	2	1687526
1985	3	0	1988	3	2006818	1991	3	2008452
1985	4	0	1988	4	1494187	1991	4	1368599
1985	5	0	1988	5	956578	1991	5	803890
1985	6	0	1988	6	1400732	1991	6	0
1985	7	1247903	1988	7	2008427	1991	7	1352310
1985	8	1963363	1988	8	2008495	1991	8	540061
1985	9	1710661	1988	9	1936390	1991	9	1170466
1985	10	0	1988	10	1885985	1991	10	1708110
1985	11	1411904	1988	11	1943414	1991	11	342421
1985	12	1990411	1988	12	2008362	1991	12	160881
1986	1	1984099	1989	1	2008467	1992	1	1709024
1986	2	1813645	1989	2	201767	1992	2	975691
1986	3	2000735	1989	3	0	1992	3	2008454
1986	4	1940693	1989	4	2469	1992	4	1940643
1986	5	1809751	1989	5	1833467	1992	5	1872406
1986	6	1799531	1989	6	1938383	1992	6	0
1986	7	2007906	1989	7	2004115	1992	7	0
1986	8	1670198	1989	8	2001979	1992	8	0
1986	9	1121241	1989	9	1842429	1992	9	0
1986	10	0	1989	10	1189086	1992	10	0
1986	11	0	1989	11	454022	1992	11	0
1986	12	496285	1989	12	2008374	1992	12	0
1987	1	1772860	1990	1	2008226	1993	1	968933
1987	2	757661	1990	2	1813945	1993	2	1573593
1987	3	1990243	1990	3	2008300	1993	3	2008013
1987	4	1863177	1990	4	1870658	1993	4	1870993
1987	5	2006240	1990	5	455773	1993	5	1883361
1987	6	1942702	1990	6	945947	1993	6	1859045
1987	7	1936769	1990	7	2008021	1993	7	2008410
1987	8	1977157	1990	8	1869028	1993	8	1347720
1987	9	1792000	1990	9	901501	1993	9	902578
1987	10	2008169	1990	10	0	1993	10	1355882
1987	11	1840647	1990	11	1216781	1993	11	1848160
1987	12	1879773	1990	12	1822046	1993	12	1991761

Table A-2 cont'd

Monthly Thermal Generation During The First Fourteen Fuel Cycles
of The Millstone Unit 2 Reactor

<u>Year</u>	<u>Month</u>	<u>Thermal Generation (MWt-hr)</u>	<u>Year</u>	<u>Month</u>	<u>Thermal Generation (MWt-hr)</u>	<u>Year</u>	<u>Month</u>	<u>Thermal Generation (MWt-hr)</u>
1994	1	1987044	1997	1	0	2000	1	1793229
1994	2	1808654	1997	2	0	2000	2	803563.6
1994	3	2004989	1997	3	0	2000	3	1995614
1994	4	1420483	1997	4	0	2000	4	1353661
1994	5	0	1997	5	0	2000	5	0
1994	6	736521	1997	6	0	2000	6	1640187
1994	7	1715901	1997	7	0	2000	7	2002629
1994	8	0	1997	8	0	2000	8	2003897
1994	9	1743833	1997	9	0	2000	9	1923670
1994	10	8969	1997	10	0	2000	10	1984571
1994	11	0	1997	11	0	2000	11	1936391
1994	12	0	1997	12	0	2000	12	1996304
1995	1	0	1998	1	0	2001	1	2000397
1995	2	0	1998	2	0	2001	2	1809984
1995	3	0	1998	3	0	2001	3	1974906
1995	4	0	1998	4	0	2001	4	1809699
1995	5	0	1998	5	0	2001	5	1537819
1995	6	0	1998	6	0	2001	6	1934469
1995	7	0	1998	7	0	2001	7	1991596
1995	8	1127403	1998	8	0	2001	8	1802924
1995	9	1943543	1998	9	0	2001	9	1941279
1995	10	2008689	1998	10	0	2001	10	2002856
1995	11	1921802	1998	11	0	2001	11	1860739
1995	12	1594145	1998	12	0	2001	12	1976053
1996	1	2007749	1999	1	0	2002	1	1945747
1996	2	1232628	1999	2	0	2002	2	835263.3
1996	3	0	1999	3	0	2002	3	0
1996	4	0	1999	4	0			
1996	5	0	1999	5	712321.6			
1996	6	0	1999	6	1942647			
1996	7	0	1999	7	1989754			
1996	8	0	1999	8	1986803			
1996	9	0	1999	9	1432859			
1996	10	0	1999	10	2004610			
1996	11	0	1999	11	1897621			
1996	12	0	1999	12	1918186			

Table A-3

Calculated C_j Factors at the Surveillance Capsule Center
Core Midplane Elevation

Fuel Cycle	$\phi(E > 1.0 \text{ MeV}) [\text{n/cm}^2\text{-s}]$			$C_j^{[1]}$		
	Capsule W-97	Capsule W-104	Capsule W-83	Capsule W-97	Capsule W-104	Capsule W-83
1	3.00E+10	2.10E+10	3.00E+10	0.89	0.70	0.83
2	3.35E+10	2.36E+10	3.35E+10	0.99	0.79	0.93
3	3.97E+10	2.80E+10	3.97E+10	1.18	0.93	1.10
4		2.68E+10	3.84E+10		0.90	1.07
5		2.66E+10	3.79E+10		0.89	1.05
6		3.90E+10	5.45E+10		1.30	1.51
7		4.00E+10	5.58E+10		1.34	1.55
8		4.02E+10	5.59E+10		1.34	1.55
9		3.94E+10	5.48E+10		1.31	1.52
10		1.96E+10	2.46E+10		0.65	0.68
11			2.81E+10			0.78
12			2.33E+10			0.65
13			2.47E+10			0.69
14			2.46E+10			0.68
Average	3.37E+10	2.99E+10	3.60E+10	1.00	1.00	1.00

[1] The C_j factors based on the ratio of the cycle specific fast ($E > 1.0 \text{ MeV}$) neutron flux divided by the average flux over the total irradiation period were deemed unsuitable for capsules W-104 and W-83 since individual reaction rates did not vary proportionally with the fast flux due to the removal of the thermal shield at the end of the fifth fuel cycle. As a result of this observation, the C_j terms that were utilized in the final analyses for capsules W-104 and W-83 were based on the individual reaction rates determined from the synthesized transport calculations. The final C_j which are based on individual reaction rates, are reported on the following pages of this table.

Table A-3 cont'd

Calculated C_j Factors at the Surveillance Capsule Center
Core Midplane Elevation
(Capsule W-104)

Fuel Cycle	Capsule W-104 Reaction Rates [rps/atom]				
	$^{63}\text{Cu (n},\alpha)$	$^{54}\text{Fe (n,p)}$	$^{58}\text{Ni (n,p)}$	$^{46}\text{Tl (n,p)}$	$^{238}\text{U (n,f)}$
1	2.36E-17	2.20E-15	2.92E-15	3.93E-16	8.60E-15
2	2.62E-17	2.46E-15	3.27E-15	4.38E-16	9.65E-15
3	3.07E-17	2.90E-15	3.85E-15	5.14E-16	1.14E-14
4	2.96E-17	2.79E-15	3.70E-15	4.95E-16	1.10E-14
5	2.94E-17	2.77E-15	3.68E-15	4.92E-16	1.09E-14
6	6.01E-17	5.27E-15	6.85E-15	9.99E-16	1.76E-14
7	6.15E-17	5.40E-15	7.03E-15	1.02E-15	1.80E-14
8	6.17E-17	5.43E-15	7.06E-15	1.03E-15	1.81E-14
9	6.06E-17	5.32E-15	6.92E-15	1.01E-15	1.77E-14
10	3.34E-17	2.77E-15	3.59E-15	5.43E-16	8.95E-15
Average	4.11E-17	3.68E-15	4.81E-15	6.83E-16	1.30E-14

Fuel Cycle	Capsule W-104 C_j				
	$^{63}\text{Cu (n},\alpha)$	$^{54}\text{Fe (n,p)}$	$^{58}\text{Ni (n,p)}$	$^{46}\text{Tl (n,p)}$	$^{238}\text{U (n,f)}$
1	0.57	0.60	0.61	0.58	0.66
2	0.64	0.67	0.68	0.64	0.74
3	0.75	0.79	0.80	0.75	0.88
4	0.72	0.76	0.77	0.72	0.84
5	0.72	0.75	0.76	0.72	0.84
6	1.46	1.43	1.42	1.46	1.35
7	1.50	1.47	1.46	1.50	1.39
8	1.50	1.48	1.47	1.50	1.39
9	1.48	1.45	1.44	1.48	1.37
10	0.81	0.75	0.75	0.79	0.69
Average	1.00	1.00	1.00	1.00	1.00

Table A-3 cont'd

Calculated C_i Factors at the Surveillance Capsule Center
Core Midplane Elevation
(Capsule W-83)

Fuel Cycle	Capsule W-83 Reaction Rates [rps/atom]					
	$^{54}\text{Fe (n,p)}$	$^{58}\text{Ni (n,p)}$	$^{46}\text{Ti (n,p)}$	$^{238}\text{U (n,f)}$	$^{59}\text{Co (n,\gamma)}$	$^{59}\text{Co (n,\gamma) Cd}$
1	3.07E-15	4.07E-15	5.34E-16	1.22E-14	2.90E-12	6.25E-13
2	3.41E-15	4.53E-15	5.92E-16	1.36E-14	3.26E-12	7.03E-13
3	4.01E-15	5.34E-15	6.94E-16	1.61E-14	3.89E-12	8.42E-13
4	3.90E-15	5.18E-15	6.75E-16	1.56E-14	3.77E-12	8.14E-13
5	3.84E-15	5.11E-15	6.65E-16	1.54E-14	3.71E-12	8.02E-13
6	7.17E-15	9.34E-15	1.33E-15	2.44E-14	2.50E-12	5.53E-13
7	7.33E-15	9.55E-15	1.36E-15	2.49E-14	2.56E-12	5.66E-13
8	7.35E-15	9.57E-15	1.36E-15	2.50E-14	2.56E-12	5.68E-13
9	7.21E-15	9.39E-15	1.33E-15	2.45E-14	2.51E-12	5.56E-13
10	3.37E-15	4.37E-15	6.44E-16	1.11E-14	1.07E-12	2.41E-13
11	3.82E-15	4.97E-15	7.28E-16	1.27E-14	1.24E-12	2.77E-13
12	3.19E-15	4.15E-15	6.12E-16	1.05E-14	1.02E-12	2.28E-13
13	3.38E-15	4.40E-15	6.47E-16	1.12E-14	1.08E-12	2.43E-13
14	3.38E-15	4.38E-15	6.46E-16	1.11E-14	1.08E-12	2.42E-13
Average	4.44E-15	5.82E-15	8.17E-16	1.57E-14	2.23E-12	4.89E-13

Fuel Cycle	Capsule W-83 C_i					
	$^{54}\text{Fe (n,p)}$	$^{58}\text{Ni (n,p)}$	$^{46}\text{Ti (n,p)}$	$^{238}\text{U (n,f)}$	$^{59}\text{Co (n,\gamma)}$	$^{59}\text{Co (n,\gamma) Cd}$
1	0.69	0.70	0.65	0.78	1.30	1.28
2	0.77	0.78	0.72	0.87	1.46	1.44
3	0.90	0.92	0.85	1.03	1.75	1.72
4	0.88	0.89	0.83	0.99	1.69	1.66
5	0.86	0.88	0.81	0.98	1.66	1.64
6	1.61	1.61	1.62	1.55	1.12	1.13
7	1.65	1.64	1.66	1.59	1.15	1.16
8	1.65	1.65	1.66	1.59	1.15	1.16
9	1.62	1.62	1.63	1.56	1.13	1.14
10	0.76	0.75	0.79	0.71	0.48	0.49
11	0.86	0.85	0.89	0.81	0.55	0.57
12	0.72	0.71	0.75	0.67	0.45	0.47
13	0.76	0.76	0.79	0.71	0.49	0.50
14	0.76	0.75	0.79	0.71	0.48	0.49
Average	1.00	1.00	1.00	1.00	1.00	1.00

Table A-4
Measured Sensor Activities and Reaction Rates

Surveillance Capsule W-97

<u>Reaction</u>	<u>Location</u>	<u>Measured Activity^[1] (dps/g)</u>	<u>Saturated Activity (dps/g)</u>	<u>Reaction Rate (rps/atom)</u>
$^{63}\text{Cu} (n,\alpha) ^{60}\text{Co} (\text{Cd})$	Top	1.12E+05	3.78E+05	5.76E-17
	Middle	1.10E+05	3.74E+05	5.71E-17
	Bottom	1.27E+05	4.28E+05	6.53E-17
	Average			6.00E-17
$^{54}\text{Fe} (n,p) ^{54}\text{Mn}$	Top	1.87E+06	2.60E+06	4.13E-15
	Middle	1.76E+06	2.45E+06	3.88E-15
	Bottom	1.86E+06	2.59E+06	4.10E-15
	Average			4.04E-15
$^{58}\text{Ni} (n,p) ^{58}\text{Co} (\text{Cd})$	Top	3.01E+07	3.70E+07	5.30E-15
	Middle	2.78E+07	3.42E+07	4.90E-15
	Bottom	3.05E+07	3.75E+07	5.37E-15
	Average			5.19E-15
$^{46}\text{Ti} (n,p) ^{46}\text{Sc}$	Top	6.15E+05	7.60E+05	7.55E-16
	Middle	5.48E+05	6.77E+05	6.73E-16
	Bottom	5.70E+05	7.07E+05	7.03E-16
	Average			7.10E-16
$^{238}\text{U} (n,f) ^{137}\text{Cs} (\text{Cd})$	Top	1.77E+05	2.75E+06	1.80E-14
	Middle	1.92E+05	2.98E+06	1.96E-14
	Bottom	1.87E+05	2.90E+06	1.91E-14
	Average			1.89E-14
$^{238}\text{U} (n,f) ^{137}\text{Cs} (\text{Cd})$	Including ^{235}U , ^{239}Pu , and γ , fission corrections.			1.49E-14 ^[2]
$^{59}\text{Co} (n,\gamma) ^{60}\text{Co}$	Top	2.22E+07	7.48E+07	4.88E-12
	Middle	2.52E+07	8.49E+07	5.54E-12
	Bottom	1.70E+07	5.73E+07	3.74E-12
	Average			4.72E-12
$^{59}\text{Co} (n,\gamma) ^{60}\text{Co} (\text{Cd})$	Top	2.93E+06	9.88E+06	6.44E-13
	Middle	3.02E+06	1.02E+07	6.64E-13
	Bottom	3.07E+06	1.04E+07	6.75E-13
	Average			6.61E-13

[1] Measured specific activities are decay corrected to time of reactor shutdown, i e., August 17, 1980

[2] The average $^{238}\text{U} (n,f)$ reaction rate of 1.49E-14 includes a correction factor of 0.872 to account for plutonium build-in and an additional factor of 0.903 to account for photo-fission effects in the sensor.

Table A-4 cont'd
Measured Sensor Activities and Reaction Rates

Surveillance Capsule W-104

<u>Reaction</u>	<u>Location</u>	Measured Activity ^[1] (dps/g)	Saturated Activity (dps/g)	Reaction Rate (rps/atom)
$^{63}\text{Cu} (n,\alpha) ^{60}\text{Co} (\text{Cd})$	Top	3.28E+05	4.87E+05	5.09E-17
	Middle	3.34E+05	4.96E+05	5.18E-17
	Bottom	3.50E+05	5.20E+05	5.43E-17
	Average			5.24E-17
$^{54}\text{Fe} (n,p) ^{54}\text{Mn}$	Top	3.02E+07	4.09E+07	3.67E-15
	Middle	2.85E+07	3.86E+07	3.46E-15
	Bottom	2.90E+07	3.93E+07	3.52E-15
	Average			3.55E-15
$^{58}\text{Ni} (n,p) ^{58}\text{Co} (\text{Cd})$	Top	3.15E+07	5.07E+07	4.88E-15
	Middle	2.65E+07	4.27E+07	4.10E-15
	Bottom	3.03E+07	4.88E+07	4.69E-15
	Average			4.56E-15
$^{46}\text{Ti} (n,p) ^{46}\text{Sc}$	Top	6.22E+06	9.55E+06	7.29E-16
	Middle	5.47E+06	8.40E+06	6.41E-16
	Bottom	5.75E+06	8.83E+06	6.74E-16
	Average			6.81E-16
$^{238}\text{U} (n,f) ^{137}\text{Cs} (\text{Cd})$	Top	4.77E+05	2.39E+06	1.57E-14
	Middle	4.77E+05	2.39E+06	1.57E-14
	Bottom	5.09E+05	2.55E+06	1.68E-14
	Average			1.61E-14
$^{238}\text{U} (n,f) ^{137}\text{Cs} (\text{Cd})$	Including ^{235}U , ^{239}Pu , and γ , fission corrections.			1.15E-14 ^[2]

[1] Measured specific activities are assumed to be decay corrected to time of reactor shutdown, i.e., September 14, 1990.

[2] The average $^{238}\text{U} (n,f)$ reaction rate of 1.15E-14 includes a correction factor of 0.848 to account for plutonium build-in and an additional factor of 0.844 to account for photo-fission effects in the sensor.

Table A-4 cont'd
Measured Sensor Activities and Reaction Rates

Surveillance Capsule W-83

<u>Reaction</u>	<u>Location</u>	<u>Measured Activity^[1] (dps/g)</u>	<u>Saturated Activity (dps/g)</u>	<u>Reaction Rate (rps/atom)</u>
$^{54}\text{Fe} (n,p) ^{54}\text{Mn}$	Top	1.21E+06	3.02E+06	4.79E-15
	Middle	1.10E+06	2.75E+06	4.36E-15
	Bottom	1.10E+06	2.75E+06	4.36E-15
	Average			4.50E-15
$^{58}\text{Ni} (n,p) ^{58}\text{Co} (\text{Cd})$	Top	4.08E+06	4.28E+07	6.13E-15
	Middle	3.62E+06	3.80E+07	5.44E-15
	Bottom	3.80E+06	3.99E+07	5.71E-15
	Average			5.76E-15
$^{46}\text{Ti} (n,p) ^{46}\text{Sc}$	Top	1.25E+05	9.14E+05	9.08E-16
	Middle	1.09E+05	7.97E+05	7.92E-16
	Bottom	1.10E+05	8.04E+05	7.99E-16
	Average			8.33E-16
$^{238}\text{U} (n,f) ^{137}\text{Cs} (\text{Cd})$	Top	1.55E+05	6.21E+05	4.08E-15
	Middle	1.80E+05	7.22E+05	4.74E-15
	Bottom	1.51E+05	6.05E+05	3.98E-15
	Average			4.27E-15
$^{238}\text{U} (n,f) ^{137}\text{Cs} (\text{Cd})$	Including ^{235}U , ^{239}Pu , and γ , fission corrections.			2.96E-15 ^[2]
$^{59}\text{Co} (n,\gamma) ^{60}\text{Co}$	Top	1.59E+07	5.05E+07	3.29E-12
	Middle	1.69E+07	5.36E+07	3.50E-12
	Bottom	1.12E+07	3.55E+07	2.32E-12
	Average			3.04E-12
$^{59}\text{Co} (n,\gamma) ^{60}\text{Co} (\text{Cd})$	Top	1.83E+06	5.72E+06	3.73E-13
	Middle	1.84E+06	5.76E+06	3.76E-13
	Bottom	1.82E+06	5.69E+06	3.71E-13
	Average			3.73E-13

[1] Measured specific activities are decay corrected to September 9, 2002.

[2] The average $^{238}\text{U} (n,f)$ reaction rate of 2.96E-15 includes a correction factor of 0.819 to account for plutonium build-in and an additional factor of 0.846 to account for photo-fission effects in the sensor

Table A-5

Comparison of Measured, Calculated, and Best Estimate
Reaction Rates At The Surveillance Capsule Center

Capsule W-97

Reaction	Reaction Rate [rps/atom]			M/C	M/BE
	Measured	Calculated	Best Estimate		
$^{54}\text{Fe}(\text{n,p})^{54}\text{Mn}$	4.04E-15	3.50E-15	4.03E-15	1.15	1.00
$^{58}\text{Ni}(\text{n,p})^{58}\text{Co} (\text{Cd})$	5.19E-15	4.66E-15	5.30E-15	1.11	0.98
$^{46}\text{Ti}(\text{n,p})^{46}\text{Sc}$	7.10E-16	5.70E-16	6.84E-16	1.25	1.04
$^{238}\text{U}(\text{n,f})^{137}\text{Cs} (\text{Cd})$	1.49E-14	1.40E-14	1.56E-14	1.06	0.95
$^{59}\text{Co}(\text{n},\gamma)^{60}\text{Co}$	4.72E-12	3.28E-12	4.70E-12	1.44	1.01
$^{59}\text{Co}(\text{n},\gamma)^{60}\text{Co} (\text{Cd})$	6.61E-13	6.78E-13	6.65E-13	0.98	0.99

Capsule W-104

Reaction	Reaction Rate [rps/atom]			M/C	M/BE
	Measured	Calculated	Best Estimate		
$^{54}\text{Fe}(\text{n,p})^{54}\text{Mn}$	3.55E-15	3.76E-15	3.58E-15	0.94	0.99
$^{58}\text{Ni}(\text{n,p})^{58}\text{Co} (\text{Cd})$	4.56E-15	4.92E-15	4.65E-15	0.93	0.98
$^{46}\text{Ti}(\text{n,p})^{46}\text{Sc}$	6.81E-16	6.57E-16	6.54E-16	1.04	1.04
$^{238}\text{U}(\text{n,f})^{137}\text{Cs} (\text{Cd})$	1.15E-14	1.33E-14	1.23E-14	0.87	0.94

Capsule W-83

Reaction	Reaction Rate [rps/atom]			M/C	M/BE
	Measured	Calculated	Best Estimate		
$^{54}\text{Fe}(\text{n,p})^{54}\text{Mn}$	4.50E-15	4.54E-15	4.53E-15	0.99	0.99
$^{58}\text{Ni}(\text{n,p})^{58}\text{Co} (\text{Cd})$	5.75E-15	5.95E-15	5.89E-15	0.97	0.98
$^{46}\text{Ti}(\text{n,p})^{46}\text{Sc}$	8.33E-16	7.85E-16	8.07E-16	1.06	1.03
$^{59}\text{Co}(\text{n},\gamma)^{60}\text{Co}$	3.04E-12	2.23E-12	3.02E-12	1.36	1.01
$^{59}\text{Co}(\text{n},\gamma)^{60}\text{Co} (\text{Cd})$	3.73E-13	4.67E-13	3.77E-13	0.80	0.98

Table A-6

Comparison of Calculated and Best Estimate Exposure Rates
At The Surveillance Capsule Center

Capsule ID	$\phi(E > 1.0 \text{ MeV}) [\text{n/cm}^2\text{-s}]$			
	Calculated ^[1]	Best Estimate	Uncertainty (1 σ)	BE/C
W-97	3.37E+10	3.76E+10	6%	1.12
W-104	3.00E+10	2.80E+10	6%	0.94
W-83	3.60E+10	3.58E+10	7%	0.99

[1] Calculated results are based on the synthesized transport calculations taken at the core midplane following the completion of each respective capsules irradiation period

Capsule ID	Iron Atom Displacement Rate [dpa/s]			
	Calculated ^[1]	Best Estimate	Uncertainty (1 σ)	BE/C
W-97	5.17E-11	5.70E-11	6 %	1.10
W-104	4.46E-11	4.15E-11	6 %	0.93
W-83	5.32E-11	5.26E-11	6 %	0.99

[1] Calculated results are based on the synthesized transport calculations taken at the core midplane following the completion of each respective capsules irradiation period.

Table A-7

Comparison of Measured/Calculated (M/C) Sensor Reaction Rate
Ratios Including all Fast Neutron Threshold Reactions

Reaction	M/C Ratio		
	Capsule W-97	Capsule W-104	Capsule W-83
$^{54}\text{Fe}(\text{n,p})^{54}\text{Mn}$	1.15	0.94	0.99
$^{58}\text{Ni}(\text{n,p})^{58}\text{Co}$ (Cd)	1.11	0.93	0.97
$^{46}\text{Ti}(\text{n,p})^{46}\text{Sc}$	1.25 ^[1]	1.04 ^[1]	1.06 ^[1]
$^{238}\text{U}(\text{n,p})^{137}\text{Cs}$ (Cd)	1.06	0.87	N/A ^[2]
Average	1.11 ^[3]	0.91 ^[3]	0.98 ^[3]
% Standard Deviation	4.1 ^[3]	4.1 ^[3]	1.4 ^[3]

[1] The M/C values for the ^{46}Ti sensors are listed but not used in the average M/C ratio due to a bias present in the SNLRML cross-section data as discussed in Section A.1.3 For additional information, these calculations were repeated using the ^{46}Ti dosimetry cross-section from the BUGLE-96 data library set. The results of these calculations were M/C ratios of 1.19, 1.00, and 1.02 for Capsules W-97, W-104, and W-83, respectively

[2] The cadmium-covered uranium measurement from Capsule W-83 was rejected

[3] The overall average M/C ratio for the set of 8 sensor measurements is 1.00 with an associated standard deviation of 9.6%

Table A-8

Comparison of Best Estimate/Calculated (BE/C) Exposure Rate Ratios

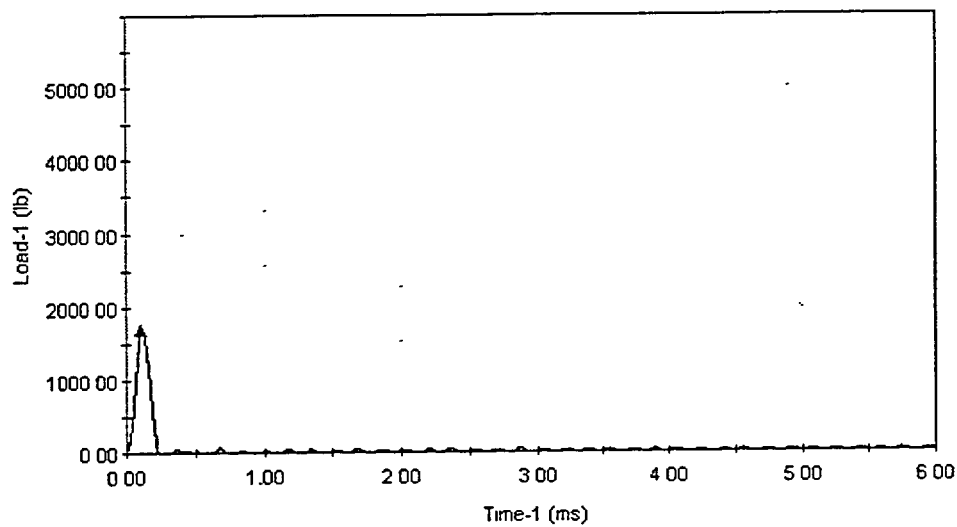
Capsule ID	BE/C Ratio	
	$\phi(E > 1.0 \text{ MeV})$	dpa/s
W-97	1.12	1.10
W-104	0.94	0.93
W-83	0.99	0.99
Average	1.02	1.01
% Standard Deviation	9.1	8.6

Appendix A References

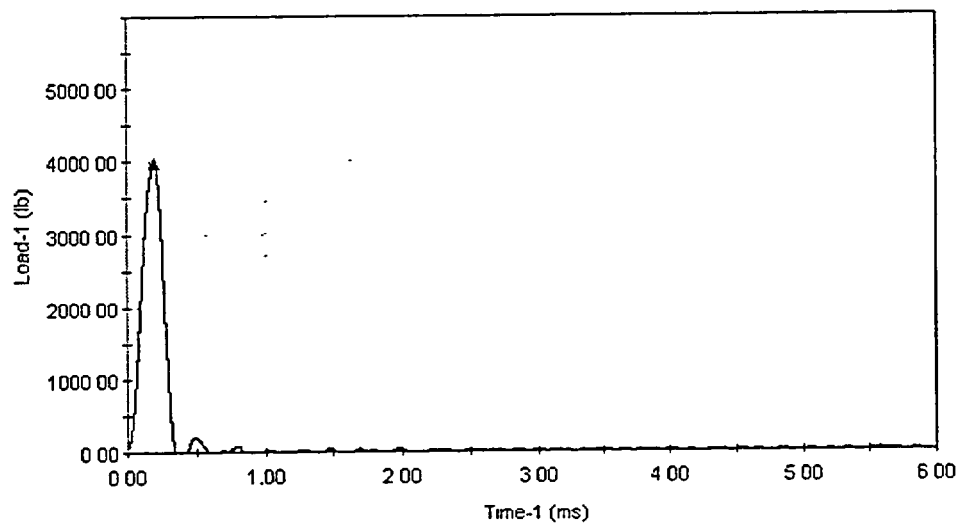
- A-1. Regulatory Guide RG-1.190, "Calculational and Dosimetry Methods for Determining Pressure Vessel Neutron Fluence," U. S. Nuclear Regulatory Commission, Office of Nuclear Regulatory Research, March 2001.
- A-2. TR-N-MCM-008, "Northeast Utilities Service Company Millstone Nuclear Unit No. 2 Post-Irradiation Evaluation of Reactor Vessel Surveillance Capsule W-97," S. T. Byrne, Combustion Engineering Inc., April 1982.
- A-3. BAW-2142, "Analysis of Capsule W-104 Northeast Nuclear Energy Company Millstone Nuclear Power Station, Unit No. 2 - Reactor Vessel Material Surveillance Program," A. L. Lowe, Jr., et al., B&W Nuclear Service Company, November 1991.
- A-4. A. Schmittroth, *FERRET Data Analysis Core*, HEDL-TME 79-40, Hanford Engineering Development Laboratory, Richland, WA, September 1979.
- A-5. RSIC Data Library Collection DLC-178, "SNLRML Recommended Dosimetry Cross-Section Compendium", July 1994.

APPENDIX B
INSTRUMENTED CHARPY IMPACT TEST CURVES

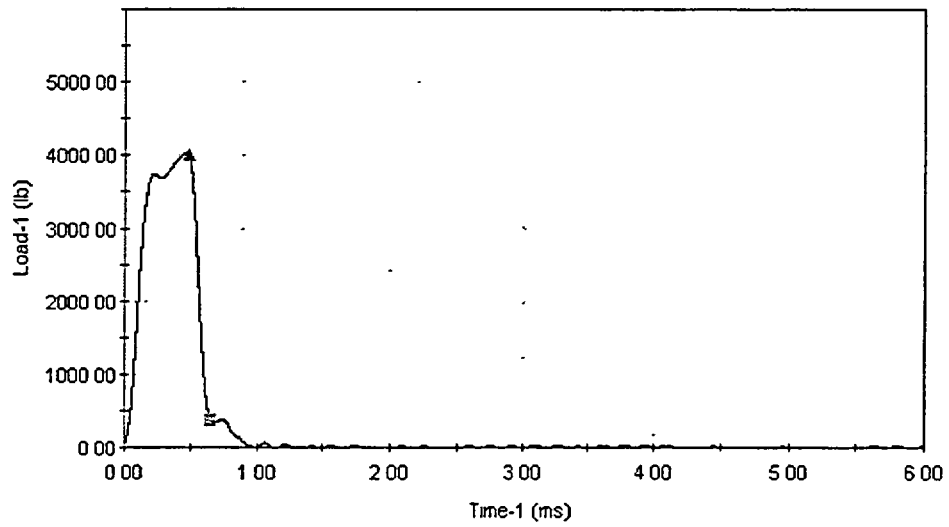
[Each of the following plots is titled as "Specimen Number, Test Temperature"]



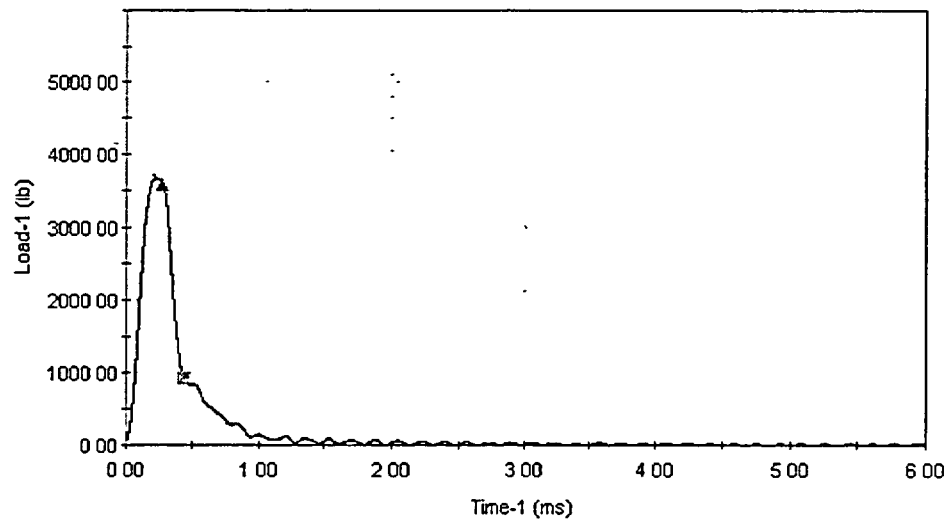
132, 0°F



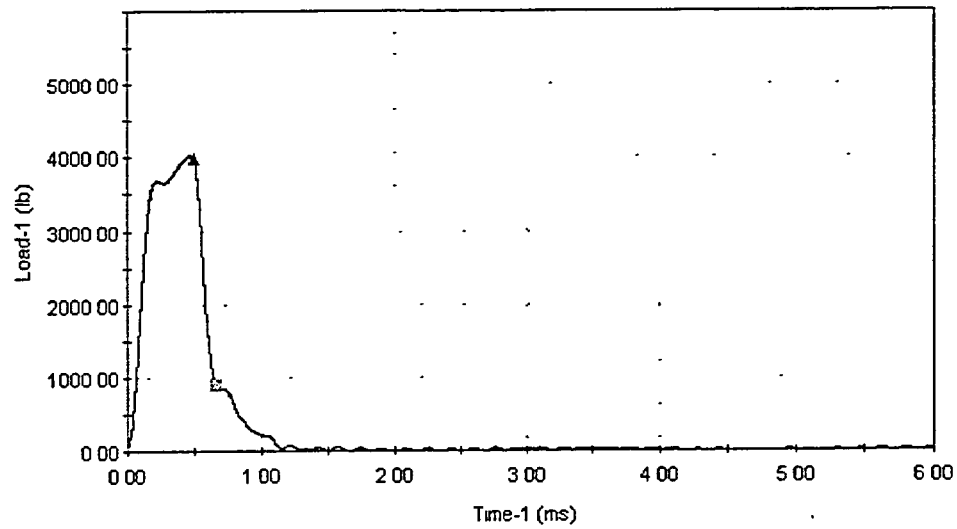
146, 75°F



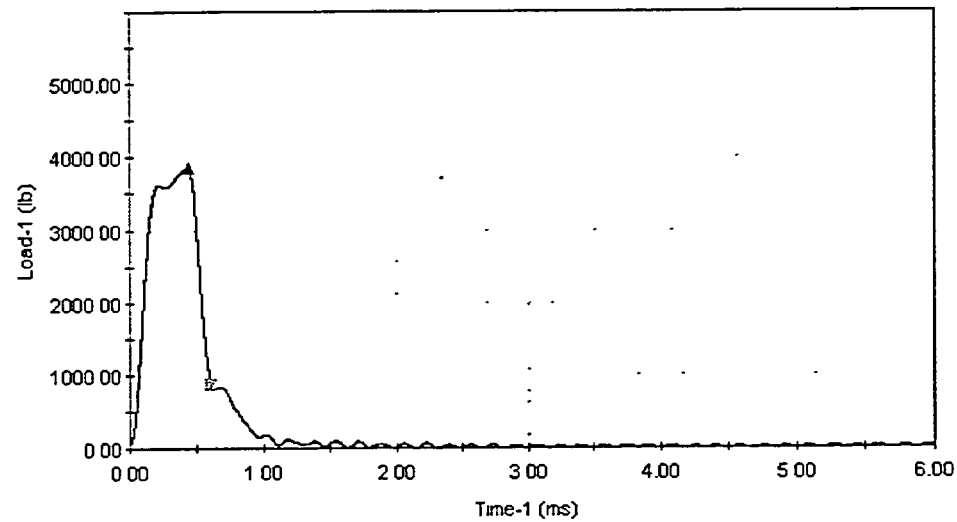
117, 130°F



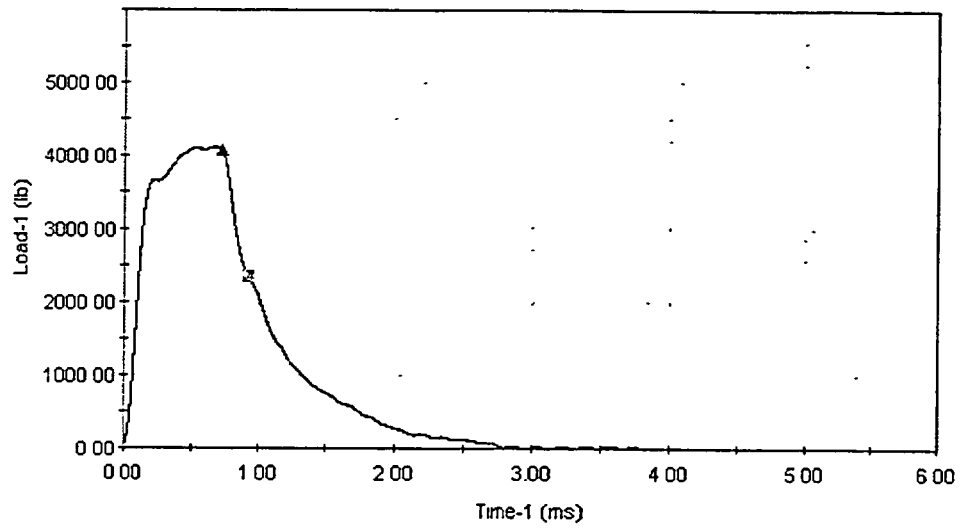
13C, 175°F



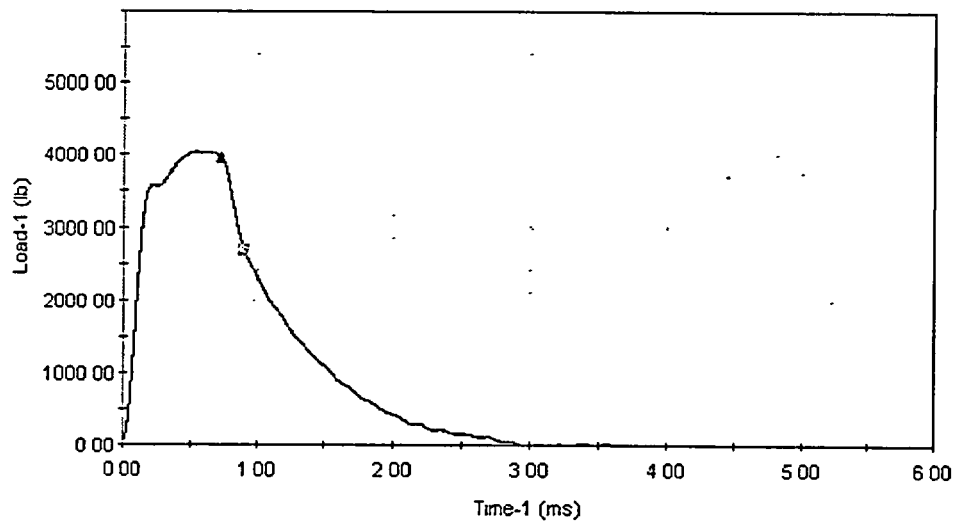
121, 175°F



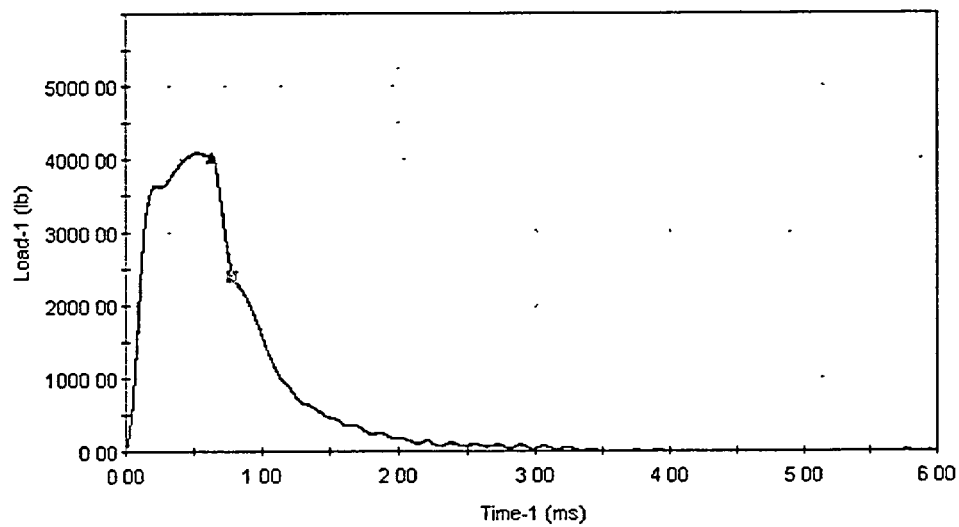
136, 200°F



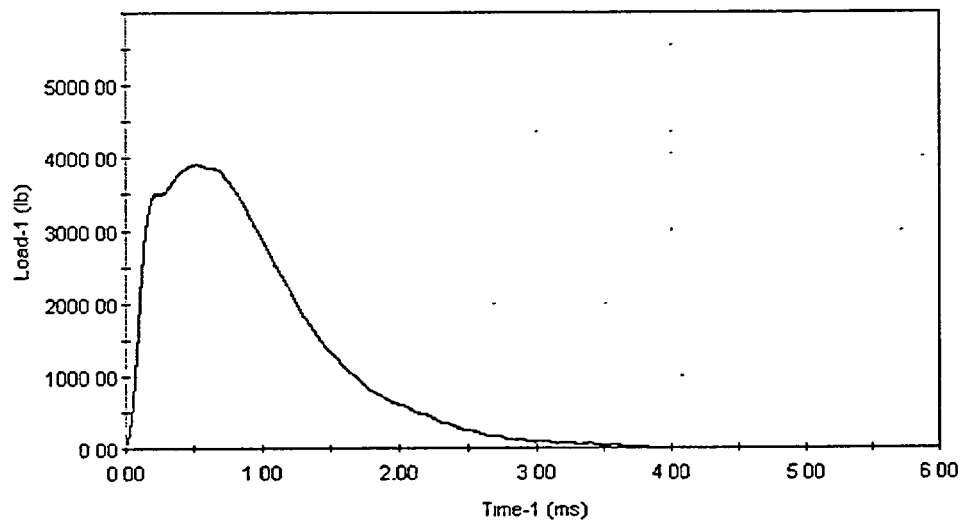
156, 215°F



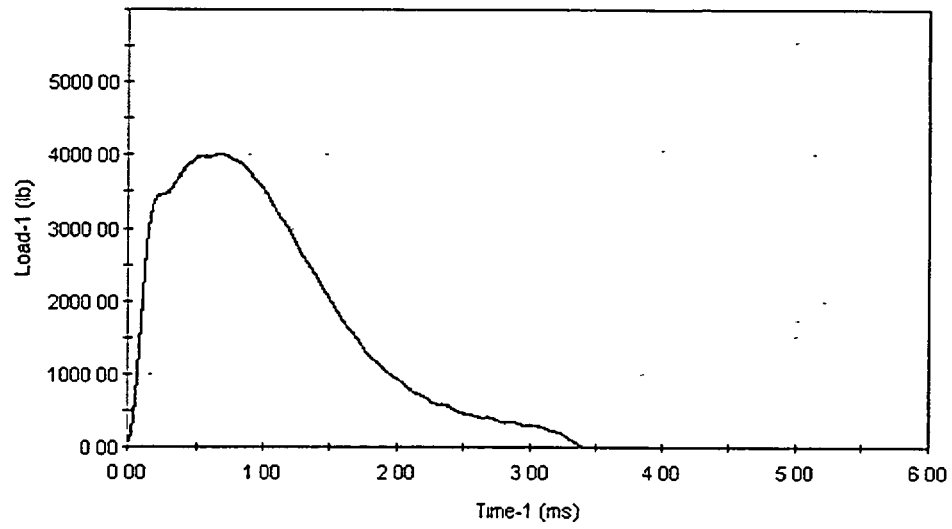
165, 225°F



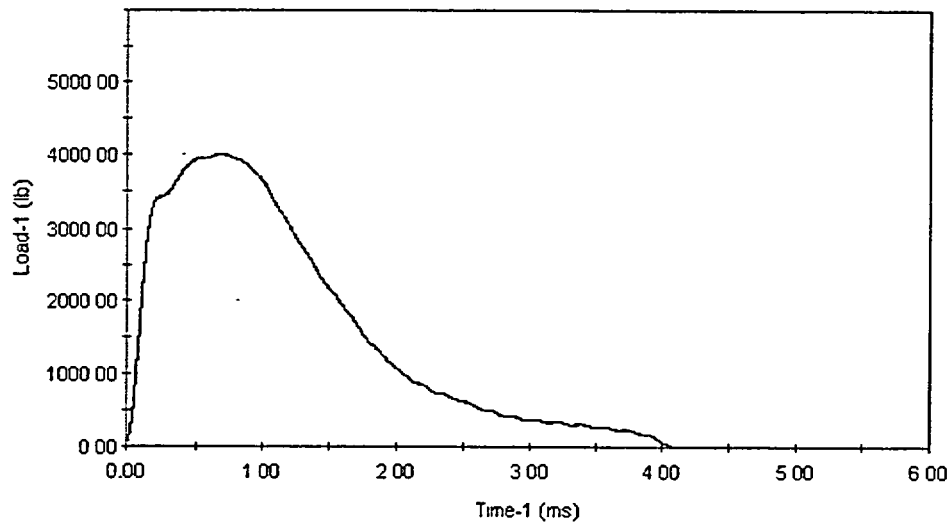
131, 250°F



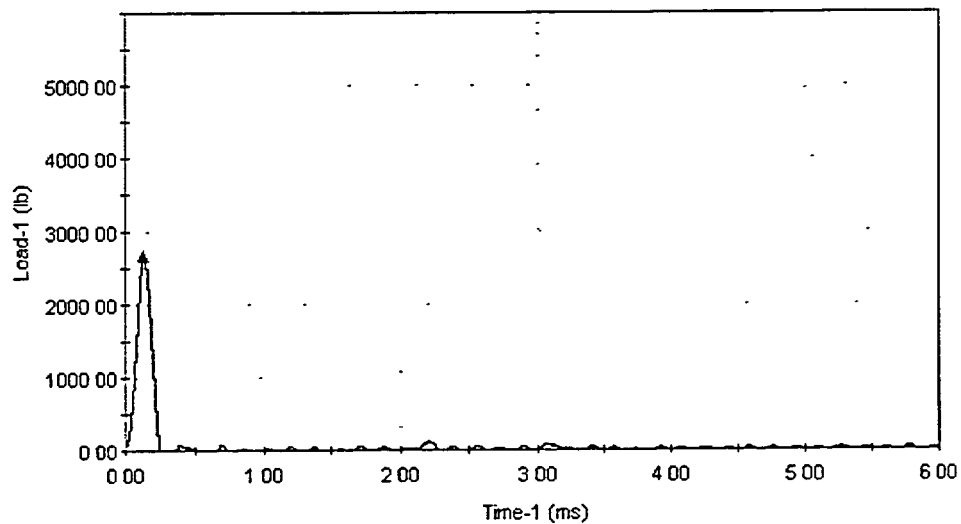
12K, 300°F



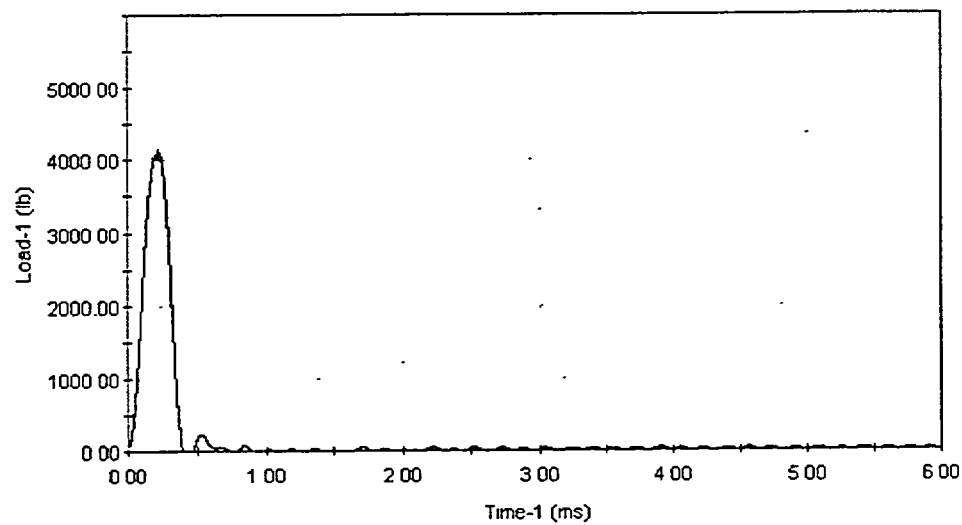
126, 325°F



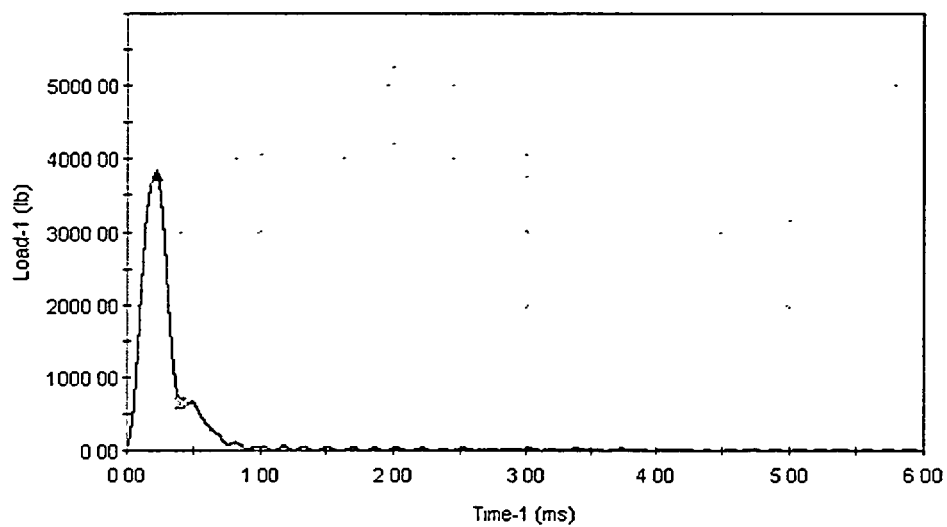
16D, 350°F



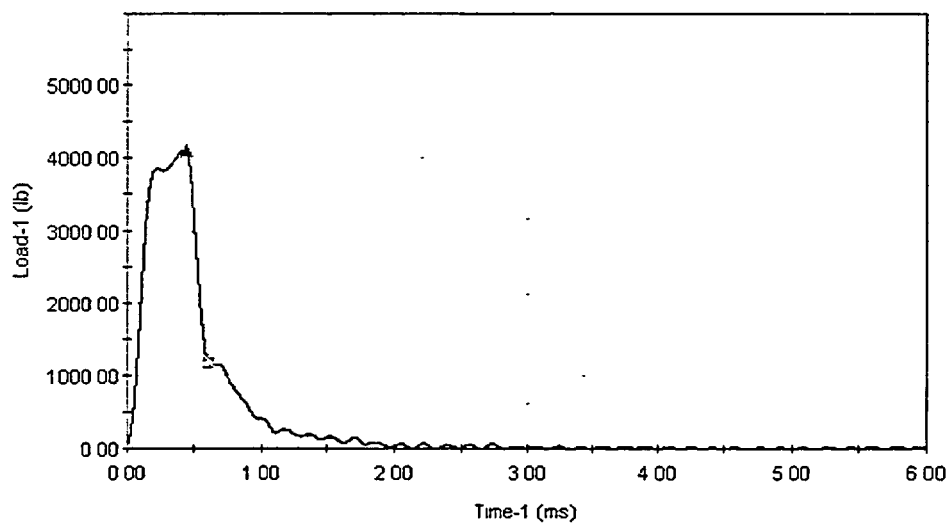
224, 0°F



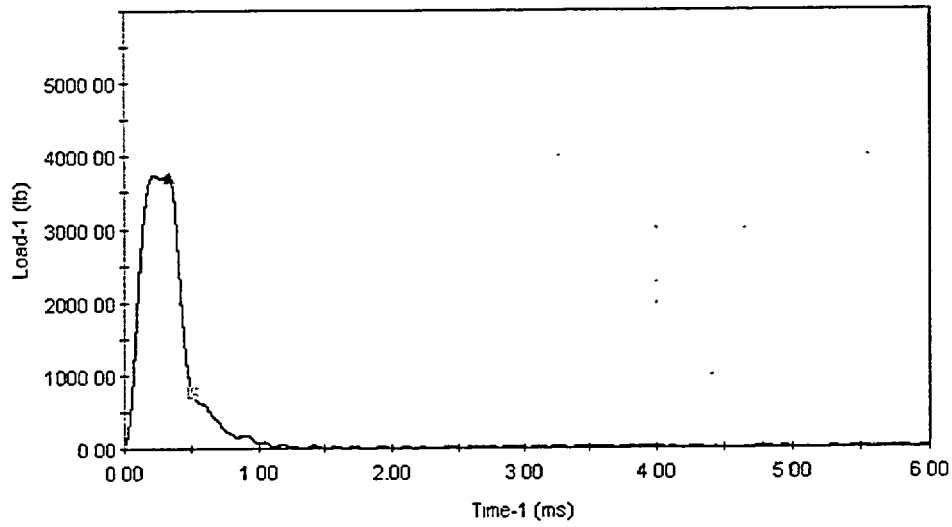
231, 75°F



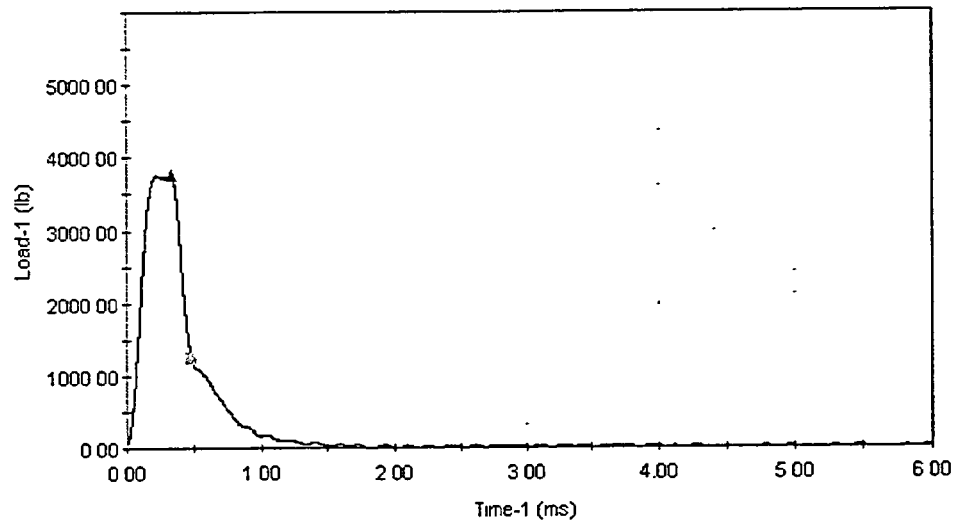
21L, 130°F



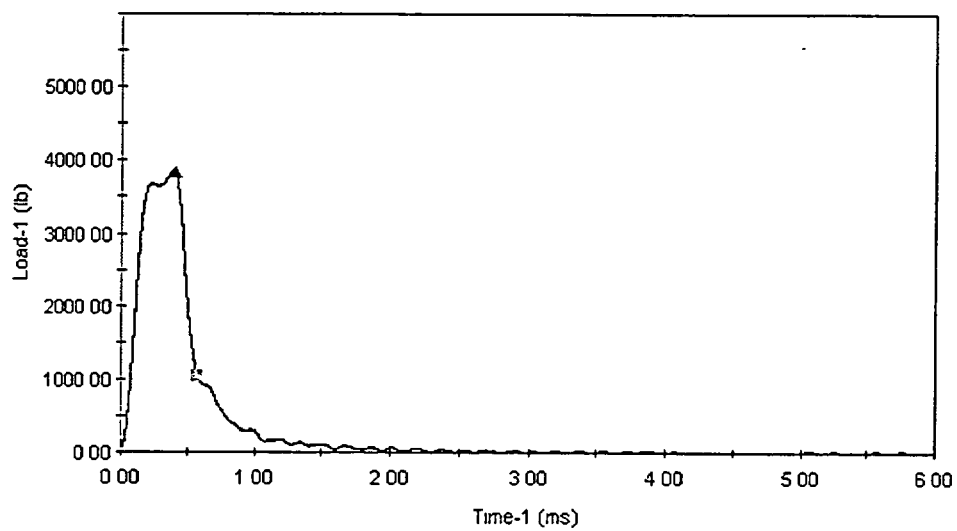
253, 150°F



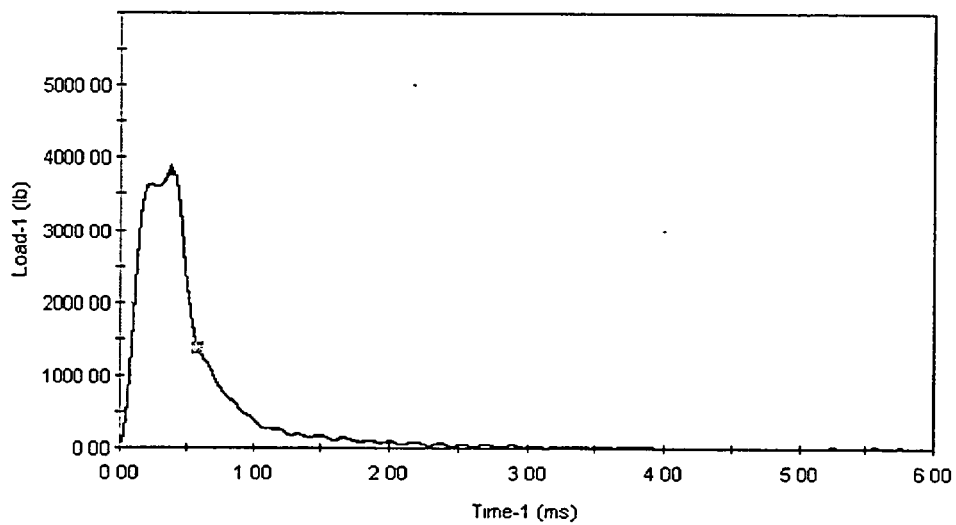
213, 150°F



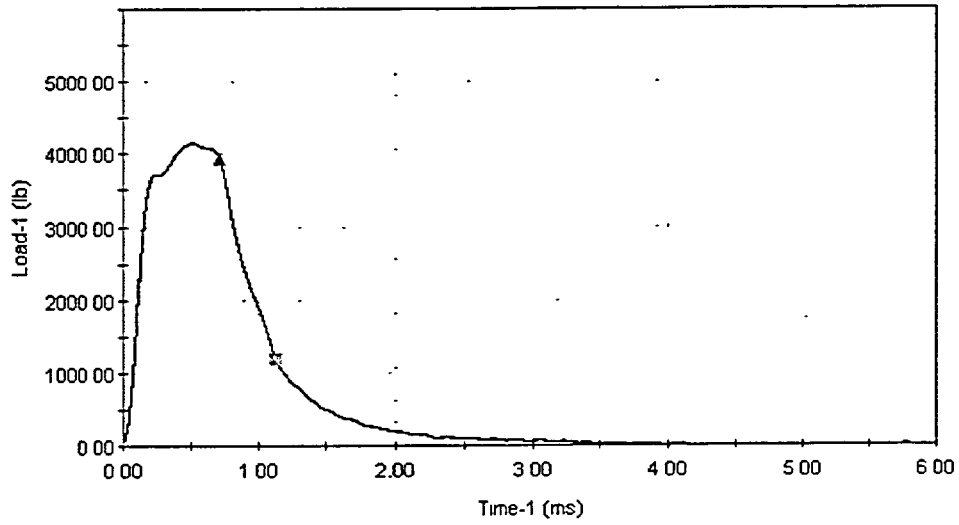
245, 175°F



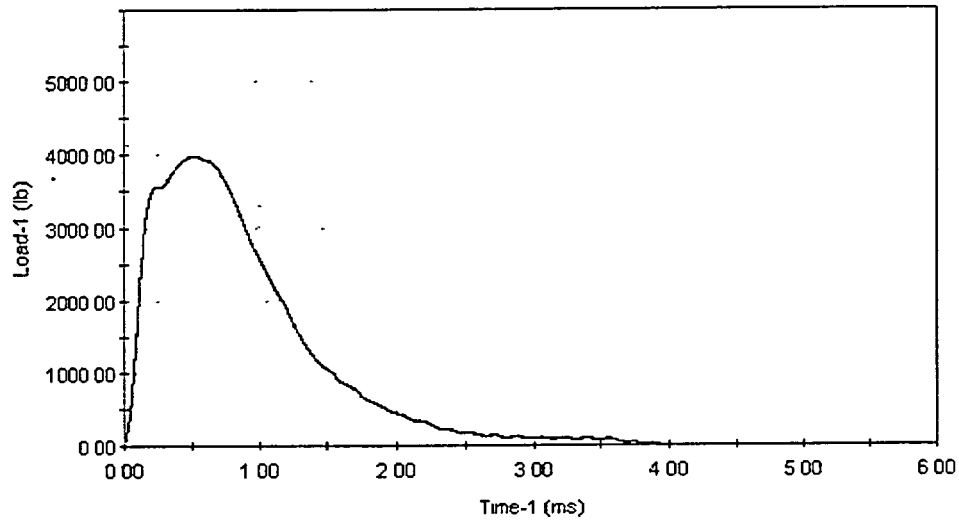
212, 175°F



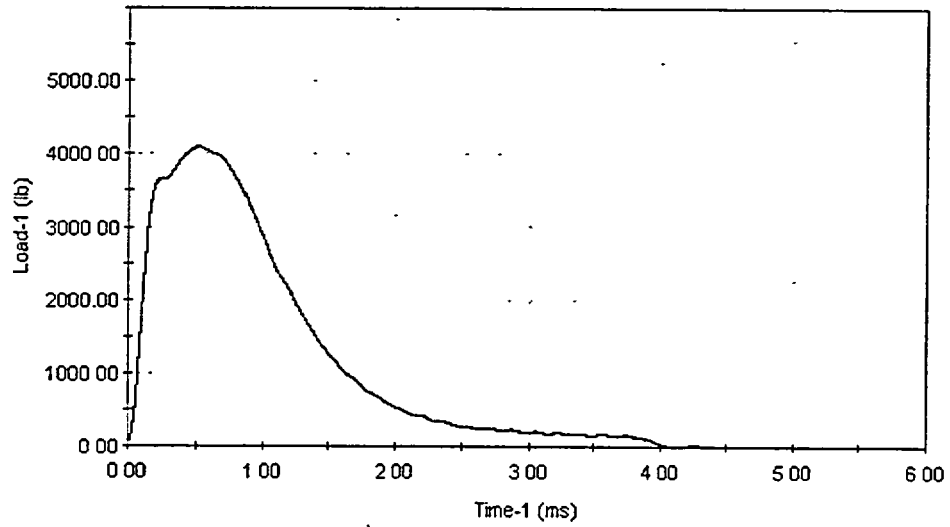
211, 200°F



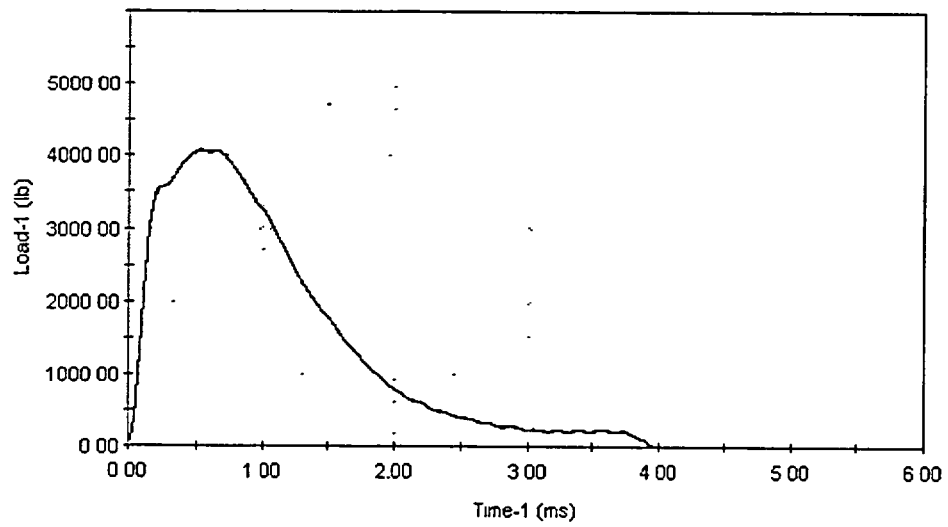
23L, 225°F



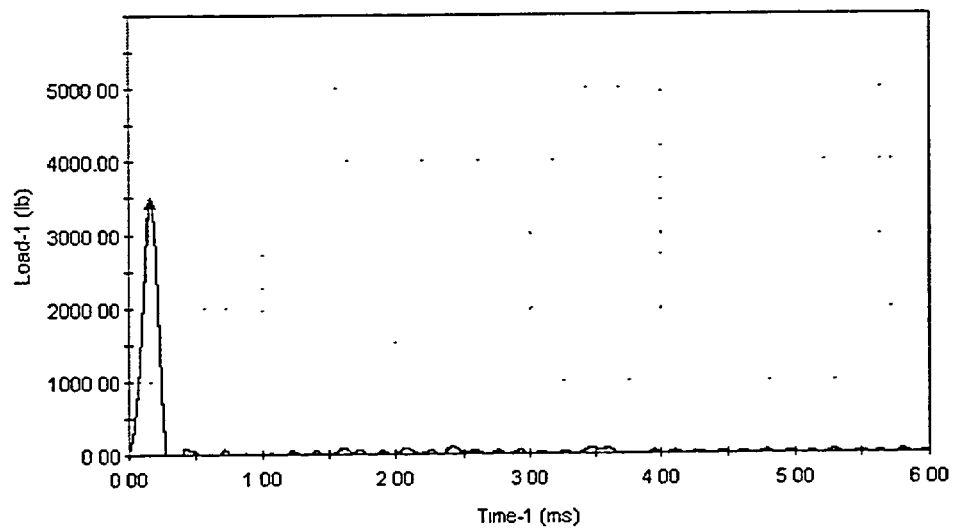
214, 275°F



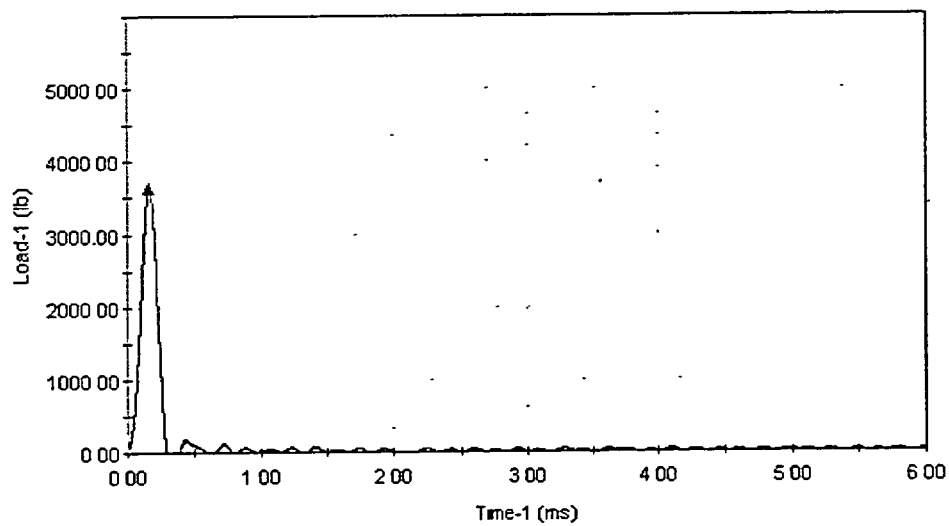
24K, 300°F



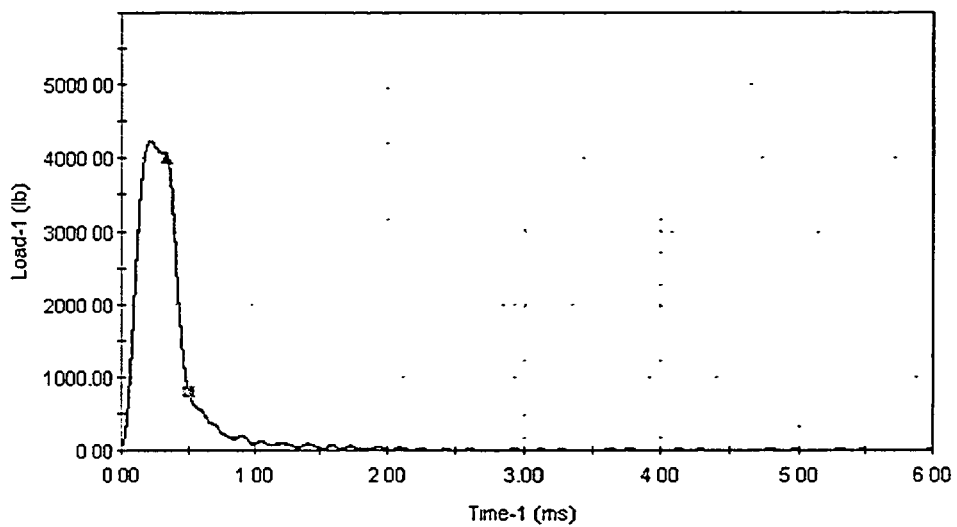
22A, 325°F



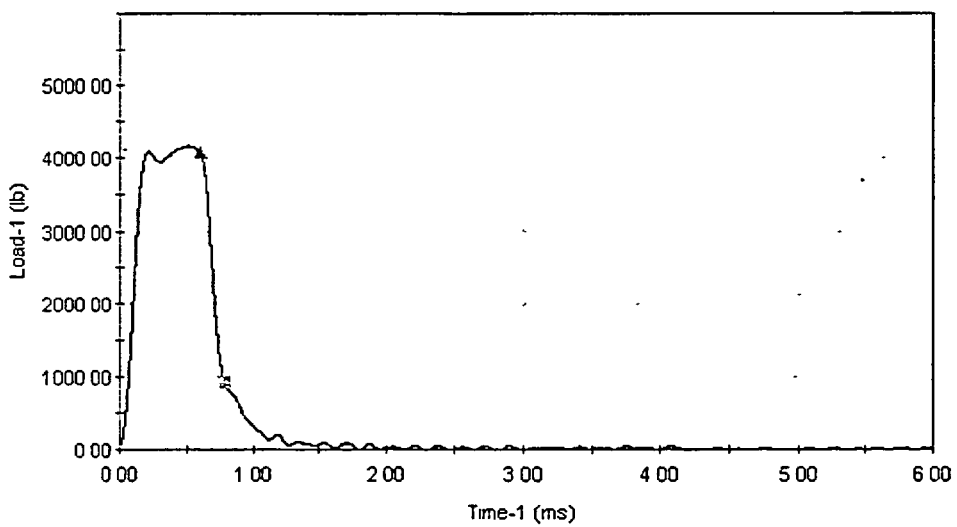
314, -50°F



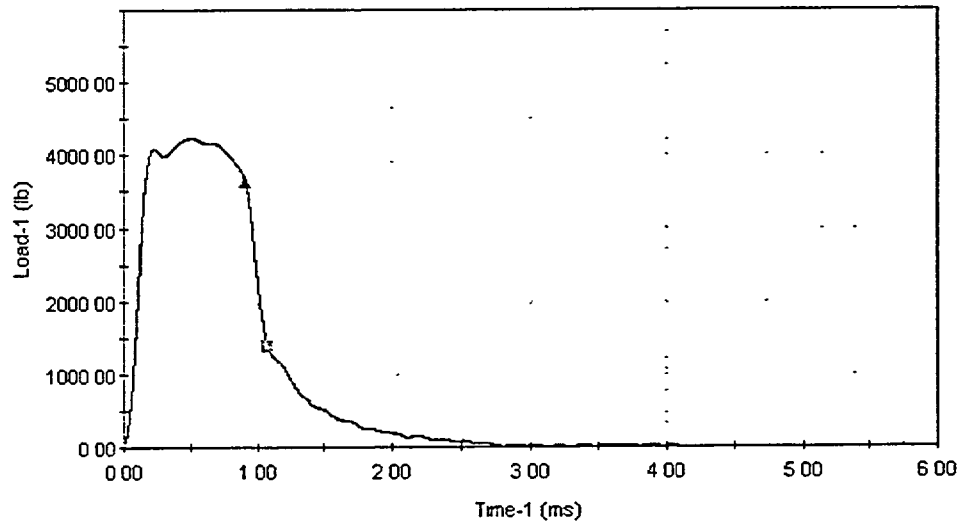
33K, 0°F



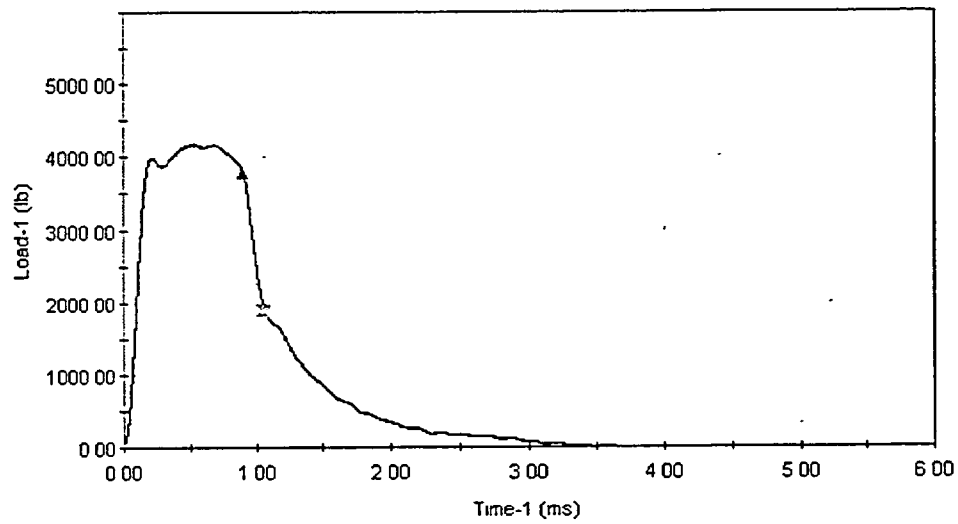
34L, 30°F



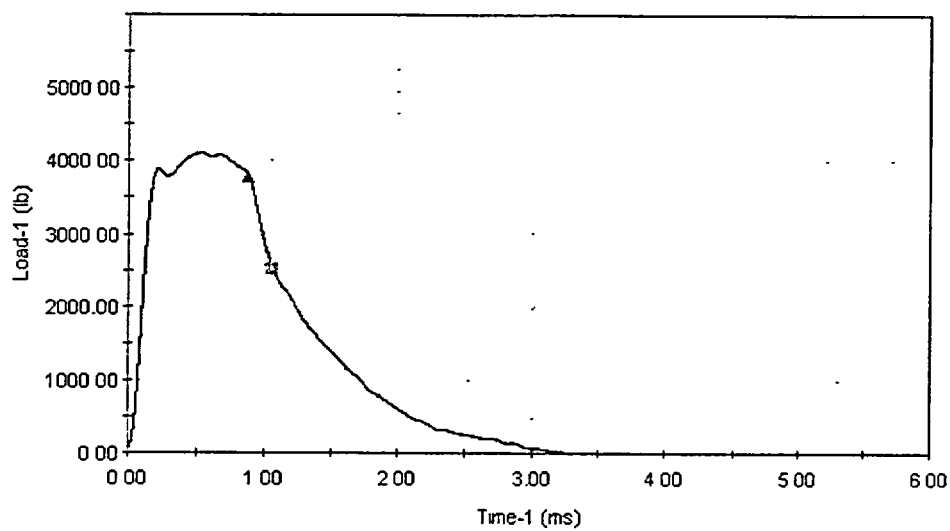
311, 50°F



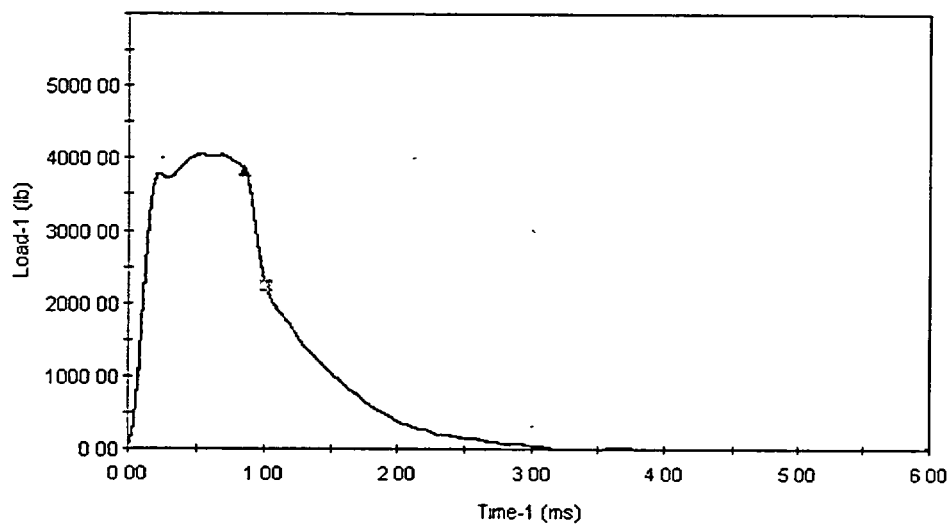
32A, 75°F



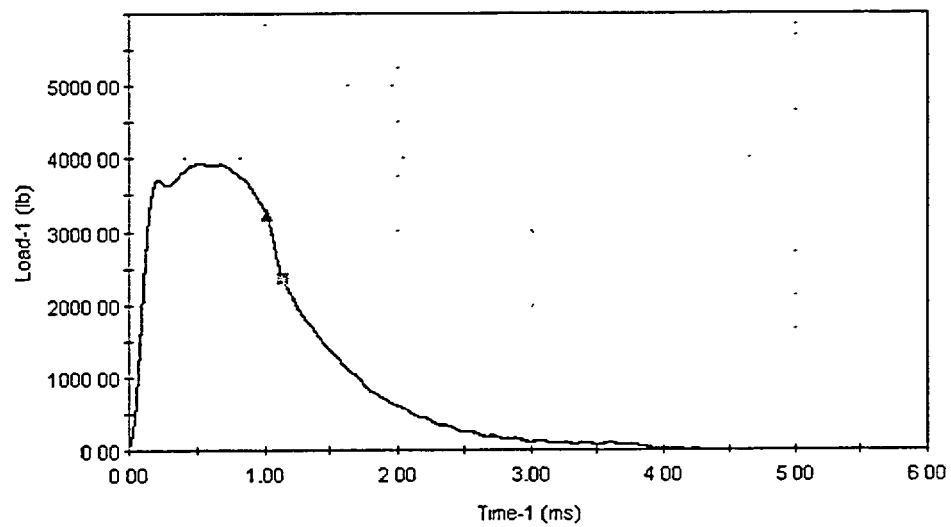
36D, 100°F



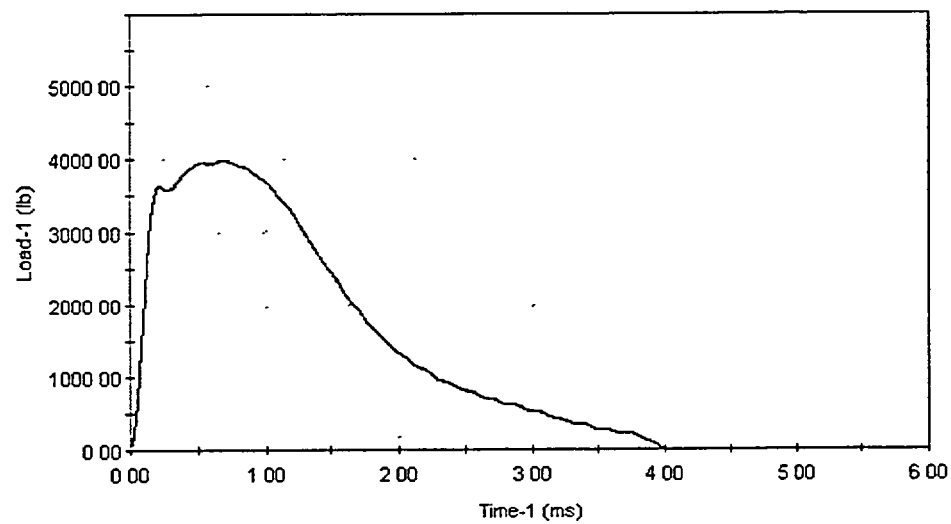
36E, 125°F



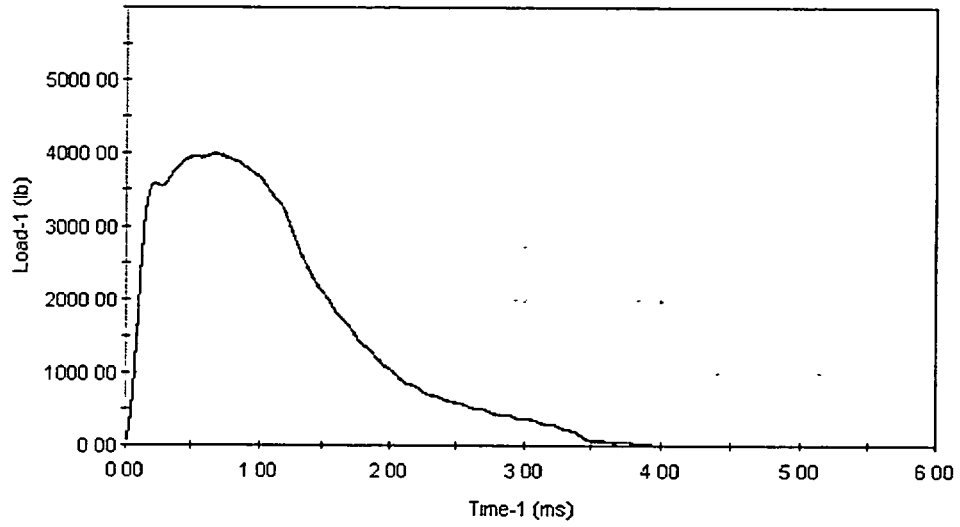
34C, 150°F



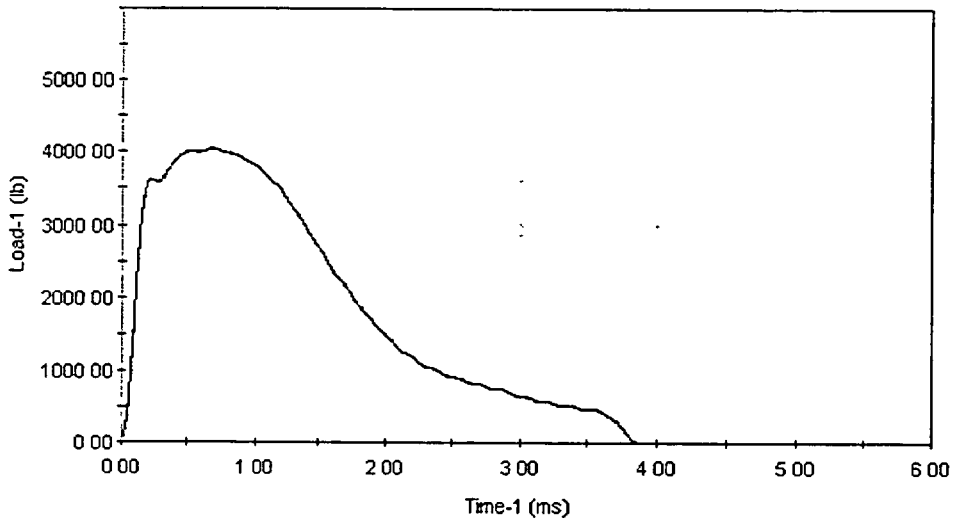
337, 200°F



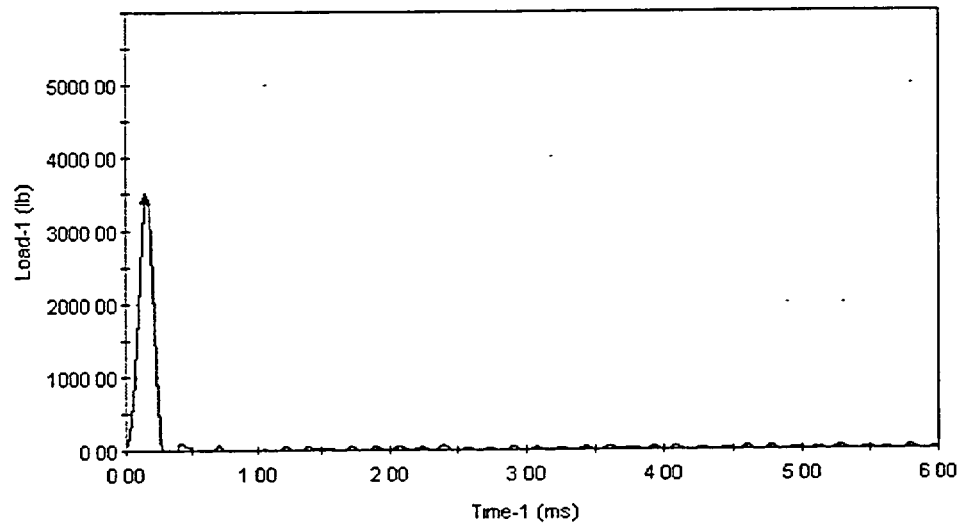
323, 225°F



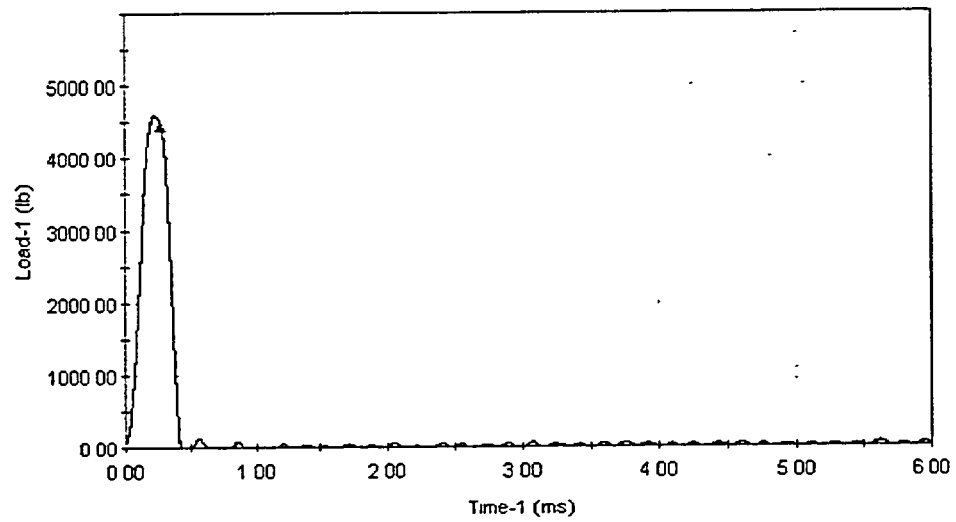
336, 250°F



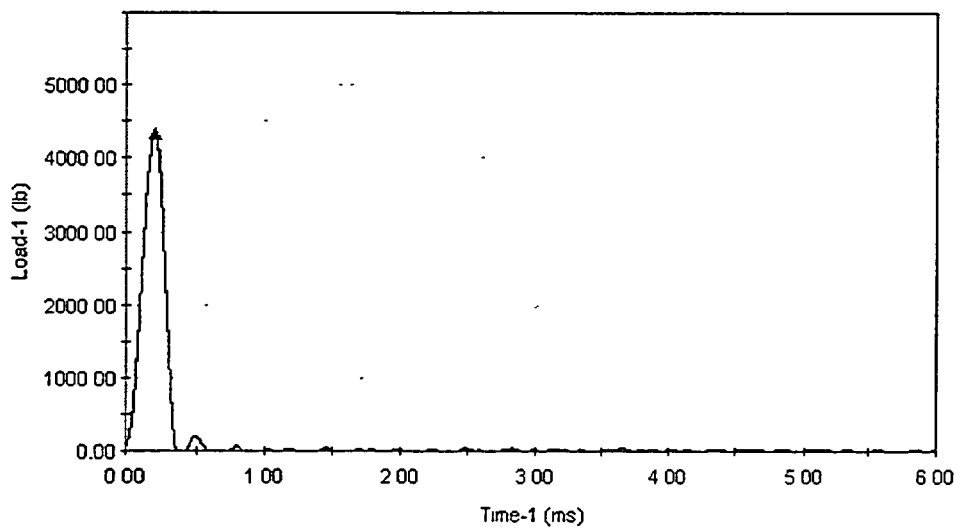
312, 250°F



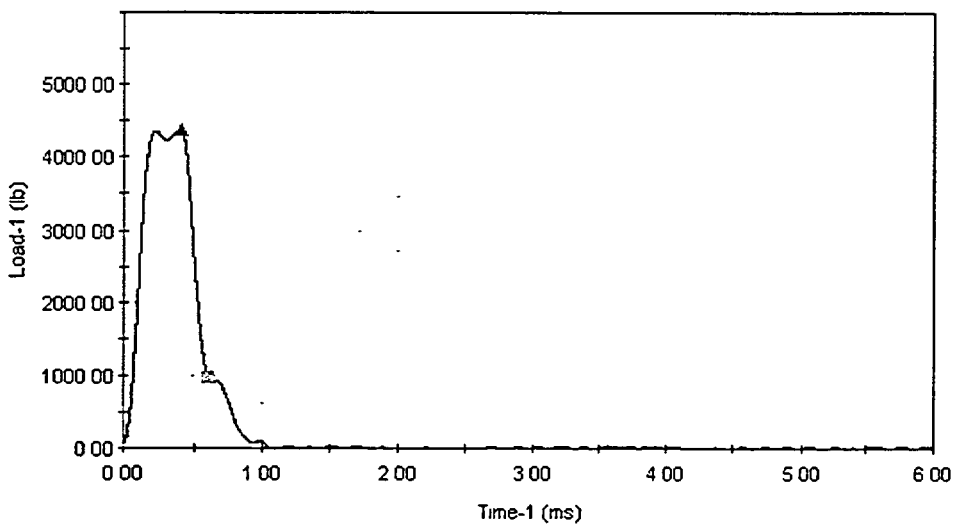
42T, -75°F



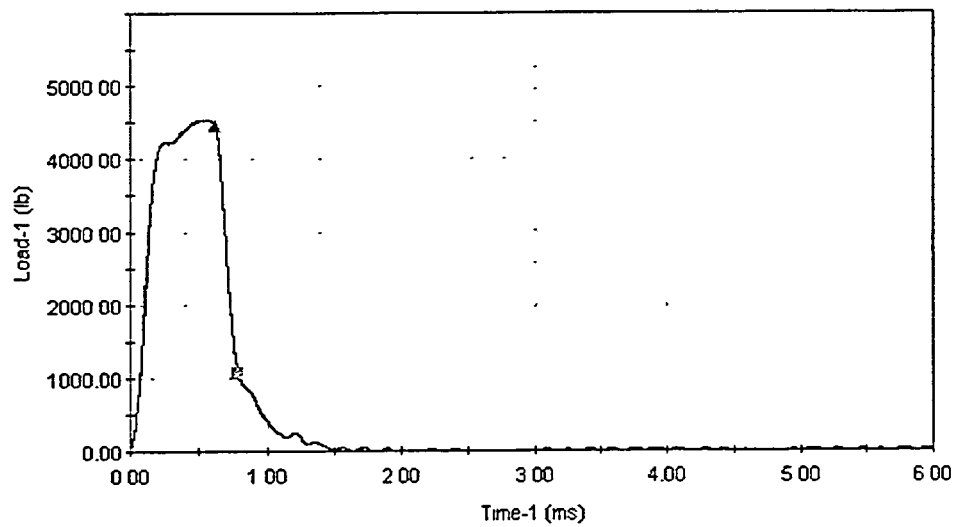
46E, -25°F



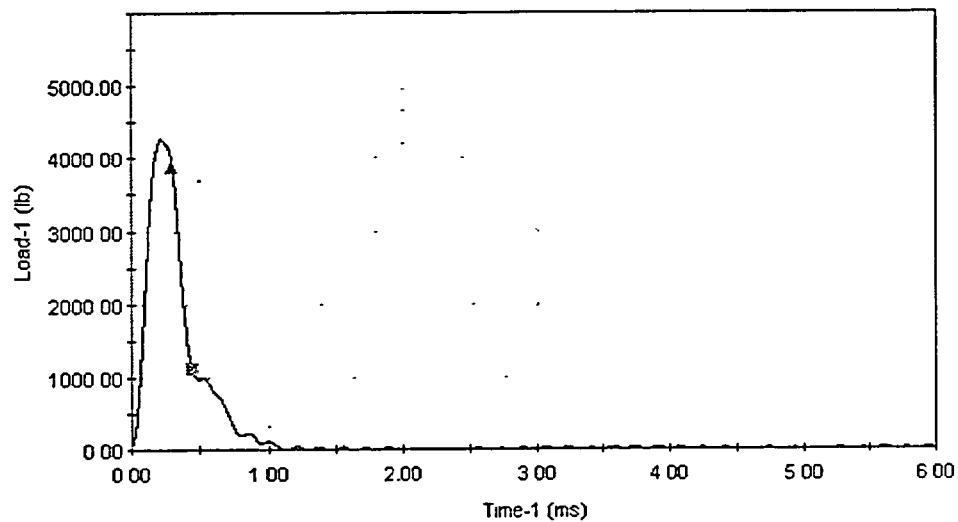
42P, 0°F



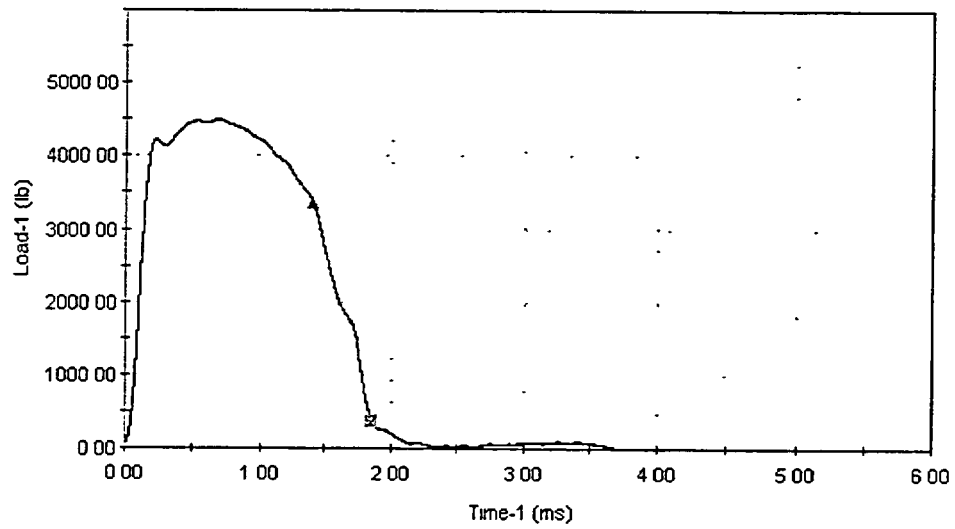
41E, 25°F



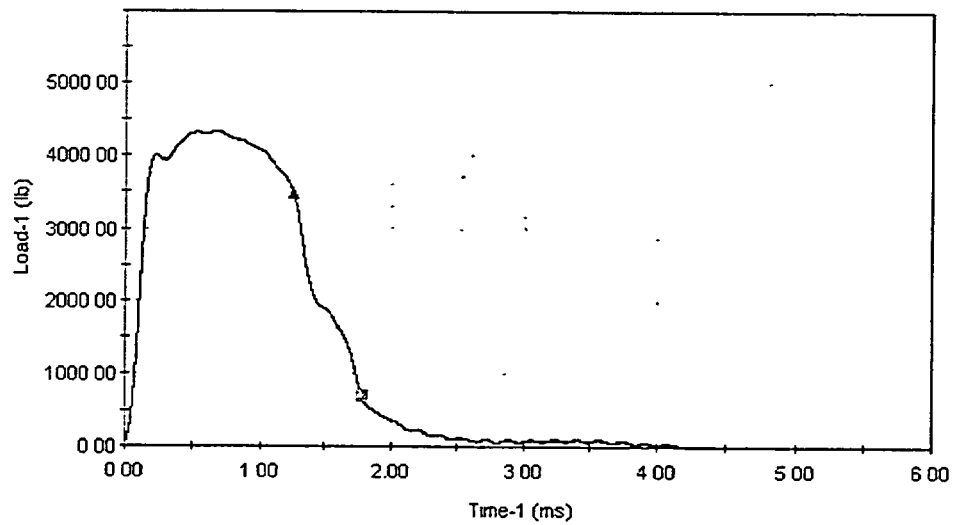
41T, 50°F



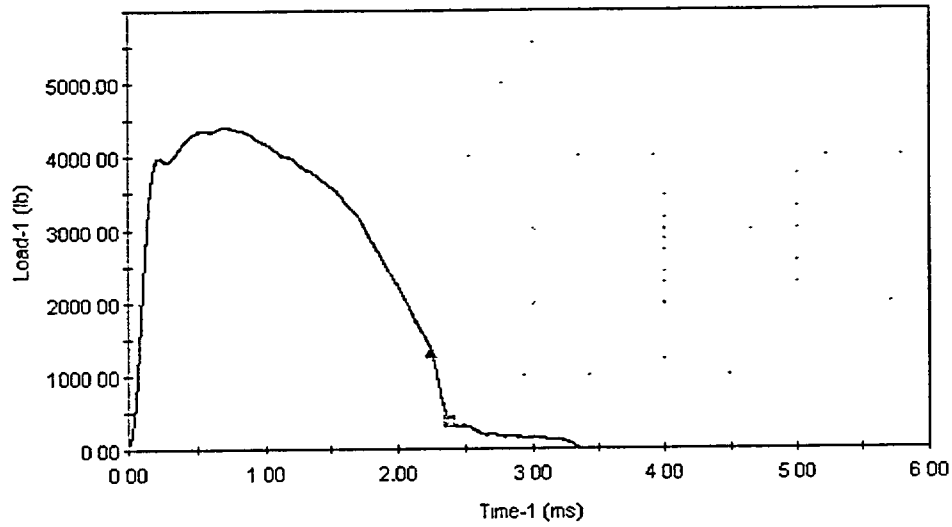
42U, 75°F



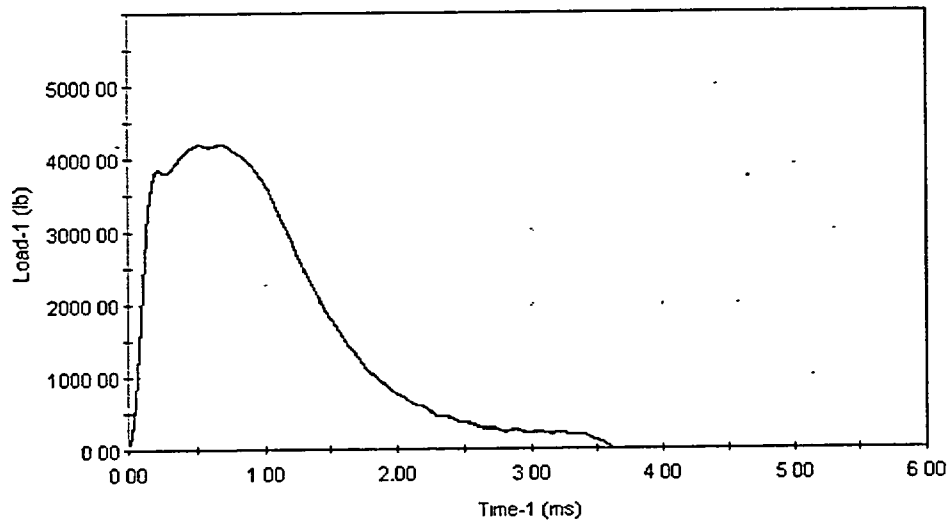
427, 100°F



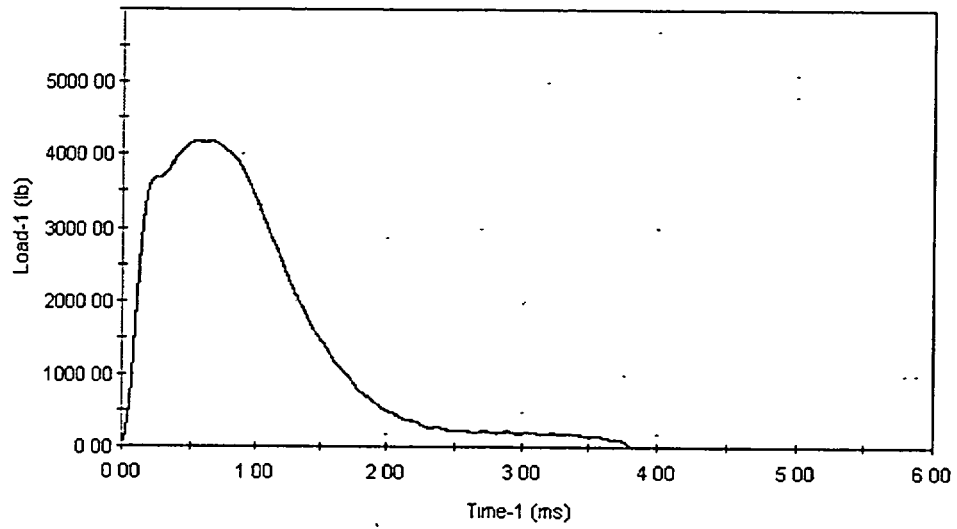
43K, 150°F



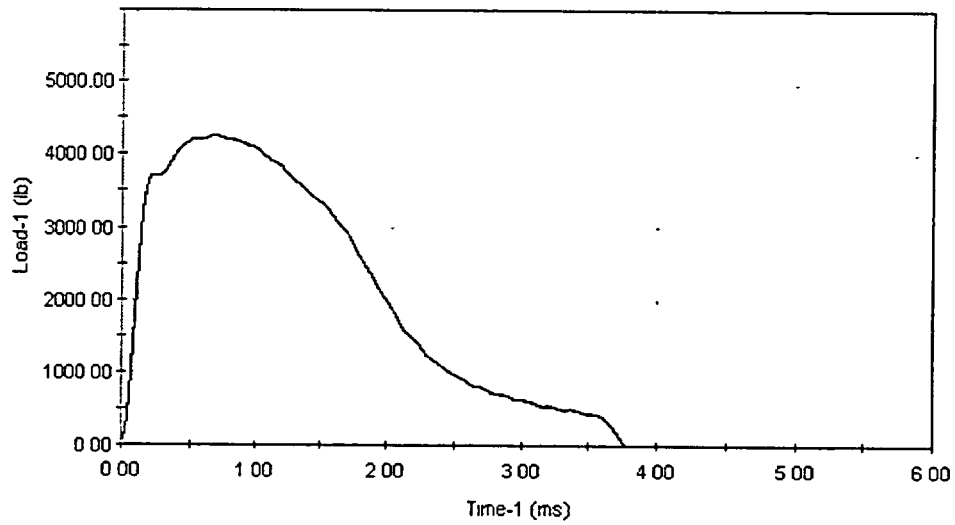
41U, 200°F



46B, 250°F



45K, 300°F



44C, 325°F

APPENDIX C

**CHARPY V-NOTCH PLOTS FOR EACH CAPSULE
USING HYPERBOLIC TANGENT CURVE-FITTING METHOD**

LOWER SHELL PLATE C-506-1 UNIRR (LONG)

CVGRAPH 41 Hyperbolic Tangent Curve Printed at 152101 on 10-10-2002

Page 1

Coefficients of Curve 1

A = 66.59

B = 64.4

C = 52.79

T0 = 78.28

$$\text{Equation is: } \text{CVN} = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf Energy: 131 Fixed Temp. at 30 ft-lbs: 442 Temp. at 50 ft-lbs: 64.3 Lower Shelf Energy: 2.19 Fixed

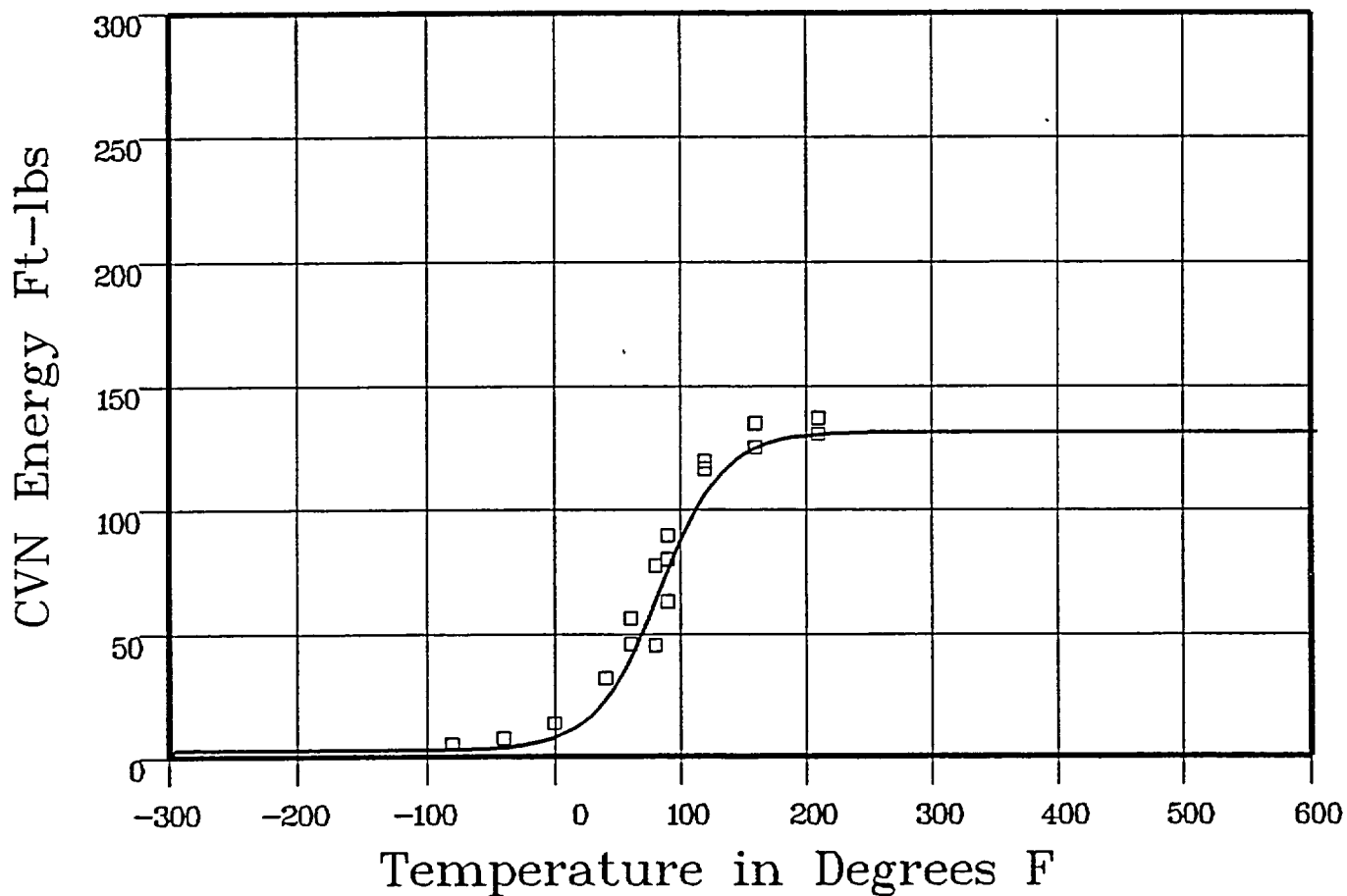
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: UNIRR

Total Fluence:



Plant: ML2 Cap: UNIRR Material: PLATE SA533B1 Ori: LT Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
-80	4.5	2.51	1.98
-40	7	3.64	3.35
0	13.5	8.51	4.98
40	32	26.66	5.33
60	45.5	45.15	.34
60	56	45.15	10.84
80	45	68.69	-23.69
80	77	68.69	8.3
90	79.5	80.66	-11.6

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 UNIRR (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: UNIRR

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
90	89	80.66	8.33
90	625	80.66	-18.16
120	119	109	9.99
120	116	109	6.99
160	1245	125.42	-92
160	1345	125.42	9.07
210	130	130.12	-12
210	1365	130.12	6.37
		SUM of RESIDUALS =	31.84

LOWER SHELL PLATE C-506-1 CAPSULE 97 (LONG)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 1521:01 on 10-10-2002

Page 1

Coefficients of Curve 2

A = 48.09

B = 45.9

C = 87.56

T0 = 146.48

$$\text{Equation is: } \text{CVN} = A + B * [\tanh((T - T_0)/C)]$$

Upper Shelf Energy: 94 Fixed Temp. at 30 ft-lbs: 109.9 Temp. at 50 ft-lbs: 150.1 Lower Shelf Energy: 219 Fixed

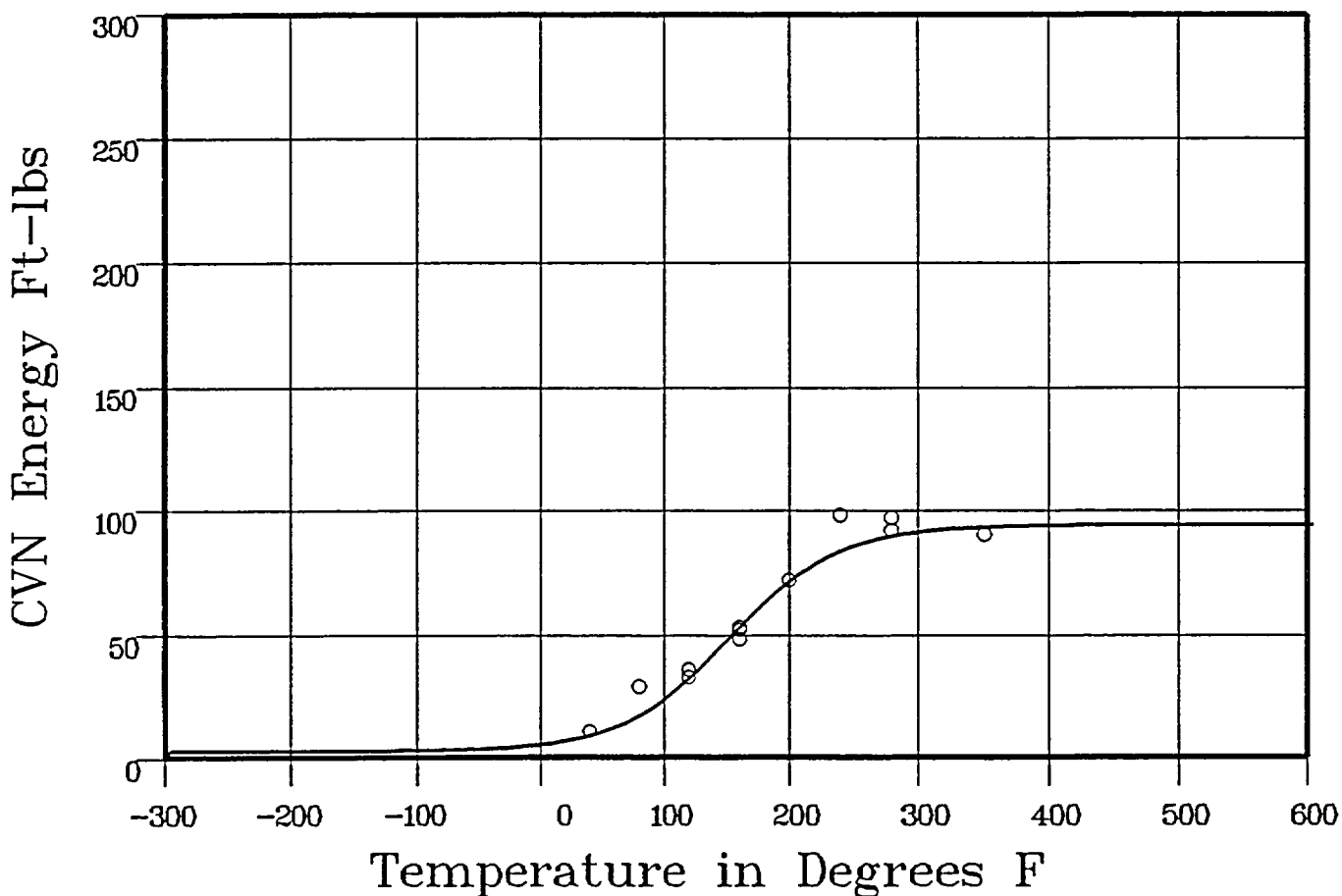
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-97

Total Fluence:



Plant: ML2 Cap: W-97 Data Set(s) Plotted Material: PLATE SA533B1 Ori: LT Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
40	11	9.61	1.38
80	29	18.69	10.3
120	33	34.62	-16.2
120	36	34.62	1.37
160	53	55.12	-2.12
160	52	55.12	-3.12
160	48	55.12	-7.12
200	72	73.11	-1.11

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 97 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-97

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
240	98	84.3	13.69
280	97	89.84	7.15
280	92	89.84	2.15
350	90	93.12	-3.12
		SUM of RESIDUALS =	17.81

LOWER SHELL PLATE C-506-1 CAPSULE 104 (LONG)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 15:21:01 on 10-10-2002

Page 1

Coefficients of Curve 3

A = 48.59	B = 46.4	C = 91.59	T0 = 170.8
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$$\text{Equation is: } \text{CVN} = A + B * [\tanh((T - T_0)/C)]$$

Upper Shelf Energy: 95 Fixed Temp. at 30 ft-lbs: 131.9 Temp. at 50 ft-lbs: 173.5 Lower Shelf Energy: 219 Fixed

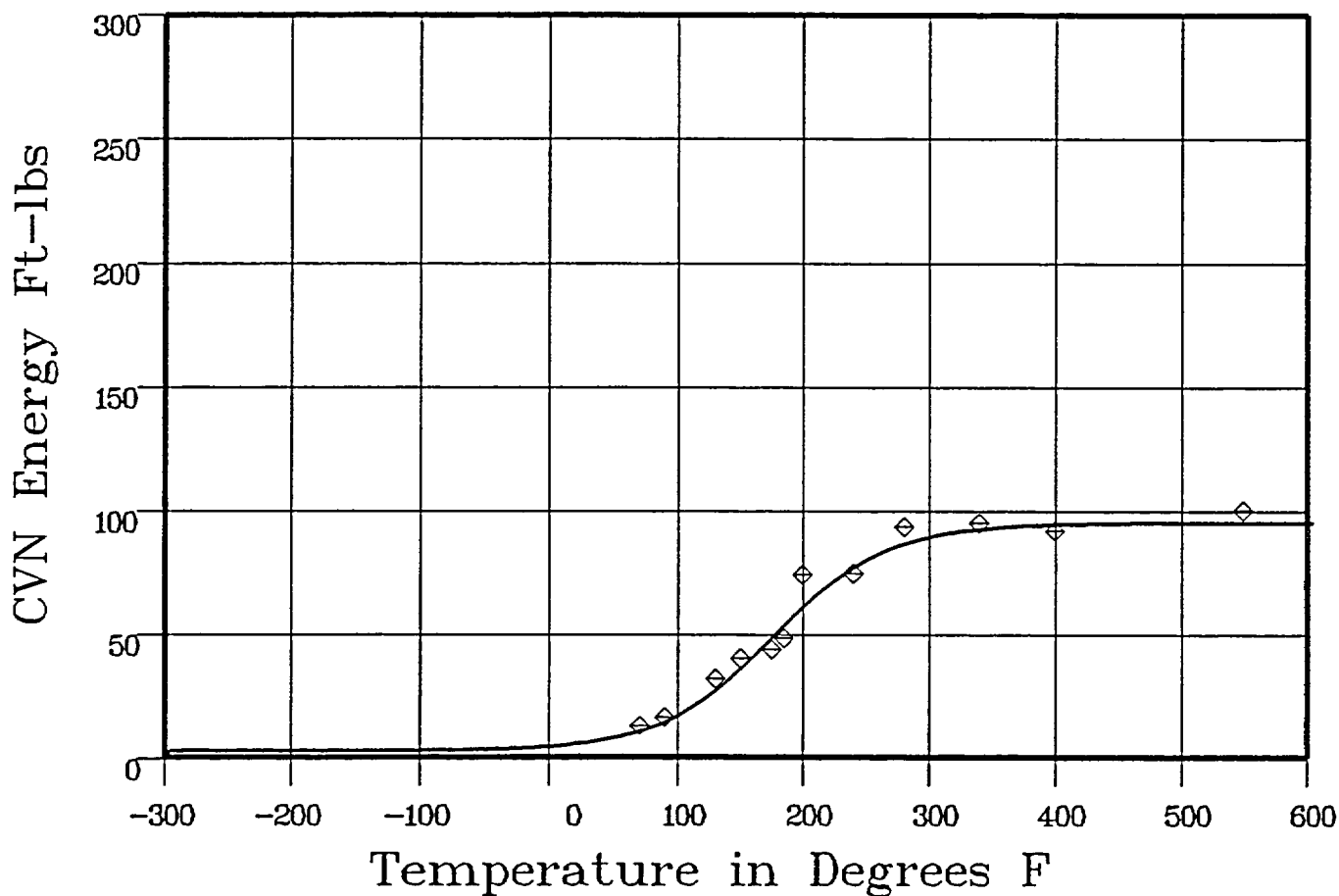
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-104

Total Fluence:



Plant: ML2 Cap: W-104 Material: PLATE SA533B1 Ori: LT Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
70	13	11.44	1.55
90	16.5	15.77	.72
130	32	29.19	2.8
150	40	38.24	1.75
175	43.5	50.72	-7.22
185	48	55.73	-7.73
200	74	62.91	11.08
240	74.5	78.22	-3.72

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 104 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-104

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
280	93.5	87.17	6.32
340	95	92.74	2.25
400	92	94.38	-2.38
550	100	94.97	5.02
			SUM of RESIDUALS = 10.46

LOWER SHELL PLATE C-506-1 CAPSULE 83 (LONG)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 152101 on 10-10-2002

Page 1

Coefficients of Curve 4

A = 47.09	B = 44.9	C = 88.27	T0 = 198.75
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$$\text{Equation is: } CVN = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf Energy: 92 Fixed Temp. at 30 ft-lbs: 163.3 Temp. at 50 ft-lbs: 204.4 Lower Shelf Energy: 219 Fixed

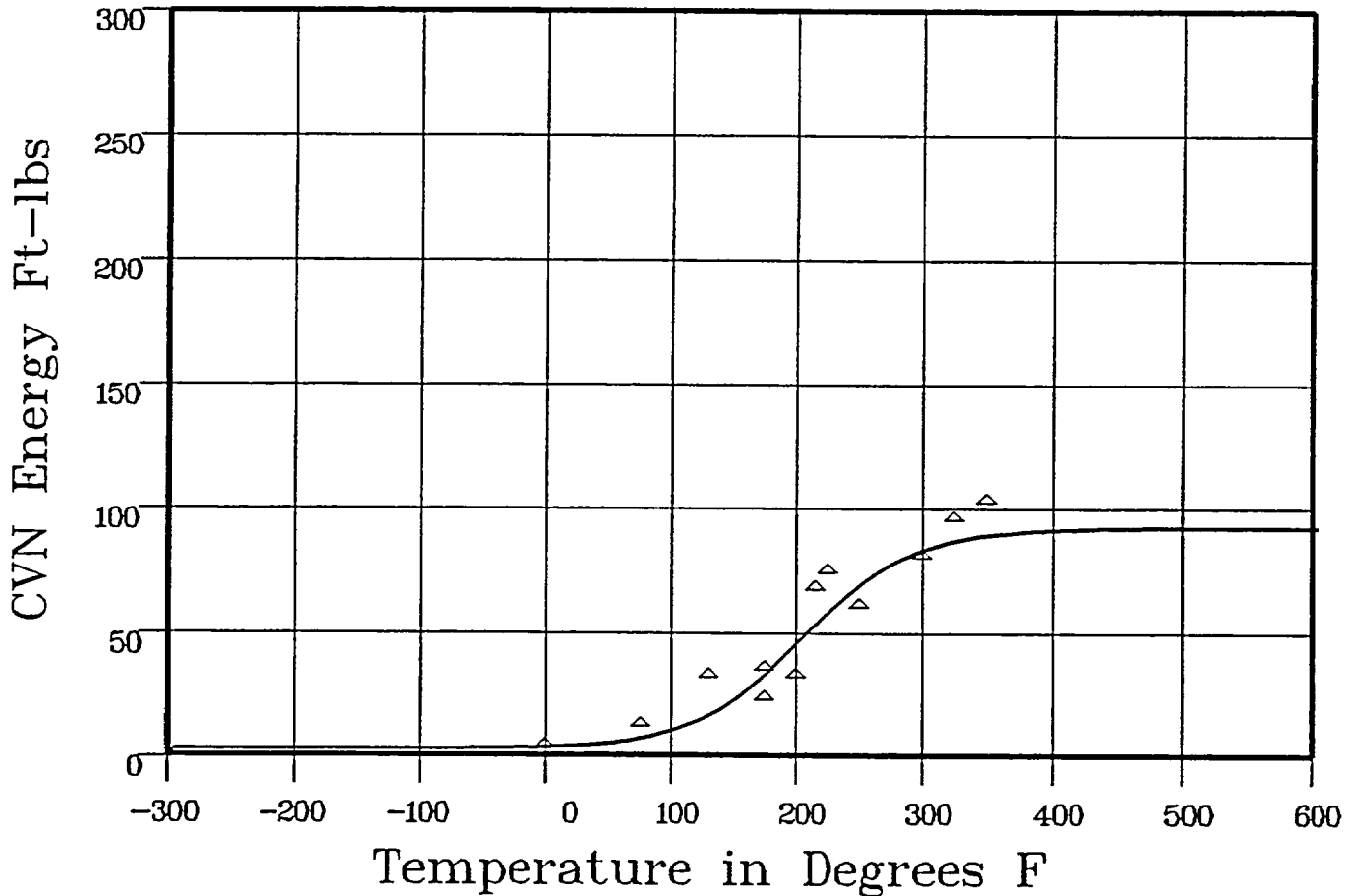
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-83

Total Fluence:



Plant: ML2 Cap: W-83 Material: PLATE SA533B1 Ori: LT Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
0	3	3.18	-18
75	12	7.32	4.67
130	32	17.82	14.17
175	35	35.3	-3
175	23	35.3	-12.3
200	32	47.73	-15.73
215	67	55.27	11.72

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 83 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-83

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
225	74	60.07	13.92
250	60	70.58	-10.58
300	80	83.77	-3.77
325	95	87.13	7.86
350	102	89.17	12.82
		SUM of RESIDUALS =	22.3

LOWER SHELL PLATE C-506-1 UNIRR (LONG)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 15:30:01 on 10-10-2002

Page 1

Coefficients of Curve 1

A = 47.72	B = 46.72	C = 6885	T0 = 64.21
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$$\text{Equation is: } LE = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf LE: 94.44

Temperature at LE 35: 44.9

Lower Shelf LE: 1 Fixed

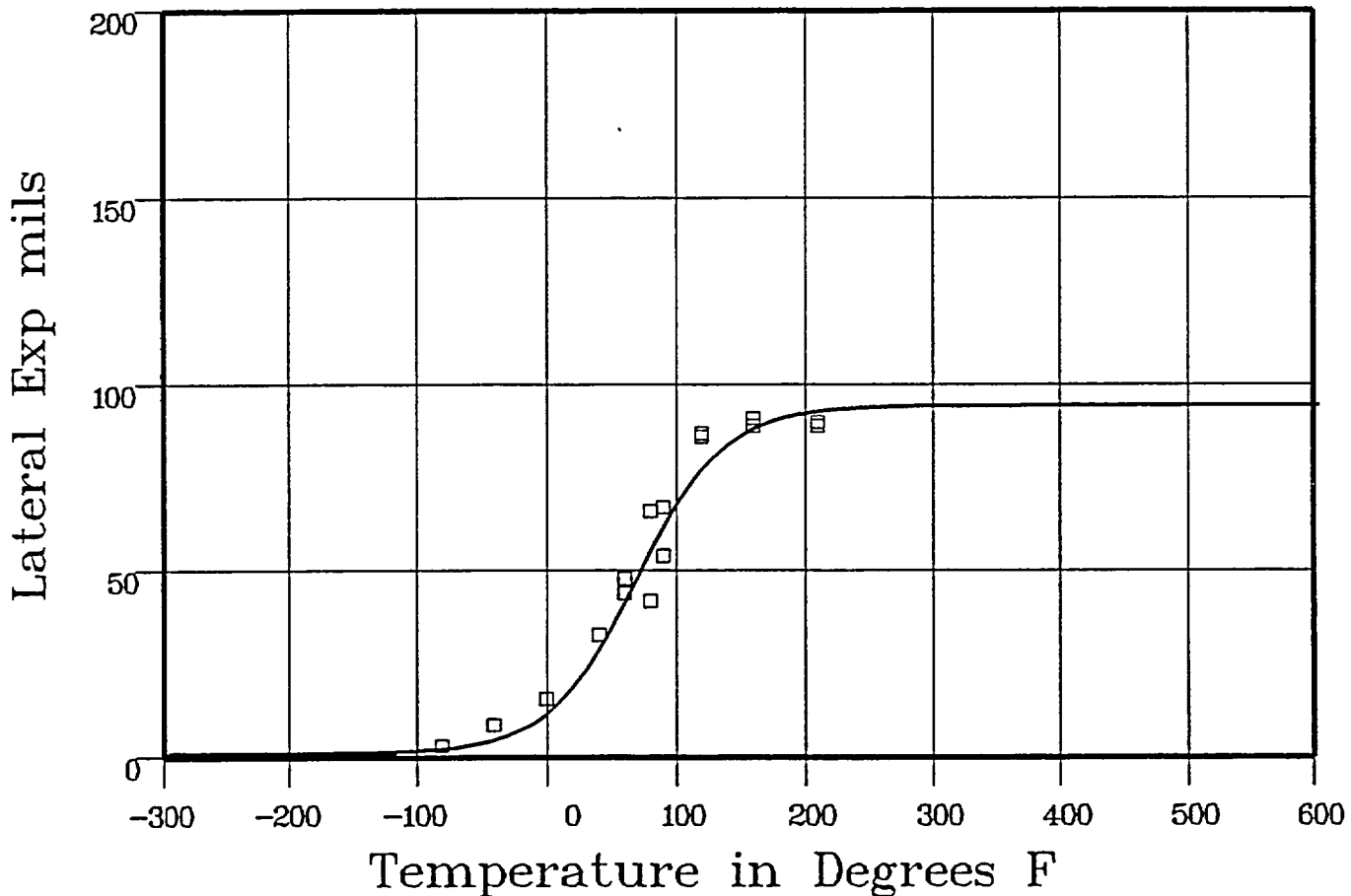
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: UNIRR

Total Fluence:



Plant: ML2 Cap: UNIRR Data Set(s) Plotted Material: PLATE SA533B1 Ori: LT Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
-80	3	2.39	.6
-40	9	5.31	3.68
0	16	13.52	2.47
40	33	31.93	1.06
60	44	44.86	-.86
60	48	44.86	3.13
80	42	58.24	-16.24
80	66	58.24	7.75
90	67	64.44	2.55

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 UNIRR (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: UNIRR

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed LE	Differential
90	67	64.44	255
90	54	64.44	-10.44
120	86	79.01	6.98
120	87	79.01	7.98
160	89	88.99	0
160	91	88.99	2
210	90	93.11	-3.11
210	89	93.11	-4.11
			SUM of RESIDUALS = 6.03

LOWER SHELL PLATE C-506-1 CAPSULE 97 (LONG)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 15:30:01 on 10-10-2002

Page 1

Coefficients of Curve 2

A = 43.28	B = 42.28	C = 110.42	T0 = 140.91
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$$\text{Equation is: } LE = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf LE: 85.56

Temperature at LE 35: 119

Lower Shelf LE: 1 Fixed

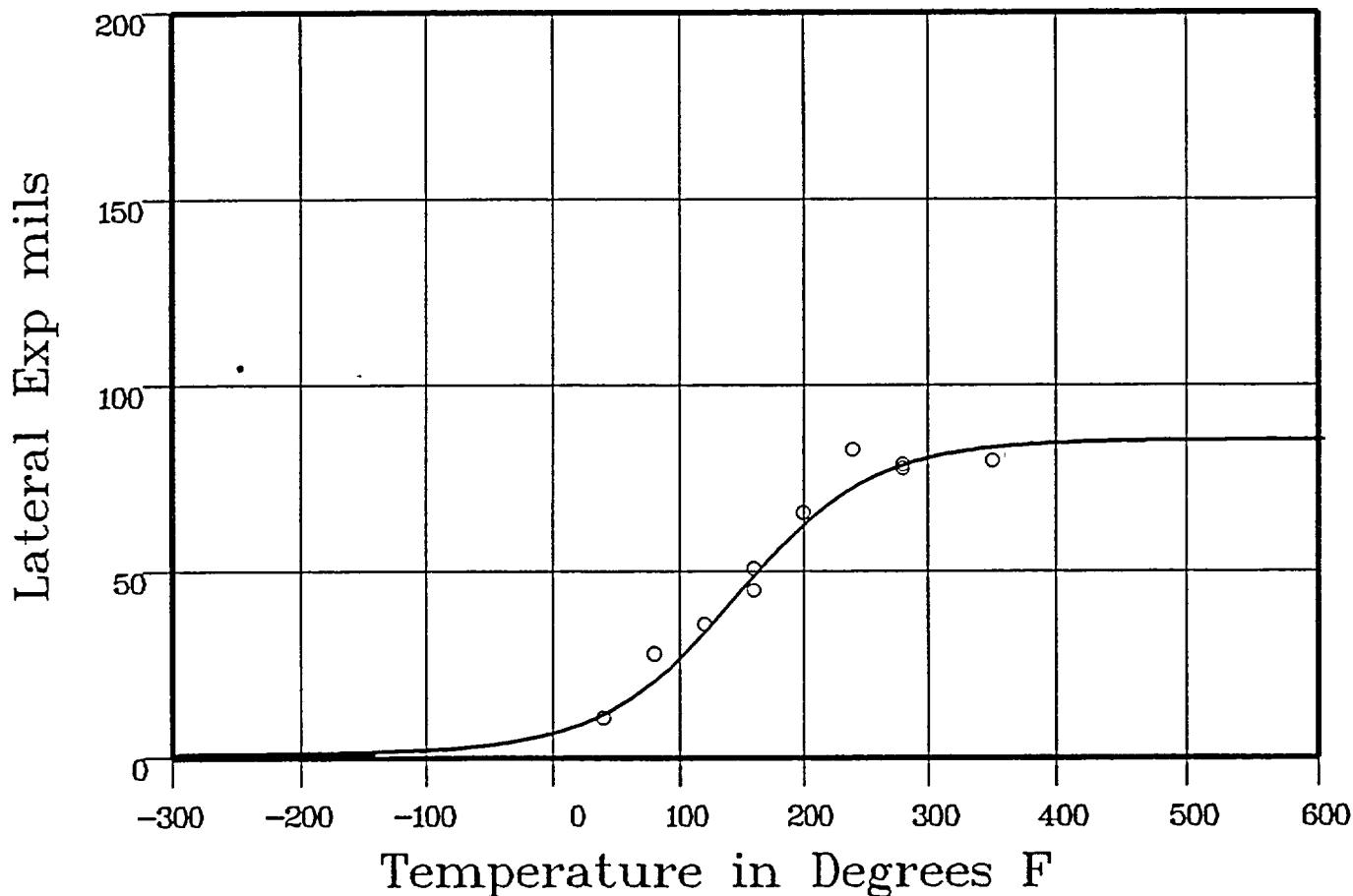
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-97

Total Fluence:



Plant: ML2 Cap: W-97 Material: PLATE SA533B1 Ori: LT Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
40	11	12.71	-1.71
80	28	22.06	5.93
120	36	35.36	.63
120	36	35.36	.63
160	51	50.51	.48
160	45	50.51	-5.51
160	45	50.51	-5.51
200	66	63.96	2.03

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 97 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-97

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed LE	Differential
240	83	73.51	9.48
280	79	79.26	-26
280	78	79.26	-126
350	80	83.69	-369

SUM of RESIDUALS = 124

LOWER SHELL PLATE C-506-1 CAPSULE 104 (LONG)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 15:30:01 on 10-10-2002

Page 1

Coefficients of Curve 3

A = 43.78

B = 42.78

C = 111.2

T0 = 162.89

Equation is: $LE = A + B * [\tanh((T - T0)/C)]$

Upper Shelf LE: 86.57

Temperature at LE 35: 139.7

Lower Shelf LE: 1 Fixed

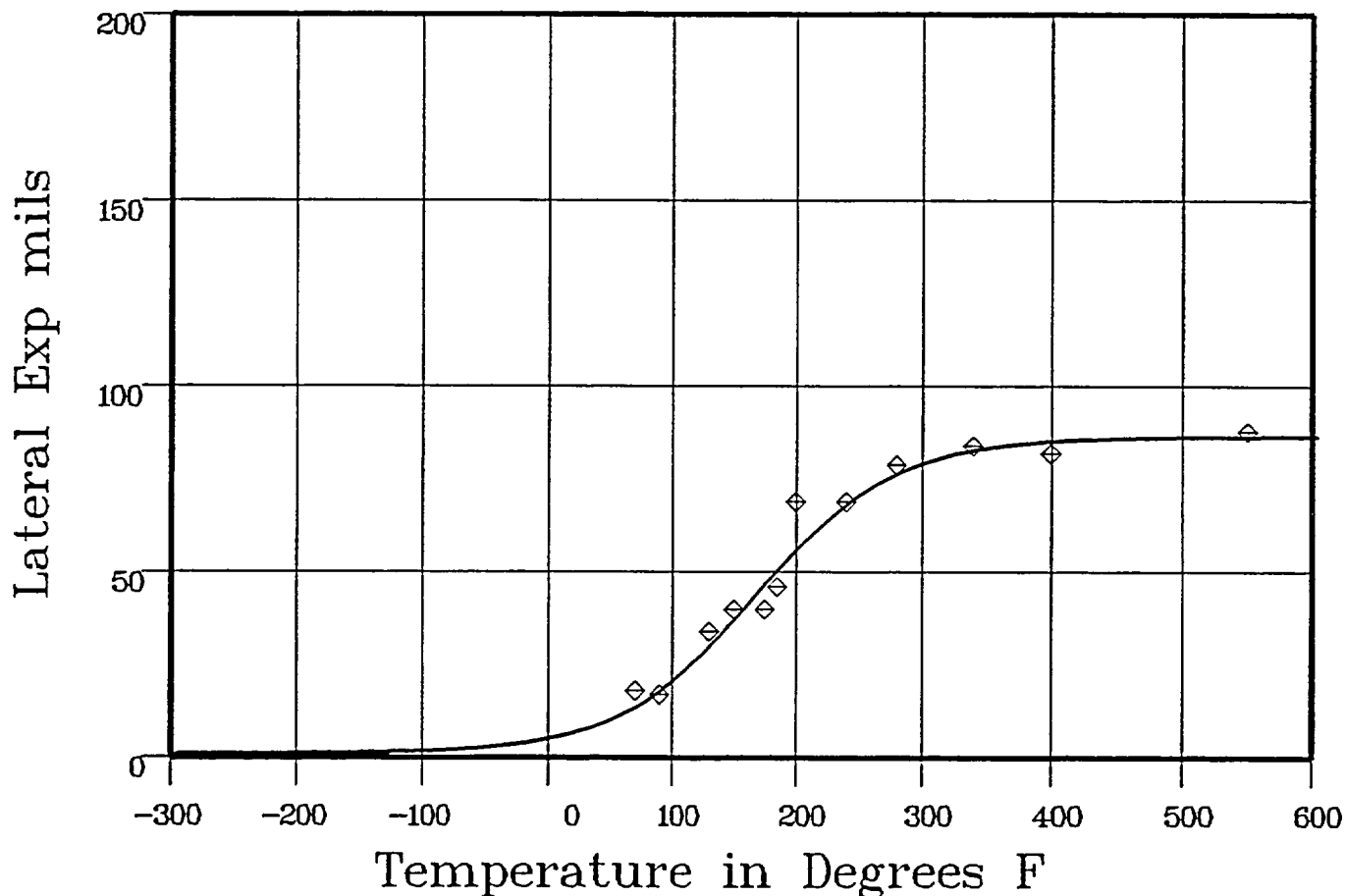
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-104

Total Fluence:



Plant: ML2 Cap: W-104 Material: PLATE SA533B1 Ori: LT Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
70	18	14.55	3.44
90	17	19.17	-2.17
130	34	31.48	2.51
150	40	38.84	1.15
175	40	48.42	-8.42
185	46	52.18	-6.18
200	69	57.55	11.44
240	69	69.46	-4.6

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 104 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-104

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed LE	Differential
280	79	77.28	1.71
340	84	83.17	.82
400	82	85.38	-3.38
550	88	86.49	1.5
			SUM of RESIDUALS = 1.95

LOWER SHELL PLATE C-506-1 CAPSULE 83 (LONG)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 15:30:01 on 10-10-2002

Page 1

Coefficients of Curve 4

A = 43.34	B = 42.34	C = 120.32	T0 = 215.33
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$$\text{Equation is: } LE = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf LE: 85.68

Temperature at LE 35: 191.3

Lower Shelf LE: 1 Fixed

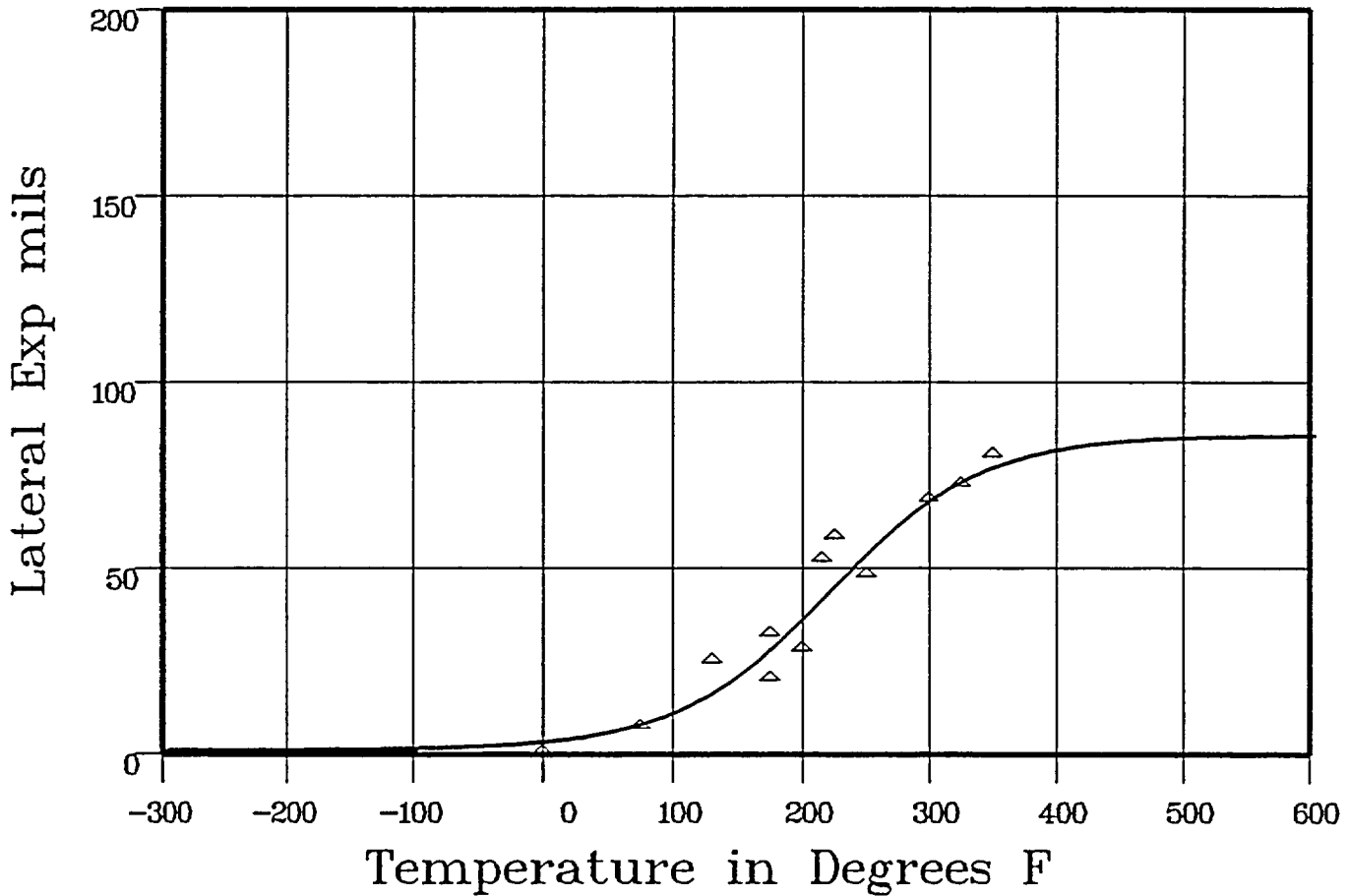
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-83

Total Fluence:



Plant: ML2 Cap: W-83 Material: PLATE SA533B1 Ori: LT Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
0	0	3.29	-3.29
75	7	8.49	-1.49
130	25	17.5	7.49
175	32	29.65	2.34
175	20	29.65	-9.65
200	28	37.97	-9.97
215	52	43.22	8.77

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 83 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-83

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed LE	Differential
225	58	46.73	11.26
250	48	55.21	-7.21
300	68	69.03	-1.03
325	72	73.9	-1.9
350	80	77.52	2.47
			SUM of RESIDUALS = -2.24

LOWER SHELL PLATE C-506-1 UNIRR (LONG)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 15:36:10 on 10-10-2002

Page 1

Coefficients of Curve 1

A = 50	B = 50	C = 67.59	T0 = 65.62
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Equation is $\text{Shear}\% = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 65.6

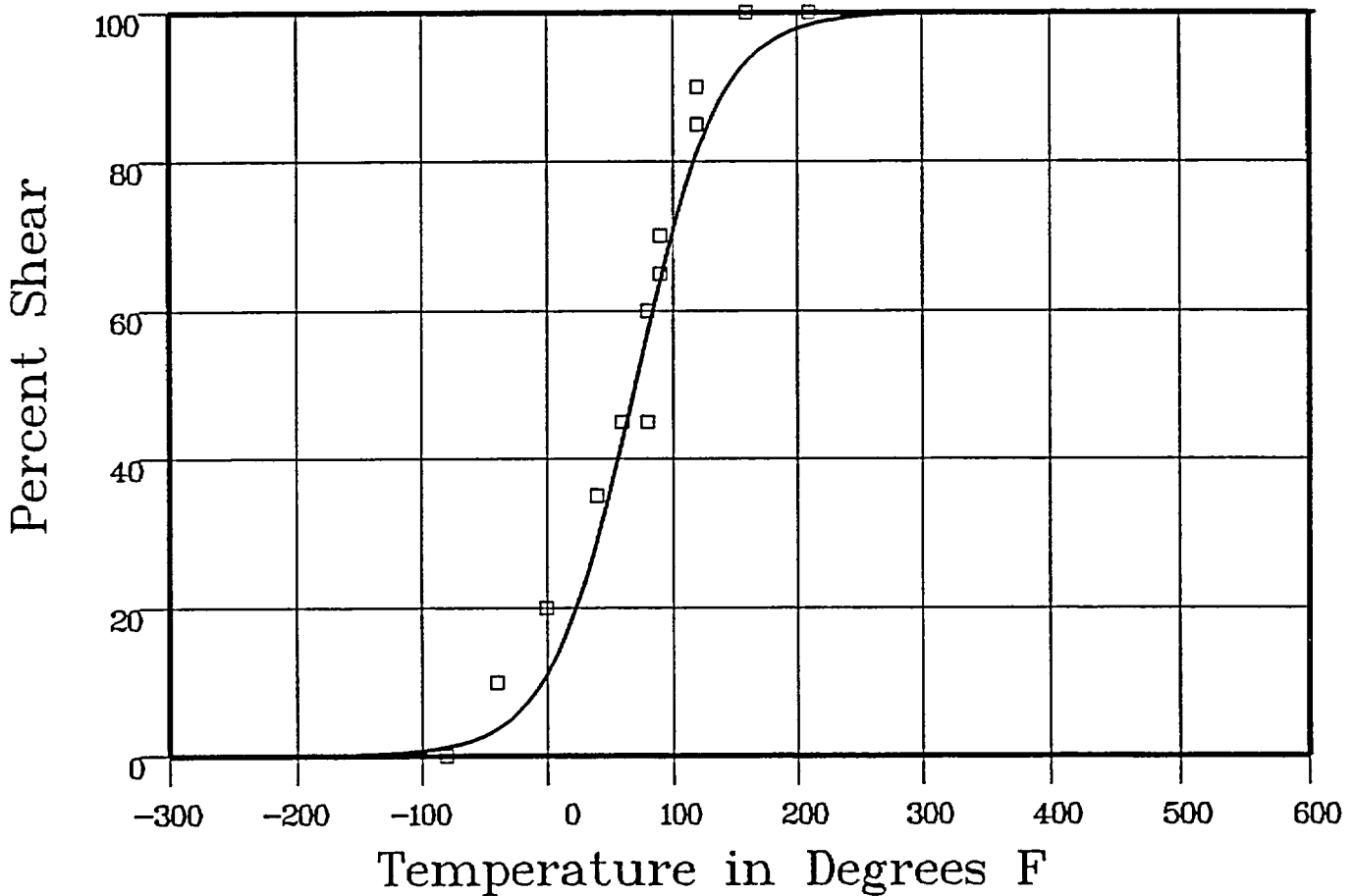
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: UNIRR

Total Fluence:



Data Set(s) Plotted

Plant: ML2

Cap: UNIRR

Material: PLATE SA533B1

Ori: LT

Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
-80	0	1.32	-1.32
-40	10	4.2	5.79
0	20	12.54	7.45
40	35	31.9	3.09
60	45	45.84	-8.4
60	45	45.84	-8.4
80	45	60.47	-15.47
80	60	60.47	-4.7
90	70	67.28	2.71

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 UNIRR (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: UNIRR

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
90	70	67.28	2.71
90	65	67.28	-2.28
120	90	83.32	6.67
120	85	83.32	1.67
160	100	94.22	5.77
160	100	94.22	5.77
210	100	98.62	1.37
210	100	98.62	1.37
			SUM of RESIDUALS = 23.14

LOWER SHELL PLATE C-506-1 CAPSULE 97 (LONG)

CVGRAPH 41 Hyperbolic Tangent Curve Printed at 15:36:10 on 10-10-2002

Page 1

Coefficients of Curve 2

A = 50

B = 50

C = 58.22

T0 = 177.65

Equation is: $\text{Shear}\% = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 177.6

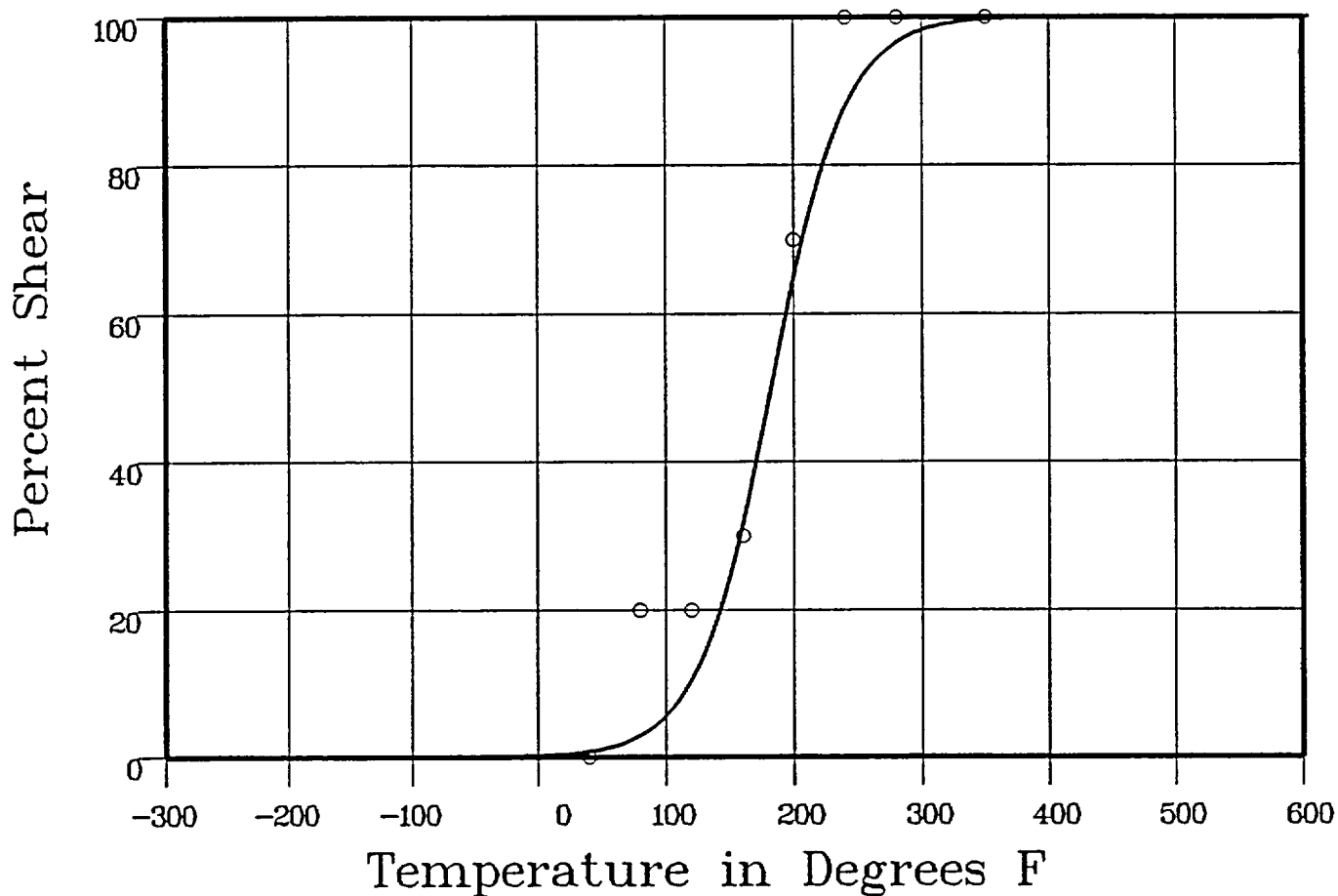
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-97

Total Fluence:



Data Set(s) Plotted
Plant: ML2 Cap: W-97 Material: PLATE SA533B1 Ori: LT Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
40	0	.87	-.87
80	20	3.37	16.62
120	20	12.12	7.87
120	20	12.12	7.87
160	30	35.28	-5.28
160	30	35.28	-5.28
160	30	35.28	-5.28
200	70	68.29	1.7

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 97 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-97 Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
240	100	89.48	10.51
280	100	97.11	2.88
280	100	97.11	2.88
350	100	99.73	.26
			SUM of RESIDUALS = 33.89

LOWER SHELL PLATE C-506-1 CAPSULE 104 (LONG)

CVGRAPH 41 Hyperbolic Tangent Curve Printed at 15:36:10 on 10-10-2002

Page 1

Coefficients of Curve 3

A = 50	B = 50	C = 104.92	T0 = 172.55
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Equation is $\text{Shear\%} = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 172.5

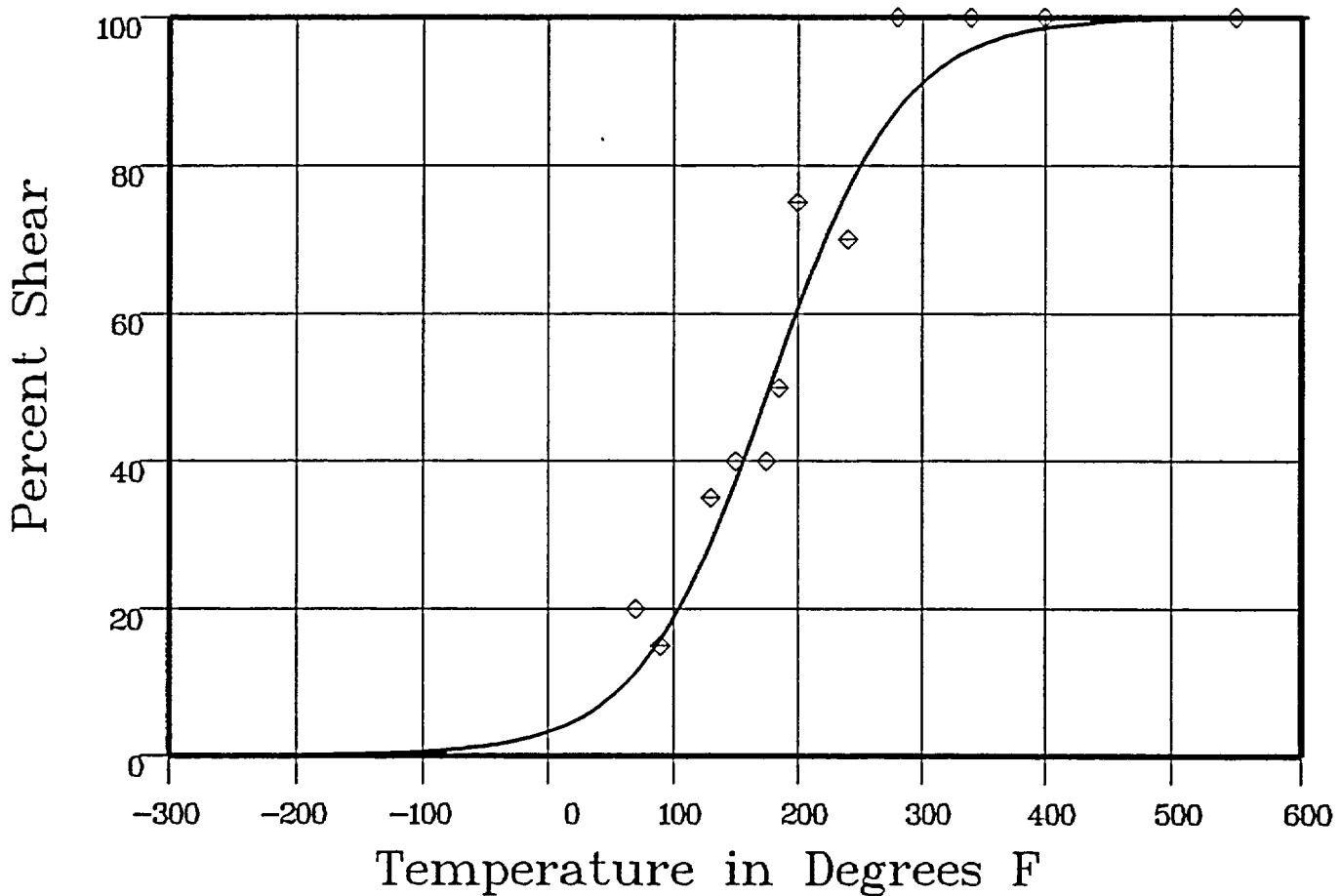
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-104

Total Fluence:



Plant: ML2 Cap: W-104 Material: PLATE SA533B1 Ori: LT Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
70	20	12.4	7.59
90	15	17.16	-2.16
130	35	30.76	4.23
150	40	39.41	.58
175	40	51.16	-11.16
185	50	55.9	-5.9
200	75	62.78	12.21
240	70	78.33	-8.33

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 104 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-104

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
280	100	88.57	11.42
340	100	96.05	3.94
400	100	98.7	1.29
550	100	99.92	.07
			SUM of RESIDUALS = 13.8

LOWER SHELL PLATE C-506-1 CAPSULE 83 (LONG)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 15:36:10 on 10-10-2002

Page 1

Coefficients of Curve 4

A = 50

B = 50

C = 88.42

T0 = 196.02

Equation is: $\text{Shear\%} = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 196

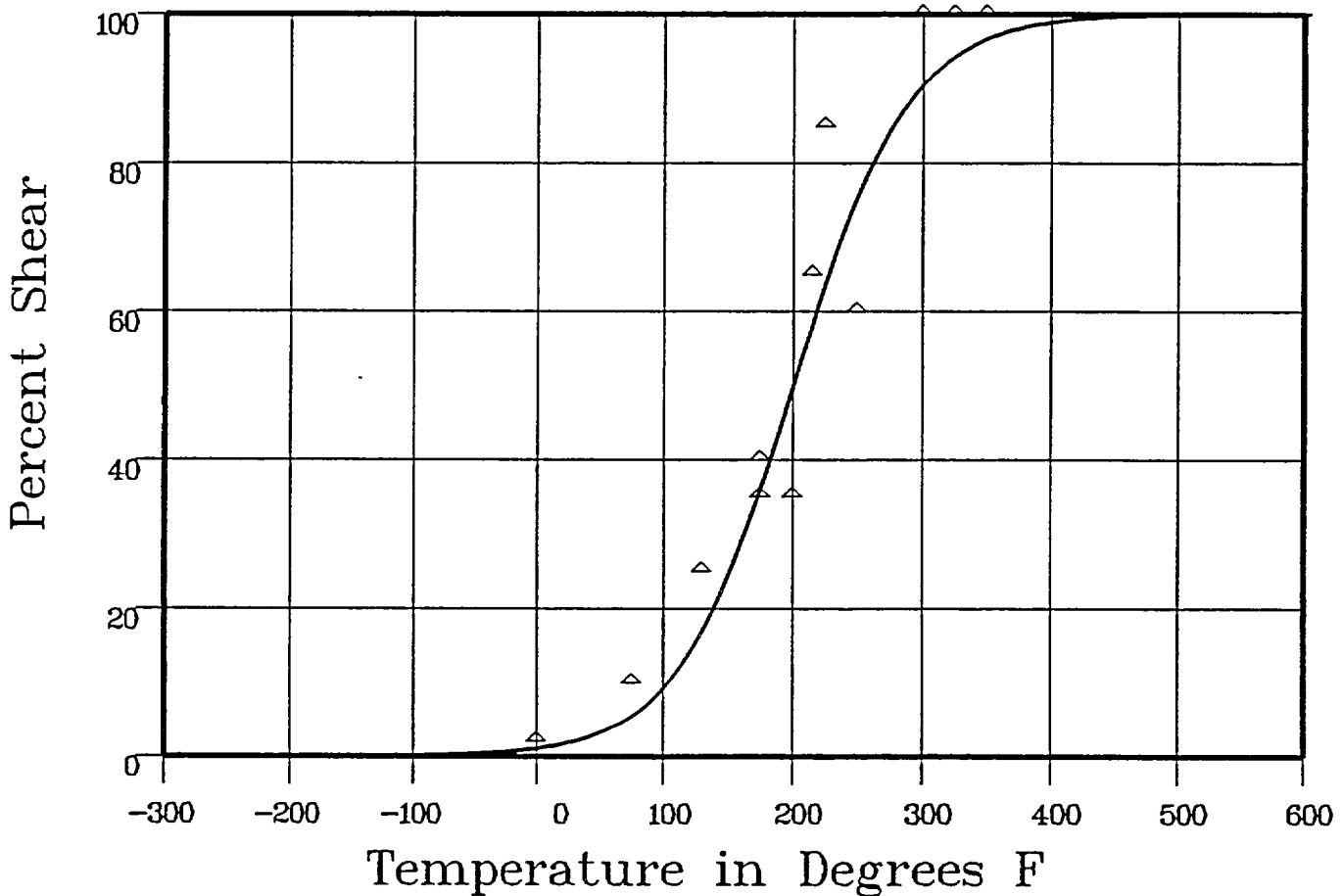
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-83

Total Fluence:



Data Set(s) Plotted
Plant: ML2 Cap: W-83 Material: PLATE SA533B1 Ori: LT Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
0	2	1.17	.82
75	10	6.08	3.91
130	25	18.34	6.65
175	35	38.33	-3.33
175	40	38.33	1.66
200	35	52.24	-17.24
215	65	60.56	4.43

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 83 (LONG)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: LT

Capsule: W-83

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
225	85	65.82	19.17
250	60	77.22	-17.22
300	100	91.3	8.69
325	100	94.86	5.13
350	100	97.01	2.98
		SUM of RESIDUALS =	15.68

LOWER SHELL PLATE C-506-1 UNIRR (TRANS)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 104352 on 10-14-2002

Page 1

Coefficients of Curve 1

A = 55.09

B = 52.9

C = 80.63

T0 = 60

$$\text{Equation is: } CVN = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf Energy: 108 Fixed Temp. at 30 ft-lbs: 184 Temp. at 50 ft-lbs: 522 Lower Shelf Energy: 219 Fixed

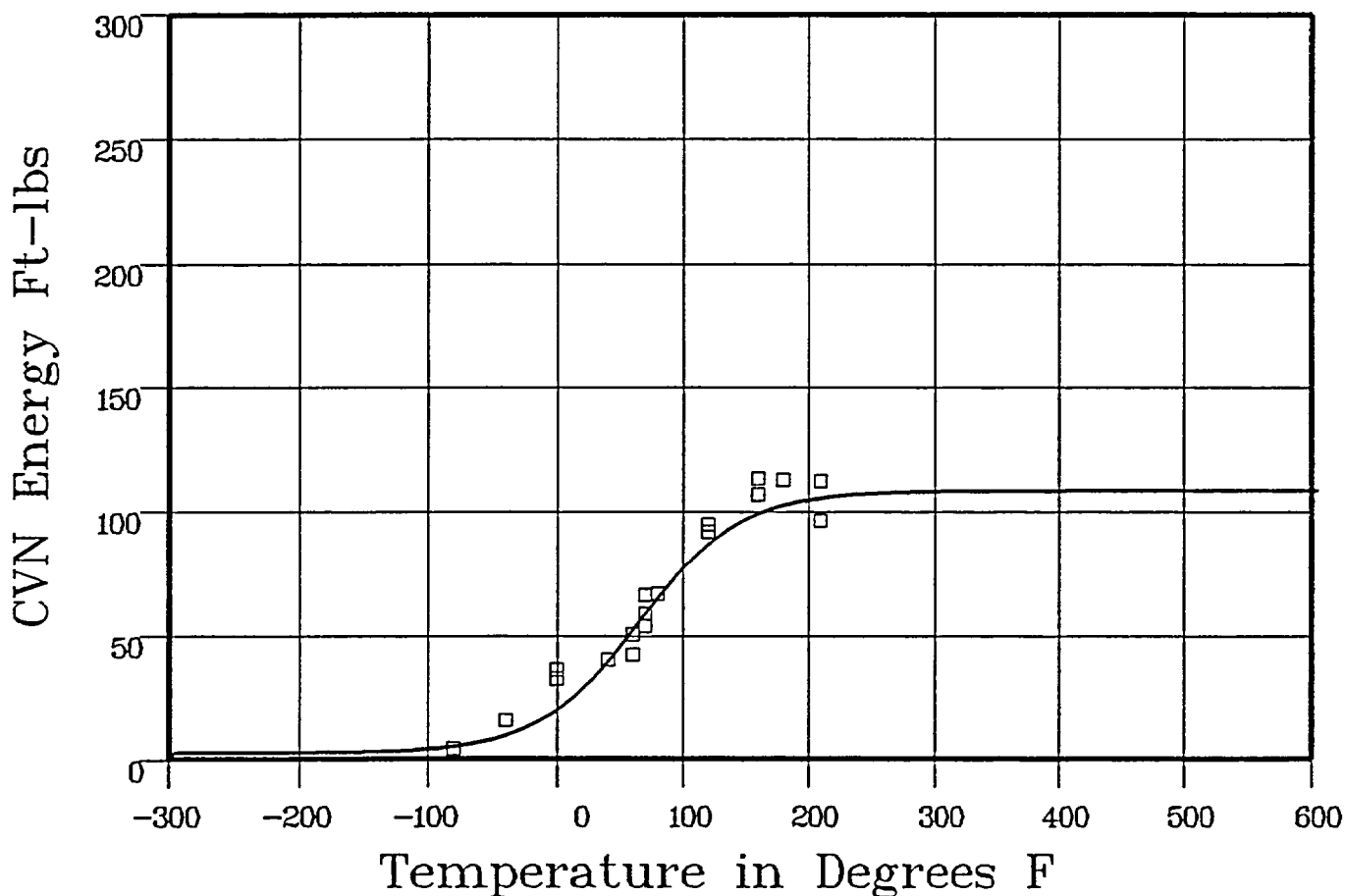
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: UNIRR

Total Fluence:



Plant: ML2 Cap: UNIRR Data Set(s) Plotted Material: PLATE SA533B1 Ori: TL Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
-80	4	5.38	-1.38
-40	15.5	10.37	5.12
0	32.5	21.68	10.81
0	36	21.68	14.31
40	40	42.24	-2.24
60	50	55.09	-5.09
60	42	55.09	-13.09
70	58.5	61.62	-3.12
70	66	61.62	4.37

*** Data continued on next page ***

LOWER SHELL PLATE C-506-1 UNIRR (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: UNIRR

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
70	53.5	61.62	-8.12
80	66.5	67.95	-1.45
120	94.5	88.51	5.98
120	91.5	88.51	2.98
160	113	99.82	13.17
160	106.5	99.82	6.67
180	112.5	102.86	9.63
210	112	105.49	6.5
210	96	105.49	-9.49
			SUM of RESIDUALS = 35.53

LOWER SHELL PLATE C-506-1 CAPSULE 97 (TRANS)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:43:52 on 10-14-2002

Page 1

Coefficients of Curve 2

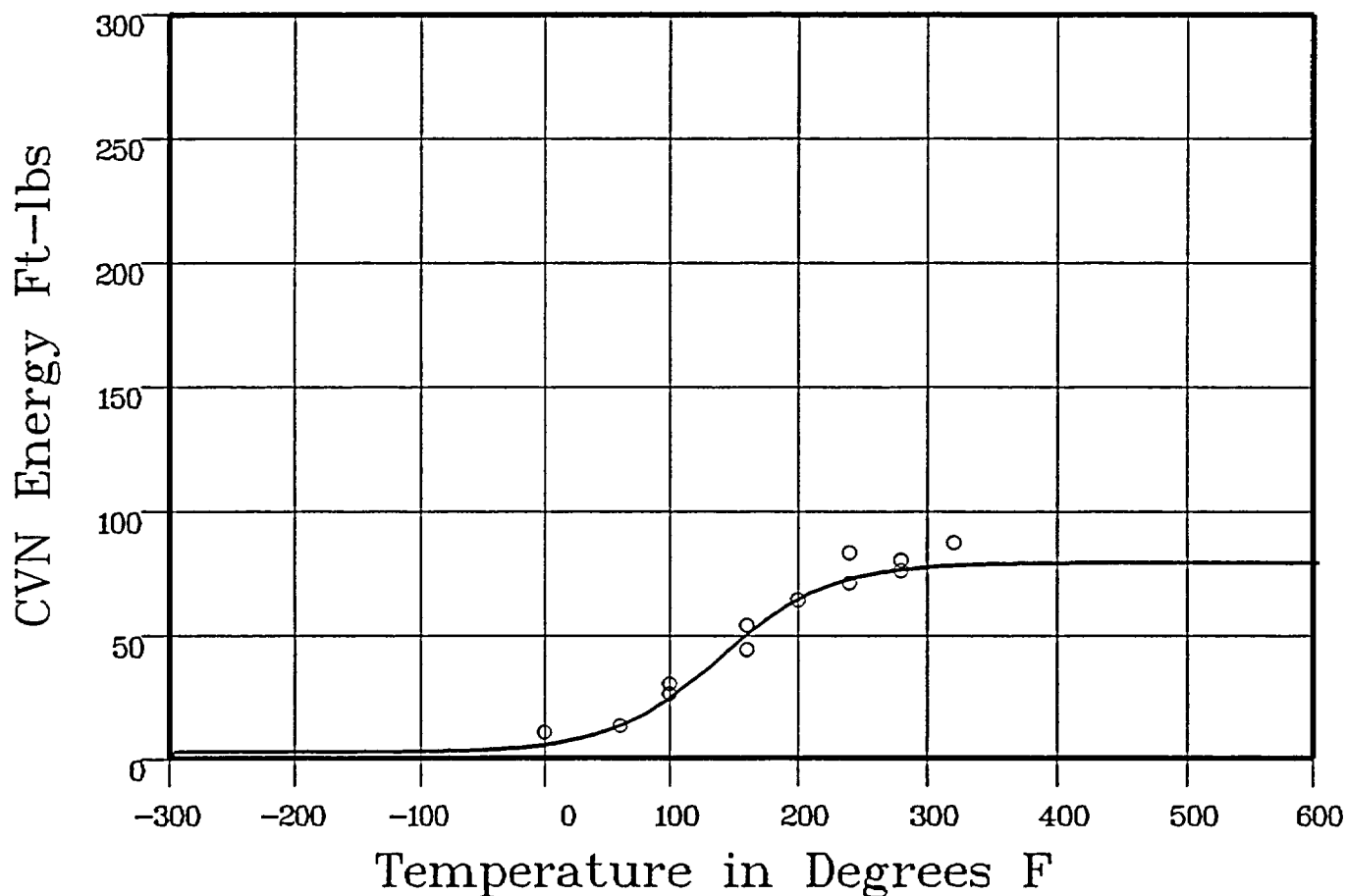
A = 40.59	B = 38.4	C = 85.91	T0 = 133.59
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$$\text{Equation is: } \text{CVN} = A + B * [\tanh((T - T_0)/C)]$$

Upper Shelf Energy: 79 Fixed Temp. at 30 ft-lbs 1092 Temp. at 50 ft-lbs: 155 Lower Shelf Energy: 219 Fixed

Material: PLATE SA533B1 Heat Number: C-5667-1 Orientation: TL

Capsule: W-97 Total Fluence:



Plant: ML2 Cap: W-97 Data Set(s) Plotted Material: PLATE SA533B1 Ori: TL Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
0	10	5.47	4.52
60	13	13.93	-93
100	26	26.3	-3
100	30	26.3	3.69
160	44	52.04	-804
160	54	52.04	1.95
200	64	65.5	-15
240	83	73.04	9.95

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 97 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: W-97

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
240	71	73.04	-2.04
280	76	76.53	-53
280	80	76.53	3.46
320	87	78.01	8.98

SUM of RESIDUALS = 19.19

LOWER SHELL PLATE C-506-1 CAPSULE 83 (TRANS)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:43:52 on 10-14-2002

Page 1

Coefficients of Curve 3

A = 43.09

B = 40.9

C = 81.51

T0 = 191.25

$$\text{Equation is: } CVN = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf Energy: 84 Fixed Temp. at 30 ft-lbs: 164.1 Temp. at 50 ft-lbs: 205.1 Lower Shelf Energy: 2.19 Fixed

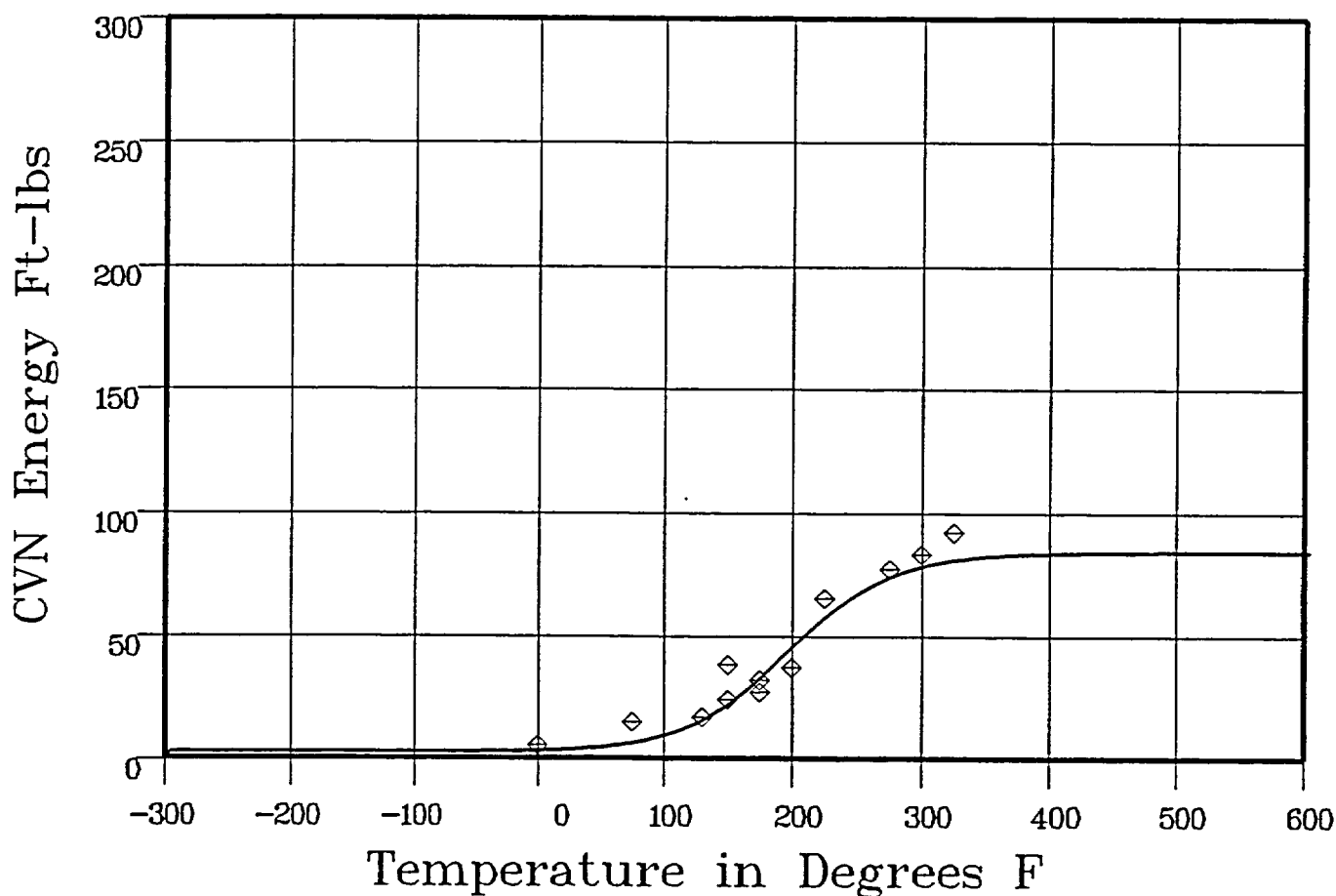
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: W-83

Total Fluence:



Plant: ML2 Cap: W-83 Material: PLATE SA533B1 Ori: TL Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
0	5	2.94	2.05
75	15	6.66	8.33
130	17	17.08	-0.08
150	24	24	0
150	38	24	13.99
175	27	35.05	-8.05
175	32	35.05	-3.05
200	37	47.47	-10.47

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 83 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: W-83

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
225	65	59.12	5.87
275	77	74.7	2.29
300	83	78.69	4.3
325	92	81.03	10.96
		SUM of RESIDUALS =	26.14

LOWER SHELL PLATE C-506-1 UNIRR (TRANS)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 120401 on 10-14-2002

Page 1

Coefficients of Curve 1

A = 44.2	B = 43.2	C = 102.74	T0 = 46.87
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$$\text{Equation is: } LE = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf LE: 87.4

Temperature at LE 35: 24.6

Lower Shelf LE: 1 Fixed

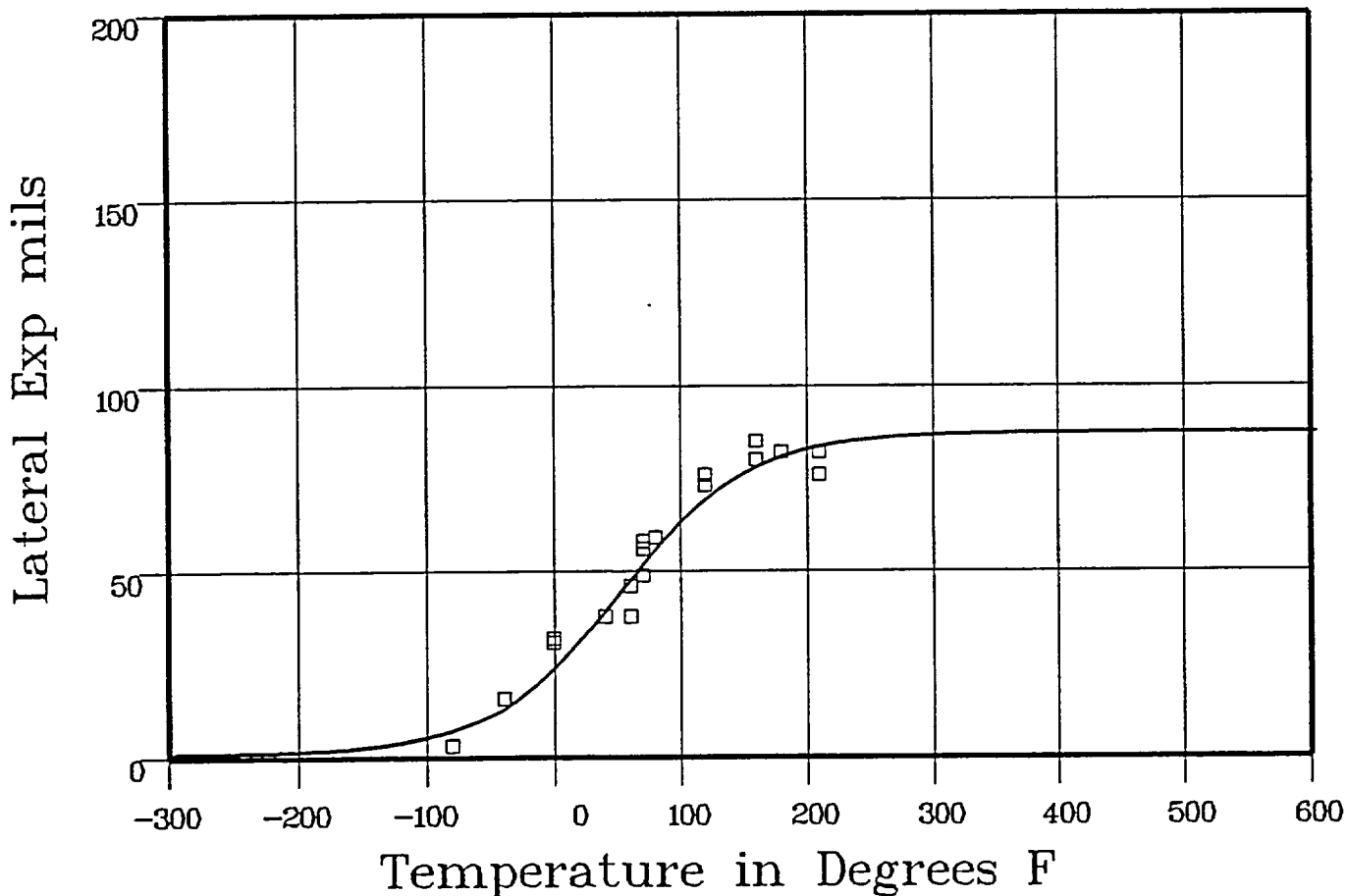
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: UNIRR

Total Fluence:



Plant: ML2 Cap: UNIRR Data Set(s) Plotted Material: PLATE SA533B1 Ori: TL Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
-80	3	7.74	-4.74
-40	16	14.44	155
0	31	25.75	524
0	32	25.75	624
40	38	41.31	-3.31
60	46	49.69	-3.69
60	38	49.69	-11.69
70	56	53.76	2.23
70	58	53.76	4.23

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 UNIRR (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: UNIRR

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed L.E.	Differential
70	49	53.76	-4.76
80	59	57.66	1.33
120	73	70.63	2.36
120	76	70.63	5.36
160	85	78.8	6.19
160	80	78.8	1.19
180	82	81.38	.61
210	82	83.93	-1.93
210	76	83.93	-7.93
			SUM of RESIDUALS = -1.48

LOWER SHELL PLATE C-506-1 CAPSULE 97 (TRANS)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 12:04:01 on 10-14-2002

Page 1

Coefficients of Curve 2

A = 39.62	B = 33.62	C = 110.69	T0 = 135.93
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$$\text{Equation is: } LE = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf LE: 78.24

Temperature at LE 35: 122.6

Lower Shelf LE: 1 Fixed

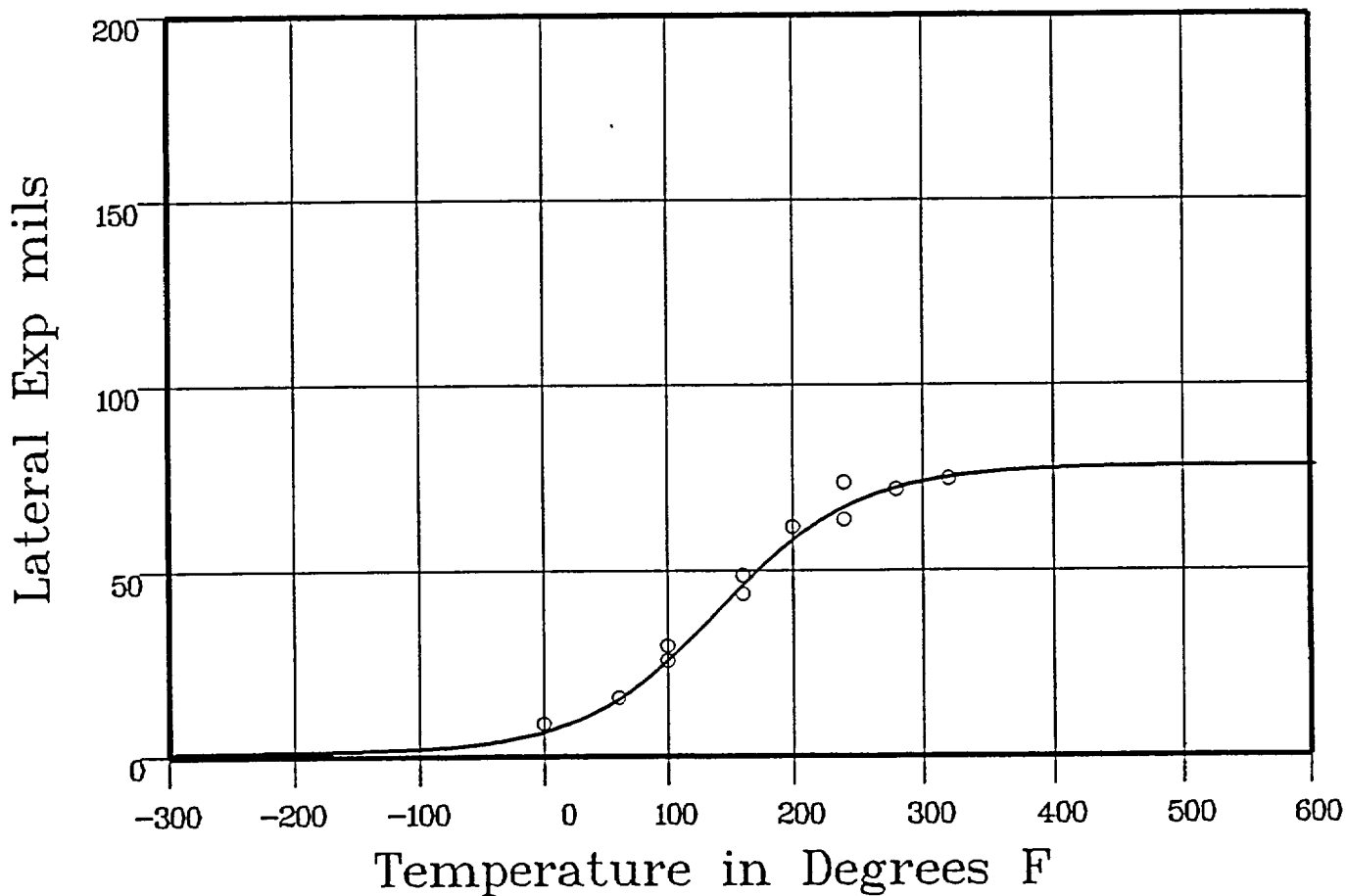
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: W-97

Total Fluence:



Plant: ML2 Cap: W-97 Data Set(s) Plotted Material: PLATE SA533B1 Ori: TL Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
0	9	7.1	1.89
60	16	16.62	-6.2
100	26	27.5	-1.5
100	30	27.5	2.49
160	44	47.88	-3.88
160	49	47.88	1.11
200	62	59.77	2.22
240	74	68.01	5.98

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 97 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: W-97

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed LE	Differential
240	64	68.01	-4.01
280	72	72.91	-9.1
280	72	72.91	-9.1
320	75	75.55	-5.5
			SUM of RESIDUALS = 13

LOWER SHELL PLATE C-506-1 CAPSULE 83 (TRANS)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 120401 on 10-14-2002

Page 1

Coefficients of Curve 3

A = 42.61	B = 41.61	C = 120.45	T0 = 219.14
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$$\text{Equation is: } LE = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf LE: 84.22

Temperature at LE 35: 196.8

Lower Shelf LE: 1 Fixed

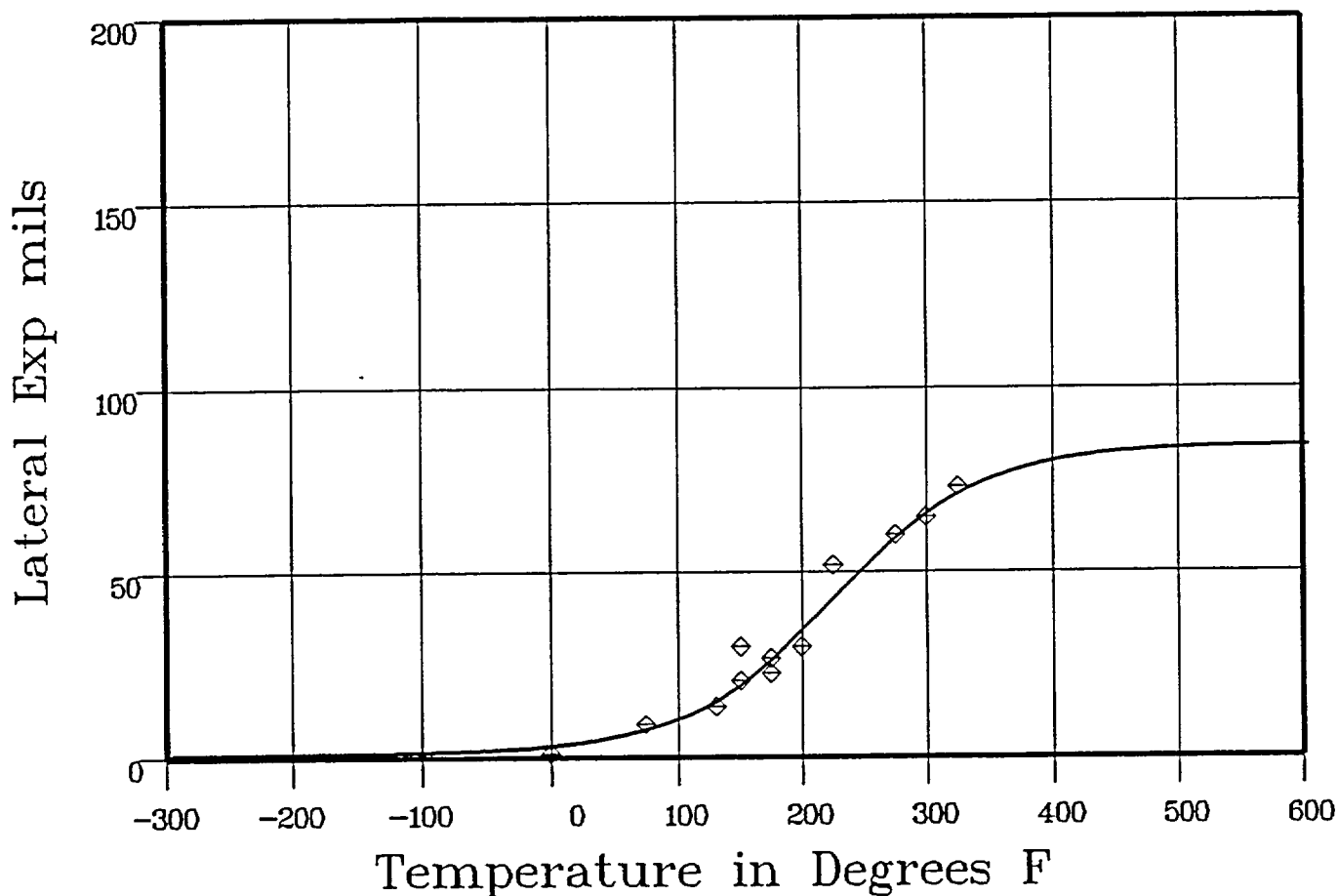
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: W-83

Total Fluence:



Plant: ML2 Cap: W-83 Material: PLATE SA533B1 Ori: TL Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
0	1	3.13	-2.13
75	9	7.96	1.03
130	14	16.43	-2.43
150	21	21.04	-0.04
150	30	21.04	8.95
175	23	28.01	-5.01
175	27	28.01	-1.01
200	30	36.05	-6.05

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 83 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: W-83

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed L.E.	Differential
225	52	44.63	7.36
275	60	60.63	-.63
300	65	66.99	-1.99
325	73	71.98	1.01
		SUM of RESIDUALS =	-.96

LOWER SHELL PLATE C-506-1 UNIRR (TRANS)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 12:13:13 on 10-14-2002

Page 1

Coefficients of Curve 1

A = 50

B = 50

C = 76.64

T0 = 59.53

Equation is: $\text{Shear}\% = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 59.5

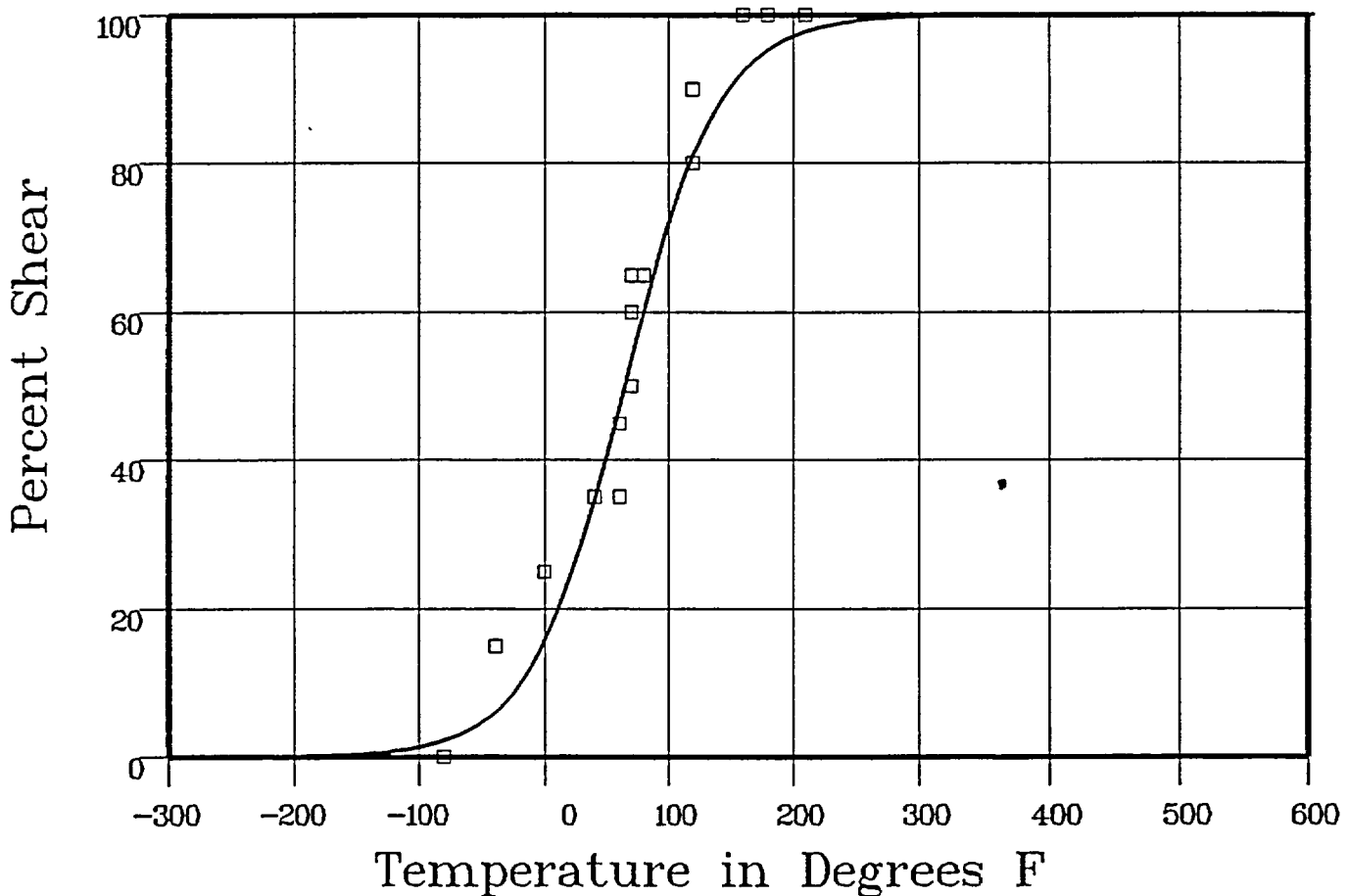
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: UNIRR

Total Fluence:



Plant: ML2 Cap: UNIRR Data Set(s) Plotted Material: PLATE SA533B1 Ori: TL Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
-80	0	2.55	-2.55
-40	15	6.93	8.06
0	25	17.45	7.54
0	25	17.45	7.54
40	35	37.52	-2.52
60	45	50.3	-5.3
60	35	50.3	-15.3
70	60	56.78	3.21
70	65	56.78	8.21

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 UNIRR (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: UNIRR

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
70	50	56.78	-6.78
80	65	63.04	1.95
120	80	82.89	-2.89
120	90	82.89	7.1
160	100	93.22	6.77
160	100	93.22	6.77
180	100	95.86	4.13
210	100	98.06	1.93
210	100	98.06	1.93
			SUM of RESIDUALS = 29.81

LOWER SHELL PLATE C-506-1 CAPSULE 97 (TRANS)

CVGRAPH 41 Hyperbolic Tangent Curve Printed at 12:13:13 on 10-14-2002

Page 1

Coefficients of Curve 2

A = 50	B = 50	C = 65.94	T0 = 165.52
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$$\text{Equation is: Shear\%} = A + B * [\tanh((T - T0)/C)]$$

Temperature at 50% Shear: 165.5

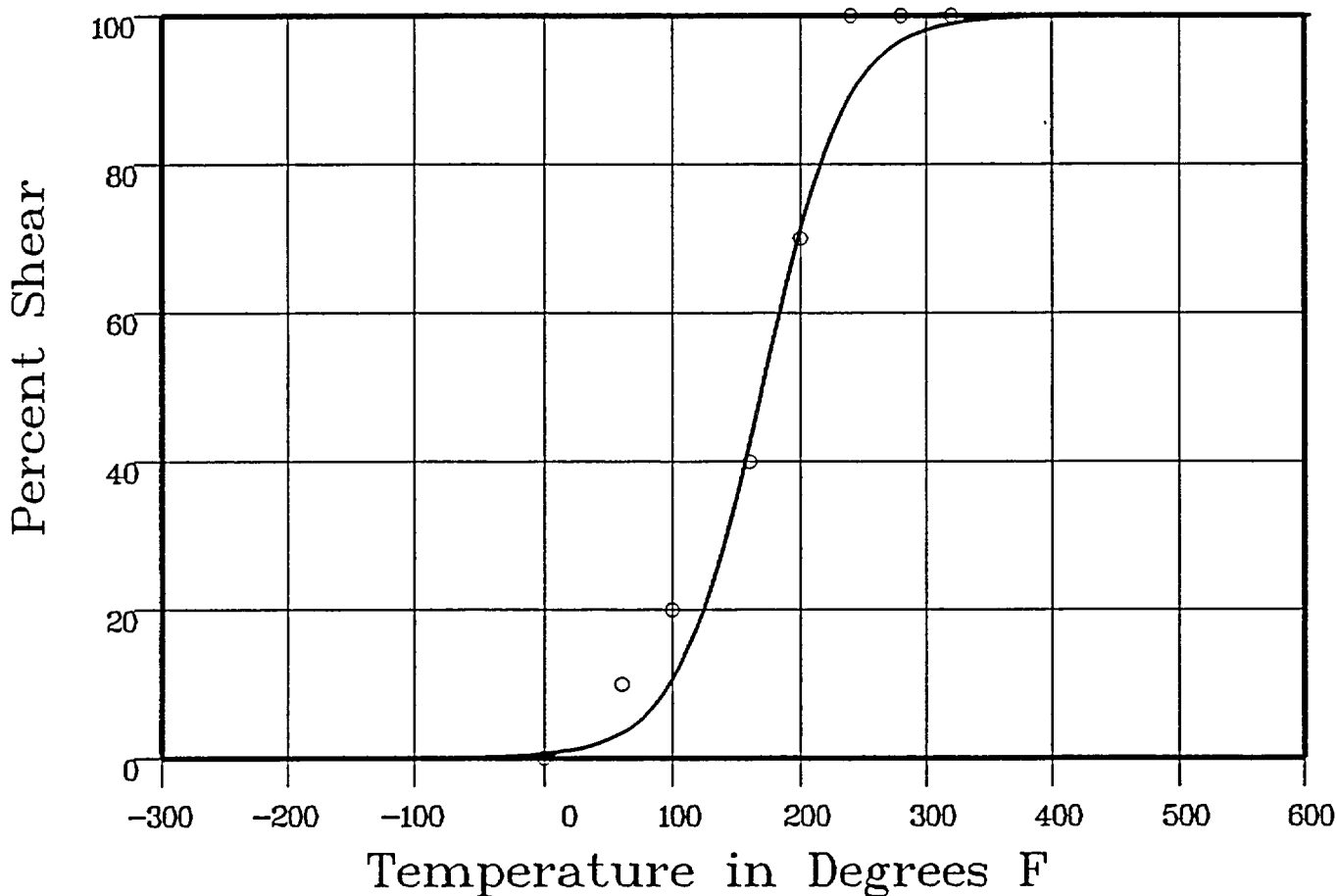
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: W-97

Total Fluence:



Plant: ML2 Cap: W-97 Material: PLATE SA533B1 Ori: TL Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
0	0	.65	-.65
60	10	3.91	6.08
100	20	12.05	7.94
100	20	12.05	7.94
160	40	45.81	-5.81
160	40	45.81	-5.81
200	70	73.99	-3.99
240	100	90.54	9.45

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 97 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: W-97

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
240	100	90.54	9.45
280	100	96.98	3.01
280	100	96.98	3.01
320	100	99.08	.91

SUM of RESIDUALS = 31.55

LOWER SHELL PLATE C-506-1 CAPSULE 97 (TRANS)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 12:13:13 on 10-14-2002

Page 1

Coefficients of Curve 3

A = 50	B = 50	C = 812	T0 = 191.71
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Equation is: $\text{Shear}\% = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 191.7

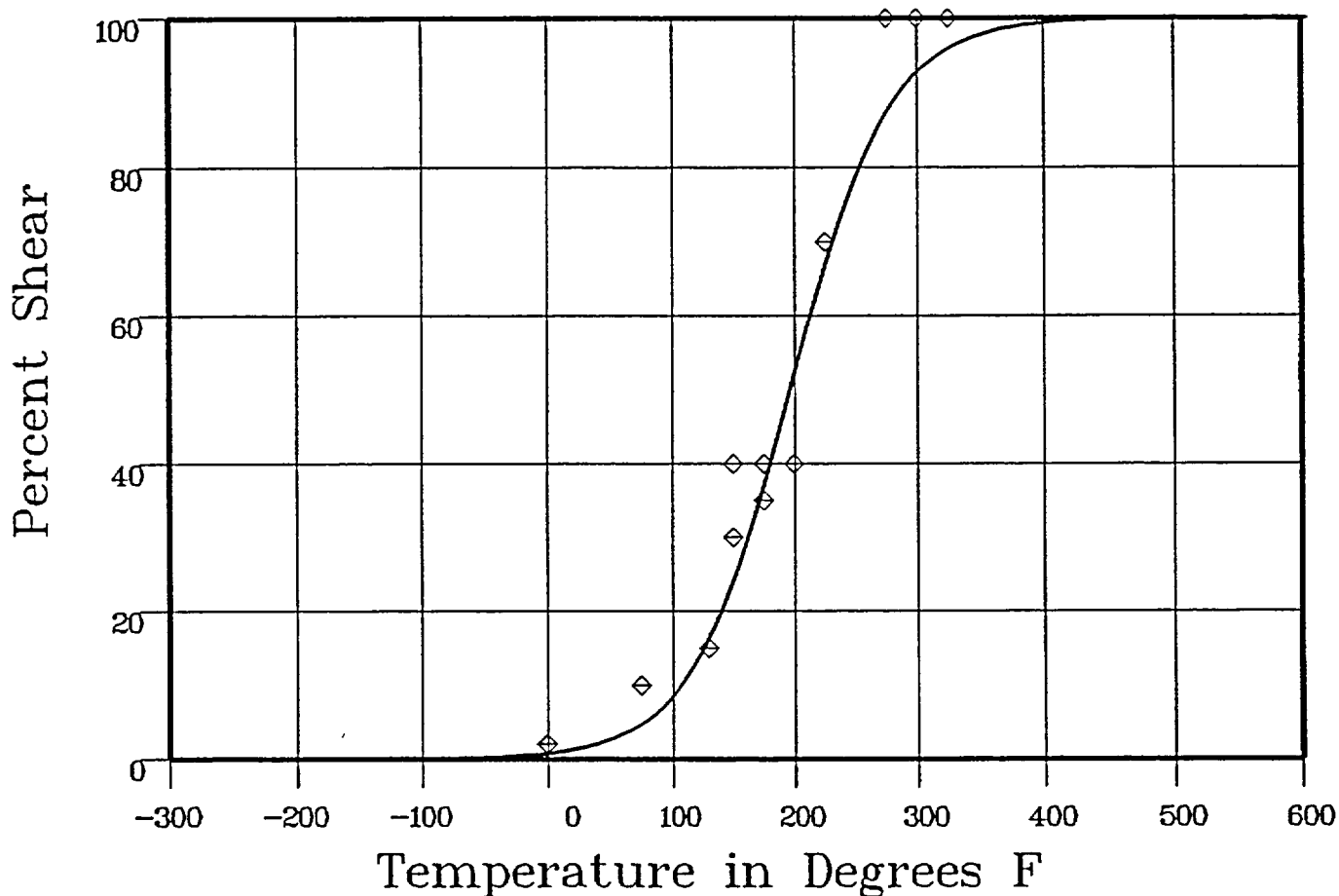
Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: W-83

Total Fluence:



Data Set(s) Plotted
 Plant: ML2 Cap: W-83 Material: PLATE SA533B1 Ori: TL Heat #: C-5667-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
0	2	.88	111
75	10	5.34	4.65
130	15	17.94	-2.94
150	30	26.35	3.64
150	40	26.35	13.64
175	35	39.84	-4.84
175	40	39.84	.15
200	40	55.08	-15.08

**** Data continued on next page ****

LOWER SHELL PLATE C-506-1 CAPSULE 97 (TRANS)

Page 2

Material: PLATE SA533B1

Heat Number: C-5667-1

Orientation: TL

Capsule: W-83

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
225	70	69.41	58
275	100	88.6	11.39
300	100	93.5	6.49
325	100	96.38	3.61
			SUM of RESIDUALS = 22.42

UNIRRADIATED (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 11:50:19 on 10-14-2002

Page 1

Coefficients of Curve 1

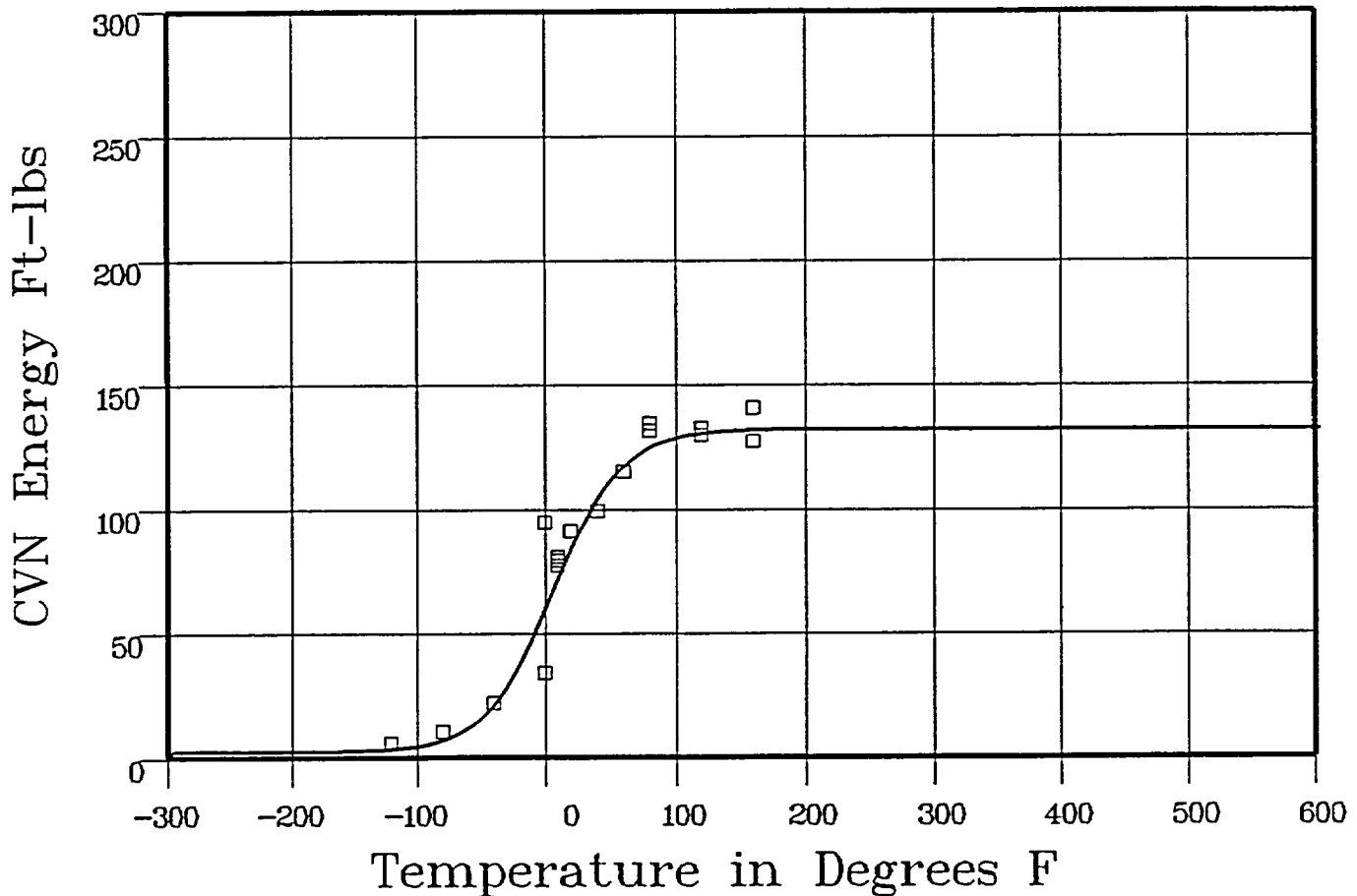
A = 67.09	B = 64.9	C = 52.34	T0 = 1.87
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$$\text{Equation is: } \text{CVN} = A + B * [\tanh((T - T_0)/C)]$$

Upper Shelf Energy: 132 Fixed Temp. at 30 ft-lbs: -32.1 Temp. at 50 ft-lbs: -12.2 Lower Shelf Energy: 219 Fixed

Material: WELD L 124/0091 Heat Number: 90136/10137 Orientation:

Capsule: UNIRR Total Fluence:



Plant: ML2 Cap: UNIRR Data Set(s) Plotted Material: WELD L 124/0091 Ori: Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
-120	55	3.42	2.07
-80	10	7.64	2.35
-40	22	24	-2
0	34	64.77	-30.77
0	94.5	64.77	29.72
10	77	77.09	-0.09
10	79	77.09	1.9
10	80.5	77.09	3.4
20	91	88.71	2.28

**** Data continued on next page ****

UNIRRADIATED (WELD)

Page 2

Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: UNIRR

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
40	99	107.46	-8.46
60	115	119.29	-4.29
80	134.5	125.75	8.74
80	131.5	125.75	5.74
120	132.5	130.59	1.9
120	129.5	130.59	-1.09
160	140.5	131.69	8.8
160	127	131.69	-4.69
			SUM of RESIDUALS = 15.54

CAPSULE 97 (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:11:31 on 11-21-2002

Page 1

Coefficients of Curve 2

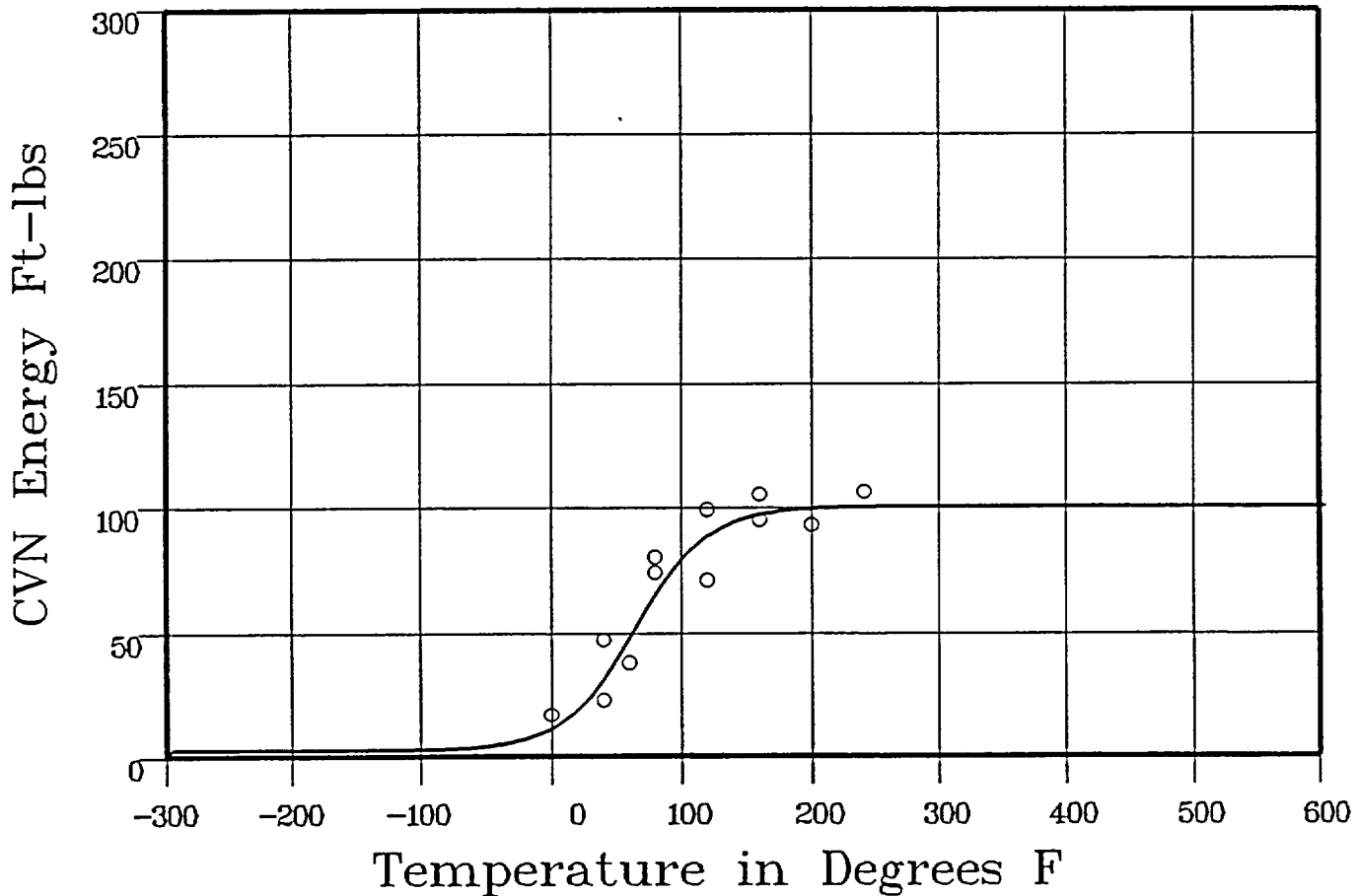
A = 5109	B = 48.9	C = 55.76	T0 = 59.53
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Equation is: $CVN = A + B * [\tanh((T - T_0)/C)]$

Upper Shelf Energy: 100 Fixed Temp. at 30 ft-lbs: 33.7 Temp. at 50 ft-lbs: 58.2 Lower Shelf Energy: 2.19 Fixed

Material: WELD L 124/0091 Heat Number: 90136/10137 Orientation:

Capsule: W-97 Total Fluence:



Data Set(s) Plotted
Plant: ML2 Cap: W-97 Material: WELD L 124/0091 Ori: Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
0	17	1254	4.45
40	23	34.64	-11.64
40	47	34.64	12.35
60	38	51.51	-13.51
80	74	68.28	5.71
80	80	68.28	11.71
120	99	89.96	9.03
120	71	89.96	-18.96

**** Data continued on next page ****

CAPSULE 97 (WELD)

Page 2

Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-97

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
160	95	97.4	-2.4
160	105	97.4	7.59
200	93	99.36	-6.36
240	106	99.84	6.15
		SUM of RESIDUALS =	4.13

CAPSULE 104 (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 11:50:19 on 10-14-2002

Page 1

Coefficients of Curve 3

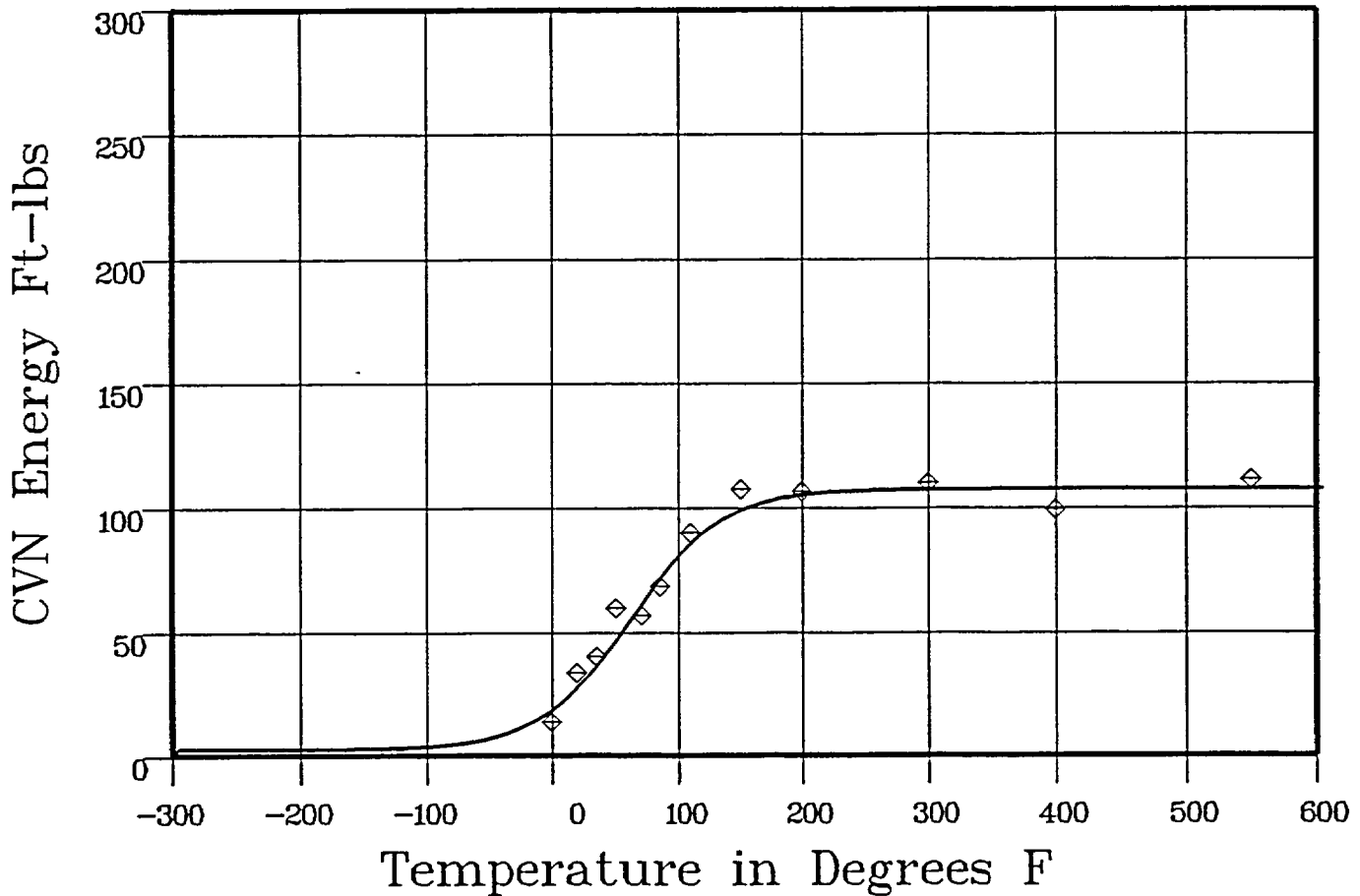
A = 54.59	B = 52.4	C = 72.12	T0 = 56.71
-----------	----------	-----------	------------

Equation is: $CVN = A + B * [\tanh((T - T0)/C)]$

Upper Shelf Energy: 107 Fixed Temp. at 30 ft-lbs: 199 Temp. at 50 ft-lbs: 503 Lower Shelf Energy: 219 Fixed

Material: WELD L 124/0091 Heat Number: 90136/10137 Orientation:

Capsule: W-104 Total Fluence:



Data Set(s) Plotted
Plant: ML2 Cap: W-104 Material: WELD L 124/0091 Ori: Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
0	135	202	-6.7
20	33.5	30.01	3.48
35	40	39.28	.71
50	59.5	49.73	9.76
70	56.5	64.14	-7.64
85	68	74.15	-6.15
110	89.5	87.52	1.97
150	107	99.66	7.33

**** Data continued on next page ****

CAPSULE 104 (WELD)

Page 2

Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-104

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
200	106	105.06	.93
300	109.5	106.87	2.62
400	99	106.99	-7.99
550	111	106.99	4

SUM of RESIDUALS = 2.34

CAPSULE 83 (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 1150:19 on 10-14-2002

Page 1

Coefficients of Curve 4

A = 55.59	B = 53.4	C = 97.76	T0 = 75
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$$\text{Equation is: } \text{CVN} = A + B * [\tanh((T - T_0)/C)]$$

Upper Shelf Energy: 109 Fixed Temp. at 30 ft-lbs: 23.9 Temp. at 50 ft-lbs: 64.7 Lower Shelf Energy: 2.19 Fixed

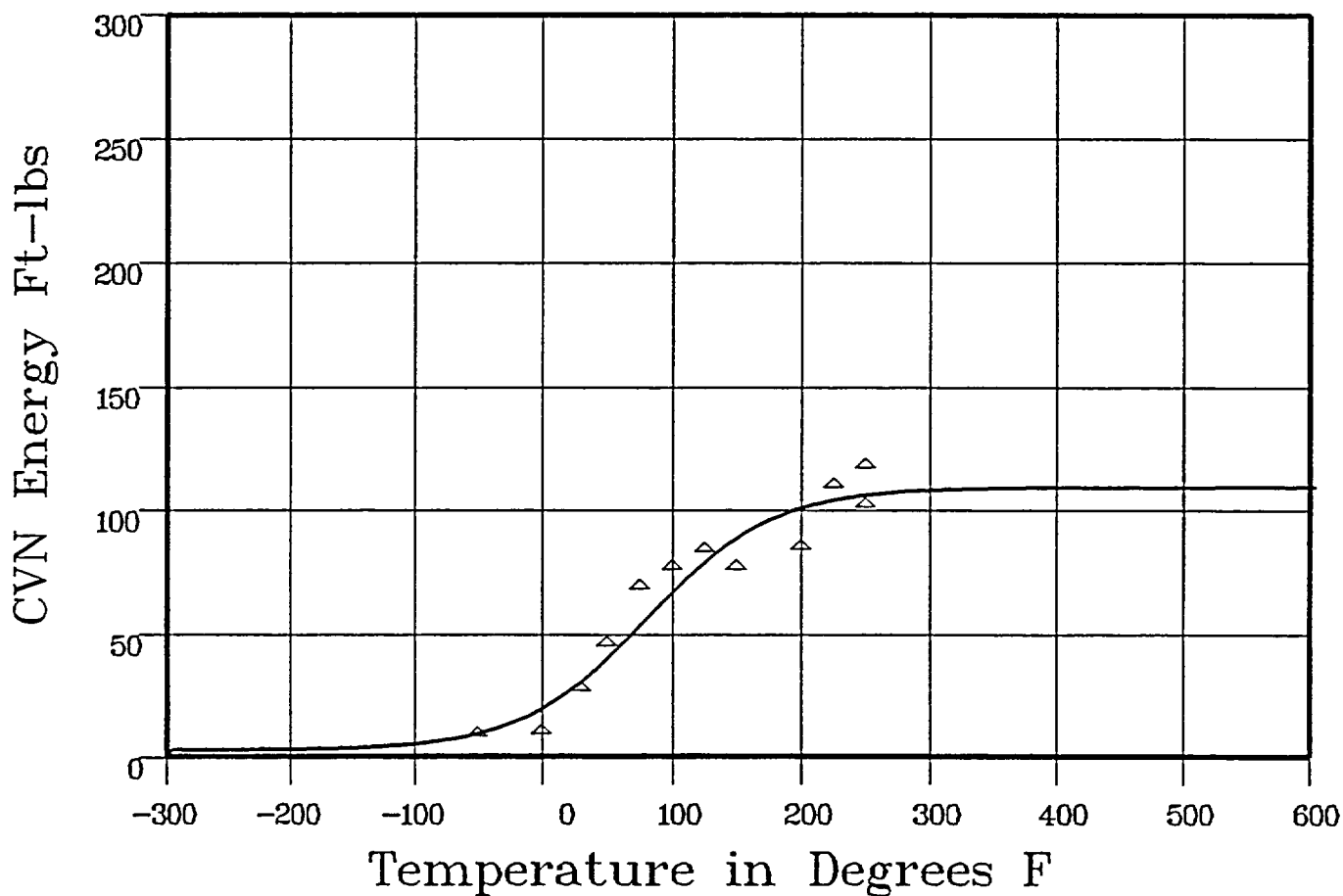
Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-83

Total Fluence:



Plant: ML2 Cap: W-83 Data Set(s) Plotted Material: WELD L 124/0091 Ori: Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
-50	8	9.88	-1.88
0	9	21.14	-12.14
30	27	32.62	-5.62
50	45	42.23	2.76
75	68	55.59	12.4
100	76	68.96	7.03
125	83	80.75	2.24

**** Data continued on next page ****

CAPSULE 83 (WELD)

Page 2

Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-83

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
150	76	90.05	-14.05
200	84	101.31	-17.31
225	109	104.25	4.74
250	101	106.1	-5.1
250	117	106.1	10.89
			SUM of RESIDUALS = -16.03

UNIRRADIATED (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 12:20:58 on 10-14-2002

Page 1

Coefficients of Curve 1

A = 47.66

B = 46.66

C = 53.35

T0 = -9.66

$$\text{Equation is: } LE = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf LE: 94.33

Temperature at LE 35: -24.5

Lower Shelf LE: 1 Fixed

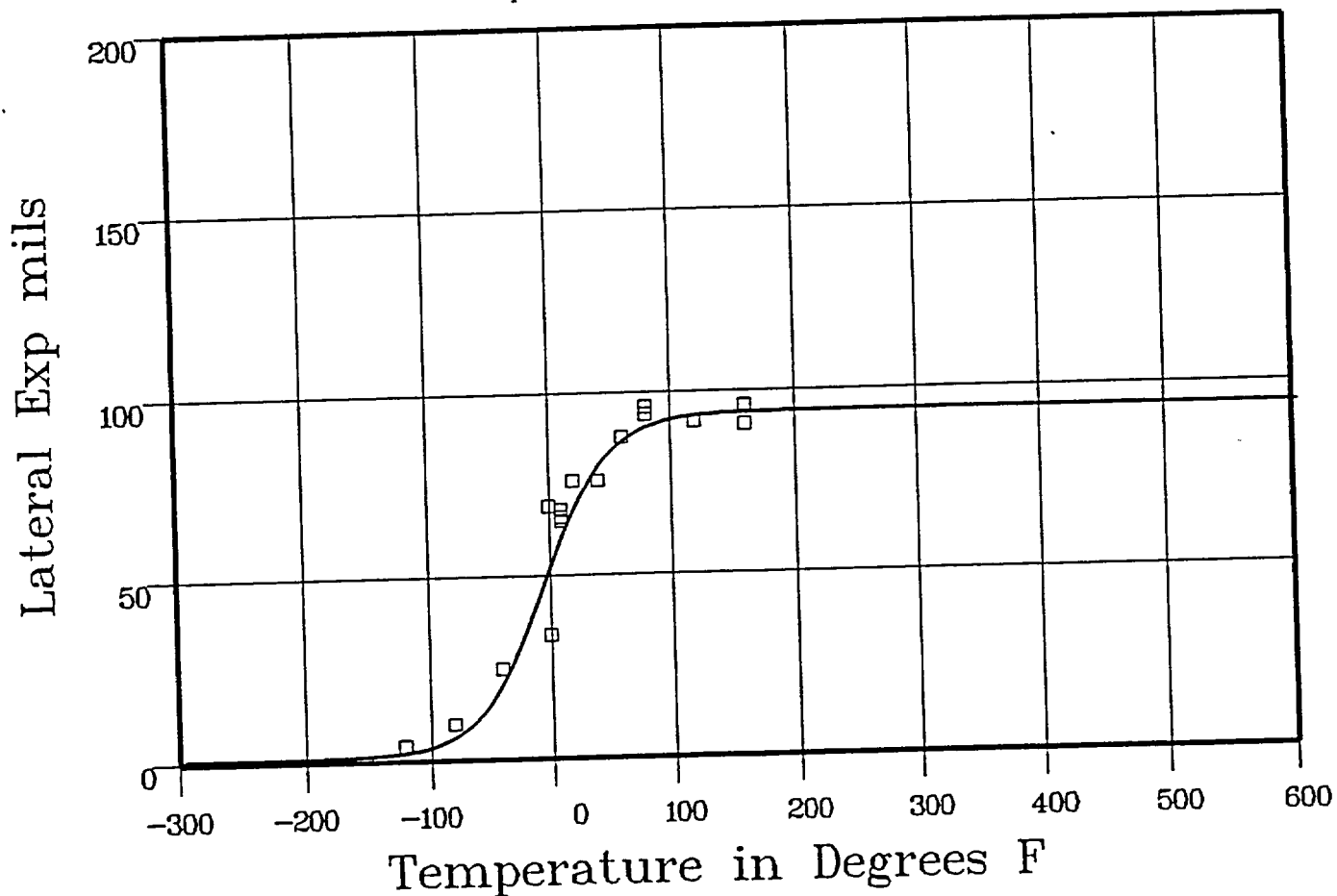
Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: UNIRR

Total Fluence:



Plant: ML2 Cap: UNIRR Data Set(s) Plotted Material: WELD L 124/0091 Ori: Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
-120	4	2.46	153
-80	10	7.23	2.76
-40	25	23.66	133
0	34	56.03	-22.03
0	69	56.03	12.96
10	68	64.13	3.86
10	66	64.13	1.86
10	65	64.13	.86
20	76	71.23	4.76

**** Data continued on next page ****

CAPSULE 97 (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 12:20:58 on 10-14-2002

Page 1

Coefficients of Curve 2

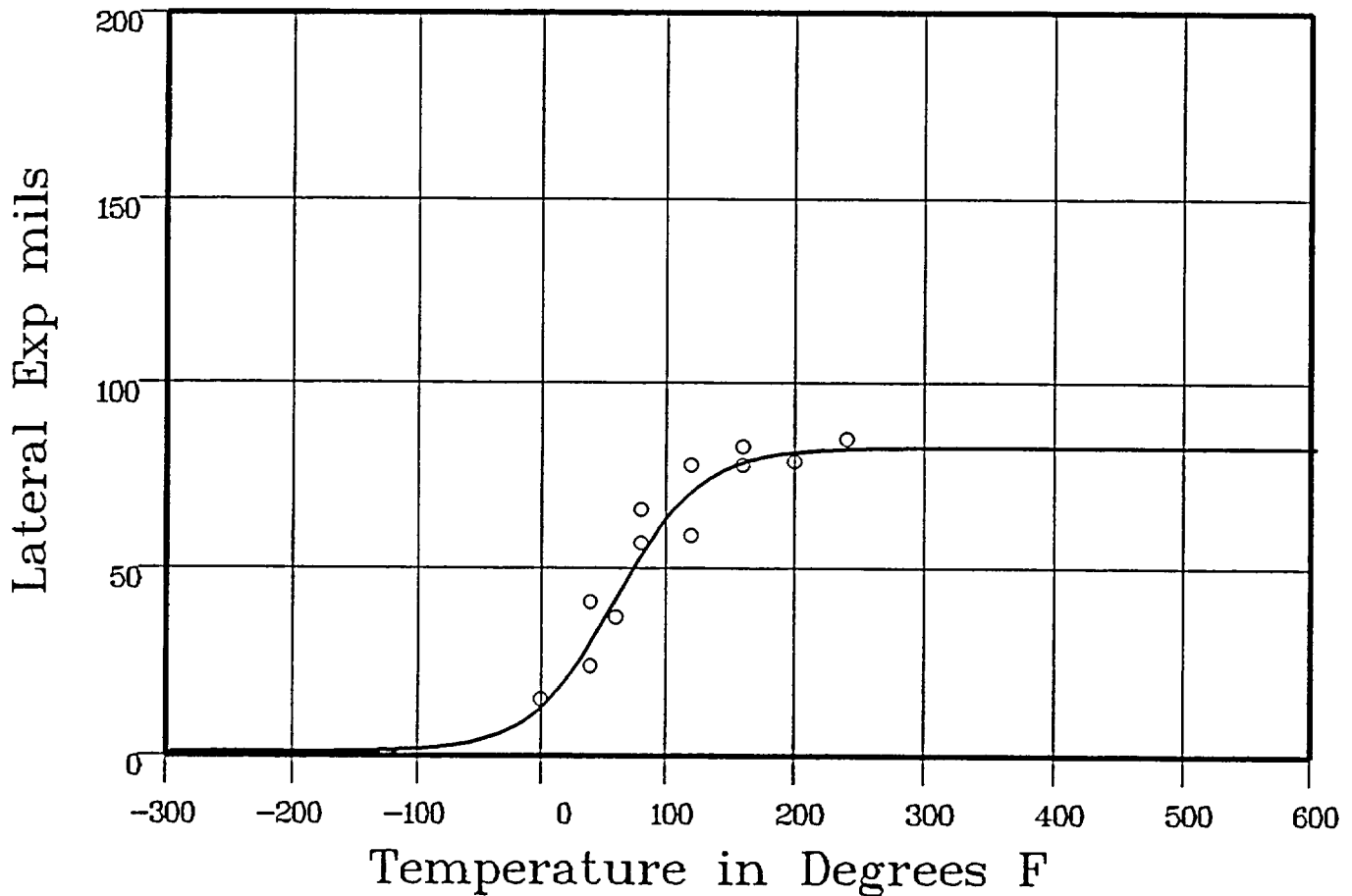
A = 41.86	B = 40.86	C = 67.98	T0 = 55.78
-----------	-----------	-----------	------------

Equation is: $LE = A + B * [\tanh((T - T0)/C)]$

Upper Shelf LE: 82.72 Temperature at LE 35: 44.2 Lower Shelf LE: 1 Fixed

Material: WELD L 124/0091 Heat Number: 90136/10137 Orientation:

Capsule: W-97 Total Fluence:



Data Set(s) Plotted
 Plant: ML2 Cap: W-97 Material: WELD L 124/0091 Ori: Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
0	15	14.26	.73
40	24	32.54	-8.54
40	41	32.54	8.45
60	37	44.39	-7.39
80	57	55.83	1.16
80	66	55.83	10.16
120	78	71.99	6
120	59	71.99	-12.99

**** Data continued on next page ****

CAPSULE 97 (WELD)

Page 2

Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-97

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed L.E.	Differential
160	78	79.08	-1.08
160	83	79.08	3.91
200	79	81.56	-2.56
240	85	82.36	2.63
			SUM of RESIDUALS = .48

CAPSULE 104 (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 12:20:58 on 10-14-2002

Page 1

Coefficients of Curve 3

A = 45.77	B = 44.77	C = 81.91	T0 = 55.31
-----------	-----------	-----------	------------

$$\text{Equation is: } LE = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf LE: 90.54

Temperature at LE 35: 35.2

Lower Shelf LE: 1 Fixed

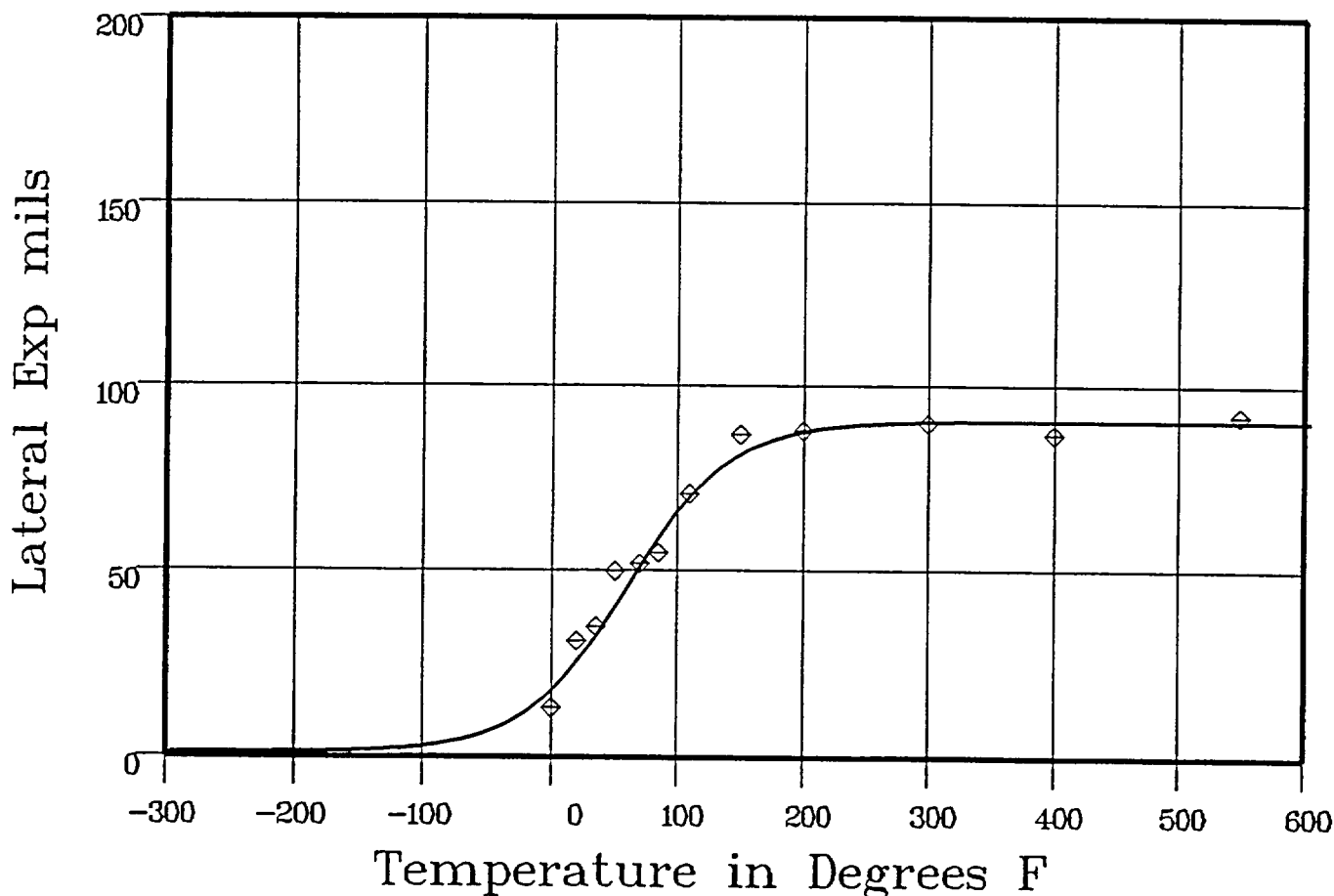
Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-104

Total Fluence:



Data Set(s) Plotted

Plant: ML2 Cap: W-104 Material: WELD L 124/0091 Ori: Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
0	13	19.42	-6.42
20	31	27.58	3.41
35	35	34.89	1
50	50	42.87	7.12
70	52	53.71	-1.71
85	55	61.32	-6.32
110	71	71.89	-8.9
150	87	82.47	4.52

**** Data continued on next page ****

CAPSULE 104 (WELD)

Page 2

Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-104

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed L.E.	Differential
200	88	88	0
300	90	90.31	-31
400	87	90.52	-352
550	92	90.54	145
			SUM of RESIDUALS = -257

CAPSULE 83 (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 122058 on 10-14-2002

Page 1

Coefficients of Curve 4

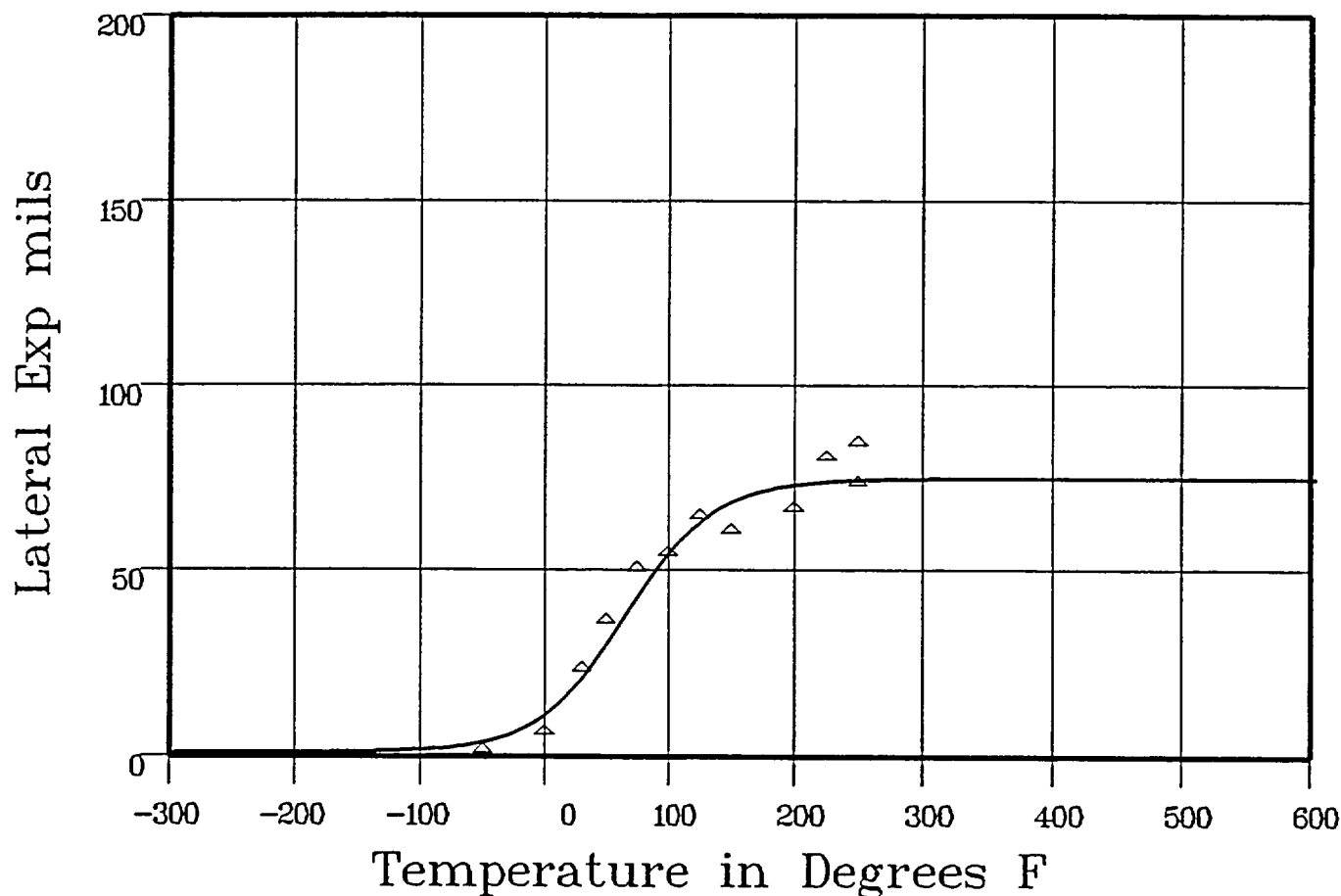
A = 37.89	B = 36.89	C = 71.98	T0 = 60.93
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Equation is: $LE = A + B * | \tanh((T - T0)/C) |$

Upper Shelf LE: 74.78 Temperature at LE 35: 55.2 Lower Shelf LE: 1 Fixed

Material: WELD L 124/0091 Heat Number: 90136/10137 Orientation:

Capsule: W-83 Total Fluence:



Data Set(s) Plotted
 Plant: ML2 Cap: W-83 Material: WELD L 124/0091 Ori: Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
-50	1	4.23	-3.23
0	6	12.46	-6.46
30	23	22.94	.05
50	36	32.32	3.67
75	50	45	4.99
100	54	56.15	-2.15
125	64	64.13	-13

*** Data continued on next page ***

CAPSULE 83 (WELD)

Page 2

Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-83

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed LE	Differential
150	60	69.05	-9.05
200	66	73.26	-7.26
225	80	74.01	5.98
250	73	74.4	-14
250	84	74.4	9.59

SUM of RESIDUALS = -5.41

UNIRRADIATED (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 12:28:54 on 10-14-2002

Page 1

Coefficients of Curve 1

A = 50	B = 50	C = 61.03	T0 = -12.18
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$$\text{Equation is: Shear\%} = A + B * [\tanh((T - T0)/C)]$$

Temperature at 50% Shear: -12.1

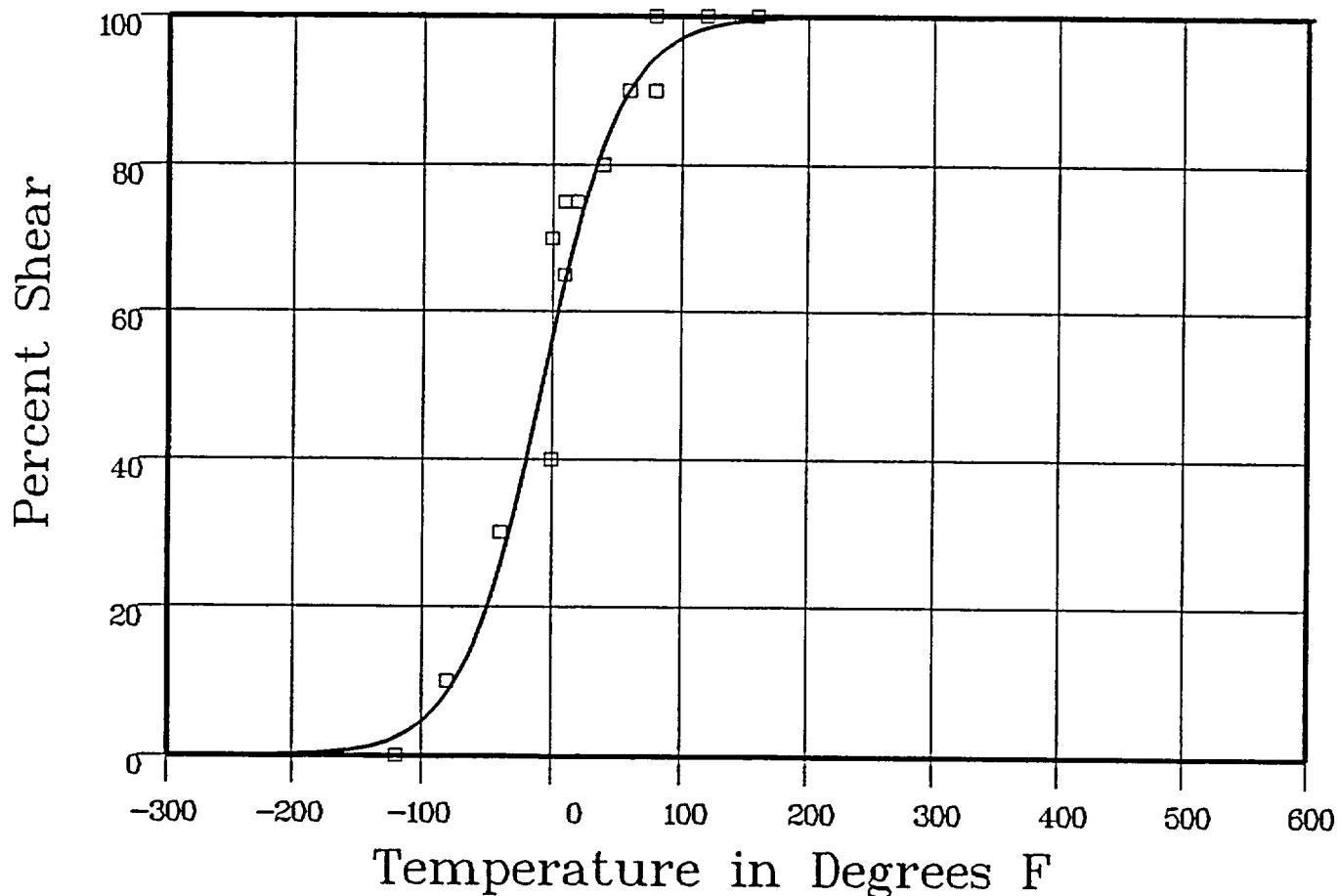
Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: UNIRR

Total Fluence:



Plant: ML2 Cap: UNIRR Material: WELD L 124/0091 Ori: Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
-120	0	2.83	-2.83
-80	10	9.77	22
-40	30	28.67	132
0	40	59.85	-19.85
0	70	59.85	10.14
10	65	67.41	-2.41
10	75	67.41	7.58
10	75	67.41	7.58
20	75	74.16	83

**** Data continued on next page ****

UNIRRADIATED (WELD)

Page 2

Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: UNIRR

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
40	80	84.68	-4.68
60	90	91.41	-1.41
80	90	95.34	-5.34
80	100	95.34	4.65
120	100	98.7	129
120	100	98.7	129
160	100	99.64	.35
160	100	99.64	.35

SUM of RESIDUALS = -9

CAPSULE 97 (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 122854 on 10-14-2002

Page 1

Coefficients of Curve 2

A = 50	B = 50	C = 4887	T0 = 67.7
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Equation is: $\text{Shear\%} = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 67.7

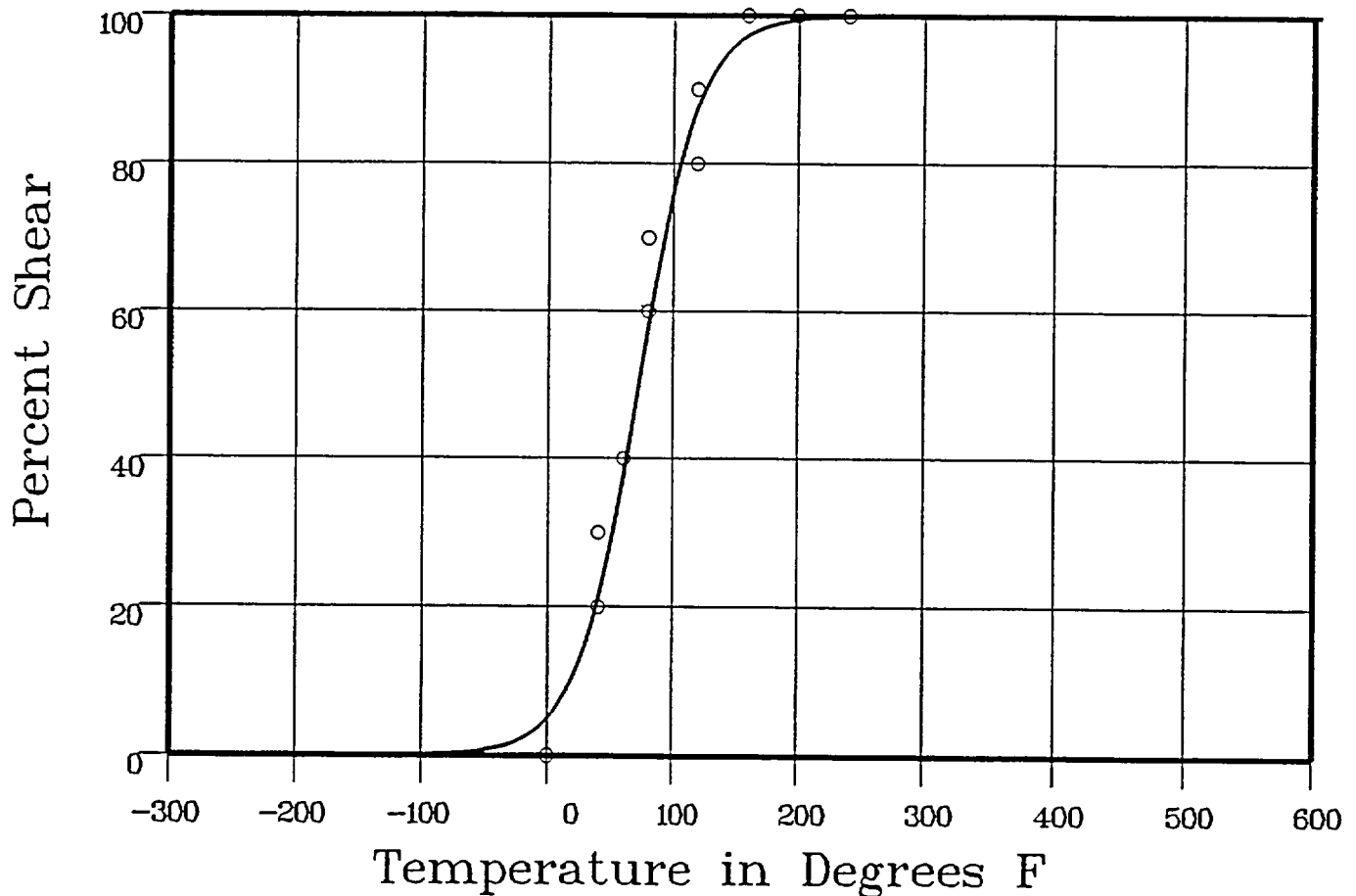
Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-97

Total Fluence:



Data Set(s) Plotted
 Plant: ML2 Cap: W-97 Material: WELD L 124/0091 Ori: Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
0	0	5.89	-5.89
40	20	24.34	-4.34
40	30	24.34	5.65
60	40	42.18	-2.18
80	60	62.32	-2.32
80	70	62.32	7.67
120	90	89.47	52
120	80	89.47	-9.47

**** Data continued on next page ****

CAPSULE 97 (WELD)

Page 2

Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-97

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
160	100	97.76	2.23
160	100	97.76	2.23
200	100	99.55	.44
240	100	99.91	.08
			SUM of RESIDUALS = -5.35

CAPSULE 104 (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 12:28:54 on 10-14-2002

Page 1

Coefficients of Curve 3

A = 50	B = 50	C = 73.56	T0 = 49.21
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Equation is: $\text{Shear}\% = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 49.2

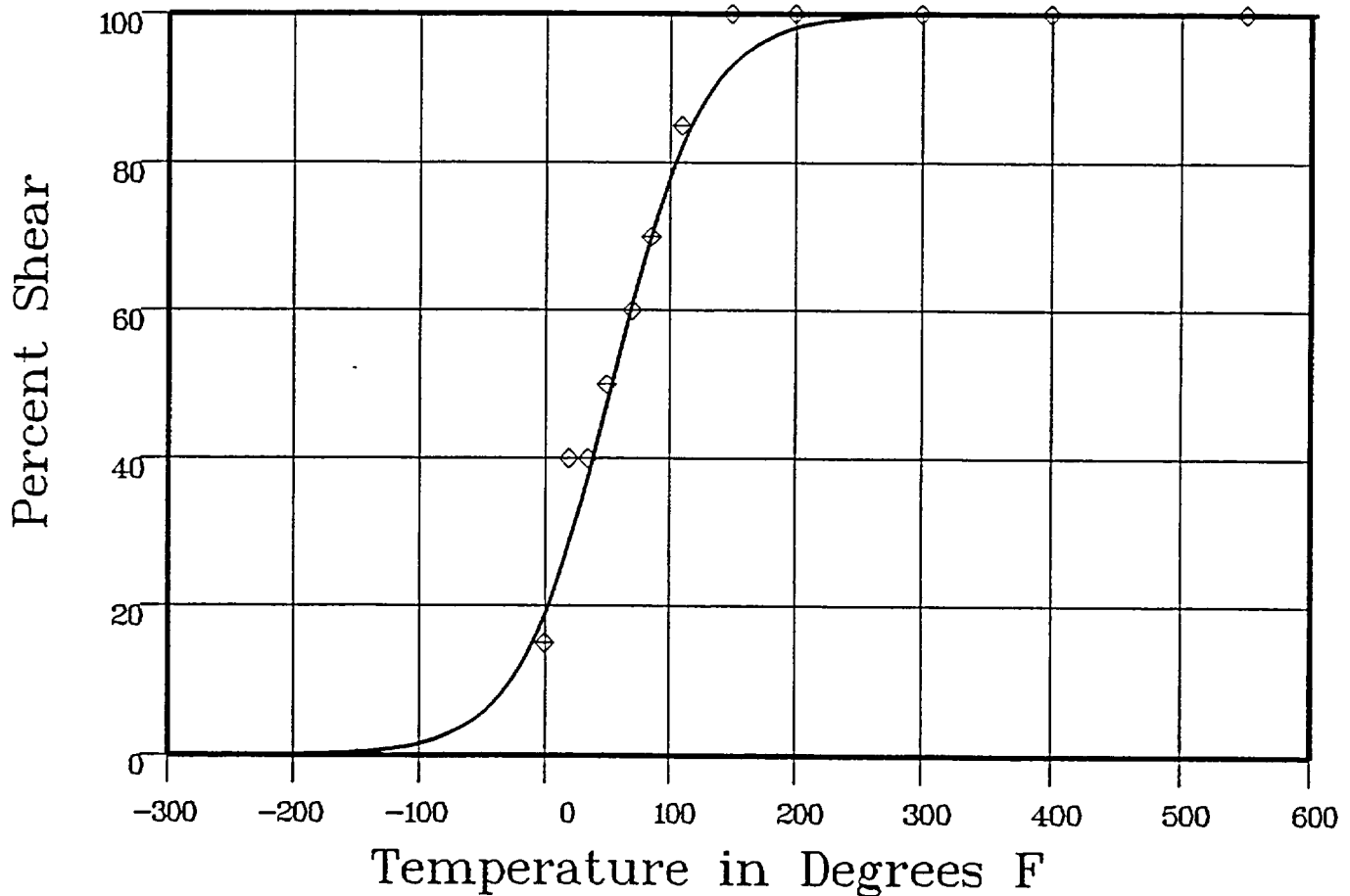
Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-104

Total Fluence:



Data Set(s) Plotted

Plant: ML2

Cap: W-104

Material: WELD L 124/0091

Ori:

Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
0	15	20.78	-5.78
20	40	31.12	8.87
35	40	40.45	-4.5
50	50	50.53	-5.3
70	60	63.76	-3.76
85	70	72.56	-2.56
110	85	83.92	1.07
150	100	93.93	6.06

**** Data continued on next page ****

CAPSULE 104 (WELD)

Page 2

Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-104

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
200	100	98.36	1.63
300	100	99.89	.1
400	100	99.99	0
550	100	99.99	0

SUM of RESIDUALS = 4.67

CAPSULE 83 (WELD)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 12:28:54 on 10-14-2002

Page 1

Coefficients of Curve 4

A = 50	B = 50	C = 70.42	T0 = 58.12
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Equation is: $\text{Shear}\% = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 58.1

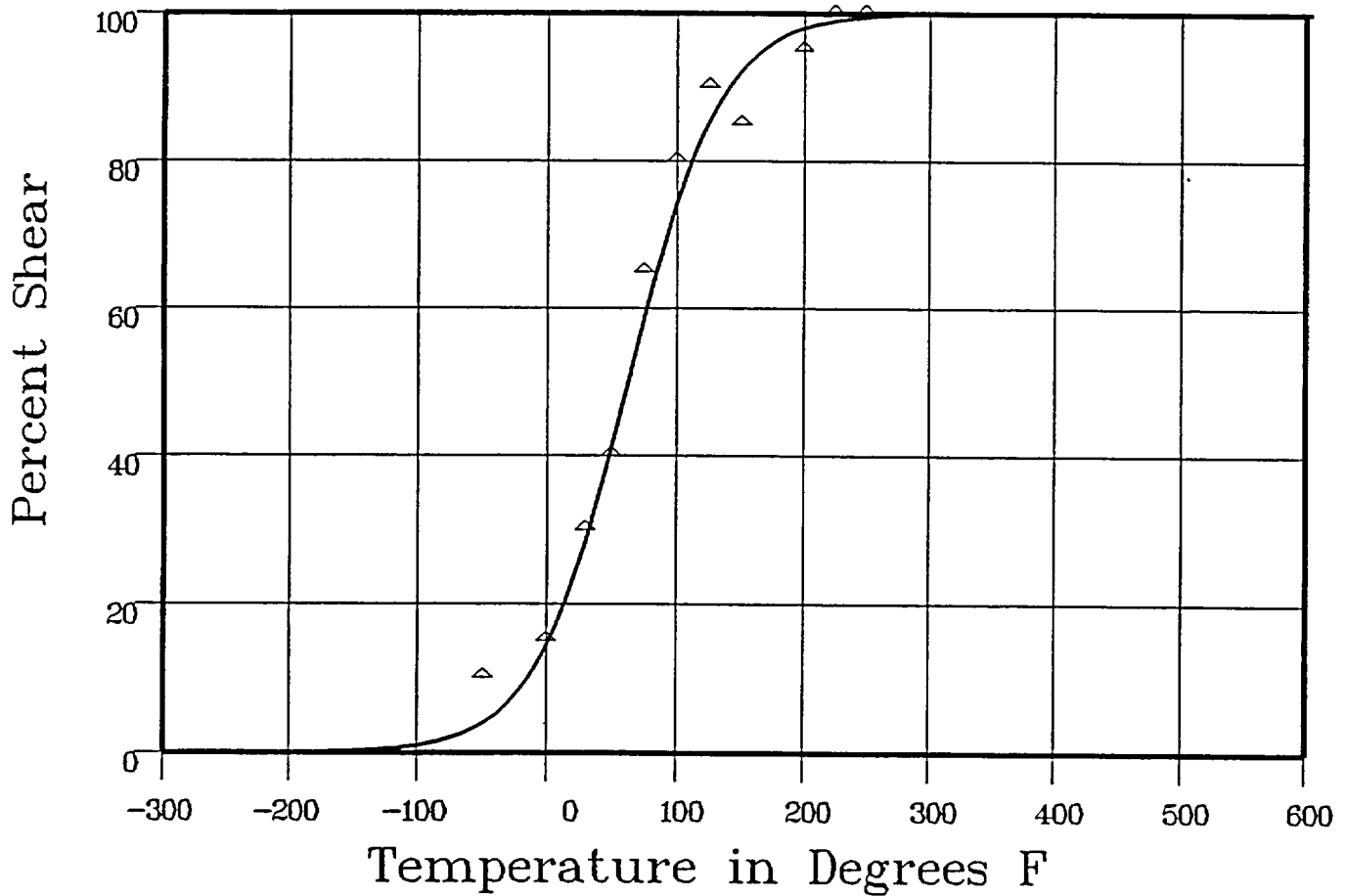
Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-83

Total Fluence:



Data Set(s) Plotted

Plant: ML2

Cap: W-83

Material: WELD L 124/0091

Ori:

Heat #: 90136/10137

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
-50	10	4.43	5.56
0	15	16.1	-1.1
30	30	31.02	-1.02
50	40	44.25	-4.25
75	65	61.75	3.24
100	80	76.66	3.33
125	90	86.98	3.01

**** Data continued on next page ****

CAPSULE 83 (WELD)

Page 2

Material: WELD L 124/0091

Heat Number: 90136/10137

Orientation:

Capsule: W-83

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
150	85	93.14	-8.14
200	95	98.25	-3.25
225	100	99.13	.86
250	100	99.57	.42
250	100	99.57	.42

SUM of RESIDUALS = -.89

UNIRRADIATED (HEAT AFFECTED ZONE)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 09:57:39 on 10-15-2002

Page 1

Coefficients of Curve 1

A = 65.59

B = 63.4

C = 52.83

T0 = 22.03

Equation is: $CVN = A + B * [\tanh((T - T0)/C)]$

Upper Shelf Energy: 129 Fixed Temp. at 30 ft-lbs: -115 Temp. at 50 ft-lbs: 8.7 Lower Shelf Energy: 2.19 Fixed

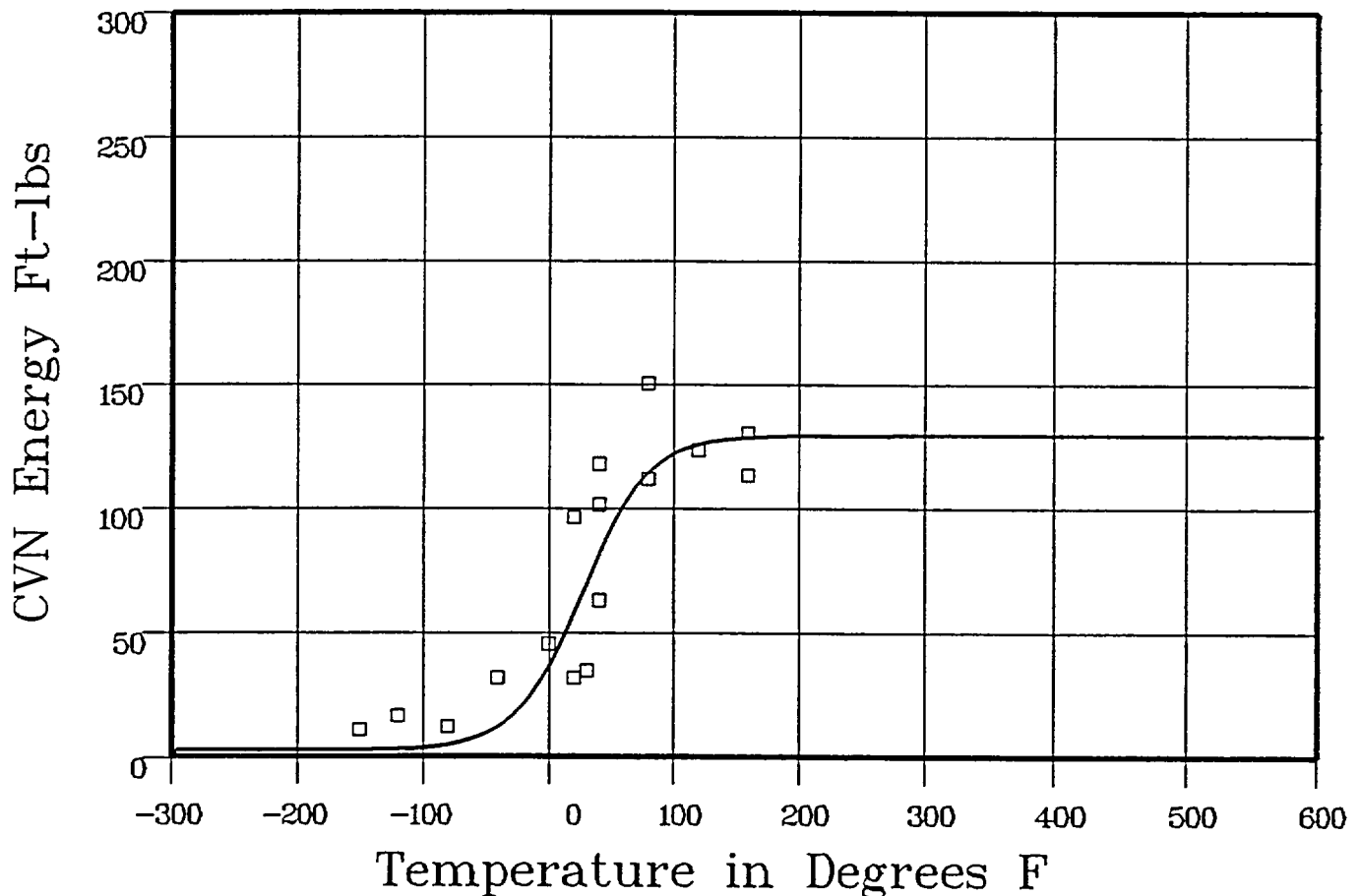
Material: HEAT AFFECTED ZONE SA533B1

Heat Number: C-506-1

Orientation:

Capsule: UNIRR

Total Fluence:



Data Set(s) Plotted

Plant: ML2

Cap: UNIRR

Material: HEAT AFFECTED ZONE SA533B1

Ori:

Heat #: C-506-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
-150	10	2.38	7.61
-120	16	2.78	13.21
-80	11.5	4.81	6.68
-40	31.5	13.26	18.23
0	45	40.59	4.4
20	96	63.16	32.83
20	31.5	63.16	-31.66
30	34.5	75.08	-40.58
40	62.5	86.36	-23.86

**** Data continued on next page ****

UNIRRADIATED (HEAT AFFECTED ZONE)

Page 2

Material: HEAT AFFECTED ZONE SA533B1 Heat Number: C-506-1 Orientation:

Capsule: UNIRR Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
40	117.5	86.36	31.13
40	101	86.36	14.63
80	111.5	116.28	-4.78
80	150	116.28	33.71
120	123	125.96	-2.96
160	113	128.31	-15.31
160	130	128.31	168
			SUM of RESIDUALS = 44.96

CAPSULE 97 (HEAT AFFECTED ZONE)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 09:57:39 on 10-15-2002

Page 1

Coefficients of Curve 2

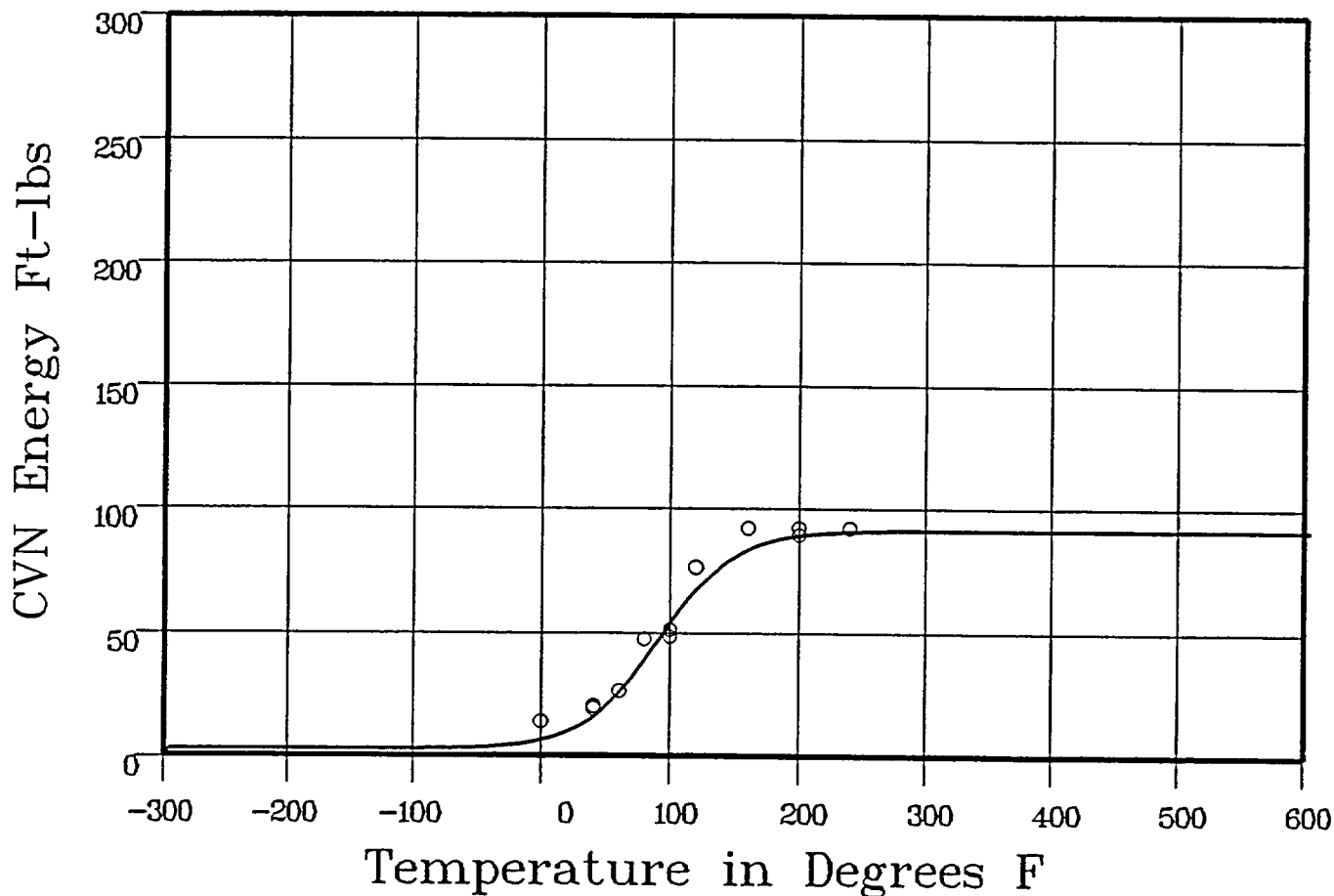
A = 46.59	B = 44.4	C = 59.81	T0 = 86.24
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Equation is: $CVN = A + B * [\tanh((T - T_0)/C)]$

Upper Shelf Energy: 91 Fixed Temp. at 30 ft-lbs: 62.7 Temp. at 50 ft-lbs: 90.8 Lower Shelf Energy: 2.19 Fixed

Material: HEAT AFFECTED ZONE SA533B1 Heat Number: C-506-1 Orientation:

Capsule: W-97 Total Fluence:



Data Set(s) Plotted

Plant: ML2 Cap: W-97 Material: HEAT AFFECTED ZONE SA533B1 Ori: Heat #: C-506-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
0	14	6.9	7.09
40	19	17.79	12
40	20	17.79	2.2
60	26	28.27	-2.27
80	47	41.97	5.02
100	51	56.63	-5.63
100	48	56.63	-8.63
120	76	69.29	6.7

**** Data continued on next page ****

CAPSULE 97 (HEAT AFFECTED ZONE)

Page 2

Material: HEAT AFFECTED ZONE SA533B1

Heat Number: C-506-1

Orientation:

Capsule: W-97

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
160	92	84.04	7.95
200	89	89.06	-.06
200	92	89.06	2.93
240	92	90.48	1.51
			SUM of RESIDUALS = 18.03

CAPSULE 83 (HEAT AFFECTED ZONE)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 09:57:39 on 10-15-2002

Page 1

Coefficients of Curve 3

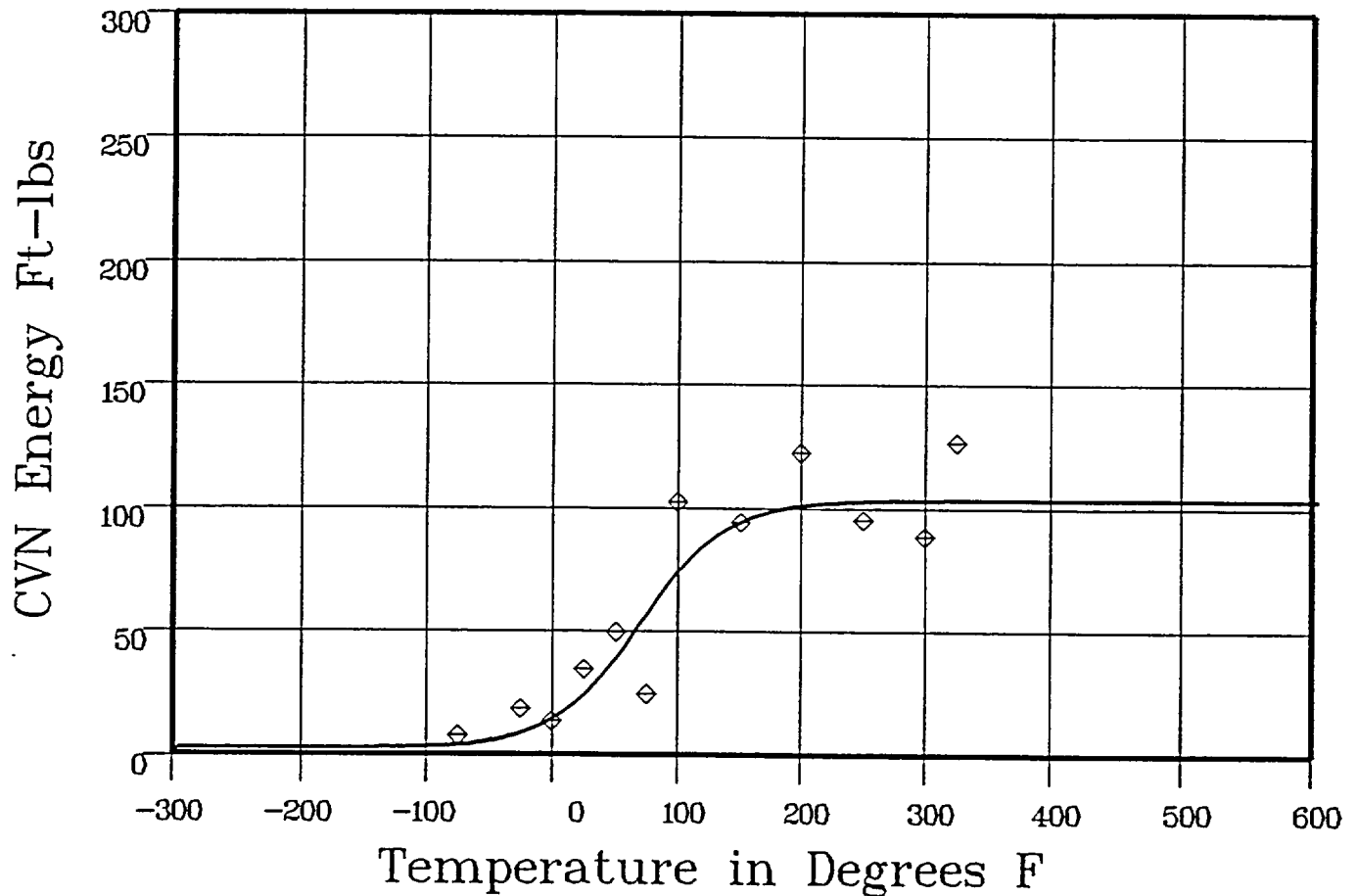
A = 52.59	B = 50.4	C = 68.85	T0 = 64.9
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Equation is: $CVN = A + B * [\tanh((T - T0)/C)]$

Upper Shelf Energy: 103 Fixed Temp. at 30 ft-lbs: 316 Temp. at 50 ft-lbs: 613 Lower Shelf Energy: 2.19 Fixed

Material: HEAT AFFD ZONE SA533B1 Heat Number: C-506-1 Orientation:

Capsule: W-83 Total Fluence:



Data Set(s) Plotted

Plant: ML2 Cap: W-83 Material: HEAT AFFD ZONE SA533B1 Ori: Heat #: C-506-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
-75	7	39	3.09
-25	18	9.09	8.9
0	13	15.48	-2.48
25	34	26.27	7.72
50	49	41.85	7.14
75	24	59.93	-35.93
100	102	76.27	25.72
150	94	95.15	-1.15

**** Data continued on next page ****

CAPSULE 83 (HEAT AFFECTED ZONE)

Page 2

Material: HEAT AFFECTED ZONE SA533B1 Heat Number: C-506-1 Orientation:

Capsule: W-83 Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
200	122	101.04	20.95
250	95	102.53	-7.53
300	88	102.89	-14.89
325	126	102.94	23.05
		SUM of RESIDUALS =	34.59

UNIRRADIATED (HEAT AFFECTED ZONE)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:01:12 on 10-15-2002

Page 1

Coefficients of Curve 1

A = 45.69

B = 44.69

C = 73.87

T0 = 18.75

Equation is: $LE = A + B * [\tanh((T - T0)/C)]$

Upper Shelf LE: 90.39

Temperature at LE 35: .7

Lower Shelf LE: 1 Fixed

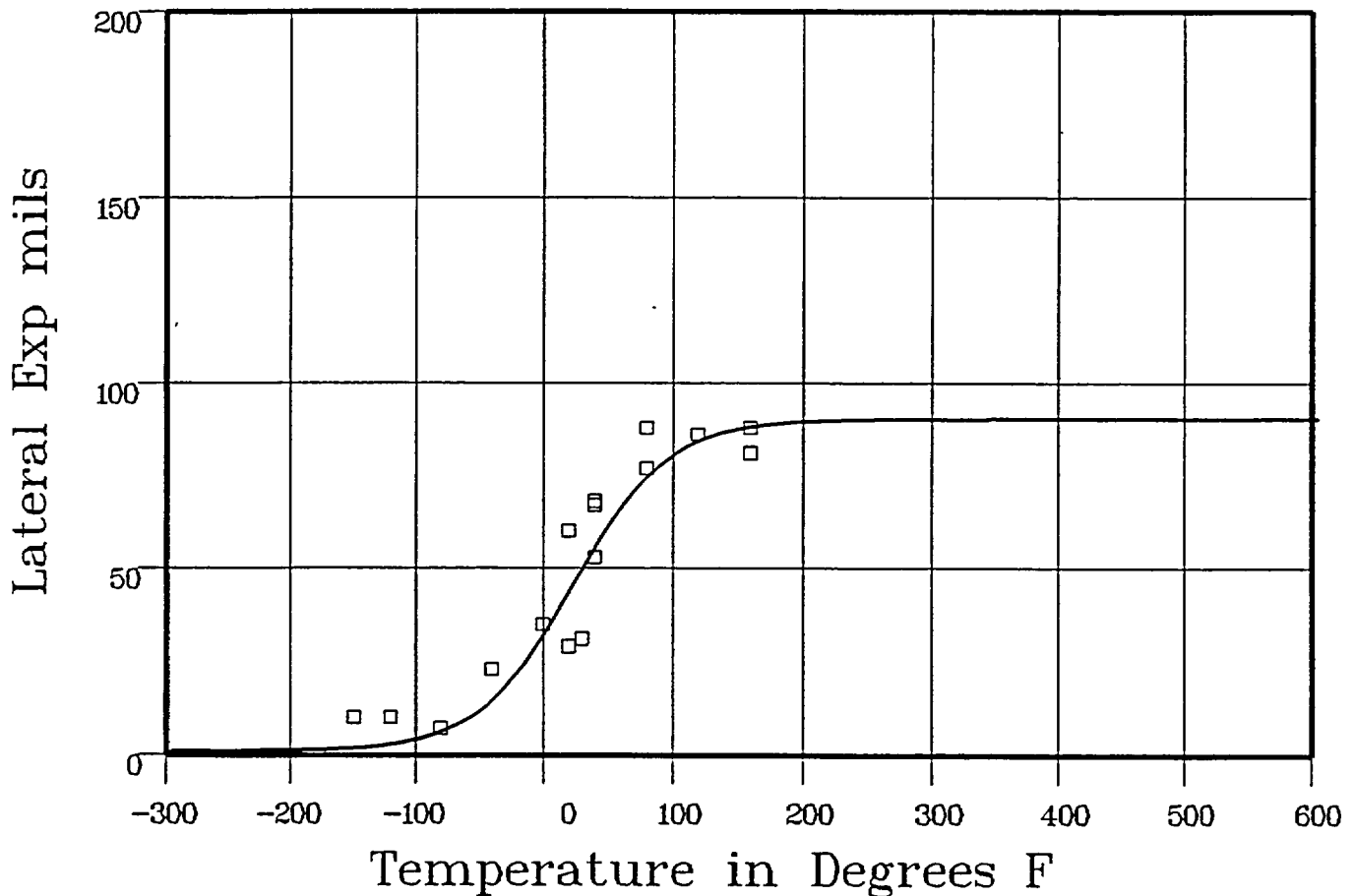
Material: HEAT AFFD ZONE SA533B1

Heat Number: C-506-1

Orientation:

Capsule: UNIRR

Total Fluence:



Data Set(s) Plotted

Plant: ML2

Cap: UNIRR

Material: HEAT AFFD ZONE SA533B1

Ori:

Heat #: C-506-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
-150	10	1.91	8.08
-120	10	3.04	6.95
-80	7	6.77	.22
-40	23	16.13	6.86
0	35	34.58	.41
20	60	46.45	13.54
20	29	46.45	-17.45
30	31	52.45	-21.45
40	53	58.21	-5.21

**** Data continued on next page ****

UNIRRADIATED (HEAT AFFECTED ZONE)

Page 2

Material: HEAT AFFECTED ZONE SA533B1 Heat Number: C-506-1 Orientation:

Capsule: UNIRR Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed LE	Differential
40	68	58.21	9.78
40	67	58.21	8.78
80	77	76.09	.9
80	88	76.09	11.9
120	86	84.97	102
160	81	88.48	-7.48
160	88	88.48	-48
			SUM of RESIDUALS = 1642

CAPSULE 97 (HEAT AFFECTED ZONE)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:01:12 on 10-15-2002

Page 1

Coefficients of Curve 2

A = 3867	B = 37.67	C = 79.81	T0 = 84.17
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Equation is: $LE = A + B * [\tanh((T - T0)/C)]$

Upper Shelf LE: 76.34

Temperature at LE 35: 76.3

Lower Shelf LE: 1 Fixed

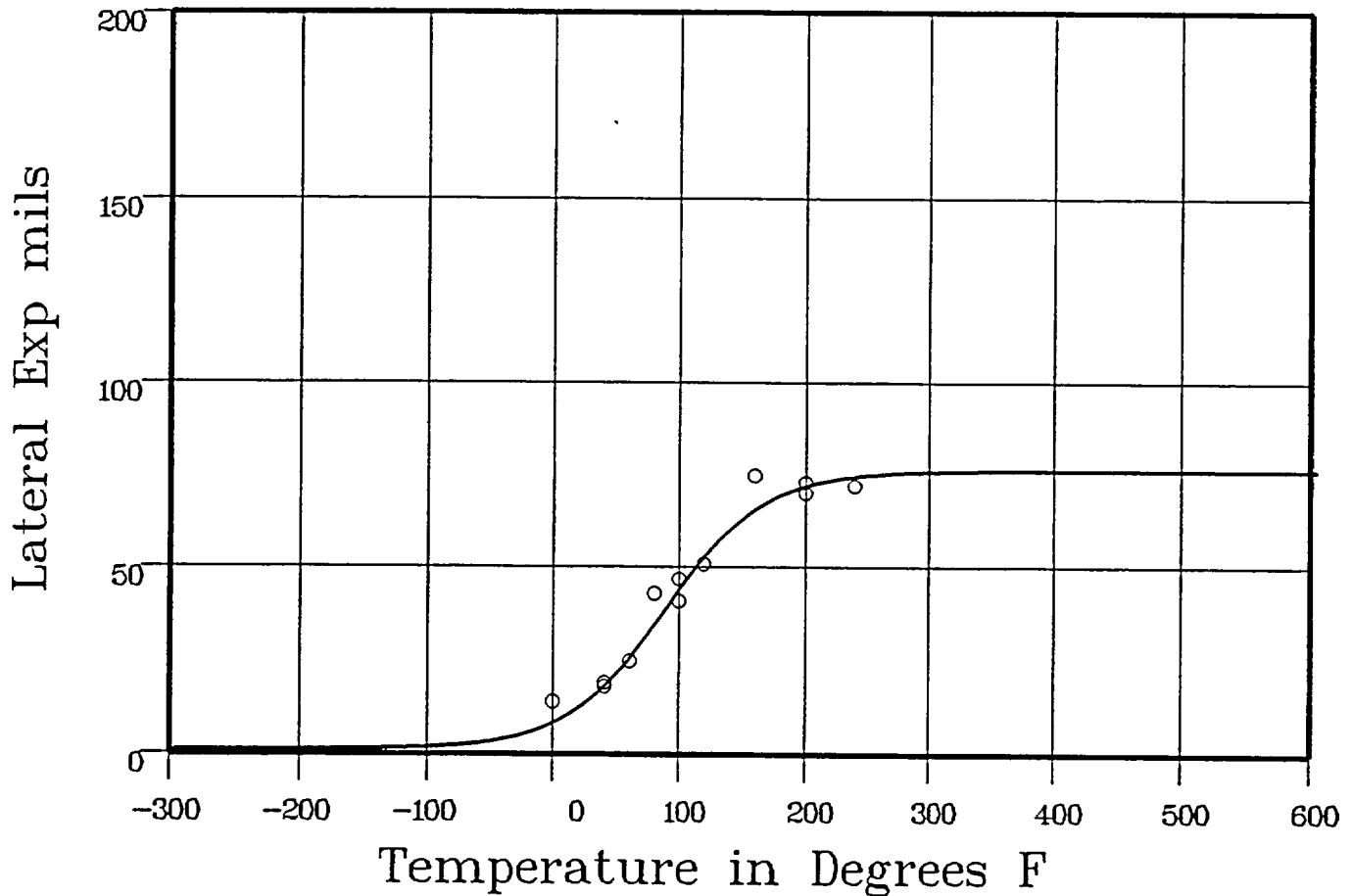
Material: HEAT AFFD ZONE SA533B1

Heat Number: C-506-1

Orientation:

Capsule: W-97

Total Fluence:



Data Set(s) Plotted

Plant: ML2

Cap: W-97

Material: HEAT AFFD ZONE SA533B1

Ori:

Heat #: C-506-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
0	14	9.15	4.84
40	18	19.71	-1.71
40	19	19.71	-.71
60	25	27.59	-2.59
80	43	36.7	6.29
100	47	46.04	.95
100	41	46.04	-5.04
120	51	54.52	-3.52

**** Data continued on next page ****

CAPSULE 97 (HEAT AFFECTED ZONE)

Page 2

Material: HEAT AFFD ZONE SA533B1

Heat Number: C-506-1

Orientation:

Capsule: W-97

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed LE	Differential
160	75	66.53	8.46
200	70	72.42	-2.42
200	73	72.42	.57
240	72	74.85	-2.85
			SUM of RESIDUALS = 2.26

CAPSULE 83 (HEAT AFFECTED ZONE)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:01:12 on 10-15-2002

Page 1

Coefficients of Curve 3

A = 34.95	B = 33.95	C = 81.03	T0 = 75.29
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Equation is: $LE = A + B * [\tanh((T - T0)/C)]$

Upper Shelf LE: 689

Temperature at LE 35: 75.4

Lower Shelf LE: 1 Fixed

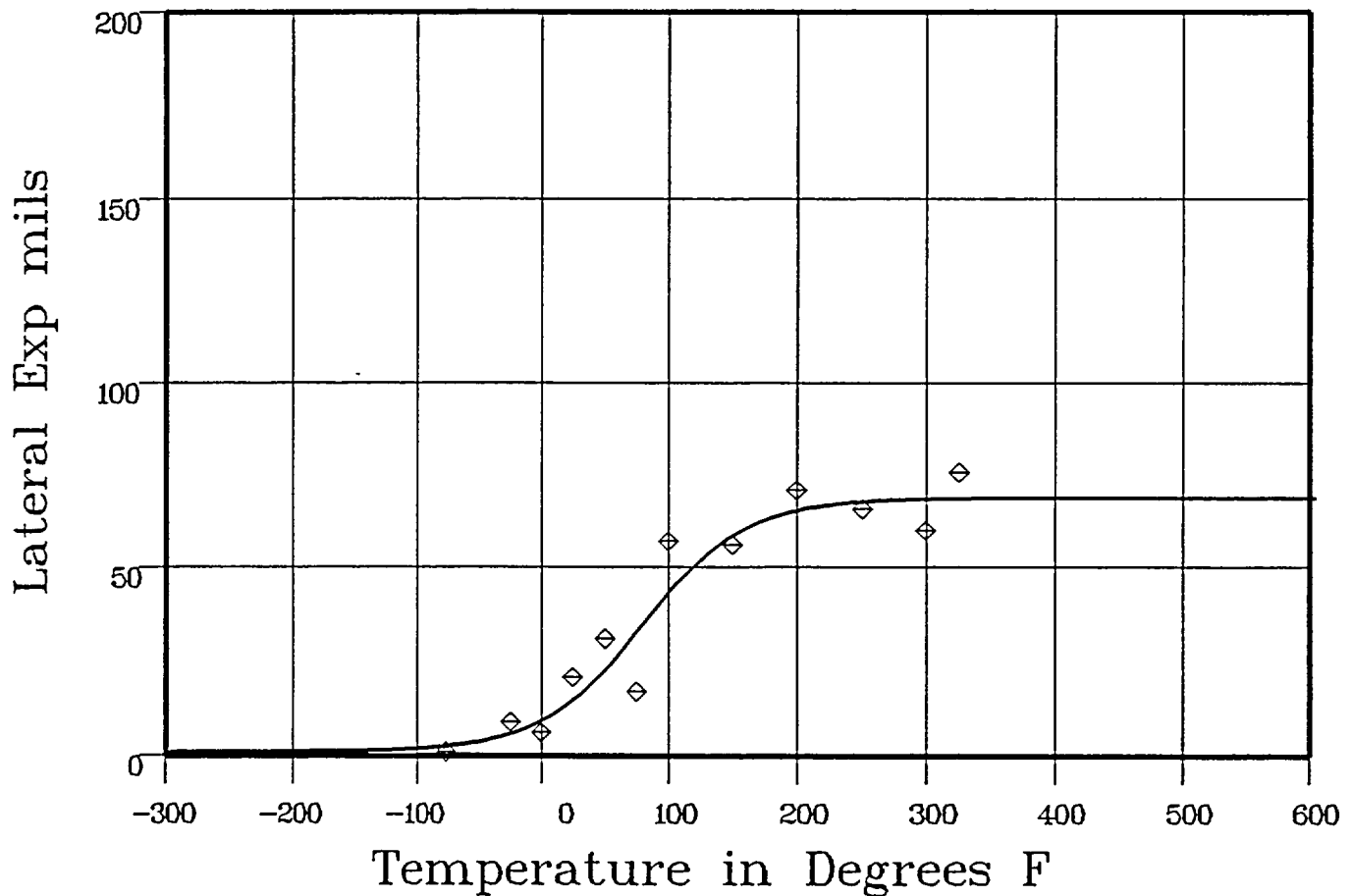
Material: HEAT AFFD ZONE SA533B1

Heat Number: C-506-1

Orientation:

Capsule: W-83

Total Fluence:



Data Set(s) Plotted

Plant: ML2

Cap: W-83

Material: HEAT AFFD ZONE SA533B1

Ori:

Heat #: C-506-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
-75	1	2.62	-1.62
-25	9	6.26	2.73
0	6	10.16	-4.16
25	21	16.22	4.77
50	31	24.68	6.31
75	17	34.82	-17.82
100	57	44.99	12
150	56	59.62	-3.62

**** Data continued on next page ****

CAPSULE 83 (HEAT AFFECTED ZONE)

Page 2

Material: HEAT AFFD ZONE SA533B1 Heat Number: C-506-1 Orientation:

Capsule: W-83 Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed LE	Differential
200	71	65.91	5.08
250	66	68	-2
300	60	68.64	-8.64
325	76	68.76	7.23
SUM of RESIDUALS =			.26

UNIRRADIATED (HEAT AFFECTED ZONE)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:04:13 on 10-15-2002

Page 1

Coefficients of Curve 1

A = 50	B = 50	C = 82.38	T0 = 7.96
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$$\text{Equation is: Shear\%} = A + B * [\tanh((T - T_0)/C)]$$

Temperature at 50% Shear: 7.9

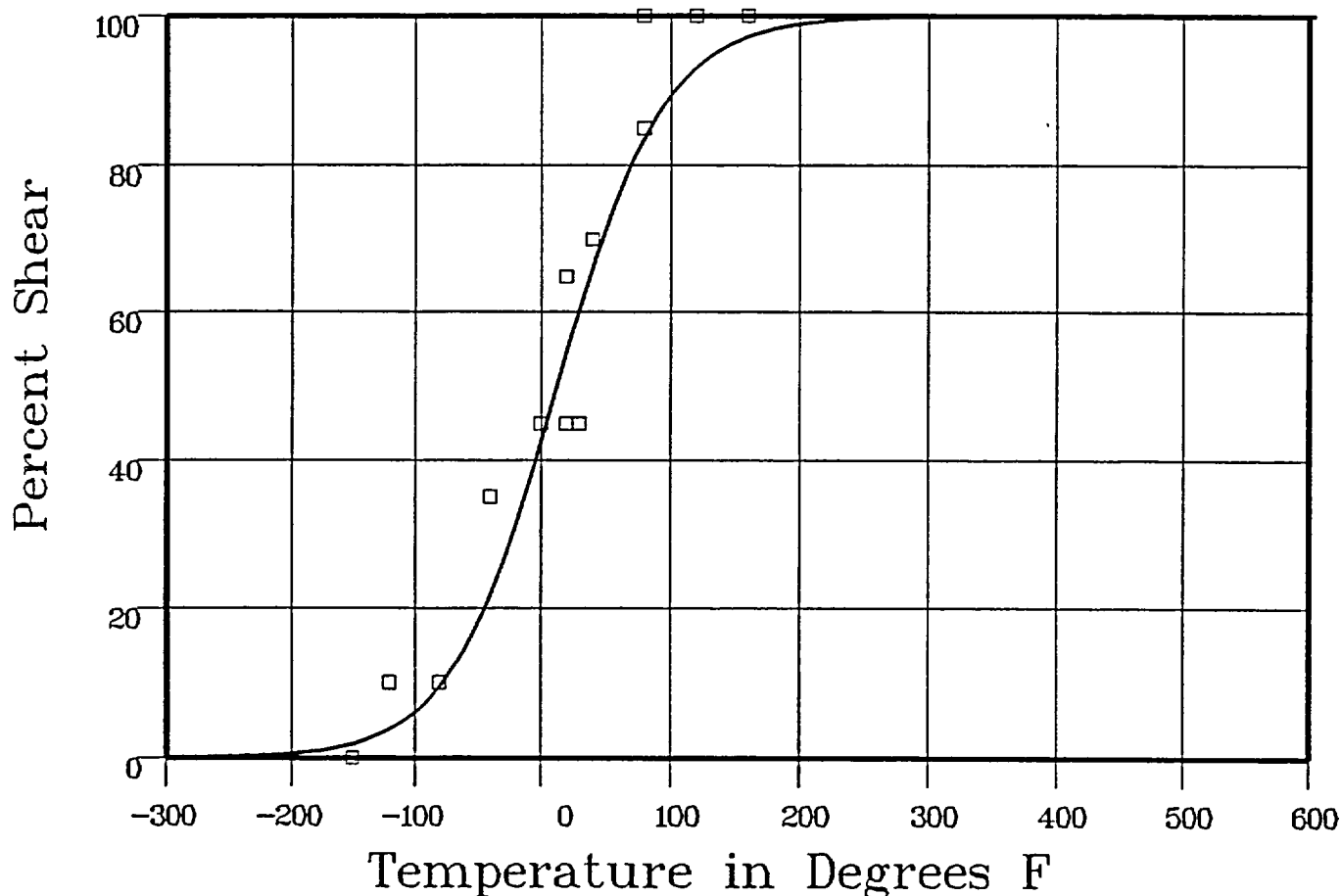
Material: HEAT AFFD ZONE SA533B1

Heat Number: C-506-1

Orientation:

Capsule: UNIRR

Total Fluence:



Data Set(s) Plotted

Plant: ML2

Cap: UNIRR

Material: HEAT AFFD ZONE SA533B1

Ori:

Heat #: C-506-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
-150	0	2.11	-2.11
-120	10	4.28	5.71
-80	10	10.56	-5.6
-40	35	23.78	11.21
0	45	45.17	-1.7
20	65	57.25	7.74
20	45	57.25	-12.25
30	45	63.06	-18.06
40	70	68.51	1.48

**** Data continued on next page ****

UNIRRADIATED (HEAT AFFECTED ZONE)

Page 2

Material: HEAT AFFD ZONE SA533B1 Heat Number: C-506-1 Orientation:

Capsule: UNIRR Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
40	70	68.51	1.48
40	70	68.51	1.48
80	85	85.17	-17
80	100	85.17	14.82
120	100	93.81	6.18
160	100	97.56	2.43
160	100	97.56	2.43
			SUM of RESIDUALS = 2165

CAPSULE 97 (HEAT AFFECTED ZONE)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:04:13 on 10-15-2002

Page 1

Coefficients of Curve 2

A = 50	B = 50	C = 67.46	T0 = 96.67
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Equation is: $\text{Shear}\% = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 96.6

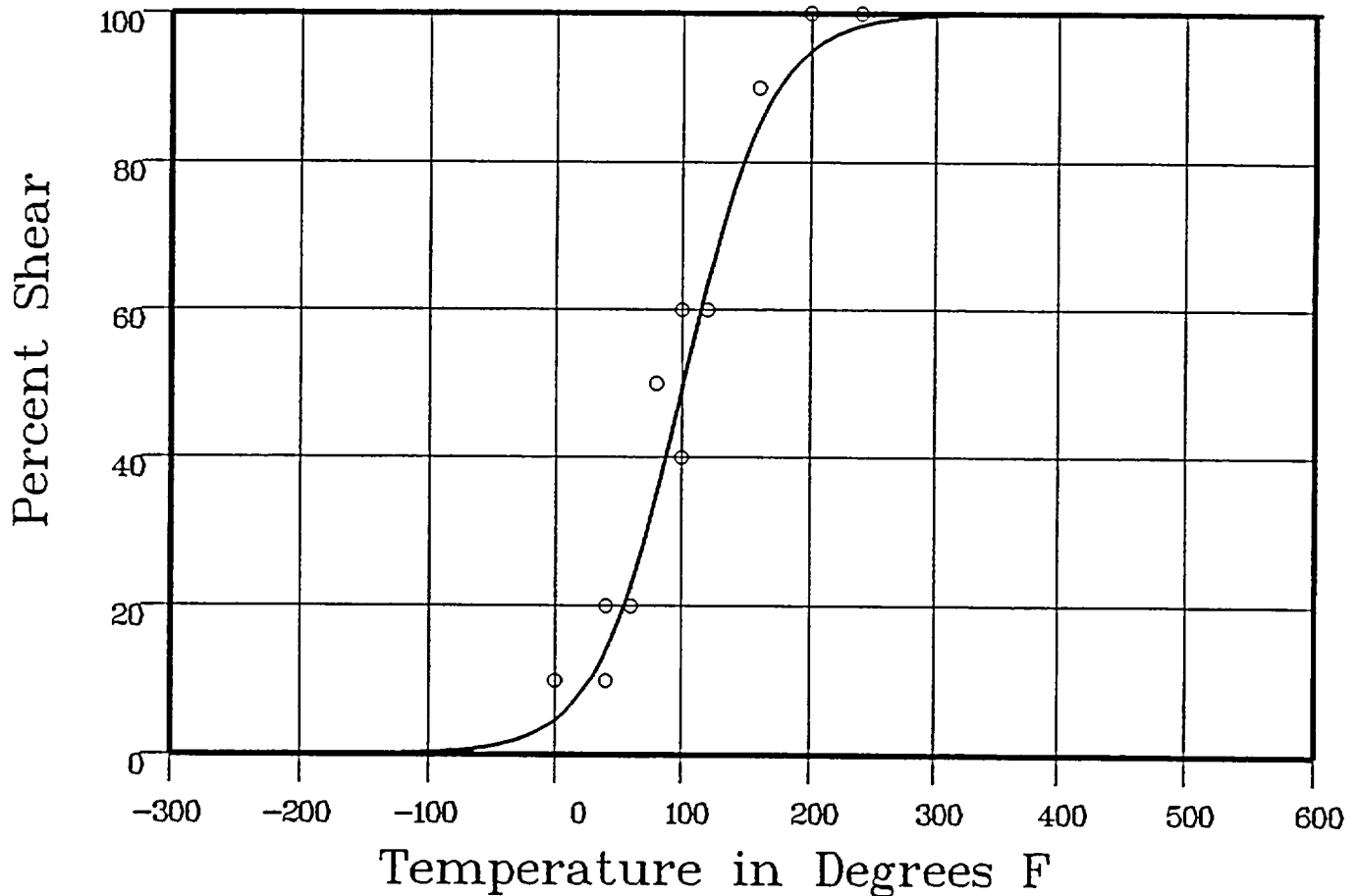
Material: HEAT AFFECTED ZONE SA533B1

Heat Number: C-506-1

Orientation:

Capsule: W-97

Total Fluence:



Data Set(s) Plotted

Plant: ML2

Cap: W-97

Material: HEAT AFFECTED ZONE SA533B1

Ori:

Heat #: C-506-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
0	10	5.38	4.61
40	20	15.7	4.29
40	10	15.7	-5.7
60	20	25.21	-5.21
80	50	37.88	12.11
100	60	52.45	7.54
100	40	52.45	-12.45
120	60	66.62	-6.62

**** Data continued on next page ****

CAPSULE 97 (HEAT AFFECTED ZONE)

Page 2

Material: HEAT AFFD ZONE SA533B1 Heat Number: C-506-1 Orientation:

Capsule: W-97 Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
160	90	86.72	3.27
200	100	95.53	4.46
200	100	95.53	4.46
240	100	98.59	1.4
			SUM of RESIDUALS = 12.17

CAPSULE 83 (HEAT AFFECTED ZONE)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:04:13 on 10-15-2002

Page 1

Coefficients of Curve 3

A = 50	B = 50	C = 101.88	T0 = 69.37
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Equation is: $\text{Shear\%} = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 69.3

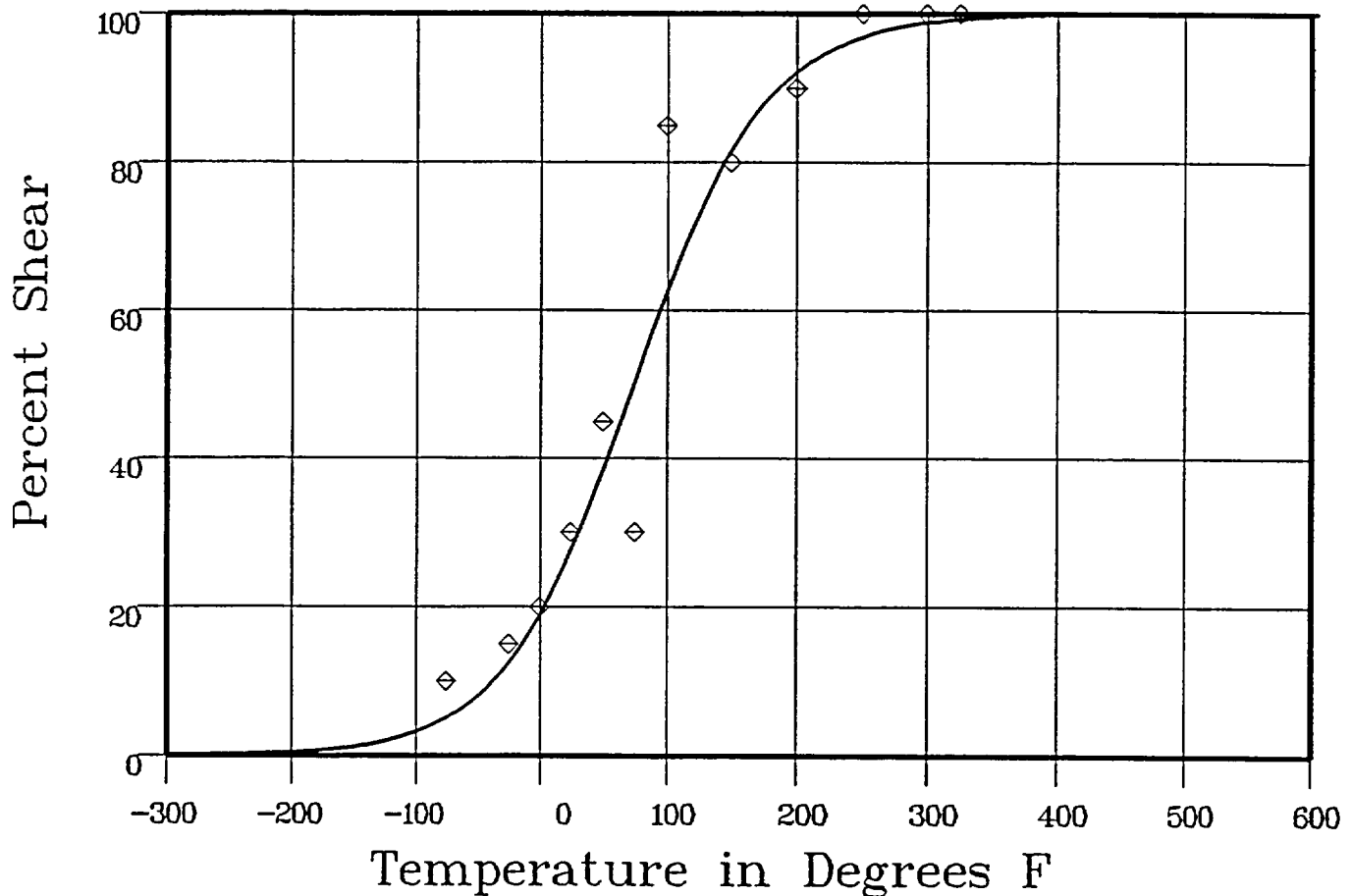
Material: HEAT AFFD ZONE SA533B1

Heat Number: C-506-1

Orientation:

Capsule: W-83

Total Fluence:



Data Set(s) Plotted

Plant: ML2

Cap: W-83

Material: HEAT AFFD ZONE SA533B1

Ori:

Heat #: C-506-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
-75	10	5.55	4.44
-25	15	13.55	1.44
0	20	20.39	-3.9
25	30	29.5	.49
50	45	40.6	4.39
75	30	52.75	-22.75
100	85	64.59	20.4
150	80	82.95	-2.95

**** Data continued on next page ****

CAPSULE 83 (HEAT AFFECTED ZONE)

Page 2

Material: HEAT AFFD ZONE SA533B1

Heat Number: C-506-1

Orientation:

Capsule: W-83

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
200	90	92.85	-2.85
250	100	97.19	2.8
300	100	98.93	106
325	100	99.34	.65

SUM of RESIDUALS = 6.76

UNIRRADIATED (STANDARD REFERENCE MATERIAL)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:08:44 on 10-15-2002

Page 1

Coefficients of Curve 1

A = 7159

B = 69.4

C = 6323

T0 = 75.29

$$\text{Equation is: } CVN = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf Energy: 141 Fixed Temp. at 30 ft-lbs: 315 Temp. at 50 ft-lbs: 54.9 Lower Shelf Energy: 219 Fixed

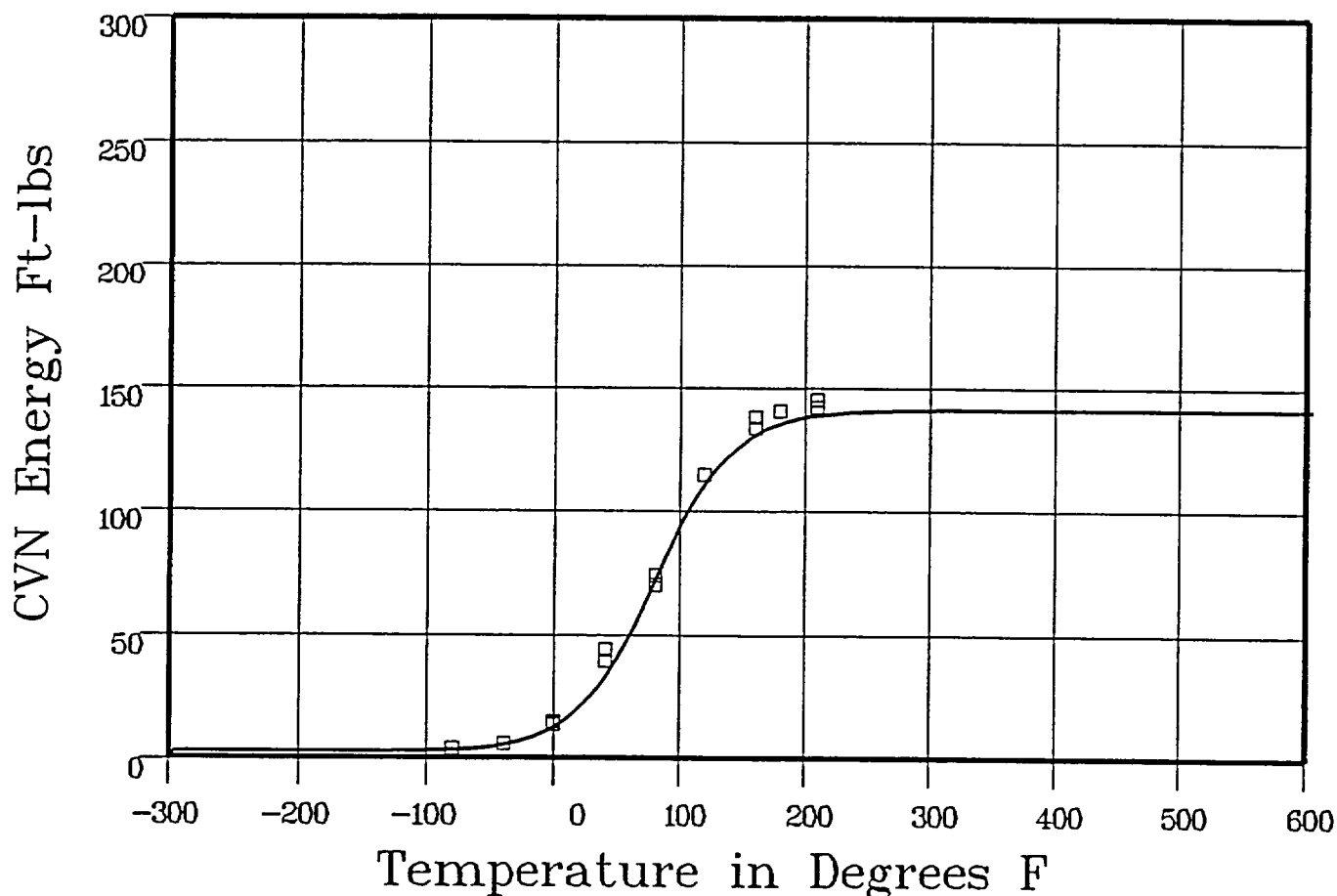
Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: UNIRR

Total Fluence:



Plant: ML2 Cap: UNIRR Data Set(s) Plotted Material: SRM HSST01 Ori: LT Heat #: A1008-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
-80	3.5	3.21	28
-40	5.5	5.72	-22
0	13.5	13.94	-44
0	14.5	13.94	55
40	39	36.44	255
40	43.5	36.44	7.05
80	73.5	76.75	-3.25
80	69.5	76.75	-7.25
120	114.5	113.84	.65

**** Data continued on next page ****

UNIRRADIATED (STANDARD REFERENCE MATERIAL)

Page 2

Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: UNIRR

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input CVN Energy	Computed CVN Energy	Differential
120	114.5	113.84	.65
160	138	132.08	5.91
160	133.5	132.08	1.41
180	140.5	136.11	4.38
210	145	139.06	5.93
210	142	139.06	2.93
			SUM of RESIDUALS = 21.14

CAPSULE 104 (STANDARD REFERENCE MATERIAL)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:08:44 on 10-15-2002

Page 1

Coefficients of Curve 2

A = 47.09	B = 44.9	C = 74.52	T0 = 194.82
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Equation is: $CVN = A + B * [\tanh((T - T0)/C)]$

Upper Shelf Energy: 92 Fixed Temp. at 30 ft-lbs: 164.9 Temp. at 50 ft-lbs: 199.6 Lower Shelf Energy: 2.19 Fixed

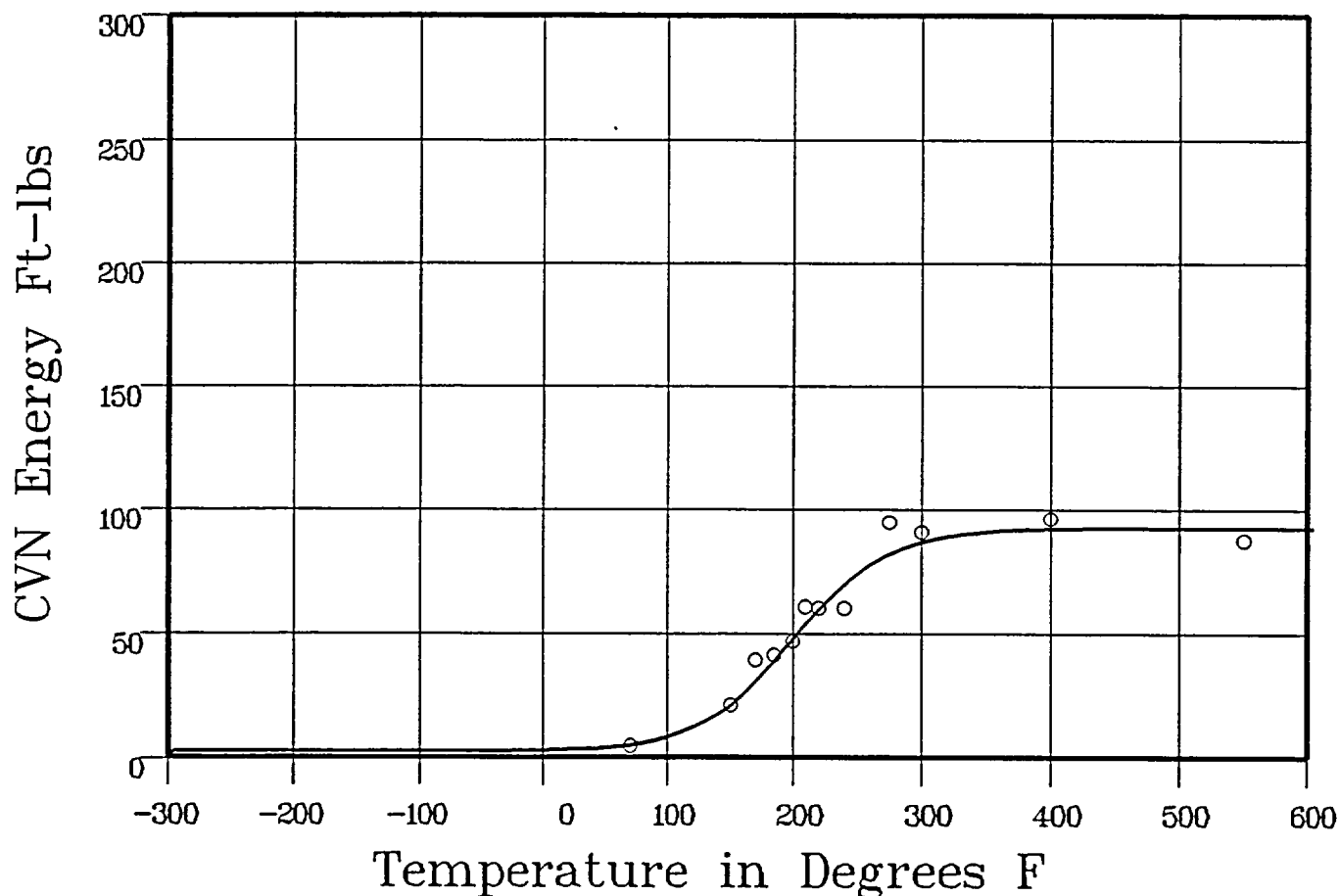
Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: W-104

Total Fluence:



Data Set(s) Plotted
 Plant: ML2 Cap: W-104 Material: SRM HSST01 Ori: LT Heat #: A1008-1

Charpy V-Notch Data

Temperature	Input CVN Energy	Computed CVN Energy	Differential
70	45	524	-74
150	21	22.94	-1.94
170	39	32.67	6.32
185	41	41.21	-21
200	46.5	50.21	-3.71
210	60.5	56.11	4.38
220	60	61.71	-1.71
240	60	71.4	-11.4

**** Data continued on next page ****

CAPSULE 104 (STANDARD REFERENCE MATERIAL)

Page 2

Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: W-104

Total Fluence:

Charpy V-Notch Data (Continued)

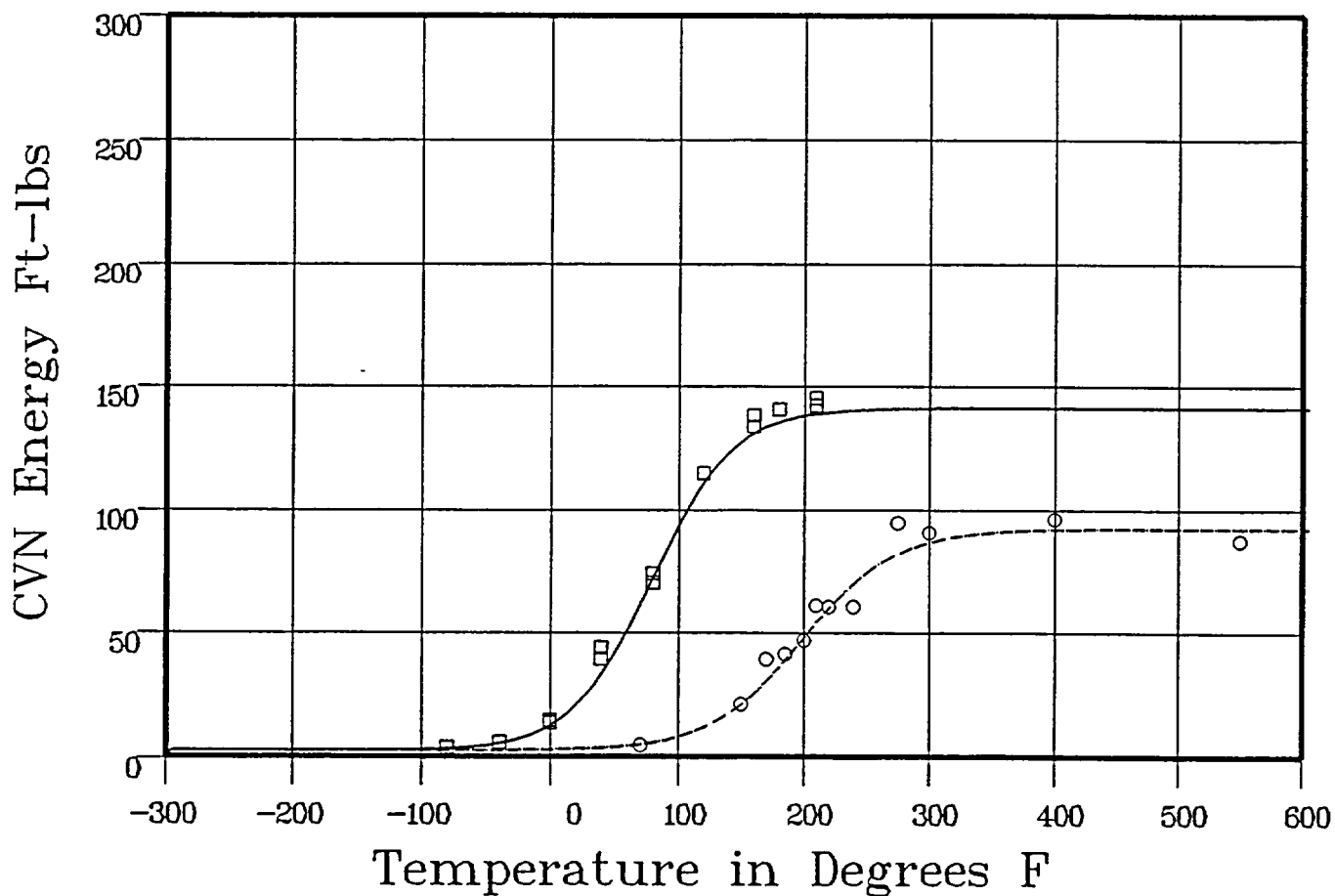
Temperature	Input CVN Energy	Computed CVN Energy	Differential
275	94.5	82.64	11.85
300	90.5	86.96	3.53
400	96	91.63	4.36
550	87	91.99	-4.99
		SUM of RESIDUALS =	5.73

STANDARD REFERENCE MATERIAL

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:08:44 on 10-15-2002

Results

Curve	Fluence	LSE	d-LSE	USE	d-USE	T @ 30	d-T @ 30	T @ 50	d-T @ 50
1	0	2.19	0	141	0	315.1	0	54.93	0
2	0	2.19	0	92	-49	164.93	133.41	199.64	144.7



Curve Legend

1 \square ———

2 \circ - - - - -

Data Set(s) Plotted

Curve	Plant	Capsule	Material	Ori.	Heat#
1	ML2	UNIRR	SRM HSST01	LT	A1008-1
2	ML2	W-104	SRM HSST01	LT	A1008-1

UNIRRADIATED (STANDARD REFERENCE MATERIAL)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:12:01 on 10-15-2002

Page 1

Coefficients of Curve 1

A = 47.42	B = 46.42	C = 71.51	T0 = 60.46
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$$\text{Equation is: } LE = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf LE: 93.84

Temperature at LE 35: 40.8

Lower Shelf LE: 1 Fixed

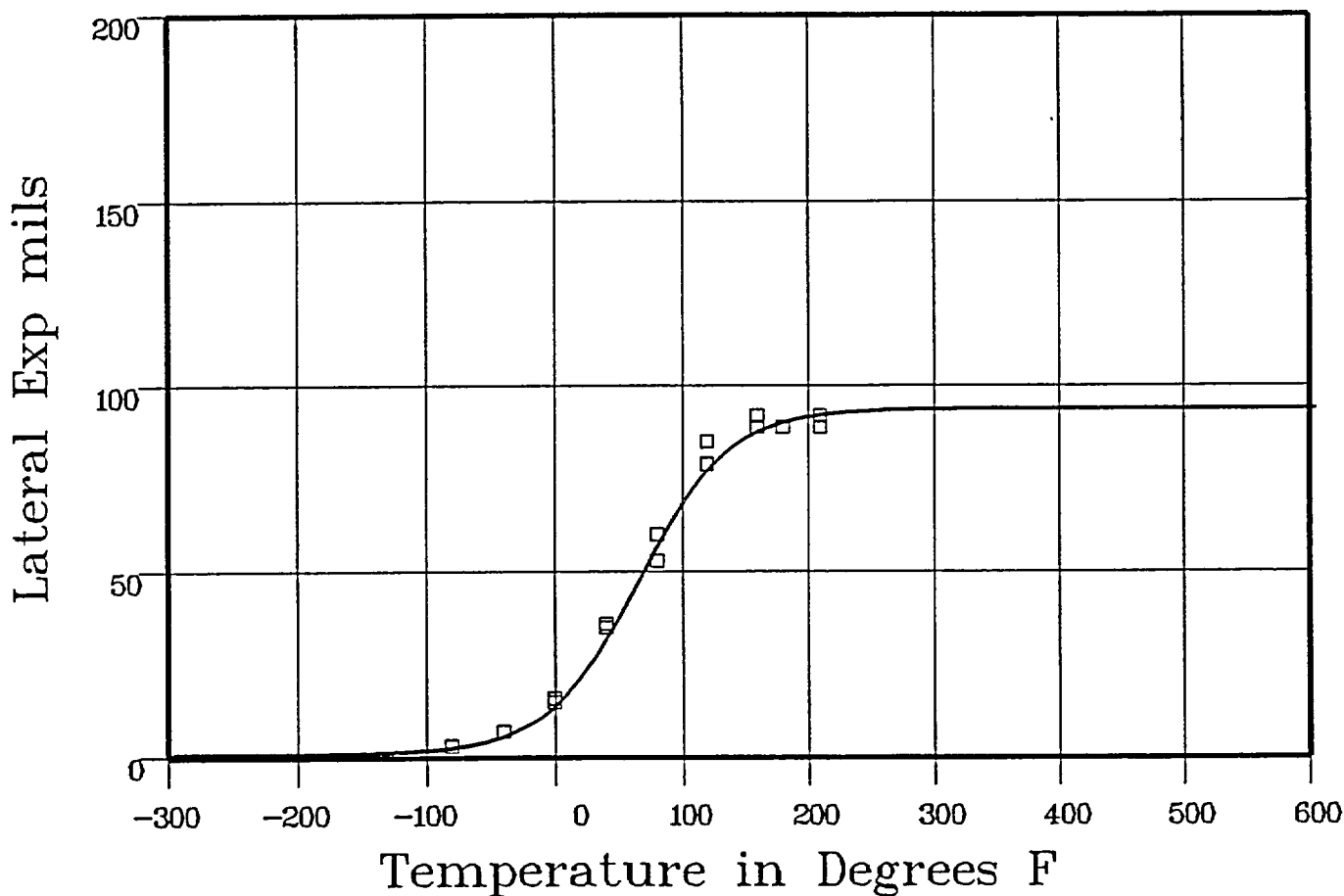
Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: UNIRR

Total Fluence:



Plant: ML2 Cap: UNIRR Data Set(s) Plotted Material: SRM HSST01 Ori: LT Heat #: A1008-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
-80	3	2.79	2
-40	7	6.27	.72
0	15	15.44	-44
0	16	15.44	55
40	35	34.48	51
40	36	34.48	151
80	60	59.79	2
80	53	59.79	-6.79
120	85	79.07	5.92

**** Data continued on next page ****

UNIRRADIATED. (STANDARD REFERENCE MATERIAL)

Page 2

Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: UNIRR

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Lateral Expansion	Computed L.E.	Differential
120	79	79.07	-.07
160	89	88.43	.56
160	92	88.43	3.56
180	89	90.67	-1.67
210	92	92.44	-.44
210	89	92.44	-3.44
SUM of RESIDUALS =			.87

CAPSULE 104 (STANDARD REFERENCE MATERIAL)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:12:01 on 10-15-2002

Page 1

Coefficients of Curve 2

A = 41.83	B = 40.83	C = 87.26	T0 = 190.72
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$$\text{Equation is: } LE = A + B * [\tanh((T - T0)/C)]$$

Upper Shelf LE: 82.67

Temperature at LE 35: 175.9

Lower Shelf LE: 1 Fixed

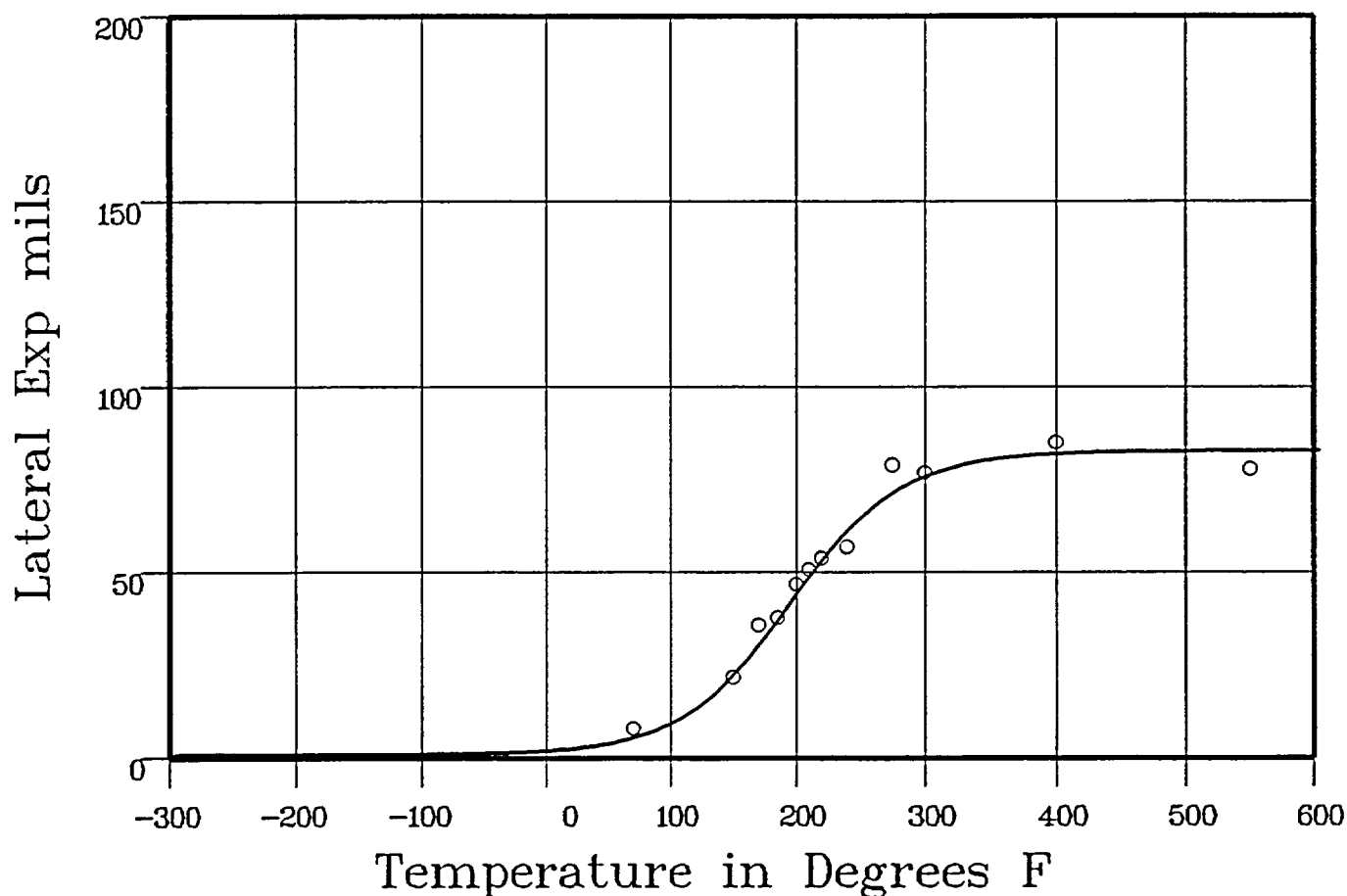
Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: W-104

Total Fluence:



Plant: ML2 Cap: W-104 Material: SRM HSST01 Ori: LT Heat #: A1008-1

Charpy V-Notch Data

Temperature	Input Lateral Expansion	Computed LE	Differential
70	8	5.83	2.16
150	22	24.05	-2.05
170	36	32.31	3.68
185	38	39.16	-1.16
200	47	46.16	.83
210	51	50.71	.28
220	54	55.04	-1.04
240	57	62.72	-5.72

**** Data continued on next page ****

CAPSULE 104 (STANDARD REFERENCE MATERIAL)

Page 2

Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: W-104

Total Fluence:

Charpy V-Notch Data (Continued)

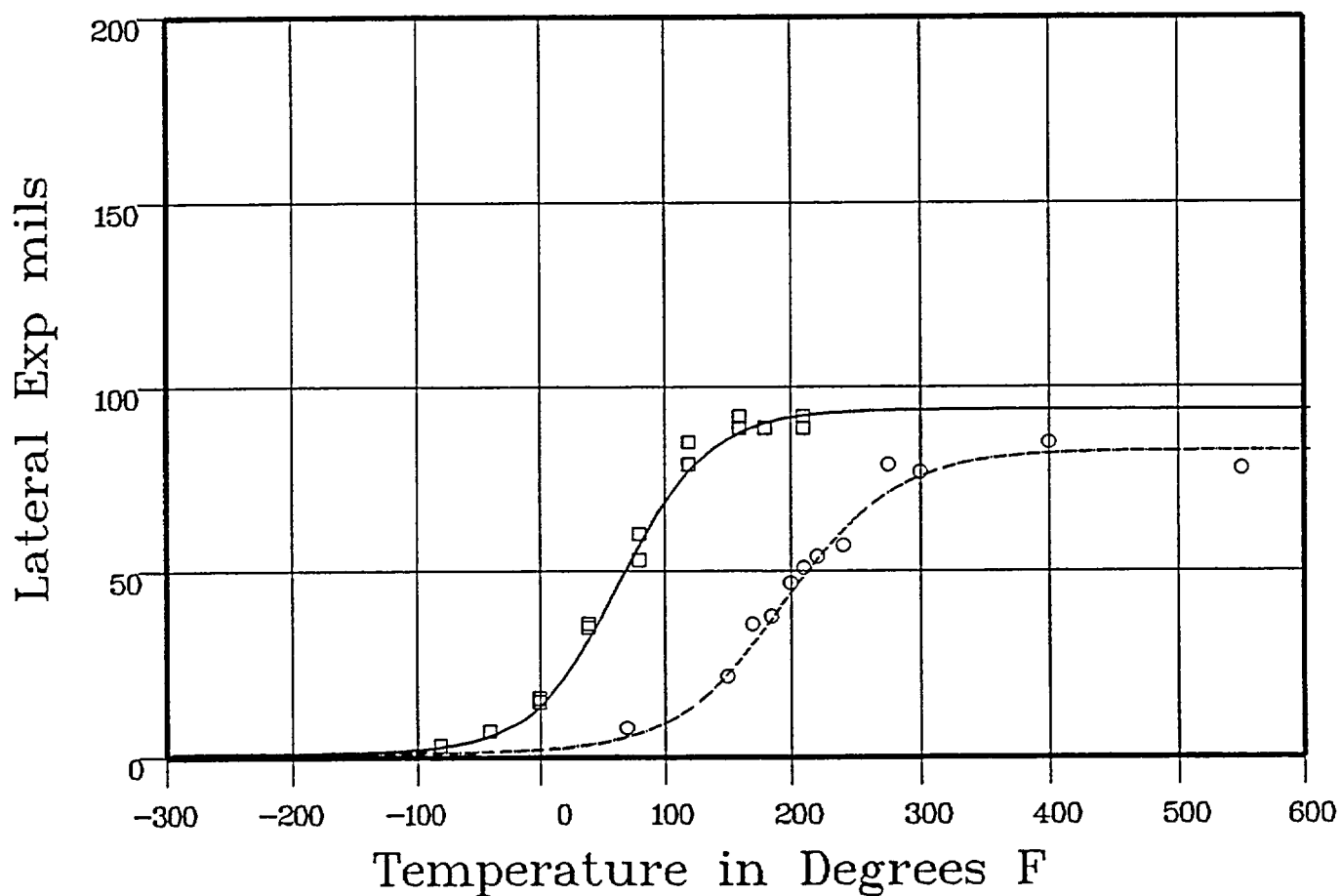
Temperature	Input Lateral Expansion	Computed L.E.	Differential
275	79	72.33	6.66
300	77	76.5	.49
400	85	82	2.99
550	78	82.65	-4.65
			SUM of RESIDUALS = 2.5

STANDARD REFERENCE MATERIAL

CVGRAPH 41 Hyperbolic Tangent Curve Printed at 10:12:01 on 10-15-2002

Results

Curve	Fluence	USE	d-USE	T @ LE35	d-T @ LE35
1	0	93.84	0	40.85	0
2	0	82.67	-11.17	175.97	135.11



Curve Legend

1 \square ———

2 \circ - - - - -

Data Set(s) Plotted

Curve	Plant	Capsule	Material	Ori.	Heat#
1	ML2	UNIRR	SRM HSST01	LT	A1008-1
2	ML2	W-104	SRM HSST01	LT	A1008-1

UNIRRADIATED (STANDARD REFERENCE MATERIAL)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:14:42 on 10-15-2002

Page 1

Coefficients of Curve 1

A = 50	B = 50	C = 64.08	T0 = 80.15
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Equation is: $\text{Shear}\% = A + B * [\tanh((T - T0)/C)]$

Temperature at 50% Shear: 80.1

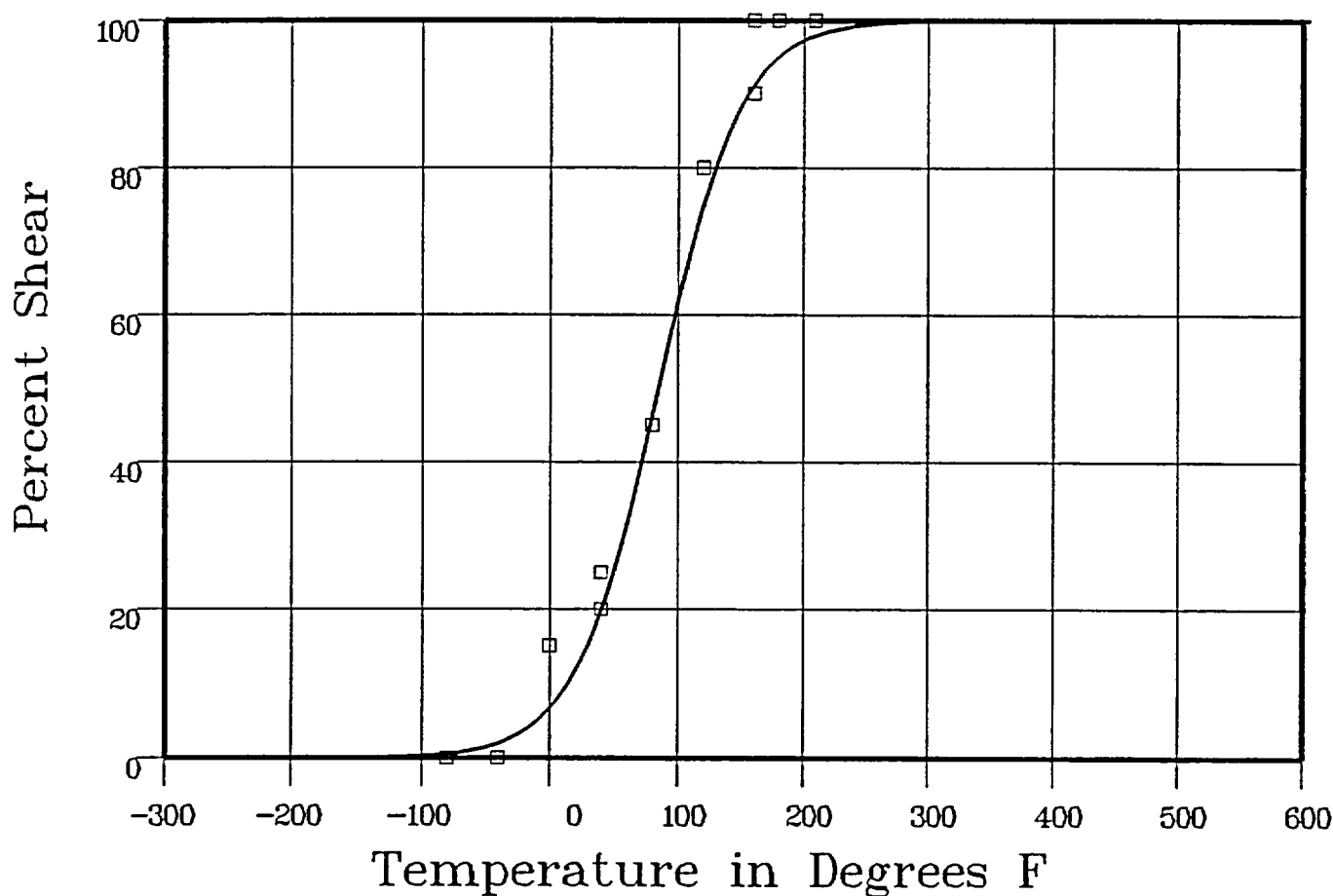
Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: UNIRR

Total Fluence:



Plant: ML2 Cap: UNIRR Data Set(s) Plotted Material: SRM HSST01 Ori: LT Heat #: A1008-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
-80	0	.67	-.67
-40	0	2.29	-2.29
0	15	7.57	7.42
0	15	7.57	7.42
40	20	22.21	-2.21
40	25	22.21	2.78
80	45	49.87	-4.87
80	45	49.87	-4.87
120	80	77.61	2.38

**** Data continued on next page ****

UNIRRADIATED (STANDARD REFERENCE MATERIAL)

Page 2

Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: UNIRR

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
120	80	77.61	2.38
160	100	92.35	7.64
160	90	92.35	-2.35
180	100	95.75	4.24
210	100	98.29	1.7
210	100	98.29	1.7
			SUM of RESIDUALS = 20.41

CAPSULE 104 (STANDARD REFERENCE MATERIAL)

CVGRAPH 4.1 Hyperbolic Tangent Curve Printed at 10:14:42 on 10-15-2002

Page 1

Coefficients of Curve 2

A = 50	B = 50	C = 77.3	T0 = 199.21
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Equation is: $\text{Shear}\% = A + B * [\tanh((T - T_0)/C)]$

Temperature at 50% Shear: 199.2

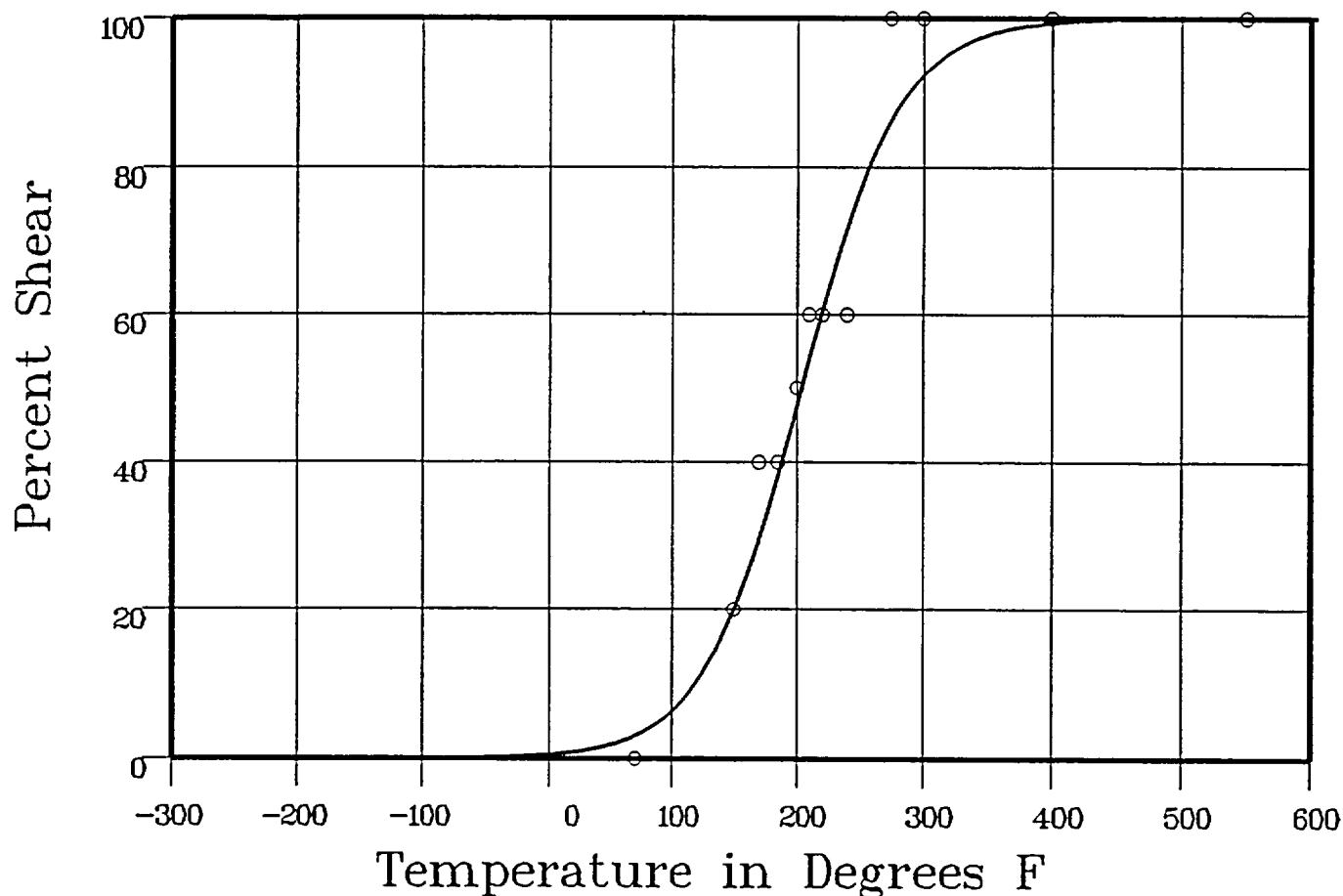
Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: W-104

Total Fluence:



Data Set(s) Plotted
 Plant: ML2 Cap: W-104 Material: SRM HSST01 Ori: LT Heat #: A1008-1

Charpy V-Notch Data

Temperature	Input Percent Shear	Computed Percent Shear	Differential
70	0	3.41	-3.41
150	20	21.86	-1.86
170	40	31.95	8.04
185	40	40.9	-.9
200	50	50.5	-.5
210	60	56.92	3.07
220	60	63.12	-3.12
240	60	74.17	-14.17

**** Data continued on next page ****

CAPSULE 104 (STANDARD REFERENCE MATERIAL)

Page 2

Material: SRM HSST01

Heat Number: A1008-1

Orientation: LT

Capsule: W-104

Total Fluence:

Charpy V-Notch Data (Continued)

Temperature	Input Percent Shear	Computed Percent Shear	Differential
275	100	87.65	12.34
300	100	93.13	6.86
400	100	99.44	.55
550	100	99.98	.01

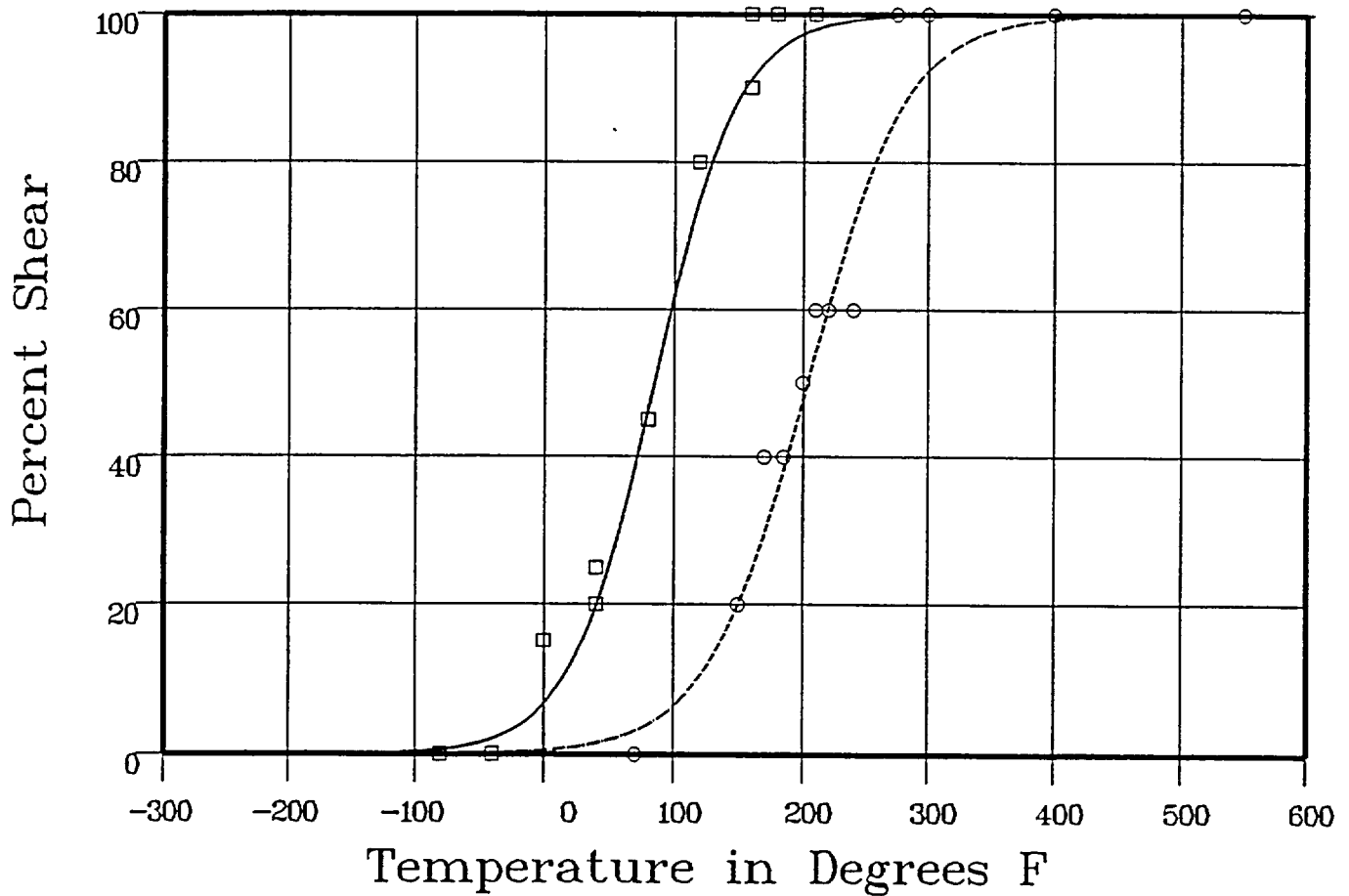
SUM of RESIDUALS = 6.89

STANDARD REFERENCE MATERIAL

CVGRAPH 41 Hyperbolic Tangent Curve Printed at 10:14:42 on 10-15-2002

Results

Curve	Fluence	T @ 50% Shear	d-T @ 50% Shear
1	0	80.15	0
2	0	199.21	119.06



Curve Legend

1 \square ———

2 \circ - - - - -

Data Set(s) Plotted

Curve	Plant	Capsule	Material	Ori.	Heat#
1	ML2	UNIRR	SRM HSST01	LT	A1008-1
2	ML2	W-104	SRM HSST01	LT	A1008-1

APPENDIX D

MILLSTONE UNIT 2 SURVEILLANCE PROGRAM

CREDIBILITY ANALYSIS

SURVEILLANCE DATA CREDIBILITY EVALUATION

INTRODUCTION:

Regulatory Guide 1.99, Revision 2, describes general procedures acceptable to the NRC staff for calculating the effects of neutron radiation embrittlement of the low-alloy steels currently used for light-water-cooled reactor vessels. Position C.2 of Regulatory Guide 1.99, Revision 2, describes the methodology for calculating the adjusted reference temperature and Charpy upper-shelf energy of reactor vessel beltline materials using surveillance capsule data. The methods of Position C.2 can only be applied when two or more credible surveillance data sets become available from the reactor in question.

To date there have been three surveillance capsules removed from Millstone Unit 2. To use these surveillance data sets, they must be shown to be credible. In accordance with Regulatory Guide 1.99, Revision 2, there are five requirements that must be met for the surveillance data to be judged credible. The purpose of this evaluation is to apply these credibility requirements to the reactor vessel surveillance data obtained from Millstone Unit 2 and determine if these surveillance data sets are credible.

EVALUATION

Criterion 1: Materials in the capsules should be those judged most likely to be controlling with regard to radiation embrittlement. The beltline region of the reactor vessel is defined in Appendix G to 10 CFR Part 50, "Fracture Toughness Requirements", December 19, 1995 to be:

"the reactor vessel (shell material including welds, heat affected zones, and plates or forgings) that directly surrounds the effective height of the active core and adjacent regions of the reactor vessel that are predicted to experience sufficient neutron radiation damage to be considered in the selection of the most limiting material with regard to radiation damage."

The Millstone Unit 2 reactor vessel consists of the following beltline region materials:

- Intermediate Shell Plate C-506-1, -2, -3 (Heats C-5843-1, -2, -3)
- Lower Shell Plate C-506-1, -2, -3 (Heats C-5667-1, -2, -3)
- Intermediate to Lower Girth Seam 9-203 (Heats 10137 and 90136)
- Intermediate and Lower Shell Longitudinal Seams 2-203 and 3-203 (Heat A-8746)

The Millstone Unit 2 surveillance programs was based on ASTM E-185-70 and utilizes test specimens from Lower Shell Plate C-506-1 and Intermediate to Lower Girth Seam 9-203 Heat # 10137/90136 Flux Type Linde 0091.

At the time when the surveillance program material was selected it was believed that copper and phosphorus were the elements most important to embrittlement of reactor vessel steels. Lower Shell Plate C-506-1 had the highest Copper content (0.15%) and one of the highest Initial RT_{NDT} values and was selected as the surveillance program base metal. It should be noted that the lower shell plate C-506-1 is currently the limiting beltline material on the Millstone Unit 2 Reactor Vessel.

The Intermediate to Lower Girth Seam 9-203 was fabricated from 2 heats of weld wire; Heat # 10137 (OD of weld) and 90136 (ID of weld). The surveillance weld was made from the same heat and the same manner as the Intermediate to Lower Shell Girth Weld, i.e. 2 heats. The average Cu/Ni between the two heats would be 0.25 Cu and 0.06 Ni, which is the highest Cu value of all beltline welds and was therefore selected as the surveillance program weld material.

Since the base metal and weld metal selected for the Millstone Unit 2 Surveillance Program represent the limiting plate and weld in the beltline region, this criterion is met.

Criterion 2: Scatter in the plots of Charpy energy versus temperature for the irradiated and unirradiated conditions should be small enough to permit the determination of the 30 ft-lb temperature and upper shelf energy unambiguously.

Plots of Charpy energy versus temperature for the unirradiated and irradiated condition are presented in Appendix C of this calcnote.

Based on engineering judgment, the scatter in the data presented in these plots is small enough to permit the determination of the 30 ft-lb temperature and the upper shelf energy of the Millstone Unit 2 surveillance materials unambiguously. Therefore, the Millstone Unit 2 surveillance program meets this criterion.

Criterion 3: When there are two or more sets of surveillance data from one reactor, the scatter of ΔRT_{NDT} values about a best-fit line drawn as described in Regulatory Position 2.1 normally should be less than 28°F for welds and 17°F for base metal. Even if the fluence range is large (two or more orders of magnitude), the scatter should not exceed twice those values. Even if the data fail this criterion for use in shift calculations, they may be credible for determining decrease in upper shelf energy if the upper shelf can be clearly determined, following the definition given in ASTM E185-82.

The functional form of the least squares method as described in Regulatory Position 2.1 will be utilized to determine a best-fit line for this data and to determine if the scatter of these ΔRT_{NDT} values about this line is less than 28°F for welds and less than 17°F for the plate.

Following is the calculation of the best fit line as described in Regulatory Position 2.1 of Regulatory Guide 1.99, Revision 2.

Table D-1
Millstone Unit 2 Chemistry Factors Based on Surveillance Data
(Reg. Guide 1.99, Rev. 2, Position 2.1)

Material	Capsule	Capsule $f^{(a)}$	FF $^{(b)}$	$\Delta RT_{NDT}^{(c)}$	FF* ΔRT_{NDT}	FF ²
Lower Shell Plate C-506 Longitudinal (Heat # C-5667-1)	W-97	0.324	0.69	65.75	45.37	0.476
	W-104	0.949	0.99	87.67	86.79	0.98
	W-83	1.74	1.15	119.12	136.99	1.323
Lower Shell Plate C-506 Transverse (Heat # C-5667-1)	W-97	0.324	0.69	90.83	62.67	0.476
	W-83	1.74	1.15	145.78	167.65	1.323
Sum =					499.47	4.578
$CF = \sum(FF * RT_{NDT}) - \sum(FF^2) = (499.47) + (4.578) = 109.1 \text{ } ^\circ\text{F}$						
Intermediate to Lower Girth Seam 9-203 (Heat # 10137 & 90136)	W-97	0.324	0.69	65.93	45.49	0.476
	W-104	0.949	0.99	52.12	51.59	0.98
	W-83	1.74	1.15	56.09	64.50	1.323
Sum =					161.58	2.779
$CF = \sum(FF * RT_{NDT}) - \sum(FF^2) = (161.58) + (2.779) = 58.14 \text{ } ^\circ\text{F}$						

Notes:

(a) f = best estimate fluence values. ($1 \times 10^{19} \text{ n/cm}^2$, $E > 1.0 \text{ MeV}$).

(b) FF = fluence factor = $f^{(0.28 - 0.1 \cdot \log f)}$.

(c) ΔRT_{NDT} values are the measured 30 ft-lb shift values taken from App. C

The scatter of ΔRT_{NDT} values about the functional form of a best-fit line drawn as described in Regulatory Position 2.1 is presented in Table B-2.

Table D-2:
Turkey Point Unit 3 Surveillance Capsule Data Scatter about the Best-Fit Line for
Surveillance Forging Materials.

Material	Capsule	CF ^(a) (Slope _{best fit})	FF ^(b)	$\Delta RT_{NDT}^{(c)}$	ΔRT_{NDT} (°F)	Scatter ΔRT_{NDT} (°F)	<17°F (Base Metals) <28°F (Weld)
Lower Shell Plate C-506 Longitudinal (Heat # C-5667-1)	W-97	109.1	0.69	65.75	75.28	9.53	Yes
	W-104	109.1	0.99	87.67	108	20.33	No
	W-83	109.1	1.15	119.12	125.47	6.35	Yes
Lower Shell Plate C-506 Transverse (Heat # C-5667-1)	W-97	109.1	0.69	90.83	75.28	-15.55	Yes
	W-83	109.1	1.15	145.78	125.47	-20.31	No
Intermediate to Lower Girth Seam 9-203 (Heat # 10137 & 90136)	W-97	58.14	0.69	65.93	40.12	-25.81	Yes
	W-104	58.14	0.99	52.12	57.55	5.43	Yes
	W-83	58.14	1.15	56.09	66.86	10.77	Yes

Notes:

- (a) f = Calculated fluence from capsule W-83 dosimetry analysis results ($\times 10^{19}$ n/cm², $E > 1.0$ MeV) See Section 6.
(b) FF = fluence factor = $f^{(0.28 - 0.1 \cdot \log f)}$.
(c) ΔRT_{NDT} values are the measured 30 ft-lb. shift values (Appendix C) and do not include the adjustment ratio procedure of Reg. Guide 1.99 Revision 2, Position 2.1, since this calculation is based on the actual surveillance weld metal measured shift values

$$\text{Best Fit } \Delta RT_{NDT} = (\text{Slope}_{\text{best fit}}) * (\text{Fluence Factor})$$

CONCLUSION:

Table D-2 indicates that two of the five measured plate ΔRT_{NDT} values are outside the 1σ scatter band. Therefore the plate data is not credible. Table D-2 also indicates that all of the measured weld ΔRT_{NDT} values are within the 1σ scatter band. Therefore the weld data meets this criteria.

Criterion 4: The irradiation temperature of the Charpy specimens in the capsule should match the vessel wall temperature at the cladding/base metal interface within +/- 25°F.

The location of the specimens with respect to the reactor vessel beltline provides assurance that the reactor vessel wall and the specimens experience equivalent operating conditions such that the temperatures will not differ by more than 25°F.

Criterion 5: The surveillance data for the correlation monitor material in the capsule should fall within the scatter band of the database for that material.

The Millstone Unit 2 surveillance program does contain correlation monitor material. According to Table 14 of NUREG/CR-6413^[12], Millstone Unit 2 Correlation Monitor Material for Capsule 104 had a residual value of 10°F, which is less than the ± 34 (2 Sigma) scatter band allowance for plate HSST02 (A533B-1) material (per figure 9 of the NUREG report). Note: The fluence & ΔT_{NDT} has been updated since the issue of the NUREG Report, however, these changes would not cause the Correlation Monitor Material to exceed the scatter band.

CONCLUSION:

Based on the preceding responses to all five criteria of Regulatory Guide 1.99, Revision 2, Section B and 10CFR 50.61, the Millstone Unit 2 surveillance weld data is credible. However due not meeting criterion 3 the Millstone Unit 2 surveillance plate data is not credible.